September 18, 2003

Mr. Mark Peifer Site Vice-President Duane Arnold Energy Center Nuclear Management Company, LLC 3277 DAEC Road Palo, IA 52324

SUBJECT: DUANE ARNOLD ENERGY CENTER NRC EVALUATIONS OF CHANGES, EXPERIMENTS, OR TESTS AND PERMANENT PLANT MODIFICATIONS INSPECTION REPORT 50-331/03-09(DRS)

Dear Mr. Peifer:

On August 29, 2003, the U.S. Nuclear Regulatory Commission (NRC) completed a routine baseline inspection at your Duane Arnold Energy Center. The enclosed report documents the inspection findings, which were discussed on August 29, 2003, with you and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Specifically, this inspection focused on the baseline biennial inspections for evaluations of changes, tests, or experiments (10 CFR 50.59) and permanent plant modifications. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be available electronically for public inspection in the NRC Public Document Room or from the Publically Available Records (PARS) component

M. Peifer

of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <u>http://www.nrc.gov/reading-rm/adams.html</u> (the Public Electronic Reading Room).

Sincerely,

/RA/

Julio F. Lara, Chief Electrical Engineering Branch Division of Reactor Safety

Docket No. 50-331 License No. DPR-49

- Enclosure: Inspection Report 50-331/03-09(DRS) w/Attachment: Supplemental Information
- cc w/encl: E. Protsch, Executive Vice President -Energy Delivery, Alliant; President, IES Utilities, Inc.
 - J. Cowan, Chief Nuclear Officer
 - J. Forbes, Senior Vice President
 - J. Bjorseth, Plant Manager
 - S. Catron, Manager, Regulatory Affairs
 - J. Rogoff, Esquire General Counsel
 - B. Lacy, Nuclear Asset Manager
 - D. McGhee, Iowa Department of Commerce
 - Chairman, Linn County Board of Supervisors
 - State Liaison Officer
 - Chairperson, Iowa Utilities Board
 - The Honorable Charles W. Larson, Jr.
 - Iowa State Representative

M. Peifer

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Iowa State Representative

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No:	50-331
License No:	DPR-49
Report No:	50-331/03-09(DRS)
Licensee:	Alliant, IES Utilities, Inc.
Facility:	Duane Arnold Energy Center
Location:	3277 DAEC Road Palo, Iowa 52324-9785
Dates:	August 25, 2003 through August 29, 2003
Inspectors:	G. Hausman, Senior Reactor Inspector, LeadT. Bilik, Reactor InspectorD. Schrum, Reactor InspectorJ. Panchison, Mechanical Engineering Contractor
Approved by:	Julio F. Lara, Chief Electrical Engineering Branch Division of Reactor Safety

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SUMMARY OF FINDINGS

IR 05000331/2003-009(DRS); IES Utilities, Inc.; 08/25/03 - 08/29/03; Duane Arnold Energy Center; Routine Baseline Inspection Report.

This report covers a five day period of announced baseline inspection on evaluations of changes, tests, or experiments and permanent plant modifications. The inspection was conducted by Region III inspectors and an engineering contractor. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process" (SDP). Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. Inspector-Identified and Self-Revealed Findings

Cornerstone: Initiating Events

No findings of significance were identified.

Cornerstone: Mitigating Systems

No findings of significance were identified.

Cornerstone: Barrier Integrity

No findings of significance were identified.

B. <u>Licensee-Identified Violations</u>

No findings of significance were identified.

REPORT DETAILS

Summary of Plant Status

Unit 1 operated at or near full power throughout the inspection period.

1. **REACTOR SAFETY**

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

.1 <u>Review of 50.59 Evaluations and Screenings</u>

a. <u>Inspection Scope</u>

The inspectors reviewed 10 evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, setpoint changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed 19 screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

b. Findings

No findings of significance were identified.

1R17 <u>Permanent Plant Modifications</u> (71111.17)

- .1 <u>Review of Recent Modifications</u>
- a. Inspection Scope

The inspectors reviewed 10 permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

b. <u>Findings</u>

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems (71152)

- .1 <u>Routine Review of Condition Reports</u>
- a. Inspection Scope

The inspectors reviewed a selected sample of condition reports associated with Duane Arnold Energy Center's permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings. The inspectors reviewed these issues to verify an appropriate threshold for identifying issues and to evaluate the effectiveness of corrective actions. In addition, condition reports written on issues identified during the inspection were reviewed to verify adequate problem identification and incorporation of the problem into the corrective action system. The specific corrective action documents that were sampled and reviewed by the team are listed in the attachment to this report.

b. Findings

No findings of significance were identified.

40A6 Meetings

.1 Exit Meeting

The inspectors presented the inspection results to Mr. Peifer and other members of licensee management at the conclusion of the inspection on August 29, 2003. The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

.2 Interim Exit Meetings

No interim exits were conducted.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

<u>Licensee</u>

- T. Anderson, Maintenance
- J. Bjorseth, Plant Manger
- C. Bleau, Design Engineering (Electrical/I&C Team Supervisor)
- D. Brigl, Systems Engineering
- S. Catron, Regulatory Affairs Manager
- E. Christopher, Program Engineering
- P. Collingsworth, Systems Engineering
- D. Curtland, Engineering Director
- S. Haller, Systems Engineering Manager
- R. Harter, Performance Improvement Manager
- B. Hopkins, Program Engineering
- D. Miller, Employee Concerns Manager
- K. Morgan, Nuclear Assessment Manager
- B. Murrell, Regulatory Affairs/Licensing
- A. Nebola, Training
- M. Peifer, Site Vice President
- J. Probst, Program Engineering
- W. Simmons, Maintenance Manager
- J. Stutzman, Training
- L. Swenzinski, Regulatory Affairs/Licensing
- M. Wood, Design Engineering (Mechanical Team Supervisor)

Nuclear Regulatory Commission

- S. Caudill, Resident Inspector
- D. Hills, Region III Mechanical Engineering Branch Chief
- G. Wilson, Senior Resident Inspector

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

<u>Opened</u>

None.

<u>Closed</u>

None.

Discussed

None.

LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety but rather that selected sections of portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

CALCULATIONS

Document Number	Title or Description	Revision or Date
CAL-VC8-009	Drywell Cooling Load Calculation	0
MC-25E	Develop FW System Curve	January 12, 1974

CORRECTIVE ACTION DOCUMENTS GENERATED DURING THE INSPECTION

Document Number	Title or Description	Revision or Date
CAP 028802	10CFR50.59 Safety Evaluation 99-041, RHR Pump	August 27, 2003
	Seal Water Coolers Not Required at DAEC, Should	
	Be Reviewed for Possible Revision	
CAP 028847	50.59 Screening Records Were Not Stand Alone	August 28, 2003
	Documents	-
CAP 028848	OI-416, Section 10 May Need Precaution Statement	August 28, 2003
CAP 028849	The Corrective Action Program's Method of Links	August 28, 2003
	and Daughter Documents Made it Very Difficult to	-
	Assess What Activities Had Been Completed	
CAP 028851	Evaluate Replacement of KY4201 and KY4202	August 29, 2003

CORRECTIVE ACTION DOCUMENTS GENERATED PRIOR TO INSPECTION

Document Number	Title or Description	Revision or Date
AR 3025	RHRSW Strainer High dp	August 6, 2002
CAP 011318	UFSAR Change Requests for Plant Modifications	August 13, 2001
CAP 011327	NRC Inspection Found Overlap of ECP Scopes	August 16, 2001
CAP 011328	ESW Flow to RHR Pump Seal Coolers	August 16, 2001
CAP 011329	SE for Removal of HPCI/RCIC Auto Open Logic	December 14, 2002
CAP 011330	Inadequate Engineering Evaluations Identified for	August 16, 2001
	the Majority of Field Variances Written for ECP 1622 and ECP 1604	-
CAP 011464	Reactor Building Sump Maintenance Rule 10 CFR 50.59	August 29, 2001
CAP 011468	Form an Engineering Team to Identify Design Basis Calculations, Modifications, and Programs That Pertain to RHR, RHRSW, ESW, and EDG Systems	August 24, 2001
CAP 011478 CAP 012068	Review of Revised 50.59 Process Implementation Inadequate Followup Action for 10 CFR 50.59 Evaluation 01-023	September 4, 2001 November 14, 2001

CORRECTIVE ACTION DOCUMENTS GENERATED PRIOR TO INSPECTION

Document Number	Title or Description	Revision or Date					
CAP 012462	10 CFR 50.59 Evaluation 01-026 (spTP 202 PUP	January 10, 2002					
	Power Ascension Testing) Requires Revision						
CAP 013469	Insure That 10CFR50.59 Screening and a	May 20, 2002					
	10 CFR 50.59 Evaluation Are Completed						
CAP 014503	Temporary Modification 02-059 Did Not Address the	October 1, 2002					
	Seismic Qualification of a Jumper Cable Installation						
CAP 014525	Error in Developing Temporary Modification 02-061	October 4, 2002					
	Delayed Exit from a Core Spray LCO						
CAP 014710	Inadequate 10 CFR 50.59 Screening to Change	November 14, 2002					
	Modes of Closed Operation in the Prerequisites						
CAP 020061	RHRSW Strainer High dp While Running A and C	August 6, 2002					
	RHRSW Pumps						
CAP 025226	Administrative Clarification on 50.59 Screen	January 16, 2003					
	Questions w/Pre-existing NRC SER						
CAP 025315	Using 10 CFR 50.59 Screenings Written for	January 28, 2003					
	EMAs/DDCs for Procedure Changes						
CAP 026474	DCF Approved by Owner and OSS (OI 149) Without	March 28, 2003					
	a 10 CFR 50.59 Screening						
CAP 027142	Focused Area Self-Assessment of 10 CFR 50.59	April 22, 2003					
	Process						
CAP 027189	10 CFR 50.59 Screening Course Exam	April 24, 2003					
040 007440	Administered with Invalid Exam Questions	May 40, 0000					
CAP 027440	AOV Reliability Improvement Mod to Install Flexible	May 16, 2003					
	Air Lines at Valve	May 47,0000					
CAP 027457	Trend of Improperly Performed Mods to the DAEC	May 17, 2003					
CAP 027594	No Show for 10 CFR 50.59 Screening Class (60030)	May 28, 2003					
CAP 028265	Applicability of 10 CFR 50.59 and NMC/DAEC	July 17, 2003					
	Supply Chain Procedures	-					

DRAWINGS

Document Number	Title or Description	Revision or Date
BECH-M129	River Water Supply System	July 14, 2003

EVALUATIONS

Document Number	Title or Description	Revision or Date
SE 97-113	Install Flush Connections on the Piping to and from	March 17, 1998
	the Reactor Building Equipment Drain Sump Heat	
	Exchanger and Install an Isolation Valve on the	
	Drain Line from this Unit	

EVALUATIONS

Document Number	Title or Description	Revision or Date
SE 98-014	UFSAR Change Request 98-122 revised the RWCU	December 18, 1998
	System Supply Isolation Valves MO2700 & MO2701	
	Design Closure Times from 20 Seconds to	
	22 Seconds	
SE 98-075	BECH-M133 Updated to Remove HS3301A From	July 31, 1998
	the Control Circuit of 1P048	
SE 99-041	ESW to RHR Pump Seal Coolers	January 1, 1900
SE 99-058	SE for Extending Fire Pump LCO Time	December 21, 1999
SE 01-025	ECP 1628, Reactor Building Vent Shaft and Control	November 20, 2001
	Building Air Intake Radiation Monitor Replacement	
SE 01-026	Cycle 18 Power Ascension Test to 1790 Mw _t	January 29, 2002
SE 02-001	Revise Qual SC-100 & SC-101, EQ Documentation	May 28, 2002
SE 02-002	Operation at 100 percent Power with RHRSW	August 10, 2002
	Strainer Bypassed	
SE 03-001	River H ₂ O Supply Valves, Safety Related Seismic	June 17, 2003
	Boundary (V-29-42 & 43 to Become Normally Open)	

MODIFICATIONS

Document Number	<u>Title or Description</u>	Revision or Date
ECP 1616	MO2046 & MO1947 Control Logic Modification	April 24, 2000
ECP 1640	Circulating Water Pumps	
ECP 1648	Replace Reactor Building Vent Shaft/Control	November 28, 2001
	Building Air Intake Radiation Monitoring System with	
	Digital Radiation Monitoring System	
ECP 1650	Drywell Cooler Replacement	April 25, 2003
ECP 1661	Install Oil Lie off of Compressor 1K003	
EMA A36191	Add a Connection with an Isolation Valve to the Inlet	April 21, 2000
	Line from the Reactor Building Equipment Drain	
	Sump to the 1E033 Heat Exchanger	
EMA A45051	Decommission the Relative Humidity Indication	July 30, 2001
	Instrument Loops for the a SBGT Train	
EMA A45791	Replace 64D10 Ground Replay with Magnacraft	
EMA A53462	Provide Permanent Power Supply to the Heat	October 24, 2002
	Tracing on the off Gas after Filter Vent Line	
EMA A57819	Butterfly Valve Installed Will Provide Better Sealing	June 5, 2002
	than the Sluice Gate Being Replaced	

PROCEDURES

Document Number	Title or Description	Revision or Date
ACP 103.0	Design Control Program	14
ACP 103.2	Safety Evaluation Applicability Review Process	14
ACP 103.2	10 CFR 50.59 Screening Process	19
ACP 103.3	Safety Evaluation Process	10

PROCEDURES

PROCEDURES		
	Title or Description	Revision or Date
ACP 103.3	10 CFR 50.59 Evaluation Process	16
ACP 109.0	Engineering Change Packages	28
ACP 109.1	Engineered Maintenance Action	26
ACP 1203.07	External Design Interfaces	6
ACP 1203.11	Design Input Requirements	12
ACP 1203.21	Engineering Calculations	13
ACP 1203.23	Engineering Drawings/Documents	15
ACP 1203.25	Engineering Specifications	5
ACP 1203.26	Engineering Acceptance Requirements	8
ACP 1203.31	Design Verification	10
ACP 1203.32	Second Level Review and Acceptance of Procured Modifications	9
ACP 1203.42	Field Variances	10
ACP 1203.43	Field Change Notice	9
ACP 1203.45	ECP Closure Activities	18
ACP 1203.50	Engineering Programs/Checklists	0
ACP 1203.51	R Stamp Applicability	1
ACP 1203.52	Assessment of Potential Impact on EOPs/SAGs	0
ACP 1203.53	Fire Protection	4
ACP 1203.54	ALARA Design Considerations	1
ACP 1203.55	Control of Heavy Loads Analysis	1
ACP 1203.56	Environmental Qualification Assessment	3
ACP 1203.57	Inservice Inspection Checklist	0
ACP 1203.58	Human Factors Considerations	1
ACP 1203.59	Power System Configuration and Analysis	2
ACP 1203.60	Masonry Wall Analysis	0
ACP 1203.61	Inservice Testing Checklist	0
ACP 1203.62	Setpoint Control	3
ACP 1203.63	PRA Design Change Notification	0
ACP 1203.64	Regulatory Guide (Reg Guide) 1.97	0
ACP 1203.66	Station Blackout Applicability	0
ACP 1203.80	Capacitor Replacement Evaluation	1
ACP 1203.81	Abandoned in Place Equipment	0
OI-416	Operating Instruction - RHR Service Water System	31

REFERENCES

Document Number	Title or Description	Revision or Date
	System Importance Ranking Results and Important	June 2001
	Human Actions from the DAEC Living PRA	
	Training Summary Printout by Course from	August 22, 2003
	01/01/1990 Through 08/21/2003 for	-
	Course: PST1.14, Perf/Rev 10 CFR 50.59	
	Screening	

REFERENCES

Document Number	Title or Description	Revision or Date
	Training Summary Printout by Course from	August 22, 2003
	01/01/1990 Through 08/21/2003 for	
	Course: PST9.27, Perf/Verify 10 CFR 50.59 Eval	
Amendment No. 243 DAEC - Issuance of Amendment Regarding November 6, 2001		November 6, 2001
	Extended Power Uprate (TAC No. MB0543)	
CENPD-397-P	Improved Flow Measurement Accuracy Using	1
	Crossflow Ultrasonic Flow Measurement	
	Technology January 2000 (Proprietary Information)	
GE-NE-A22-00100-	Project Task Report Duane Arnold Energy Center	January 2, 1900
23-01-R2	Asset Enhancement Program	
	Task T0400: Containment System Response	
	December 2002 (Proprietary Information)	
NG-00-1900	DAEC Docket No: 50-331 Op. License No: DPR-49	November 16, 2000
	Technical Specification Change Request	
	(TSCR-042): Extended Power Uprate	

SCREENINGS

Document Number	Title or Description	Revision or Date
4	Change APRM Flow Biased Setpoints for Power	December 4, 2001
33	Uprate per EMA A52240 Revise Document APED-A61-089, SBO Compliance Assessments & Issue APED-A61-091 to Incorporate Power Uprate Analysis Results into the SBO Program Documentation	August 8, 2001
83	Add Air Assist to CV2851-O in the Closing Direction to Increase the Closing Force Applied to CV2851	October 25, 2001
84	Add Air Assist to CV2850-O in the Closing Direction to Increase the Closing Force Applied to CV2850	October 25, 2001
229	Change Terminology in the UFSAR Associated with 10CFR50.59 to be Consistent with the Revised Rule	September 25, 2001
243	Trip of Main Turbine on Loss of 125 Vdc	September 30, 2001
284	Revise ACP 103.1 for PUP, Clarification of 50.59 Screening Applicability, and SLO Safety Limit MCPR	November 20, 2001
385	Revise Stroke Times for CV2315 in STP 3.5.1-05, 3.5.1-06386	October 22, 2001
484	Added Steps to Start/stop RHR Pump Room Cooling Units, Modified Steps Changing RHR Injection Points During SDC and Added Subsection to Return System to Original Injection Point. Added Information That Total FPC Flow May Be Increased up to 2400 gpm	November 9, 2001
496	Revise BECH-M175 to Change Max dp Across the Outside Air Intake Filter from 1.8 in Wg to 2.0 in Wg	November 14, 2001

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SCREENINGS

Document Number	Title or Description	Revision or Date
532	Change Feedpump Low Pressure Suction Trip	November 20, 2001
	Setpoints from 250 psi to 225 psi	
598	Change Required As Left Diesel Cooler dp to	December 12, 2001
	Greater Than or Equal to 69.3 Inches of Water	
605	Cancel STPs, Revision 16 of the Offsite Dose	December 13, 2001
	Assessment Manual, No Longer Has a Requirement	
	to Do Surveillances on the 1T-269 Level	
	Instruments	_
649	Allow For Operation of Three RBCCW Pumps in All	December 27, 2001
	Reactor Modes	
1685	Revision to IPOI-3	October 2, 2002
1839	Adds P&LS for Pump Flows and Strainer Venting.	November 16, 2002
	Also Changes SW Flow Rates for Pump Protection	
2012	Install 0.109 Orifices in the P Port of the FASV's for	January 15, 2003
	Main Steam Valves	
2159	Address Issues Identified During NRC Inspection on	February 16, 2003
	Smoke in Control Room	
2514	Drywell Cooling Trends Indicate That All Drywell	May 27, 2003
	Coolers May Not Be Needed (Isolation of Non-	
	essential Coolers)	

LIST OF ACRONYMS USED

ADAMSAgency-Wide Document Access and Management SystemALARAAs Low As is Reasonably AchievableAOVAir Operated ValveAPEDAdministrative Procedure Engineering DirectiveAPRMAverage Power Range MonitorARAction RequestBECHBechtelCAPCorrective Action ProgramCFRCode of Federal RegulationsCVControl ValveDAECDuane Arnold Energy CenterDPRDemonstration Power ReactorDRSDivision of Reactor SafetyECPEngineering Change PackageDCFDocument Change FormDDCDesign Document ChangeEDGEmergency Diesel GeneratorEMAEngineered Maintenance ActionEOPFast Acting Solenoid ValveFPCFuel Pool CoolingFWFeedwatergpmgallons per minutegovGovermentHPCIHigh-Pressure Coolant InjectionHSHand SwitchhtmlHypertext Markup LanguagehttpHypection Manual ChapterIPEIndividual Plant EvaluationIRInspection ReportLCOLimiting Condition of OperationLLCLimiting Condition of OperationLLCLimited Liability CompanyMCPRMinimum Critical Power RatioNRCNuclear Reactor RegulationNRCNuclear Reactor RegulationNRCNuclear Reactor SystemPRAProbabilistic Risk Assessment	ACP	Administrative Control Procedure
AOVAir Operated ValveAPEDAdministrative Procedure Engineering DirectiveAPRMAverage Power Range MonitorARAction RequestBECHBechtelCAPCorrective Action ProgramCFRCode of Federal RegulationsCVControl ValveDAECDuane Arnold Energy CenterDPRDemonstration Power ReactorDRSDivision of Reactor SafetyECPEngineering Change PackageDCFDocument Change FormDDCDesign Document ChangeEDGEmergency Diesel GeneratorEMAEngineered Maintenance ActionEOPEssential Service WaterFASVFast Acting Solenoid ValveFPCFuel Pool CoolingFWFeedwatergpmgallons per minutegovGovermentHPCIHigh-Pressure Coolant InjectionHSHand SwitchhtmlHypertext Transfer ProtocolI&CInstrumentation and ControlINCInspection ReportLCOLimiting Condition of OperationILCLimited Liability CompanyMCPRMinimum Critical Power RatioNMCNuclear Regulatory CommissionNRROffice of Nuclear Reactor RegulationNRROffice of Nuclear Reactor RegulationNRAOffice of Nuclear Reactor RegulationNRAOffice of Nuclear Reactor RegulationNRAOffice of Nuclear Reactor RegulationNRAOffice of Nuclear Reactor Regulation<		
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PARS Publically Available Records System		
	OSS	Operations Shift Supervisor
PRA Probabilistic Risk Assessment	PARS	Publically Available Records System
	PRA	Probabilistic Risk Assessment

LIST OF ACRONYMS USED

PUP	Power Uprate
RBCCW	Reactor Building Closed Cooling Water
RHR	Residual Heat Removal
RHRSW	Residual Heat Removal Service Water
RCIC	Reactor Core Isolation Cooling
RWCU	Reactor Water Clean-Up
SAG	Severe Accident Guideline
SBGT	Standby Gas Treatment
SBO	Station Black Out
SDC	Shutdown Cooling
SDP	Significance Determination Process
SE	Safety Evaluation
SER	Safety Evaluation Report
SLO	Single-Loop Operation
STPS	Surveillance Test Procedures
SW	Service Water
TSCR	Technical Specification Change Request
UFSAR	Updated Final Safety Analysis Report
V	Valve or Volt
V	Valve or Volt
wpd	WordPerfect Document
www	World Wide Web