



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
SAM NUNN ATLANTA FEDERAL CENTER  
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March 4, 2003

Carolina Power & Light Company  
ATTN: Mr. James Scarola  
Vice President - Harris Plant  
Shearon Harris Nuclear Power Plant  
P. O. Box 165, Mail Code: Zone 1  
New Hill, North Carolina 27562-0165

SUBJECT: ANNUAL ASSESSMENT LETTER - SHEARON HARRIS NUCLEAR POWER  
PLANT (NRC INSPECTION REPORT 50-400/03-01)

Dear Mr. Scarola:

On February 5, 2003, the NRC staff completed its end-of-cycle plant performance assessment of the Shearon Harris Nuclear Power Plant. The end-of-cycle review for Harris involved the participation of all technical divisions in evaluating performance indicators (PIs) for the most recent quarter and inspection results for the period from January 1 through December 31, 2002. The purpose of this letter is to inform you of our assessment of your safety performance during this period and our plans for future inspections at your facility so you will have an opportunity to prepare for these inspections and to inform us of any planned inspections which may conflict with your plant activities.

Overall, Harris operated in a manner that preserved public health and safety and fully met all cornerstone objectives. Plant performance for the most recent quarter of the assessment period was in the Regulatory Response Column of the Action Matrix due to one White finding in the mitigating systems cornerstone. The finding dealt with foreign material in the containment sump suction line to the A residual heat removal pump that would have degraded pump performance if recirculation flow was required. This finding was identified as White in the second quarter 2002. As stated in our Mid-Cycle Letter, dated August 26, 2002, a Supplemental Inspection conducted in July 2002 determined that the problem identification, root cause, and extent of condition evaluations for this White finding were adequate.

In addition, nine unresolved items were identified during a December 2002 triennial fire protection inspection and are under review as part of the Significance Determination Process. Supplemental inspections may be warranted if these issues are determined to be of greater than very low safety significance (Green). Also, one unresolved item discussed in our Mid-Cycle Letter involving the adequacy of Hemyc/MT cable wrap fire barrier qualification tests and installed configurations continues to be reviewed.

While plant performance for the most recent quarter was within the Regulatory Response Column of the Action Matrix, there was an additional safety significant inspection finding during the first three quarters of the assessment cycle. As discussed in the Mid-Cycle letter, in the fourth quarter 2001, a White finding was identified dealing with a degraded fire barrier separating the B train

switchgear room/auxiliary control panel room and the A train cable spreading room. This finding, combined with the foreign material finding discussed above resulted in Harris being in the Degraded Cornerstone Column of the Action Matrix during the second and third quarters of 2002. A Supplemental Inspection was completed using Inspection Procedure (IP) 95001 for the fire barrier issue on September 6, 2002. Subsequent to this inspection, the staff determined that physical fire protection changes had reduced the risk significance of the degraded fire barrier.

In addition, on April 16, 2002 the staff issued a Severity Level III Notice of Violation in accordance with the NRC Enforcement Policy. This violation was identified collectively for two fire protection noncompliances. One involved the fire protection White finding discussed above and the other involved a change to the fire protection program, associated with the White finding, made without prior NRC approval. This violation was reviewed during the December 2002 triennial fire protection inspection. Further review is necessary to close the issue.

There was also a safety significant PI during the first three quarters of the assessment cycle. The Unplanned Scrams per 7000 Critical Hours PI crossed the White threshold during the third quarter 2002 due to three reactor trips that occurred during the first three quarters of 2002. The PI recrossed the threshold to Green during the fourth quarter 2002. A Supplemental Inspection using IP 95001 was completed satisfactorily on December 15, 2002.

The enclosed inspection plan details the inspections scheduled through March 31, 2004. In addition to baseline inspection activities, the plan currently includes a review of your implementation of the interim compensatory measures required by the NRC Security Order dated February 25, 2002, and a review of your activities in response to NRC Bulletin 2002-02, "Reactor Pressure Vessel (RPV) Head and Vessel Head Penetration (VHP) Nozzle Inspection Programs." The inspection plan is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of inspector arrival onsite. Routine resident inspections are not listed due to their ongoing and continuous nature. The inspections in the last six months of the inspection plan are tentative and may be revised at the mid-cycle review meeting.

As you are aware, since the terrorist attacks on September 11, 2001, the NRC has issued several orders and threat advisories to enhance security capabilities and improve guard force readiness. We have conducted inspections to review your implementation of these requirements and have monitored your actions in response to changing threat conditions. For 2003, we plan to continue these inspections, conduct portions of the security baseline inspection program, as well as perform additional inspections to evaluate your compliance with new requirements that may be ordered. Based on our final determinations in this regard, we will notify you of any inspection plan changes.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

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If circumstances arise which cause us to change the inspection plan, we will contact you to discuss the change as soon as possible. Please contact Mr. Paul Fredrickson at 404-562-4530 with any questions you may have regarding this letter or the inspection plan.

Sincerely,

*/RA/*

Loren R. Plisco, Director  
Division of Reactor Projects

Docket No. 50-400  
Licensee No. NPF-63

Enclosure: Harris Inspection/Activity Plan

cc w/encl:(See page 4)

CP&L

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cc w/encl:

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**Harris**  
**Inspection / Activity Plan**  
**03/01/2003 - 03/31/2004**

Unit Number	Inspection Activity	Title	No. of Staff on Site	Planned Dates		Inspection Type
				Start	End	
	<b>OL EP</b>	<b>- EMERGENCY EXERCISE</b>	<b>2</b>			
1	IP 7111401	Exercise Evaluation		03/03/2003	03/07/2003	Baseline Inspections
1	IP 7111404	Emergency Action Level and Emergency Plan Changes		03/03/2003	03/07/2003	Baseline Inspections
1	IP 71151	Performance Indicator Verification		03/03/2003	03/07/2003	Baseline Inspections
	<b>PS-SEC</b>	<b>- SECURITY - TI/ICM</b>	<b>3</b>			
1	IP 2515/148	Inspection of Nuclear Reactor Safeguards - Interim Compensatory Measures		04/07/2003	04/11/2003	Safety Issues
	<b>EB2-HS</b>	<b>- HEAT SINK</b>	<b>2</b>			
1	IP 7111107B	Heat Sink Performance		04/14/2003	04/18/2003	Baseline Inspections
	<b>PS-RP</b>	<b>- RADIATION PROTECTION</b>	<b>3</b>			
1	IP 7112101	Access Control to Radiologically Significant Areas		04/28/2003	05/02/2003	Baseline Inspections
1	IP 7112102	ALARA Planning and Controls		04/28/2003	05/02/2003	Baseline Inspections
1	IP 7112202	Radioactive Material Processing and Transportation		04/28/2003	05/02/2003	Baseline Inspections
1	IP 71151	Performance Indicator Verification		04/28/2003	05/02/2003	Baseline Inspections
	<b>2515/150</b>	<b>- 2515/150 REV1</b>	<b>1</b>			
1	IP 2515/150	Reactor Pressure Vessel Head and Vessel Head Penetration Nozzles (NRC Bulletin 2002-02)		05/05/2003	05/09/2003	Safety Issues
	<b>EB2-ISI</b>	<b>- INSERVICE INSPECTION-U1</b>	<b>1</b>			
1	IP 7111108P	Inservice Inspection Activities - PWR		05/12/2003	05/16/2003	Baseline Inspections
	<b>PS-RP</b>	<b>- RADIATION PROTECTION</b>	<b>2</b>			
1	IP 7112101	Access Control to Radiologically Significant Areas		05/12/2003	05/16/2003	Baseline Inspections
1	IP 7112102	ALARA Planning and Controls		05/12/2003	05/16/2003	Baseline Inspections
1	IP 7112202	Radioactive Material Processing and Transportation		05/12/2003	05/16/2003	Baseline Inspections
1	IP 71151	Performance Indicator Verification		05/12/2003	05/16/2003	Baseline Inspections
	<b>OL RQ</b>	<b>- REQUAL SIMULATOR INSPECTION</b>	<b>1</b>			
1	IP 7111111B	Licensed Operator Requalification		06/09/2003	06/12/2003	Baseline Inspections
	<b>PI&amp;R</b>	<b>- PROBLEM IDENTIFICATION &amp; RESOLUTION</b>	<b>3</b>			
1	IP 71152B	Identification and Resolution of Problems		07/07/2003	07/11/2003	Baseline Inspections
1	IP 71152B	Identification and Resolution of Problems		07/21/2003	07/25/2003	Baseline Inspections
	<b>OL PREP</b>	<b>- INITIAL EXAM PREP</b>	<b>2</b>			
1	V23211	HARRIS/EXAMS AT POWER FACILITIES		02/02/2004	02/06/2004	Not Applicable
	<b>OL EXAM</b>	<b>- INITIAL EXAM</b>	<b>2</b>			
1	V23211	HARRIS/EXAMS AT POWER FACILITIES		02/16/2004	02/27/2004	Not Applicable
	<b>EB1SSDPC</b>	<b>- SSDPC PRE-INSPECTION VISIT</b>	<b>2</b>			
1	IP 7111121	Safety System Design and Performance Capability		03/01/2004	03/05/2004	Baseline Inspections

This report does not include INPO and OUTAGE activities.  
This report shows only on-site and announced inspection procedures.

Harris  
Inspection / Activity Plan  
03/01/2003 - 03/31/2004

Unit Number	Inspection Activity	Title	No. of Staff on Site	Planned Dates Start End		Inspection Type
<b>EB1SSDPC - SAFETY SYS DESIGN &amp; PERFORMANCE CAPABILI</b>						5
1	IP 7111121	Safety System Design and Performance Capability		03/22/2004	03/26/2004	Baseline Inspections

This report does not include INPO and OUTAGE activities.  
This report shows only on-site and announced inspection procedures.