BSC

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Attachments		Total Number of Pages
Attachment A.	Event Trees	82
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DISCLAIMER

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1	PURPOSE	Norman Graves	[signature] 3/11/08
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7	RESULTS AND CONCLUSIONS	Doug Orvis	[signature] 3/11/08
Att A	EVENT TREES	Norman Graves	[signature] 3/11/08

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Att C	ACTIVE COMPONENT RELIABILITY DATA ANALYSIS	Erin Collins	3/11/08
Att D	PASSIVE EQUIPMENT FAILURE ANALYSIS	Dan Christman (Sections D1 and D3) David Bradley (Section D2)	3/11/08 3/11/08
Att E	HUMAN RELIABILITY ANALYSIS	Paul Amico Mary Presley Erin Collins Doug Orvis	3/11/08 3/11/08 3/11/08 3/11/08
Att F	FIRE ANALYSIS	Paul Amico & Laura Plumb under supervision of Paul Amico	3/11/08 3/11/08
Att G	EVENT SEQUENCE QUANTIFICATION SUMMARY TABLE	Jeff Marr	3/11/08
Att H	SAPHIRE Model and Supporting Files	Norman Graves	3/11/08

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Andrew Burningham Amy Primmer ~~William Chris Allen~~ 3/12/08	3/11/08 Amy E. Primmer 3/11/08	Section 1-7 Attachments ~~B, C, D, E, G, H~~ A THRU H ~~Attachments A and F~~	Administrative check 3/11/08	Perform checks on the Calculations and Analyses – Checklist (Attachment 6 to EG-PRO-3DP-G04B-00037 that are administrative in nature (e.g., format, procedural compliance, links in InfoWorks, DIRS, reference format, document numbering, confirmation of SAPHIRE validation, tracking number, etc.)
Alex Deng	03/11/08	Sections 1, 3, 4, and 7	Overall approach and methodology	Check that the standard approach and methodology includes changes to the methodology resulting from input from industry reviewers.
Phuoc Le / Dan Gallagher	3/11/08	Section 6.0 through 6.8 and Attachments A through H	Cut set check	Cut Set Check - Section 6.0 - 6.8 and Attachments A - H
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Dan Christman	[signature] 3/11/08	Section 6.5 and Attachment F	Specialty check: Fire Initiating Events	Fire Initiating Events - Section 6.5 and Attachment F
Doug Orvis	[signature] 3/11/08	Section 6.0	Specialty check: Section 6.0	Initiating Event Screening - Section 6.0
Laura Plumb	[signature] 3/11/08	Section 6.3.3 Miscellaneous Data	Specialty check	Check Section 6.3.3 and supporting reference and cross-references to other sections
Ching Chan	[signature] 3/11/2008	Attachment B-1 System Pivotal Events Analyses - Fault Tree Analysis - Site Prime Mover	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date
Stefhan Sherman	[signature] 3/11/08	Attachment B-2 System Pivotal Events Analyses - Fault Tree Analysis - Cask Transfer Trolley	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date

Checker	Signature/Date	Section	Type of Check	Detailed Scope of Check
M.J. Rubano For Ekachai Danupatampa	Mary Jane Rubano 3.12.08	Attachment B-3 System Pivotal Events Analyses - Fault Tree Analysis – Loading/Unloading Room Shield Door And Slide Gate	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date
Chris Hicks For Freddie Guerrero	Chris Hicks 3/11/08	Attachment B-4 System Pivotal Events Analyses - Fault Tree Analysis – Canister Transfer Machine	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date
Stephen Skochko for Karim Vakhshoori	S Sk For Karim Vakhshoori 03/11/08	Attachment B-5 System Pivotal Events Analyses - Fault Tree Analysis – Horizontal Cask Tractor and Trailer	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date
Len Swanson	Leonard Swanson 3/11/08	Attachment B-6 System Pivotal Events Analyses - Fault Tree Analysis Site Transporter	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date
Nasser Dehkordi For Ajit Hiranadani	[illegible] 3/12/08	Attachment B-7 System Pivotal Events Analyses - Fault Tree Analysis – Heating Ventilation and Air Conditioning	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to

Checker	Signature/Date	Section	Type of Check	Detailed Scope of Check
				date
Nohemi Brewer	[signature] 3/11/08	Attachment B-8 System Pivotal Events Analyses - Fault Tree Analysis AC Power	Design concurrence	Check fault tree description. Is design accurately described and do all basic events have basis in latest issued for LA information? Are success criteria accurate? Are basic events clearly phrased? Are the references to Engineering documents correct and up to date
Dan Christman	[signature] 3/11/08	Attachment C	Specialty check	Check Attachment C including the MathCad file for Bayesian update of reliability values
Doug Smith Stephen Skochko for Karim Vakhshoori Stephen Skochko	[signature] 3-11-08 [signature] For Karim Vakhshoori 03/11/08 [signature] 3/11/08	Attachment C	Detailed references and numerical inputs	This check traced input data back to references for Attachment C
Dan Christman	[signature] 3/11/08	Attachment D	Specialty check	Check Sections D 2, 6.3.2.3 and 6.3.2.4.
David Bradley	[signature] 3/11/08	Attachment D	Specialty check	Check Sections D 1, D 3, 6.3.2.1, 6.3.2.2, and 6.3.2.5
Phuoc Le	[signature] 3/11/08	Attachment E - Human Reliability Analysis	Specialty check	Section 6.4 and Attachment E
Clarence Smith	[signature] 3/12/08	Attachment E - Human Reliability Analysis	Design concurrence	Check that the Basic Scenarios in Attachment E are consistent with the concept of operations

Checker	Signature/Date	Section	Type of Check	Detailed Scope of Check
Chris Hicks For Freddie Guerrero	Chris Hicks 3/11/08	Attachment F Fire Analysis	Design concurrence	Check dimensions of rooms and area computation
Stephen Skochko For Karim Vakhshoori	SS for Karim Vakhshoori 03/11/08	Attachment F - Fire Analysis	Detailed references and numerical Inputs	Check tabulation of equipment contained in each room
Sandra Castro	Sandra Castro 3/11/08	Section 2	Detailed references and numerical Inputs	Check that all references to engineering documents are correct and up to date
Kathryn Sheffield For Elliot Bedrosian	Kathryn Sheffield 3/12/08	All sections of main body and	Detailed references and numerical Inputs	Check that data in body of analysis has been accurately copied from the sources in attachments
Steve Mikhail	[illegible] 3/11/08	All Sections and Attachments	Reference check	Check that references are to the appropriate document
Dale Dexheimer	D. Dexheimer 3/11/08	Section 6.8	Specialty check	Check consistency with Preclosure Consequence Analysis

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ACRONYMS AND ABBREVIATIONS

Acronyms

ASD	adjustable speed drive
ASME	American Society of Mechanical Engineers
ATHEANA	a technique for human event analysis
BSC	Bechtel SAIC Company, LLC
CCF	common-cause failure
CDF	cumulative density function
CFR	Code of Federal Regulations
CRCF	Canister Receipt and Closure Facility
CTM	canister transfer machine
CTT	cask transfer trolley
DHLW	defense high-level radioactive waste
DOE	U.S. Department of Energy
DPC	dual-purpose canister
DSNF	DOE spent nuclear fuel
EDGF	Emergency Diesel Generator Facility
EFC	error-forcing context
EOC	errors of commission
EOO	errors of omission
EPRI	Electric Power Research Institute
ESD	event sequence diagram
ETF	expended toughness fraction
FEA	finite element analysis
FEM	finite element modeling
FFTF	Fast Flux Test Facility
FTA	fault tree analysis
GROA	geologic repository operations area
HAZOP	hazard and operability
HCLPF	high confidence of low mean frequency of failure
HCTT	cask tractor and cask transfer trailer
HEP	human error probabilities
HEPA	high-efficiency particulate air filter
HFE	human failure event
HLW	high-level radioactive waste
HRA	human reliability analysis
HTC	a transportation cask that is never upended
HVAC	heating, ventilation, and air conditioning

ACRONYMS AND ABBREVIATIONS (Continued)

IET	initiator event tree
IHF	Initial Handling Facility
ITC	important to criticality
ITS	important to safety
LLNL	Lawrence Livermore National Laboratory
LOS	loss of shielding
LOSP	loss of offsite power
LS-DYNA	Livermore Software–Dynamic Finite Element Program
MAP	mobile access platform
MCC	motor control centers
MCO	multicanister overpack
MLD	master logic diagram
MPC	multipurpose canister
N/A	not applicable
NARA	Nuclear Action Reliability Assessment
NFPA	National Fire Protection Association
NNP	normal network protection
NNPP	Naval Nuclear Propulsion Program
NRC	U.S. Nuclear Regulatory Commission
NUREG	Nuclear Regulation (U.S. Nuclear Regulatory Commission)
PCSA	Preclosure Safety Analysis
PDF	probability density function
PEFA	passive equipment failure analysis
PFD	process flow diagram
PIF	performance influencing factor
PLC	programmable logic controller
PRA	probabilistic risk assessment
PSC	procedural safety controls
PSF	performance-shaping factor
QA	quality assurance
RF	Receipt Facility
SAPHIRE	Systems Analysis Programs for Hands-on Integrated Reliability Evaluations
SDU	steel/depleted uranium/steel
SFTM	spent fuel transfer machine
SLS	steel/lead/steel
SNF	spent nuclear fuel
SPM	site prime mover
SPMRC	site prime mover railcars
SPMTT	site prime mover truck trailers

ACRONYMS AND ABBREVIATIONS (Continued)

SRET	system response event tree
SSC	structure, system, or component
SSCs	structures, systems, and components
TAD	transportation, aging, and disposal
TEV	transport and emplacement vehicle
TRIGA	Training, Research, Isotopes, General Atomics
TTC	a transportation cask that is upended using a tilt frame
TYP-FM	type and failure mode
VTC	a transportation cask that is upended on a railcar
WHF	Wet Handling Facility
WPTT	waste package transfer trolley
YMP	Yucca Mountain Project

Abbreviations

AC	alternating current
°C	degrees Celsius
cfm	cubic feet per minute
DC	direct current
ft	foot, feet
gpm	gallons per minute
hp	horsepower
hr, hrs	hour, hours
J	joule
°K	degrees Kelvin
kV	kilovolt
m, min	minute, minutes
mph	miles per hour
s	second
V	volt
W	watt
yr,yrs	year, years

1. PURPOSE

This document on the Receipt Facility (RF) and its companion document entitled *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34), constitute a portion of the preclosure safety analysis (PCSA) that is described in its entirety in the safety analysis report that will be submitted to the U.S. Nuclear Regulatory Commission (NRC) as part of the Yucca Mountain Project (YMP) license application. These documents are part of a collection of analysis reports that encompass all waste handling activities and facilities of the geologic repository operations area (GROA) from the beginning of operations to the end of the preclosure period. The *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34) describes the identification of initiating events and the development of potential event sequences that emanate from them. This analysis uses the resulting event sequences developed in this analysis to perform a quantitative analysis of the event sequences for the purpose of categorization per the definition provided by 10 CFR (Code of Federal Regulations) Part 63 (Ref. 2.3.2).

The PCSA uses probabilistic risk assessment (PRA) technology derived from both nuclear power plant and aerospace methods and applications in order to perform analyses to comply with the risk informed aspects of 10 CFR 63.111 and 63.112 (Ref. 2.3.2) and to be responsive to the acceptance criteria articulated in the *Yucca Mountain Review Plan, Final Report* (Ref. 2.2.68). The PCSA, however, limits the use of PRA technology to identification and development of event sequences that might lead to direct exposure of workers or onsite members of the public; radiological releases that may affect the workers or public (onsite and offsite), and criticality.

The radiological consequence assessment relies on bounding inputs with deterministic methods to obtain bounding dose estimates. These were developed using broad categories of scenarios that might cause a radiological release or direct exposure to workers and the public, both onsite and offsite. These broad categories of scenarios were characterized by conservative meteorology and dispersion parameters, conservative estimates of material at risk, conservative source terms, conservative leak path factors, and filtration of releases via facility high-efficiency particulate air (HEPA) filters when applicable. After completion of the event sequence development and categorization in this analysis, each Category 1 and Category 2 event sequence was conservatively matched with one of the categories of dose estimates. The event sequence analyses also serve as input to the PCSA criticality analyses by identifying the event sequences and end states where conditions leading to criticality are in Category 1 or 2.

An event sequence is defined in 10 CFR 63.2 (Ref. 2.3.2) as:

> A series of actions and/or occurrences within the natural and engineered components of a geologic repository operations area that could potentially lead to exposure of individuals to radiation. An event sequence includes one or more initiating events and associated combinations of repository system component failures, including those produced by the action or inaction of operating personnel. Those event sequences that are expected to occur one or more times before permanent closure of the geologic repository operations area are referred to as Category 1 event sequences. Other event sequences that have at least one chance in 10,000 of occurring before permanent closure are referred to as Category 2 event sequences.

As an extrapolation of the definition of Category 2 event sequences, sequences that have less than one chance in 10,000 of occurring before permanent closure are identified as Beyond Category 2. Consequence analyses are not required for those event sequences.

10 CFR 63.112, Paragraph (e) and Subparagraph (e)(6) (Ref. 2.3.2) require analyses to identify the controls that are relied upon to limit or prevent potential event sequences or mitigate their consequences. Subparagraph (e)(6) specifically notes that the analyses include consideration of “means to prevent and control criticality.” The PCSA criticality analyses employ specialized deterministic methods that are beyond the scope of the present analysis. However, the event sequence analyses serve as an input to the PCSA criticality analyses by identifying the event sequences and end states where conditions leading to criticality are in Category 1 or 2. Some event sequence end states include the phrase “important to criticality.” This indicates that the event sequence has a potential for reactivity increase that is analyzed to determine if reactivity can exceed the upper subcriticality limit.

In order to determine the criticality potential for each waste form and associated facility and handling operations, criticality sensitivity calculations are performed. These calculations evaluate the impact on system reactivity to variations in each of the parameters important to criticality during the preclosure period. The parameters are waste form characteristics, reflection, interaction, neutron absorbers (fixed and soluble), geometry, and moderation. The criticality sensitivity calculations determine the sensitivity of the effective neutron multiplication factor (k_{eff}) to variations in any of these parameters as a function of the other parameters. The PCSA criticality analyses determined the parameters that this event sequence analysis includes. The presence of a moderator in association with a path to exposed fuel was required to be explicitly modeled in the event sequence analysis because such events could not be deterministically found to be incapable of exceeding the upper subcriticality limit. Other situations treated in the event sequence analysis for similar reasons are multiple U.S. Department of Energy (DOE) spent nuclear fuel (SNF) canisters in the Canister Receipt and Closure Facility (CRCF) in the same general location and presence of sufficient soluble boron in the pool in the Wet Handling Facility (WHF).

The initiating events considered in the PCSA define what could occur within the GROA and are limited to those events that constitute a hazard to a waste form while it is present in the GROA. Initiating events include internal events occurring during waste handling operations conducted within the GROA and external events (e.g., seismic, wind energy, or flood water events) that impose a potential hazard to a waste form, waste handling systems, or personnel within the GROA. Such initiating events are included when developing event sequences for the PCSA. However, initiating events that are associated with conditions introduced in structures, systems, and components (SSCs) before they reach the site are not within the scope of the PCSA. The excluded from consideration offsite conditions include drops of casks, canisters, or fuel assemblies during loading at a reactor site; improper drying, closing, or inerting at the reactor site; rail or road accidents during transport; tornado or missile strikes on a transportation cask; or nonconformances introduced during cask or canister manufacturing that result in a reduction of containment strength. Such potential precursors are subject to deterministic regulations such as 10 CFR Part 50 (Ref. 2.3.1), 10 CFR Part 71 (Ref. 2.3.3), and 10 CFR Part 72 (Ref. 2.3.4) and associated quality assurance (QA) programs. As a result of compliance to such regulations, the SSCs are deemed to pose no undue risk to health and safety. Although the analyses do not address quantitative probabilities to the aforementioned excluded precursors, it is clear that the use of conservative design criteria and the implementation of QA controls result in unlikely exposures to radiation.

Other boundary conditions used in the PCSA include:

- Plant operational state. The initial state of the facility is normal with each system operating within its vendor-prescribed operating conditions.
- No other simultaneous initiating events. It is standard practice to not consider the occurrence of other initiating events (human-induced or naturally occurring) during the time span of an event sequence because: (a) the probability of two simultaneous initiating events within the time window is small and, (b) each initiating event will cause operations in the waste handling facility to be terminated, which further reduces the conditional probability of the occurrence of a second initiating event, given that the first has occurred.
- Component failure mode. The failure mode of a structure, system, or component (SSC) corresponds to that required to make the initiating or pivotal event occur.
- Fundamental to the basis for the use of industry-wide reliability parameters within the PCSA, such as failure rates, is the use of SSCs within the GROA that conform to NRC accepted consensus codes and standards, and other regulatory guidance.
- Intentional malevolent acts, such as sabotage and other security threats, are not addressed in this analysis.

As stated, the scope of the preclosure safety analysis is limited to internal initiating events originating within the GROA boundary and external initiating events that have their origin outside the GROA boundary, but can affect buildings and/or equipment within the GROA. External event analyses are documented in *External Events Hazards Screening Analysis* (Ref. 2.2.28) and *Frequency Analysis of Aircraft Hazards for License Application* (Ref. 2.2.19). Internal event identification (using a master logic diagram (MLD) and hazard and operability (HAZOP) evaluation), event sequence development and grouping, and related facility details are provided in *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34), which also documents the methodology and process employed and initiates the analysis that is completed here.

This document uses event trees from the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34) to quantify the event sequences for each waste form. Quantification refers to the process of obtaining the mean frequency of each event sequence for the purpose of categorization. This document shows the categorization of each event sequence based on:

- Mean frequency associated with the event sequence frequency distribution
- Uncertainty associated with the event sequence frequency distribution

- Material at risk for each Category 1 and 2 event sequence for purposes of dose calculations
- Important to safety (ITS) SSCs
- Compliance with the nuclear safety design bases
- Procedural safety controls required for operations.

Other PCSA documents which are not referenced here cover the reliability and categorization of external events and summarize procedural safety controls and nuclear safety design bases. The main documents that will emanate from Volume I (Ref. 2.2.34) and the current analyses are:

- *ITS SSC/Non-ITS SSC Interactions Analysis* (Ref. 2.4.1)
- *Preclosure Nuclear Safety Design Bases* (Ref. 2.4.2)
- *Preclosure Procedural Safety Controls* (Ref. 2.4.3)
- *Seismic Event Sequence Quantification and Categorization* (Ref. 2.4.4).

2. REFERENCES

2.1 PROCEDURES/DIRECTIVES

2.1.1 EG-PRO-3DP-G04B-00037, REV 10. *Calculations and Analyses*. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071018.0001.

2.1.2 EG-PRO-3DP-G04B-00046, Rev. 10. *Engineering Drawings*. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080115.0014.

2.1.3 IT-PRO-0011, REV 7. *Software Management*. Las Vegas, Nevada: Bechtel SAIC Company. ACC: DOC.20070905.0007.

2.1.4 LS-PRO-0201, REV 5. *Preclosure Safety Analysis Process*. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0021.

2.2 DESIGN INPUTS

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

Design Inputs are listed in this section and the Attachment sections listed in Section 2.5.

The inputs in this Section noted with an asterisk (*) indicate that they fall into one of the designed categories described in Section 4.1, relative to suitability for intended use.

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2.2.29 *BSC 2008. *Initial Handling Facility Fire Hazard Analysis.* 51A-M0A-FP00-00100-000-00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080212.0007.

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2.3 DESIGN CONSTRAINTS

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2.3.2 10 CFR 63. 2007. Energy: Disposal of High-Level Radioactive Wastes in a Geologic Repository at Yucca Mountain, Nevada. U.S. Nuclear Regulatory Commission.

2.3.3 10 CFR 71. 2007. Energy: Packaging and Transportation of Radioactive Material. U.S. Nuclear Regulatory Commission. ACC: MOL.20070829.0114.

2.3.4 10 CFR 72. 2007. Energy: Licensing Requirements for the Independent Storage of Spent Nuclear Fuel, High-Level Radioactive Waste, and Reactor-Related Greater than Class C Waste. U.S. Nuclear Regulatory Commission.

2.4 DESIGN OUTPUTS

2.4.1 BSC 2008. *ITS SSC/Non-ITS SSC Interactions Analysis.* 000-PSA-MGR0-02300-000-00A. Las Vegas, Nevada: Bechtel SAIC Company.

2.4.2 BSC 2008. *Preclosure Nuclear Safety Design Bases.* 000-30R-MGR0-03500-000-000. Las Vegas, Nevada: Bechtel SAIC Company.

2.4.3 BSC 2008. *Preclosure Procedural Safety Controls.* 000-30R-MGR0-03600-000-000 REV 00. Las Vegas, Nevada: Bechtel SAIC Company.

2.4.4 BSC 2008. *Seismic Event Sequence Quantification and Categorization.* 000-PSA-MGR0-01100-000-00A. Las Vegas, Nevada: Bechtel SAIC Company.

2.5 ATTACHMENT REFERENCES

2.5.1 Attachment A: Design Inputs references are listed in Section 2.2 of the main report.

2.5.2 Attachment B: Design Inputs references are listed in Section B1.1, Section B2.1, Section B3.1, Section B4.1, Section B5.1, Section B6.1, Section B7.1, Section B8.1, and Section B9.1.

2.5.3 Attachment C: Design Inputs references are listed in Section C5.

2.5.4 Attachment D: Design Inputs references are listed in Section D4.1.

2.5.5 Attachment E: Design Inputs references are listed in Section E8.1.

2.5.6 Attachment F: Design Inputs references are listed in Section F2.

2.5.7 Attachment G: This attachment does not contain Design Inputs references.

2.5.8 Attachment H: This attachment does not contain Design Inputs references.

3. ASSUMPTIONS

3.1 ASSUMPTIONS REQUIRING VERIFICATION

There are no assumptions requiring verification.

3.2 ASSUMPTIONS NOT REQUIRING VERIFICATION

3.2.1 General Analysis Assumptions

Equipment and SSC designed and purchased for the Yucca Mountain repository are of the population of equipment and SSC represented in U.S. industry-wide reliability information sources. Furthermore, the uncertainty in reliability is represented by the variability of reliabilities across this population.

Rationale–Although the repository features some unique pieces of equipment at the system level (such as the site transporter and the cask transfer trolley (CTT)), at the component level, the repository relies on proven and established technologies. The industry-wide information sources include historical reliability information at the component level. Such experience is relevant to the repository because the repository relies on components that are similar to the ones represented in the information sources. In some cases, system-level information, such as crane load-drop rates, from the industry-wide information sources are used. It is appropriate to use such information because it represents similar pieces of equipment at the system level. In addition, drawing from a wide spectrum of sources takes advantage of many observations, which yield better statistical information regarding the uncertainty associated with the resulting reliability estimates.

4. METHODOLOGY

4.1 QUALITY ASSURANCE

This analysis has been prepared in accordance with *Calculations and Analyses* (Ref. 2.1.1) and *Preclosure Safety Analysis Process* (Ref. 2.1.4). Therefore, the approved version is designated as "QA: QA."

Documentation of suitability for intended use of "QA: N/A" drawings: Engineering drawings are prepared using the "QA: QA" procedure *Engineering Drawings* (Ref. 2.1.2). This means they are checked by an independent checker and reviewed for constructability and coordination before review and approval by the engineering group supervisor and the discipline engineering manager (Ref. 2.1.2, Section 3.2.2 and Attachments 3 and 5). The check, review, and approval process provides assurance that these drawings accurately document the design and operational philosophy of the facility. For this reason, they are suitable for their intended use as sources of input to this analysis.

Documentation of suitability for intended use of sketches (which are "QA: N/A"): In a few instances, sketches are used as inputs to this analysis. The use of sketches is acceptable for committed analyses, such as the present analysis, provided that the results are not used for procurement, fabrication, or construction purposes. Because the present analysis is not used for procurement, fabrication, or construction purposes, the use of sketches is acceptable. Therefore, the sketches that are used as inputs are suitable for their intended uses

Documentation of suitability for intended use of "QA: N/A" engineering calculations or analyses: Engineering calculations and analyses are prepared using the "QA: QA" procedure *Calculations and Analyses* (Ref. 2.1.1). They are checked by an independent checker and reviewed for coordination before review and approval by the engineering group supervisor and the discipline engineering manager. The check, review, and approval process provides assurance that these calculations and analyses accurately document the design and operation of the facility. For this reason, they are suitable for their intended use as sources of input to this analysis.

Documentation of suitability for intended use of engineering studies (which are "QA: N/A"): In a few instances, studies are used as inputs to this analysis. The uses of inputs from studies are made clear by the context of the discussion at the point of use. The use of studies is acceptable for committed analyses, such as the present analysis, provided that the results are not used for procurement, fabrication, or construction purposes. Because the present analysis is not used for procurement, fabrication, or construction purposes, the use of studies is acceptable. Therefore, the studies that are used as inputs are suitable for their intended uses.

Documentation of suitability for intended use of Bechtel SAIC Company, LLC (BSC) design guides (which are "QA: N/A"): The uses of inputs from design guides are made clear by the context of the discussion at the point of use. Design guides are used as inputs only when specific design documents, such as drawings, calculations, and design reports are not available at the present level of design development. Therefore, the design guides that are used as inputs are suitable for their intended uses.

Documentation of suitability for intended use of BSC engineering standards (which are "QA: N/A"): Engineering standards are used in this analysis as the basis for the numbering system for basic events. The uses of inputs from BSC engineering standards are made clear by the context of the discussion at the point of use. Therefore, the design guides that are used as inputs are suitable for their intended uses.

Documentation of suitability for intended use of BSC Interoffice memorandum: Due to the early nature of the design of some systems, the only available sources for the information used are interoffice memorandum. The information used from these sources are conservative estimates and appropriate for their intended use.

Documentation of suitability for intended use of inputs from outside sources: The uses of inputs from outside sources are made clear by the context of the discussion at the point of use. These uses fall into the following categories and are justified as follows (in addition to the justifications provided at the point of use).

1. Some inputs are cited as sources of the methods used in the analysis. These inputs are suitable for their intended uses because they represent commonly accepted methods of analysis among safety analysis practitioners or, more generally, among scientific and engineering professionals.

2. Some inputs are cited as examples of applications of methods of analysis by others. These inputs are suitable for their intended uses because they illustrate applicable methods of analysis.

3. Some inputs are cited as sources of historical safety-related data. These inputs are suitable for their intended uses because they represent historical data that is commonly accepted among safety analysis practitioners.

4. Some inputs are cited as sources of accepted practices as recommended by codes, standards, or review plans. These inputs are suitable for their intended uses because they represent codes, standards, or review plans that are commonly accepted by practitioners of the affected professional disciplines.

5. Some inputs provide information specific to the Yucca Mountain repository that was produced by organizations other than BSC. These inputs are suitable for their intended uses because they provide information that was developed for the Yucca Mountain Repository under procedures that apply to the organization that produced the information.

4.2 USE OF SOFTWARE

4.2.1 Level 1 Software

This section addresses software used in this analysis as Level 1 software, as defined in *Software Management* (Ref. 2.1.3, Attachment 12). SAPHIRE Version 7.26 STN 10325-7.26-01 (Ref. 2.2.74) is used in this analysis for PRA simulation and analyses. The SAPHIRE software is used on a personal computer running Windows XP inside a VMware virtual machine; it is also listed in the current *Qualified and Controlled Software Report*, and was obtained from Software

Configuration Management. The SAPHIRE software is specifically designed for PRA simulation and analyses, and has been verified to show that this software produces precise solutions for encoded mathematical models within the defined limits, for each parameter, employed (Ref. 2.2.40). Therefore, SAPHIRE version 7.26 is suitable for use in this analysis.

The SAPHIRE project files for this analysis are listed in Attachment H. They are contained on a compact disc, which is included as part of Attachment H. SAPHIRE project files contain all of the inputs that SAPHIRE requires to produce the outputs that are documented in this analysis.

4.2.2 Level 2 Software

This section addresses software used in this analysis that is classified as Level 2 software, as defined in *Software Management* (Ref. 2.1.3, Attachment 12). The software is used on personal computers running either Windows XP Professional or Windows 2000 operating systems.

- Word 2003, a component of Microsoft Office Professional 2003, and Visio Professional 2003 are listed in the current *Level 2 Usage Controlled Software Report*. Visio 2003 and Word 2003 are used in this analysis for the generation of graphics and text. The accuracy of the resulting graphics and text is verified by visual inspection. The precise means of verification is left to the discretion of the checker in compliance with applicable procedures.

- Excel 2003, a component of Microsoft Office Professional 2003, and Mathcad version 13.0 and 14.0 are listed in the current *Level 2 Usage Controlled Software Report*. Crystal Ball version 7.3.1 (a commercial, off-the-shelf, Excel-based risk-analysis tool) is listed on the *Controlled Software Report* and is registered for Level 2 usage. Excel 2003, Mathcad 13.0 and 14.0, and Crystal Ball 7.3.1 are used in this analysis to calculate probability distributions for selected SAPHIRE inputs and to graphically display information. Graphical representations are verified by visual inspection. The calculations are documented in sufficient detail to allow an independent replication of the computations. The user defined formulas and inputs are verified by visual inspection. The results are in some cases verified by independent replication of the computations. However, in some cases, for example, for some Excel calculations and Mathcad 13.0 and 14.0 calculations, the results are verified by visual inspection. The precise means of verification is left to the discretion of the checker in compliance with applicable procedures.

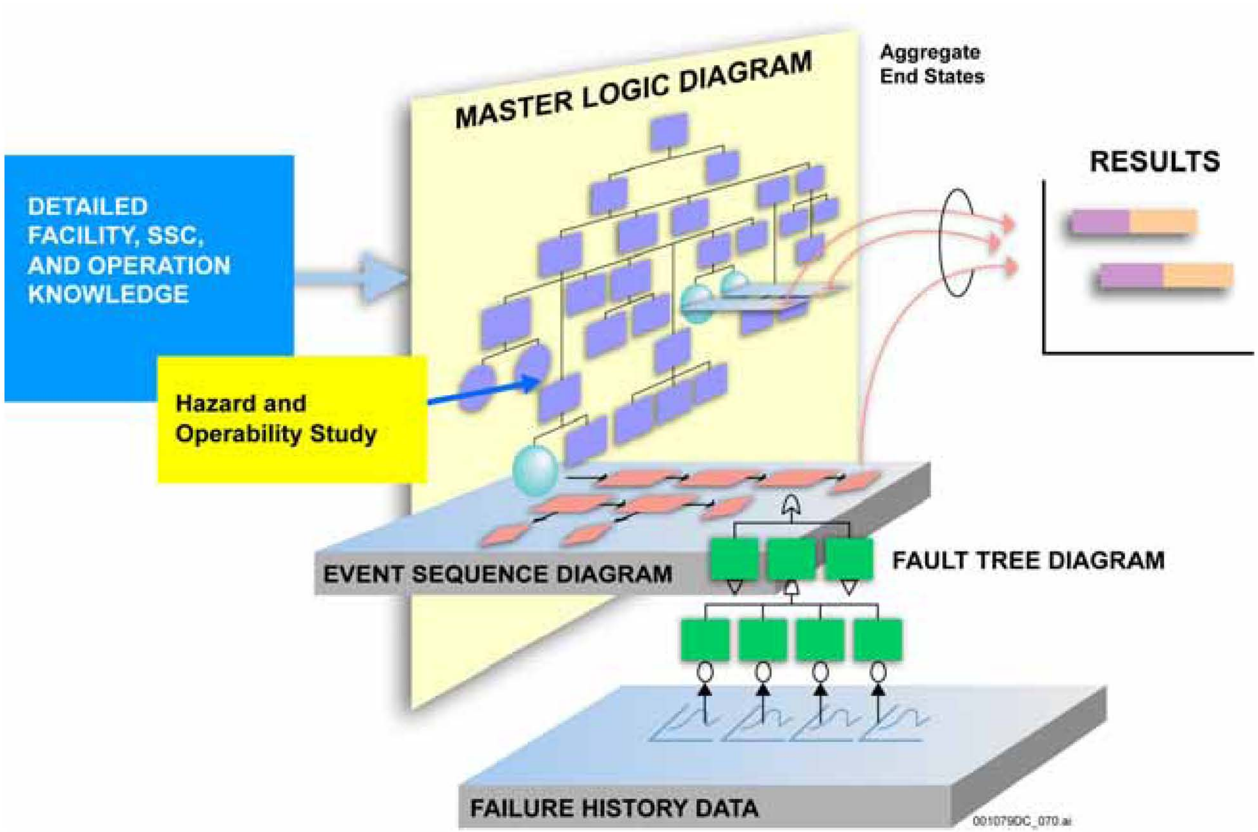
- WinZip 9.0, a file compression utility for Windows, is listed in the current *Level 2 Usage Controlled Software Report*. WinZip 9.0 is used in this analysis to compress files for presentation on compact disc in Attachment H.

4.3 DESCRIPTION OF ANALYSIS METHODS

This section presents the PCSA approach and analysis methods in the context of overall repository operations. As such, it includes a discussion of operations that may not apply to the RF. Specific features of the RF and its operations are not discussed until Section 6, where the methods described here are applied to the RF. The PCSA uses the technology of PRA as described in references such as *Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications* (Ref. 2.2.8). The PRA answers three questions:

1. What can go wrong?
2. What are the consequences?
3. How likely is it?

PRA may be thought of as an investigation into the responses of a system to perturbations or deviations from its normal operation or environment. The PCSA is a simulation of how a system acts when something goes wrong. Relationships between the methodological components of the PCSA are depicted in Figure 4.3-1. Phrases in ***bold italics*** in this section indicate methods and ideas depicted in Figure 4.3-1. Phrases in *normal italics* indicate key concepts.



Source: Modified from *Master Logic Diagram* (Ref. 2.2.77)

Figure 4.3-1. Event Sequence Analysis Process

The PCSA starts with analysts obtaining sufficient knowledge of facility design and operation, and equipment and SSC design and operation to understand how the YMP waste handling is conducted. This is largely performed and documented in *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34). An understanding of how a facility operates is a prerequisite for developing event sequences that depict how it would fail. *Success criterion* are important additional set of inputs to the PCSA. A success criterion states the minimum functionality that constitutes acceptable, safe performance. For example, a success criterion for a crane is to pick-up, transport, and put-down a cask without dropping it. The complementary statement of a success criterion is a failure mode (e.g., crane drops cask).

The basis of the PCSA is the development of ***event sequences***. An event sequence may be thought of as a string of events beginning with an *initiating event* and eventually leading to potential consequences (*end states*). Between initiating events and end states within a scenario, are *pivotal events* that determine whether and how an initiating event propagates to an end state. An event sequence answers the question "What can go wrong?" and is defined by one or more initiating events, one or more pivotal events, and one end state. Initiating events are identified by ***MLD*** development, cross-checked with an evaluation based on applied ***HAZOP*** evaluation techniques. Event sequences unfold as a combination of failures and successes of pivotal events. An end state, the termination point for an event sequence, identifies the type of radiation exposure or potential criticality, if any, that results. In this analysis, eight mutually exclusive end states are of interest:

1. "OK"–Indicates the absence of radiation exposure and potential for criticality.

2. Direct Exposure, Degraded Shielding–Applies to event sequences where a SSC providing shielding is not breached, but its shielding function is jeopardized. An example is a lead-shielded transportation cask that is dropped from a height great enough for the lead to slump toward the bottom of the cask at impact, leaving a partially shielded path for radiation to stream. This end state excludes radionuclide release.

3. Direct Exposure, Loss of Shielding–Applies to event sequences where a SSC providing shielding fails, leaving a direct path for radiation to stream. For example, this end state applies to a breached transportation cask, with a canister inside maintaining its containment function. In another example, this end state applies to shield doors inadvertently opened. This end state excludes radionuclide release.

4. Radionuclide Release, Filtered–Indicates a release of radioactive material from its confinement, through a filtered path, to the environment. The release is filtered when it is confined and filtered through the successful operation of the heating, ventilation, and air conditioning (HVAC) system over its mission time. This end state excludes moderator intrusion.

5. Radionuclide Release, Unfiltered–Indicates a release of radioactive material from its confinement, through the pool of the WHF or through an unfiltered path, to the environment. This end state excludes moderator intrusion.

6. Radionuclide Release, Filtered, Also Important to Criticality–This end state refers to a situation in which a filtered radionuclide release occurs and (unless the associated event sequence is beyond Category 2) for which a criticality investigation is indicated.

7. Radionuclide Release, Unfiltered, Also Important to Criticality–This end state refers to a situation in which an unfiltered radionuclide release occurs and (unless the associated event sequence is beyond Category 2) for which a criticality investigation is indicated.

8. Important to Criticality–This end state refers to a situation in which there has been no radionuclide release and (unless the associated event sequence is beyond Category 2) for which a criticality investigation is indicated.

The answer to the second question, "What are the consequences?" requires consideration of radiation exposure and the potential for criticality for Category 1 and Category 2 event sequences. Consideration of the consequences of event sequences that are beyond Category 2 is not required by 10 CFR Part 63 (Ref. 2.3.2). Radiation doses to individuals from direct exposure and radionuclide release are addressed in a companion consequence analysis by modeling the effects of bounding event sequences related to the various waste forms and the facilities that handle them.

The radiological consequence analysis develops a set of bounding consequences. Each bounding consequence represents a group of like event sequences. The group (or bin) is based on such factors as characteristics of the waste form involved, availability of HEPA filtration, location of occurrence (in water or air), and characteristics of the surrounding material (such as transportation cask or waste package). Each event sequence is mapped to one of the bounding consequences, for which conservative doses have been calculated.

Criticality analyses are performed to ensure that any Category 1 and Category 2 event sequences that terminate in end states that are important to criticality would not result in a criticality. In order to determine the criticality potential for each waste form and associated facility and handling operations, criticality sensitivity calculations are performed. These calculations evaluate the impact on system reactivity of variations in each of the parameters important to criticality during the preclosure period. The parameters are: waste form characteristics, reflection, interaction, neutron absorbers (fixed and soluble), geometry, and moderation. The criticality sensitivity calculations determine the sensitivity of the effective neutron multiplication factor to variations in any of these parameters as a function of the other parameters. The deterministic sensitivity analysis covers all reasonably achievable repository configurations that are important to criticality. Refer to Section 4.3.9 for detailed discussion of the treatment of criticality in event sequences.

The third question, "How likely is it?" is answered by the estimation of event sequence frequencies. The PCSA uses ***failure history*** records (for example, *Nonelectronic Parts Reliability Data* (Ref. 2.2.38) and *Nuclear Computerized Library for Assessing Reactor Reliability* (Ref. 2.2.50)), structural reliability analysis, thermal stress analysis, and engineering and scientific knowledge about the design as the basis for estimation of probabilities and frequencies. These sources coupled with the techniques of probability and statistics, for example, *Handbook of Parameter Estimation for Probabilistic Risk Assessment* (Ref. 2.2.11), are used to estimate frequencies of initiating events and event sequences and the conditional probabilities of pivotal events.

The PCSA uses ***event sequence diagrams*** (***ESDs***), *event trees*, and ***fault trees*** to develop and quantify event sequences. The ESDs and event trees are described and developed in the event sequence development analyses. The present analysis uses fault trees to disaggregate a SSC or item of equipment to a level of detail that is supported by available reliability information from failure history records. Various techniques of probability and statistics are employed to estimate failure frequencies of mechanical, electrical, electro-mechanical, and electronic equipment. Such frequencies, or *active-component* unreliabilities, provide inputs to the fault tree models of items of equipment. Fault trees are used in some instances to model initiating events and in other instances to model pivotal events.

Some pivotal events are related to structural failures of containment (e.g., canisters) and others are related to shielding (e.g., transportation casks). In these cases, probabilistic structural reliability analysis methods are employed to calculate the mean conditional probability of containment or shielding failure given the initiating event (e.g., a drop from a crane). Other pivotal events require knowledge of response to fires. Calculation of failure probabilities given a fire is accomplished by the appropriate analysis using applicable material properties and traditional methods of heat transfer analysis, structural analysis, and fire dynamics. The probabilities so derived are called *passive-equipment* failure probabilities.

All pivotal events in the PCSA are characterized by *conditional probabilities* because their values rely on the conditions set by previous events in an event sequence. For example, the failure of electrical or electronic equipment depends on the operating temperature. Therefore, if a previous event in a scenario is a failure of a cooling system, then the probability of the electronic equipment failure would depend on the operation (or not) of the cooling system.

The frequency of occurrence of an event sequence is the product of the frequency of its initiating event and the conditional probabilities of its pivotal events. This is true whether or not the frequency and probabilities are expressed as single points or probability distributions. To group together event sequences for the purpose of categorization, the frequencies of event sequences within the same ESD that result in the same end state, are summed. The concept of ***aggregating event sequences*** to obtain aggregated end state results is depicted in Figure 4.3-1.

The PCSA is described above as a system simulation. This is important in that any simulation or model is an approximate representation of reality. Approximations may lead to uncertainties regarding the frequencies of event sequences. The event sequence quantification presented in this document propagates input uncertainties to the calculated frequencies of event sequences using Monte Carlo techniques. Figure 4.3-1 illustrates the ***results*** as horizontal bars to depict the uncertainties that give rise to potential ranges of results.

As required by the performance objectives for the GROA through permanent closure in 10 CFR 63.111 (Ref. 2.3.2), each aggregated event sequence is categorized based on its frequency. Therefore, the focus of the analysis in this document is to:

1. Quantify the frequency of each initiating event that is identified in the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34).

2. Quantify the conditional probability of the pivotal events in each event sequence.

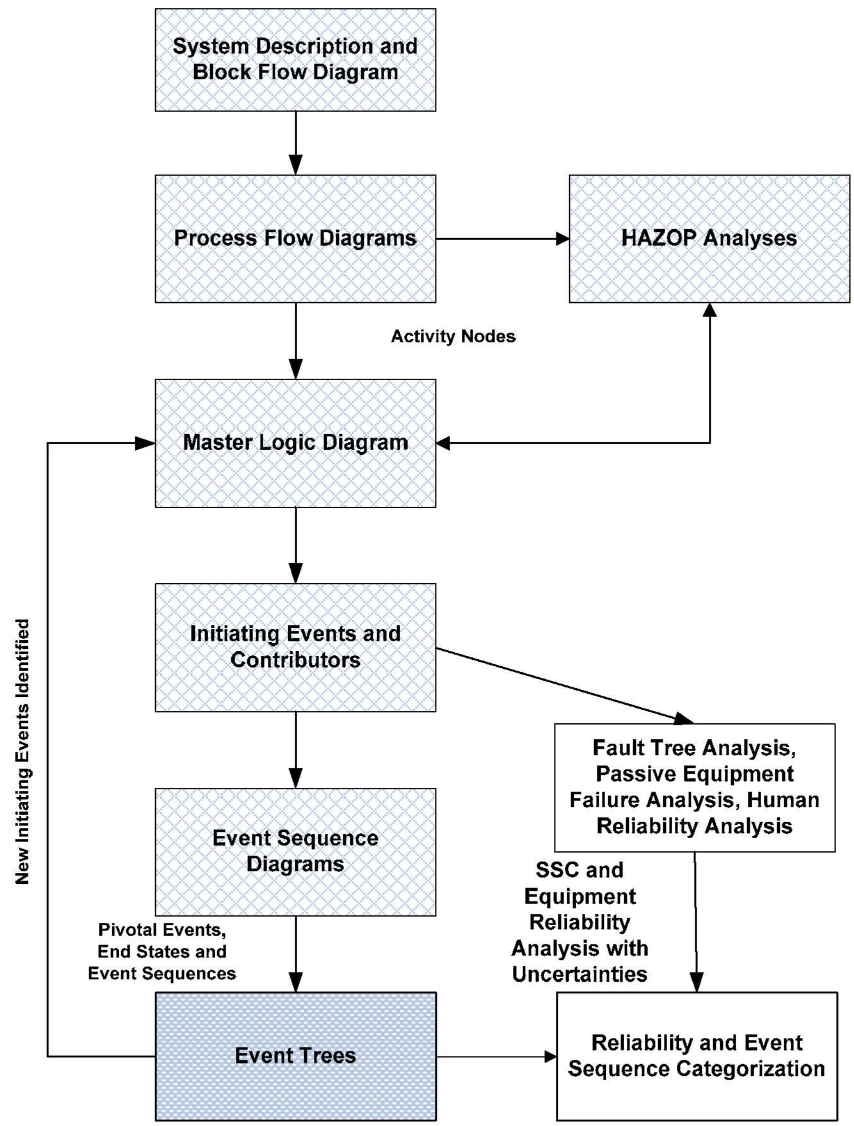
3. Calculate the frequency of each event sequence (i.e., calculate the product of the initiating event frequency and pivotal event conditional probabilities).

4. Calculate the frequencies of the aggregated event sequences.

5. Categorize the aggregated event sequences for further analysis.

The activities required to accomplish these objectives are illustrated in Figure 4.3-2 and described below.

The cross-hatched boxes in Figure 4.3-2 serve as a review of the analysis performed for the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34). The interface between the event sequence development analysis and the present categorization analysis is the set of event trees, as represented by the darkly shaded box. The event trees from the event sequence development analysis are passed as input into the present analysis. The unshaded boxes represent the analysis performed in this study, the methods of which are described later in Section 4.



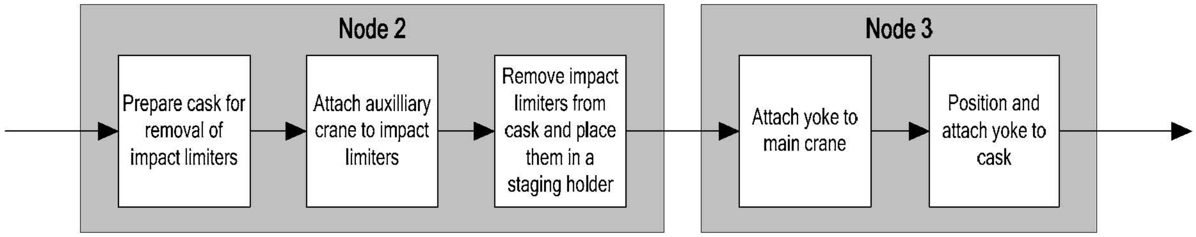
NOTE: HAZOP = hazard and operability; SSC = structure, system, or component.

Source: Modified from *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34, Figure 2).

Figure 4.3-2. PCSA Process

The event sequences that are categorized in the present analysis can be more fully understood by consulting the event sequence development analysis (Ref. 2.2.34). The remainder of this subsection presents a refresher of the event sequence development process.

A simplified process flow diagram (PFD) is developed to clearly delineate the process and sequence of operations to be considered within the analysis of the facility. An excerpt from an example PFD is shown in Figure 4.3-3. The PFD guides development of the MLD and the conduct of the HAZOP evaluation. The PFD is broken down into nodes to identify specific processes and operations that are evaluated with both a MLD and HAZOP evaluation to identify potential initiators.



NOTE: This diagram illustrates a small portion of the overall handling operations for a typical waste facility.

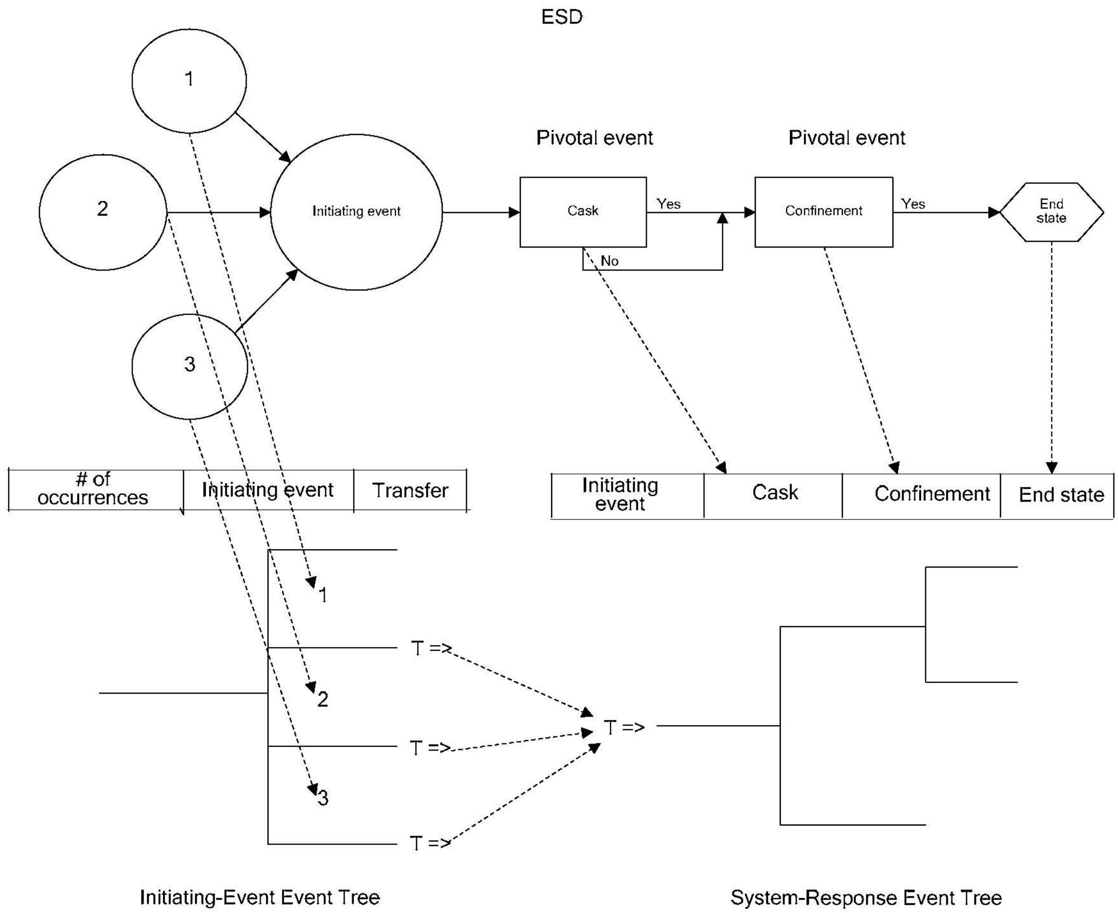
Source: Original

Figure 4.3-3. Portion of a Simplified Process Flow Diagram for a Typical Waste-Handling Facility

Development of the MLD is accomplished by deriving specific failures from a generalized statement of the undesired state. As a "top-down" analysis, the MLD starts with a top event, which represents a generalized undesired state. The top event includes direct exposure to radiation and exposure as a result of a release of radioactive material. The basic question answered by the MLD is "How can the top event occur?" Each successively lower level in the MLD hierarchy divides the identified ways in which the top event can occur with the aim of eventually identifying specific initiating events that may cause the top event. In the MLD, the initiating events are shown at the next-to-lowest level. The lowest level provides an example of contributors to the initiating event. This process for the PCSA is detailed in the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34, Section 4.3.1.2.).

The HAZOP evaluation focuses on identifying potential initiators that are depicted in the lower levels of the MLD. It is a "bottom-up" approach that supplements the "top-down" approach of the MLD. The HAZOP evaluation is also a systematic analysis of repository operations during the preclosure phase. As an early step in the performance of the HAZOP evaluation, the intended function, or intention, of each node in the PFD is defined. The intention is a statement of what the node is supposed to accomplish as part of the overall operation. The HAZOP analysts work their way through the PFD, node by node, and postulate deviations from normal operations. A "deviation" is any out-of-tolerance variation from the normal values of parameters specified for the intention. Although the repository is in some ways to be the first of its kind, the operations are based on established technologies: for example, transportation cask movement by truck and rail, crane transfers of casks and canisters, rail-based trolleys, air-based conveyances, robotic welding, and SNF pool operations. The team assembled for the HAZOP evaluation (and available on call as questions arose) had experience with such technologies and was well equipped to perform the evaluation.

The MLD and HAZOP evaluation are strongly interrelated. The MLD is cross-checked to the HAZOP evaluation. That is, the MLD is modified to include any initiators and contributors that are identified in the HAZOP evaluation but not already included in the MLD. The entire process is iterative in nature (Figure 4.3-2, iteration not shown) with insights from succeeding steps often feeding back to predecessors. The top-down MLD and the bottom-up HAZOP evaluation provide a diversity of viewpoints that add confidence that no important initiating events have been omitted. Details on implementation of the HAZOP evaluation are presented in the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34, Section 4.3.1.3). Section 4.3.1.3).



Source: Original

Figure 4.3-4. ESD, Event Tree Relationship

An overview of the pertinent human and SSC response to an initiating event is depicted in an ESD. As shown in Figure 4.3-4, an ESD represents event sequences in terms of initiating events, pivotal events, and end states. The boxes (pivotal events) represent events that have binary outcomes: success (yes) or failure (no). Because the future is uncertain, the analyst does not know which of the alternative scenarios might occur. The ESD depicts the alternative scenarios as paths that can be traced through the diagram. Each alternative path from an initiating event to an end state represents an event sequence. The events that may occur after the initiating event are identified by asking and answering the question "What can happen next?" Typically, questions about the integrity of radionuclide containment (e.g., cask, canister, or waste package) and confinement (e.g., HVAC) become pivotal events in the ESD

The initiating events that are represented in the MLD are transferred to events depicted as "little bubbles" (Figure 4.3-4, 1, 2, 3) in the ESDs. One or more initiating events identified on the MLD may be included in a single little bubble, but all of the initiating events included in the little bubble must have the same pivotal events (i.e., human and SSC responses) and the same conditional probability for each pivotal event. Initiating events represented by little bubbles may be aggregated further into "big bubbles" as depicted in Figure 4.3-4. The big bubble represents the failures associated with a major function in a specific location depicted in the PFD and establishes the level of aggregation for the categorization of the event sequence (as Category 1, Category 2, or beyond Category 2).

For example, all initiating events that challenge the containment function of a canister would include pivotal events that question the containment integrity of the canister and the availability of HVAC confinement. The knowledge to develop such ESDs and appropriately group the initiating events comes from a detailed knowledge of the SSCs and operations derived from developing the PFD, MLD, and HAZOP evaluation. The pivotal event conditional probabilities are the same for all initiating events in a little bubble. All initiating events represented by the big bubble have the same human and SSC responses, and therefore, may be represented by the same event sequences. However, the conditional probability for each pivotal event is not necessarily the same for each little bubble.

4.3.1 Event Tree Analysis and Categorization

Also illustrated in Figure 4.3-4 is the relationship of the YMP ESDs to their equivalent event trees. Event trees contain the same information as ESDs but in a form suitable to be used by software such as SAPHIRE (Ref. 2.2.40) which ultimately stores event trees, fault trees, and reliability data, and it quantifies the event sequences. Event tree depiction of ESDs provides little new information. In an event tree, each event sequence has its separate line so that the connections between initiating events and end states is more explicit than in ESDs (Ref. 2.2.63, Section 3.4.4.2). Any path from left to right that begins with the initiating event and terminates with an end state is an event sequence. Every path must be associated with an end state. As illustrated in the event tree portion of Figure 4.3-4, each intersection of a horizontal and vertical line is referred to as a node (or branch point). Each node is associated with a conditional probability of following the vertical downward branch. . By convention, the description of each branch is stated as a success, and the downward branch indicates a failure. The complement is the probability of taking the vertical upward branch, that is, the probability of success. To quantify the event sequence the initiating event frequency (or expected number of occurrences) is multiplied by the conditional probability of each subsequent pivotal event node in the event sequence until an end state is reached.

The YMP PCSA uses the concept of linked event trees (Ref. 2.2.63). Each facility has its own set of event trees. The first event tree simply represents the little bubbles, one horizontal line per little bubble. This is called the initiator event tree (IET). The second event tree contains the pivotal events and end states. This is called the system response event tree (SRET). An event sequence would start with each of the horizontal lines as if it were the initiating event on the SRET, as indicated in Figure 4.3-4. Each set of IET and SRET is quantified for each waste container type (e.g., dual-purpose canisters (DPC), transportation, aging, and disposal (TAD) canisters, U.S. Department of Energy spent nuclear fuel (DOE SNF)) that is handled in a facility. The event in the IET labeled "# of occurrences" represents the number of handlings (i.e., demands) for that initiating event. For example, each lift of a vertical transportation cask provides an opportunity for a drop. An event sequence quantification includes: the frequency (or number of occurrences) of each end state (e.g., radionuclide release), associated with a single lift, and multiplies it by the number of lifts to obtain the expected number of drops over the preclosure period. This approach is consistent with a binomial model of reliability.

Categorization of event sequences is based on the aggregated "big bubble" initiating event. Each line on the IET coupled with the SRET is quantified separately. Using Figure 4.3-4, this would mean three quantifications, corresponding to the three initiating event frequencies and three corresponding sets of pivotal event probabilities. (By SAPHIRE convention, the top line is a dummy initiating event.) Each event sequence, therefore, would have three values. In order to obtain the total frequency of an event sequence for purposes of categorization, per 10 CFR 63.111 (Ref. 2.3.2), the three frequencies are probabilistically summed. Doing this summation is equivalent to basing categorization on the big bubble. If an event sequence has only one little bubble, then only the SRET needs to be used with the initiating event in the place so denoted, in the second event tree. In this case, summation of event sequences is not necessary and not performed.

Because each event sequence is associated with a mean number of occurrences over the preclosure period, categorization is straightforward. Those event sequences that are expected to occur one or more times before permanent closure of the GROA are Category 1 event sequences. Other event sequences that have at least one chance in 10,000 of occurring but less than one occurrence before permanent closure are Category 2 event sequences. Sequences that have less than one chance in 10,000 of occurring before permanent closure are identified as beyond Category 2. As described in Section 4.3.6, event sequence quantification considers uncertainties and categorization is performed on the basis of an event sequence mean value of the underlying probability distribution. The preclosure period lasts 100 years but actual emplacement operations occupy 50% of this time (Ref. 2.2.15, Section 2.2.2.7).

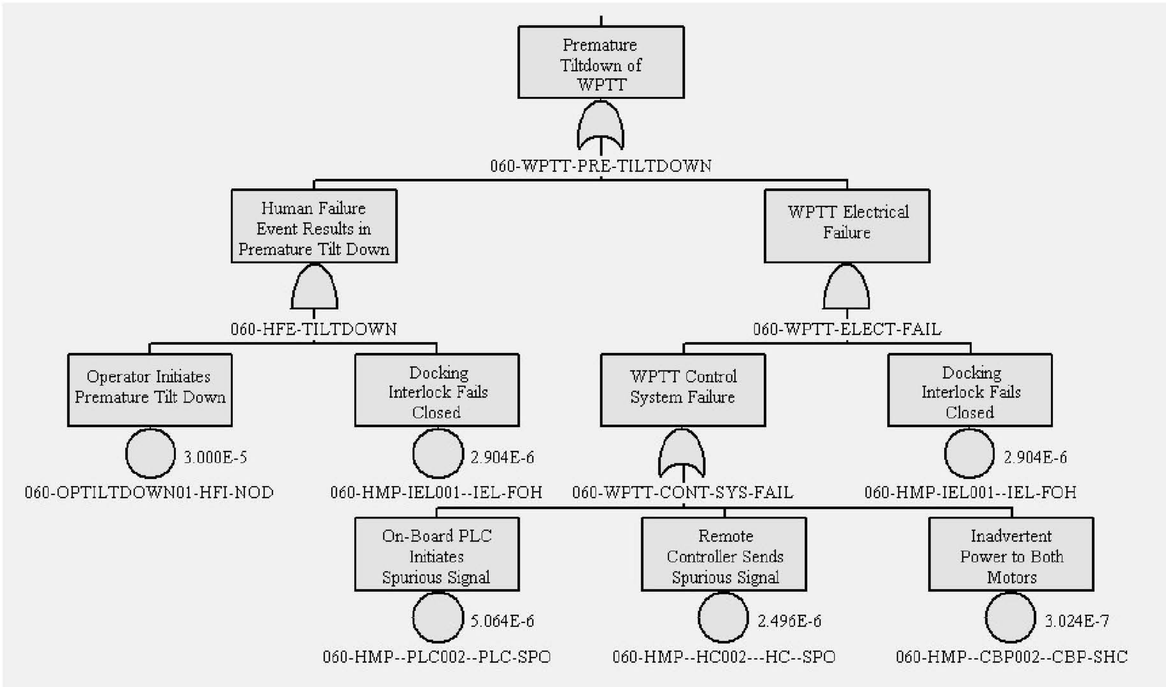
An initiating event for an event sequence may have the potential to affect several waste form types (for instance, a high-level radioactive waste (HLW) canister and a DOE standardized canister, or a TAD canister and a DPC). For example, the seismically-induced event sequence leading to a collapse of a surface facility could cause the breach of all the waste forms inside that facility. Similarly, a large fire affecting an entire facility also affects all the waste forms inside the facility. The number of occurrences over the preclosure period of an event sequence that affects more than one type of waste form is equal to the number of occurrences of the event sequence, evaluated for one of the waste form types, multiplied by the probability that the other waste form types are present at the time the initiating event occurs. Because a probability is less than or equal to one, the resulting product is not greater than the number of occurrences of the event sequence before multiplication by the probability. The number of occurrences of an event sequence is calculated for a given waste form type, without adjustment for the probability of presence of other waste form types. The results of the event sequence categorization (reported in Section 6.8.3) show that the event sequences that have the potential to cause personnel exposure to radiation from more than one type of waste form are either Category 2 event sequences resulting in a direct exposure, or beyond Category 2 event sequences resulting in a radionuclide release. In the first case, doses from direct radiation after a Category 2 event sequence have no effect on the public because of the great distances from the locations of offsite receptors. In the second case, beyond Category 2 event sequences do not require a consequence calculation. Thus, the demonstration that the performance objectives of 10 CFR 63.111 (Ref. 2.3.2) are met is not dependent on the waste form at risk in the event sequences that may involve more than one type of waste form. It is appropriate, therefore, to evaluate event sequences separately for each relevant type of waste form.

4.3.2 Initiating and Pivotal Event Analysis

The purpose of this analysis is to develop the frequency (i.e., expected number of occurrences over the 50-year operating lifetime of the facility) of each event sequence in order to categorize event sequences in accordance with 10 CFR 63.2 (Ref. 2.3.2). (In this document, the term frequency is used interchangeably with expected number when discussing event sequence quantification.) This involves developing the frequency of each initiating event and conditional probability of each pivotal event. Some pivotal events in this analysis are associated with structural or thermal events. In these cases, passive equipment failure analyses (PEFAs) are performed. The PEFAs include probabilistic structural or thermal analyses as summarized later in this section to develop mean conditional probabilities of failure directly associated with pivotal events. Often, however, the events depicted in ESDs or event trees cannot easily be mapped to such a calculation or to reliability data (e.g., failure history records). This is because large aggregates of components (e.g., systems or complicated pieces of equipment such as the waste package transfer trolley (WPTT)) may be unique to the YMP facility with little or no prior operating history. The components, however, of which it is composed, have usually been used before and there is an adequate set of reliability data for these components. The PCSA used fault trees for this mapping. As a result, the PCSA disaggregates or breaks down the initiating events and pivotal events, when needed, into a collection of simpler components. All initiating events use fault trees and the pivotal event associated with confinement is analyzed via a fault tree of the HVAC system. In effect, the use of fault trees creates a mapping between ESD or event tree events and the available reliability data.

4.3.2.1 Fault Tree Analysis

Construction of a fault tree is a deductive reasoning process that answers the question "What are all combinations of events that can cause the top event to occur?" Figure 4.3-5 demonstrates this.



NOTE: This fault tree is presented for illustrative purposes only and is not intended to represent results of the present analysis.
PLC = programmable logic controller; WPTT = waste package transfer trolley.

Source: Original

Figure 4.3-5. Example Fault Tree

This top-down analytical development defines the combinations of causes for the initiating, or pivotal events, into an event sequence, in a way that allows the probability of the events to be estimated.

As the name implies, fault tree events are usually failures or faults. Fault trees use logic or Boolean gates. Figure 4.3-5 shows two types of gates: the AND gate (mound shaped symbol with a flat bottom) and the OR gate (mound shaped symbol with a concave bottom). An AND gate passes an output up the tree if all events immediately attached to it occur. An OR gate passes an output up the tree if one or more events immediately attached to it take place. An AND gate often implies components or system features that back each other up, so that if one fails, the other continues to adequately perform the function. The success criterion of the SSC or equipment being analyzed is important in determining the appropriate use of gates.

The bottom level of the fault tree contains events with circles beneath them indicating a *basic event*. Basic events are associated with frequencies from industry-wide active equipment reliability information, passive equipment failure analysis, or human reliability analysis.

Fault trees are Boolean reduced to "minterm" form, which expresses the top event in terms of the union of minimal cut sets. Minimal cut sets, which are groups of basic events that must all occur to cause the top event in the fault tree, result from applying the Boolean Idempotency and Absorption laws. Fault tree analysis, as used in the PCSA, is well described in the NUREG-0492 (Ref. 2.2.84). Each minimal cut set represents a single basic event or a combination of two or more basic events (e.g., a logical intersection of basic events) that could result in the occurrence of the event sequence. Minimal cut sets are minimal in the sense that they contain no redundant basic events (i.e., if any basic event were removed from a minimal set, the remaining basic events together would not be sufficient to cause the top event). Section 4.3.6 continues the discussion about utilization of minimal cut sets in the quantification of event sequences.

As illustrated in Figure 4.3-5, the organization of the fault trees in the PCSA is developed to emphasize two primary elements, which together result in the occurrence of the top event: (1) human failure events, and (2) equipment failures. The human failure events include postulated unintended crew actions and omissions of crew actions. Identification and quantification of human failure events (HFEs) are performed in phases. Initial identification of HFEs led to design changes to either eliminate them or reduce the probability that they would cause the fault tree top event. For example, Figure 4.3-5 shows an HFE logically intersected with an electro-mechanical interlock such that both a crew error of commission and failure of the interlock must occur for premature WPTT tiltdown to occur.

Event trees and fault trees are complementary techniques. Often used together, they map the system response from initiating events through damage levels. Together, they delineate the necessary and sufficient conditions for the occurrence of each event sequence (and end state). Because of the complementary nature of using both inductive and deductive reasoning processes, combining event trees and fault trees allow more comprehensive, concise, and clearer event sequences to be developed and documented than using either one exclusively. The selection of and division of labor among each type of diagram depends on the analyst's opinion. In the PCSA, the choice was made to develop event trees along the lines of major functions such as crane lifts, waste container containment, HVAC and building confinement, and introduction of moderator. Fault trees disaggregate these functions into equipment and component failure modes for which unreliabilities or unavailabilities were obtained.

4.3.2.2 Passive Equipment Failure Analysis

Passive equipment (e.g., transportation casks, storage canisters, waste packages) may fail from manufacturing defects, material variability, defects introduced by handling, long-term effects such as corrosion, and normal and abnormal use. Industry codes, such as *Minimum Design Loads for Buildings and Other Structures* (Ref. 2.2.7) and *2004 ASME Boiler and Pressure Vessel Code* (Ref. 2.2.9) establish design load combinations for passive structures (such as building supports) and components (such as canisters). These codes specify design basis load combinations and provide the method to establish allowable stresses. Typical load combinations for buildings involve snow load, dead (mass) load, live occupancy load, wind load, and earthquake load. Typical load combinations for canisters and casks are found in *2004 ASME Boiler and Pressure Vessel Code* (Ref. 2.2.9) and would include, for example, preloads or pre-stresses, internal pressurization and drop loads, which are specified in terms of acceleration.

Design basis load combinations are purposefully specified to conservatively encompass anticipated normal operational conditions as well as uncertainties in material properties and analysis. Therefore, passive components, when designed to codes and standards and in the absence of significant aging, generally fail because of load combinations or individual loads that are much more severe than those anticipated by the codes. Fortunately, the conservative nature of establishing the design basis coupled with the low probability of multiple design basis loads occurring concurrently often means a significant margin or factor of safety exists between the design point and actual failure. The approach used in the PCSA takes advantage of the design margins (or factor of safety).

The development of code requirements for minimum design loads in buildings and other structures in the late 1970's considered multiple loads. A probabilistic basis for structural reliability was developed as part of the development of *Development of a Probability Based Load Criterion for American National Standard A58, Building Code Requirements for Minimum Design Loads in Buildings and Other Structures* (Ref. 2.2.44). This document refers to classic structural reliability theory. In this theory, each structure has a limit state (e.g., yield or ultimate), such that, loads and resistances are characterized by Equation 1:

$$g(x_1,x_2,\ldots x_i,\ldots x_n) = 0 \qquad \text{(Eq. 1)}$$

In Equation 1, g is termed the limit-state variable where failure is defined as $g < 0$ and the x_i are resistance (sometimes called capacity or fragility) variables or load (sometimes called stress or demand) variables. The probability of failure of a structure is given, in general, by Equation 2:

$$P_f = \int \ldots \int f_x(x_1, x_2, \ldots x_i \ldots x_n) dx_1 dx_2 \ldots dx_n \qquad \text{(Eq. 2)}$$

Where f_x is the joint probability density function of x_i and the integral is over the region in which $g < 0$. The fact that these variables are represented by probability distributions implies that absolutely precise values are not known. In other words, the variable values are uncertain. This concept is illustrated in Figure 4.3-6. Codes and standards such as *Minimum Design Loads for Buildings and Other Structures* (Ref. 2.2.7), guide the process of designing structures such that there is a margin, often called a factor of safety, between the load and capacity. The factor of safety is established in recognition that quantities, methods used to evaluate them, and tests used to ascertain material strength give rise to uncertainty. A heuristic measure of the factor of safety is the distance between the mean values of the two curves.

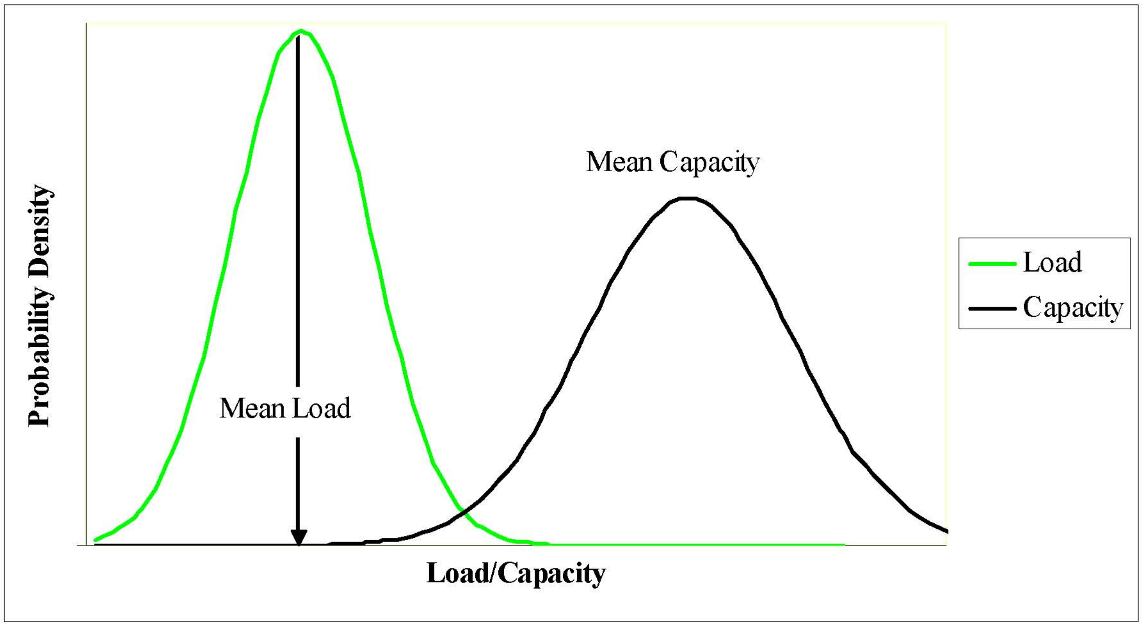


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Figure 4.3-6. Concept of Uncertainty in Load and Resistance

In the case in which Equations 1 and 2 are approximated by one variable representing capacity and the other representing load, each of which is a function of the same independent variable *y*, the more familiar load-capacity interference integral results as shown in Equation 3.

$$P_f = \int F(y)h(y)dy \qquad \text{(Eq. 3)}$$

P_f is the mean probability of failure and is appropriate for use when comparing to a probability criterion such as one in a million. In Equation 3, *F(y)* represents the cumulative density function (CDF) of structural capacity and *h(y)* represents the probability density function (PDF) of the load. The former is sometimes called the fragility function and the later is sometimes called the hazard function.

To analyze the probability of breach of a dropped canister, *y* is typically in units of strain, *F* is typically a fragility function, which provides the conditional probability of breach given a strain, and *h* is the probability density function of the strain that would emerge from the drop. For seismic risk analysis, *h* represents the seismic motion input, *y* is in units of peak ground acceleration, and *F* is the seismic fragility. The seismic analysis of the YMP structures is documented separately in *Seismic Event Sequence Quantification and Categorization* (Ref. 2.4.4). Degradation of shielding owing to impact loads uses a strain to failure criterion within the simplified approach of Equation 4, described below. For analysis of the conditional probability of breach owing to fires, *y* is temperature, *F* is developed from fire data for non-combustible structures, and *h* is developed using probabilistic heat transfer calculations. Analysis for heating up casks, canisters, and waste packages associated with loss of building forced convection cooling was similarly accomplished, but Equation 4 was used.

If load and capacity are known, then Equations 2 and 3 provide a single valued result, which is the mean probability of failure. Each function in Figure 4.3-6 is characterized by a mean value, $\overline{L}$ and $\overline{R}$, and a measure of the uncertainty, generally the standard deviation, usually denoted by σ_L and σ_R for L and R, respectively. The spread of the functions may be expressed, alternatively, by the corresponding coefficient of variation *(V)* given by the ratio of standard deviation to mean, or $V_L = \sigma_L / \overline{L}$ and $V_R = \sigma_R / \overline{R}$ for load and resistance, respectively. The coefficient of variation may be thought of as a measure of dispersion expressed in terms of the number of means.

In the PCSA, the capacity curve for developing the fragility of casks and canisters against drops was constructed by a statistical fit to tensile elongation to failure tests (Ref. 2.2.35). The load curve may be constructed by varying drop height. A cumulative distribution function may be fit to a locus of points each of which is the product of drop height frequency and strain given drop height.

Impact Events Associated with Containment Breach

A simplification of Equation 3, consistent with HLWRS-ISG-02 (Ref. 2.2.69), and shown in Equation 4 is used in the PCSA. It is illustrated in Figure 4.3-7.

$$P_f = \int_0^h F(y)dy \qquad \text{(Eq. 4)}$$

In Equation 4, ***h*** is a single value conservative load.

The load is a single value estimated by performing a calculation for a condition more severe than the mean. For example, if the normal lift height of the bottom of a canister is 23 ft, a drop height of 32.5 ft is more severe and may be conservatively applied to all drop heights equal to or below this height. The conditional probability of breach is an increasing function of drop height. Strain resulting from drops is calculated by dynamic finite element analysis using LS-DYNA for canisters and transportation cask drops (Ref. 2.2.35). Therefore, use of a higher than mean drop height for the load for all drop heights, results in a conservative estimate of breach probability. As an additional conservatism, a lower limit of breach probability of 1E-05 was placed on drops of casks, canisters, and waste packages. To perform the analyses, representative canisters and casks were selected from the variety of available designs in current use which were relatively thin walled on the sides and bottom. This added another conservative element.

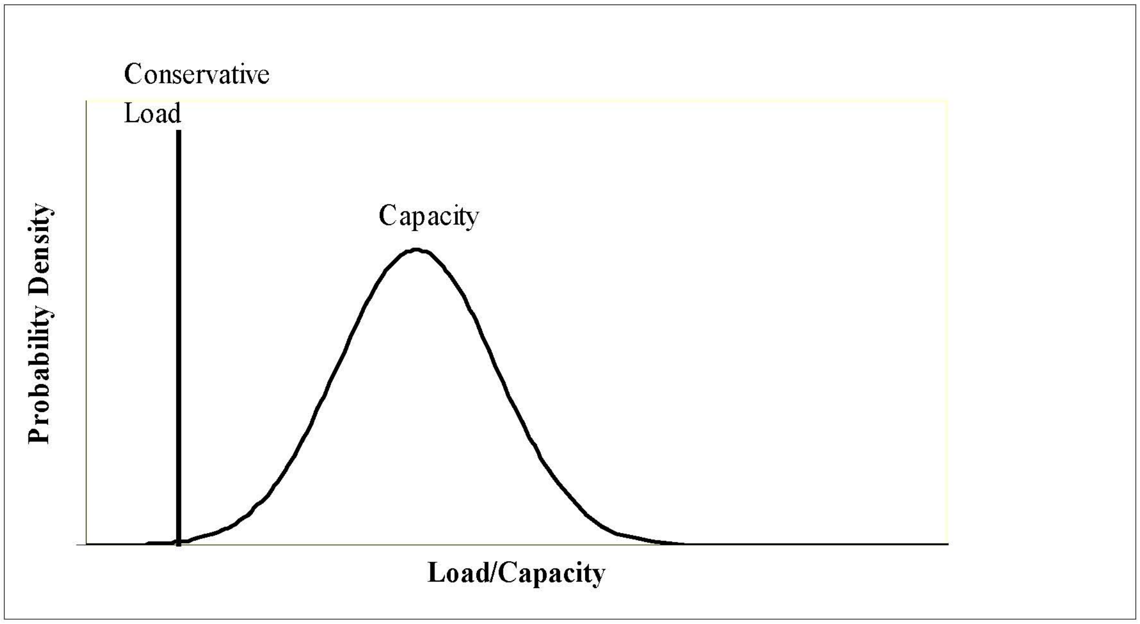

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Figure 4.3-7. Point Estimate Load Approximation Used in PCSA

The PCSA applies PEFAs to a wide variety of event sequences including those associated with:

- Canister drops
- Canister collisions with other objects and structures
- Other objects dropped on canisters
- Transportation cask drops and subsequent slapdowns (analyzed without impact limiters)
- Conveyance derailments and collisions when carrying transportation casks and canisters (conveyances would be trucks, railcars, cask transfer trolleys, and site transporters)
- Other objects dropped on transportation casks
- Waste package drops
- Waste package collision with other waste packages
- Transport and emplacement vehicle (TEV) collisions with structures and another TEV when carrying a waste package
- Objects dropped on waste packages
- Objects dropped on TEV.

Many of these, such as collisions, derailments, and objects dropped onto casks/canisters, involve far lower energy loads than drop events. For impact loads that are far less energetic than drops, the drop probability is ratioed by impact energy to estimate the less energetic situation.

Shielding Degradation Events

Impact loads (such as drops) may not be severe enough to breach a transportation cask, but might lead to degradation of shielding such that onsite nearby personnel are exposed.

The shielding degradation analysis is based primarily on results of finite-element modeling (FEM) performed for four generic transportation casks types for transportation accidents, as reported in NUREG/CR-6672 (Ref. 2.2.80). The results of the FEM analysis were used to estimate threshold drop heights and thermal conditions at which loss of shielding (LOS) may occur in repository event sequences. The four cask types include one steel monolith rail cask, one steel/depleted uranium/steel (SDU) truck cask, one steel/lead/steel (SLS) truck cask, and one SLS rail cask. The study performed structural and thermal analyses for both failure of containment boundaries and loss of shielding for accident scenarios involving rail cask and truck cask impacting unyielding targets at various impact speeds from 30 mph to greater than 120 mph. Impact orientations included side, corner, and end. The study also correlated the damage to impacts on real targets, including soil and concrete.

NUREG/CR-6672 (Ref. 2.2.80) addresses two modes of shielding degradation in accident scenarios: Deformations of lid and closure geometry that permit direct streaming of radiation; and/or reductions in cask wall thickness, or relocation of the depleted uranium or lead shielding. The shielding degradation due to lid/closure distortion can be accompanied by air-borne releases if the inner shell of the cask is also breached.

The structural analyses do not credit the energy absorption capability of impact limiters. Therefore, the results are deemed applicable to approximate the structural response of transportation and similar casks in drop scenarios for the RF.

Principal insights reported in NUREG/CR-6672 (Ref. 2.2.80) are the following:

- Monolithic steel rail casks do not exhibit any shielding degradation, but there may be some radiation streaming through gaps in closures in any of the impact scenarios.
- SDU truck cask exhibited no shielding degradation, explained by modeling that included no gaps between forged depleted uranium segments so that no displacement of depleted uranium could occur.
- The SLS rail and truck casks exhibit shielding degradation due to lead slumping. Lead slump occurs mostly on end-on impact, with a lesser amount in corner orientation. For side-on orientation, there is no significant reduction in shielding.

Therefore, this analysis focuses on SLS casks to estimate the drop or collision conditions that could result in shielding degradation from lead slumping. Since it is not possible to predict at this time the fraction of casks to be delivered during the preclosure period that will be of the steel-lead-steel type, all transportation casks are analyzed as described below.

The *Shipping Container Response to Severe Highway and Railway Accident Conditions*. NUREG/CR-4829 (Ref. 2.2.47) defines three levels of cask response, characterized by the maximum effective plastic strain within the inner shell of a transport cask. Of these, level S3 has strain levels between 2.0% and 30% which produces large distortions, seal leakage likely and lead slump likely. The minimum strain level associated with S3 was applied to the strain versus impact speed results from the FEM (Ref. 2.2.80) to establish a median threshold impact speed for the onset of shielding degradation. The threshold speeds are translated into equivalent drop heights, using calculated bottom corner drops for impact loads onto real concrete targets, not idealized rigid targets. Use of a conservative coefficient of variation, coupled with the median, allowed a lognormal fragility curve as a function of drop height (or equivalently impact speed), to be developed. Each event sequence may be characterized by a conservative impact speed. For example, the maximum speed of onsite vehicles is 2.5 mph by design (with exception of 9 mph for the site prime mover) and a cask drop height of 15 ft is unlikely, by design, to be exceeded. Using Equation 4, the fragility curve was combined with the maximum or a conservative estimate of impact speed (or equivalent drop height).

Fire Events Associated with Possible Containment Breach

Fire initiated events are included in the PCSA, which probabilistically analyzes the full range of possible fires that can occur, as well as variations in the dynamics of the heat transfer and uncertainties in the failure temperature of the target. This analysis focuses on fires that might directly impact the integrity of cask, canister, and waste package containment. Equation 3 is used for this purpose. The fragility analysis includes the uncertainty in the temperature that containment will be breached, and the uncertainty in the thermal response of the canister to the fire. In calculating the thermal response of the canister, variations in the intensity and duration of the fire are considered along with conditions that control the rate of heat transfer to the container, e.g., convective heat transfer coefficients, view factors, emissivities, etc. In calculating the failure temperature of the canister, variations in the material properties of the canister are considered, along with, variations in the loads that lead to failure. The load or demand is associated with uncertainty in the fire severity.

Fire severity is characterized by the fire temperature and duration, since these factors control the amount of energy that the fire could transfer to a cask, canister, or waste package. (In this analysis, these are referred to as targets.) The duration of the fire is taken to be the amount of time a particular container is exposed to the fire, and not necessarily the amount of time a fire burns. Probability distributions of the fire temperature and fire duration are based on the unavailability of manual or automatic suppression, which leads to an assessment that significantly overstates the risk of fires.

4.3.2.2.1 Uncertainty in Fire Duration

An uncertainty distribution for the fire duration is developed by considering test data and analytical results reported in several different sources; some specific to the YMP facilities and some providing more generic information. In general, the fire durations are found to depend upon the amount, type, and configuration of the available combustible material.

Based on a review of the available information, it is determined that two separate uncertainty distributions would be needed: one for conditions without automatic suppression and one for conditions with automatic suppression. The derivation of these two distributions is discussed below.

Uncertainty in fire duration was developed from:

- *Utilisation of Statistics to Assess Fire Risks in Buildings* (Ref. 2.2.82)
- *Heat and Mass Release for Some Transient Fuel Source Fires: A Test Report.* NUREG/CR-4680 (Ref. 2.2.60)
- *Quantitative Data on the Fire Behavior of Combustible Materials Found in Nuclear Power Plants: A Literature Review*. NUREG/CR-4679 (Ref. 2.2.61).

The derivation of the distribution of fire duration is described in Attachment D, Sections D2.1.1.2 and D2.1.1.3.

The fire temperature used in this calculation is the effective blackbody temperature of the fire. This temperature implicitly accounts for the effective emissivity of the fire, which for large fires approaches a value of 1.0 (Ref. 2.2.75, p. 2-56). Fires within a YMP facility may involve both combustible solid and liquid materials. A probability distribution for the fire temperature was derived by combining the fire severity information about compartment fires discussed in *SFPE Handbook of Fire Protection Engineering* (Ref. 2.2.75, Section 2, Chapter 2) with information about liquid hydrocarbon pool fires (Ref. 2.2.3 and Ref. 2.2.75, p. 2-56). The derivation of this distribution is described in Attachment D, Section D2.1.2. The fire temperature is normally distributed with a mean of 1,072°K (799°C) and a standard deviation of 172°K. The mean of this distribution is approximately equal to the transportation cask design basis fire temperature of 800°C specified in 10 CFR 71.73 (Ref. 2.3.3).

Fire temperature and duration are negatively correlated. Intense fires with high fire temperatures tend to be short-lived because the high temperature results from very rapid burning of the combustible material. In determining the joint probability distribution of fire duration and temperature, a negative correlation coefficient of -0.5 was used (refer to Attachment D, Section D2.1.3).

The thermal response of the canister is calculated using simplified radiative, convective, and conductive heat transfer models, which have been calibrated to more precise models. The simplified models are found to accurately match predictions for heating of the canister in either a cask or waste package. The heat transfer models are simplified in order to allow a probabilistic analysis to be performed using Monte Carlo sampling. The models consider radiative and convective heat transfer from the fire to the canister, cask, waste package, or shielded bell. This analysis conservatively models the fire completely engulfing the container.

When calculating the heat load on the target for a fully engulfing fire, radiation is the dominant mode of heat transfer between the fire and the target. The magnitude of the radiant heating of the container depends on the fire temperature, the emissivity of the container, the view factor between the fire and the container, also the duration of the fire.

The total radiant energy deposited in the container can be roughly estimated using Equation 5:

$$Q_{rad} = \varepsilon F_{cf} \sigma (T_{fire})^4 At \qquad \text{(Eq. 5)}$$

where

Q_{rad}	=	incident radiant energy over the fire duration (J)
ε	=	emissivity of the container
F_{cf}	=	container-to-fire view factor
σ	=	Stefan-Boltzmann constant ($W/m^2 K^4$)
T_{fire}	=	equivalent blackbody fire temperature (K)
A	=	container surface area (m^2)
t	=	duration of the fire (s)

The following variables in this equation are treated as uncertain: fire temperature, view factor, and fire duration. In the case of a canister inside a waste package, cask, or shielded bell, a more complicated set of equations is used to simulate outer shell heat up and subsequent heat transfer to layers of containment or shielding and then to the canister itself. The model also includes heating of the canister by decay heat from the spent fuel or high-level radioactive waste.

To estimate the uncertainty associated with target fragility, two failure modes were considered:

1. Creep-Induced Failure. Creep is the plastic deformation that takes place when a material is held at high temperature for an extended period under tensile load. This mode of failure is possible for long duration fires.

2. Limit Load Failure. This failure mode occurs when the load exerted on a material exceeds its structural strength. As the temperature of the canister increases in temperature, its strength decreases. Failure is generally predicted at some fraction (usually around 70%) of the ultimate strength.

Failure is considered to occur when either of the failure thresholds is exceeded.

Equation 3, along with the heat transfer equations, are solved using Monte Carlo simulation (described in Section 4.3.6) with the above described fragility and target fire severity probability distributions, and distributions for the uncertain heat transfer factors. For each Monte Carlo trial, the calculated maximum canister temperature is compared to the sampled target failure temperature. If the maximum temperature of the target exceeds the sampled failure temperature, then target failure is counted. The failure probability in this method is equal to the fraction of the samples for which failure is calculated.

Uncertainty in the calculated canister failure probability is given by a calculated mean and standard deviation, where the mean is simply the number of failures divided by the total number of samples and the standard deviation is given by Equation 6 for the standard deviation of a binomial distribution:

$$\sigma = \sqrt{\frac{\frac{n_{fail}}{N}\left(\frac{N - n_{fail}}{N}\right)}{N}} \qquad \text{(Eq. 6)}$$

where n_{fail} is the number of trials in which failure occurs and N is the total number of Monte Carlo trials.

Fire Event Associated with Shielding Degradation

The thermal analyses in NUREG/CR-6672 (Ref. 2.2.80) indicates that the probability of shielding degradation in a fire scenario should be based on the probability of having a fire that is equivalent to a 1,000°C engulfing fire that lasts for more than a half-hour. However, shielding degradation does not occur unless there is a coincident puncture or breach in the cask that allows a pathway for melted lead to flow out of its usual configuration. These threshold conditions apply to all cask types and would result in radiation streaming from the cask.

The transportation cask is present within the YMP facilities in only three areas: vestibules, preparation rooms, and unloading rooms. The fire ignition frequencies of these areas are summed up in Section 6.5 and Attachment F. Furthermore, the method described above for obtaining the probability distribution of fire severity from input distributions of fire temperature and fire duration, resulted in an estimate of the conditional probability of the threshold fire given a fire ignition. This is a conservative calculation because it did not include the conditional probability that a puncture or failure through the wall to the lead shielding must also occur for shielding degradation.

Other Thermal Events Associated with Possible Breach

The PCSA focuses on the potential of cask, canister, and waste package breach associated with fires. As described above, the fires of most interest were those that surround the target containment. However, heatup associated with loss of building cooling was also considered.

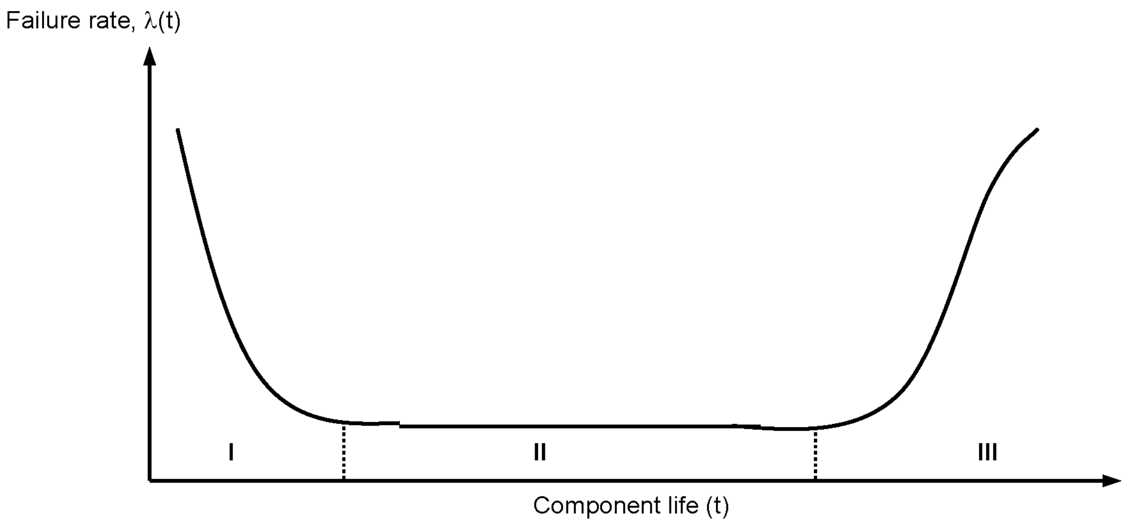
The analysis of loss of building cooling on containment integrity takes a similar, conservative, analytical approach. A bounding set of conditions and configurations are postulated, and then using the ANSYS code (Ref. 2.2.14), the maximum steady state temperature is compared to the temperature at which the component would be expected to fail. In no case is a containment barrier near its failure threshold from loss of building cooling.

4.3.3 Utilization of Industry-Wide Reliability Data

4.3.3.1 Use of Population Variability Data

The quantification of event sequence probabilities via event tree and fault tree modeling requires information on the reliability of active equipment and components, as usually represented in fault tree basic events. The PCSA attempts to anticipate event sequences before they happen, which means that associated equipment reliabilities are uncertain.

As presented in NUREG-0492 (Ref. 2.2.84, Figure X-8, p. X-23), the typical model of failure probability for a component is depicted as a “bathtub curve” illustrated in Figure 4.3-8. The curve is divided into three distinct phases. Phase I represents the component failure probability during the “burn-in” period. Phase II corresponds to the “constant failure rate function” where the exponential distribution can be applied to calculate the probability of failure within a specified “mission time.” Toward the end of the component life or the wear-out period, which is represented by Phase III of the curve; the probability of failure increases.



Source: *Fault Tree Handbook.* NUREG-0492 (Ref. 2.2.84, Figure X-8, p. X-23).

Figure 4.3-8. Component Failure Rate “Bathtub Curve” Model

As is usually done in PRA, the PCSA uses Phase II because Phase I failures are identified by burn-in testing of equipment before repository operations occur and Phase III failures are eliminated by preventive maintenance which includes manufacturer recommended replacement intervals. In Phase II, the component time-to-failure probability can be represented with the exponential distribution. The probability of failure of a given component (or system) depends on the value of the constant failure rate, λ, and the mission time, t_m, as follows in Equation 7:

$$P_F(\lambda, t_m) = 1 - \exp(-\lambda t_m) \qquad \text{(Eq. 7)}$$

When the product λt_m is small (<0.1), the failure probability may be calculated by the following Equation 8 approximation, which introduces less than a 10% error:

$$P_F(\lambda,t_m) \cong \lambda t_m \quad \text{(Eq. 8)}$$

The PCSA also uses the concept of unavailability to estimate basic event probabilities. This applies to standby equipment such as the emergency diesel generators and fire suppression. In accordance with reliability theory, that after each test the component or system is "good as new" with a "resetting" of the time-to-failure "clock" for the exponential failure model. The unavailability factor is evaluated as the probability of failure during the time between tests, τ. The average unavailability factor, or failure on demand of the standby unit, q_d, is calculated as shown in Equation 9:

$$q_d(\lambda,\tau) = \tfrac{1}{2}(\lambda\tau) \quad \text{(Eq. 9)}$$

In this model, the component failure rate is constant between tests, the test does not require any time, and the test neither introduces another failure mode nor changes the failure rate of the component.

Failure on demand is also needed for equipment, such as cranes, that is challenged in discrete steps. This model is not based on time in service; it is based on the number of times the component or system is called upon to perform its safety function.

Information about hardware failure is characterized as one of the following:

1. Historical performance of successes and failures of an identical piece of equipment under identical environmental conditions and stresses that are being analyzed (e.g., operational experience).

2. Historical performance of successes and failures of an identical piece of equipment under conditions other than those being analyzed (e.g., test data).

3. Historical performance of successes and failures of a similar piece of equipment or similar category of equipment under conditions that may or may not be those under analysis (e.g., another program's test data or data from handbooks or compilations).

4. General engineering or scientific knowledge about the design, manufacture, and operation of the equipment or an expert's experience with the equipment.

The YMP repository has not yet operated, and test information on prospective equipment has not yet been developed. The equipment and SSCs designed and purchased for the Yucca Mountain repository will be of the population of equipment and SSCs represented in U.S. industry-wide reliability information sources (Assumption 3.2.1). Furthermore, the uncertainty in reliability is represented by the variability of reliabilities across this population. Attachment C contains the list of industry-wide reliability information sources used in the PCSA.

The lack of actual operating experience, the use of industry-wide data, and the consideration of uncertainties (Ref. 2.2.69) suggested that a Bayesian approach was appropriate for the PCSA. A Bayesian approach and the use of judgment in expressing the state-of-knowledge of basic event unreliability is a well-recognized and accepted practice (Ref. 2.2.55, Ref. 2.2.11, and Ref. 2.2.63). Furthermore, to paraphrase HLWRS-ISG-02, reliability estimates for high reliability SSC may include the use of engineering judgment, supported by sufficient technical basis; and empirical reliability analyses of a SSC, could include values based on industry experience and judgment (Ref. 2.2.69).

Let λ_j be one failure rate of a set of possible failure rates of a component and E be a new body of evidence. Knowledge of the probability of λ_j given E, is represented by $P(\lambda_j/E)$. For a failure rate, frequency, or probability of active equipment, Bayes' theorem is stated as follows in Equation 10:

$$P(\lambda_j / E) = \frac{P(\lambda_j) L(E / \lambda_j)}{\sum_j P(\lambda_j) P(E / \lambda_j)} \qquad \text{(Eq. 10)}$$

In summary, this states that the knowledge of the "updated" probability of λ_j, given the new information E, equals the "prior" probability of λ_j, before any new information, times the likelihood function, $L(E/\lambda_j)$. The likelihood function is a probability that the new information really could be observed, given the failure rate λ_j. The numerator in Equation 10 is divided by a normalization factor, which must be such that the sum of the probabilities over the entire set of λ_j equals unity. If there is actual operational experience available, then the steps in an application of Bayes' theorem would be as follows: (1) estimate the prior probability using one or more of the four reliability data types; (2) obtain new information in the form of tests or experiments; (3) characterize the test information in the form of a likelihood function; and (4) perform the calculation in accordance with Equation 10 to infer the updated probability.

The PCSA used industry-wide reliability data to develop Bayesian prior distributions for each active equipment/component failure mode in the fault trees. Updates per Equation 10 will await actual test and operations. The following summarizes the methods used to develop the Bayesian prior distributions.

Using multiple reliability databases will typically cause a given active component to have various reliability estimates, each one from a different source. These various estimates can be viewed as independent samples from the same distribution, g, representing the source-to-source variability, also called population variability, of the component reliability (Ref. 2.2.11, Section 8.1). In a Bayesian approach to reliability estimation, the population-variability distribution of a component constitutes an informative prior distribution for its reliability. The population-variability distributions developed in this analysis attempt to encompass the actual component reliability distributions that will be observed at the GROA when operating experience becomes available.

A parametric empirical Bayes method is used to develop the population-variability distributions of active components considered in the PCSA. As indicated in "Bayesian Parameter Estimation in Probabilistic Risk Assessment" (Ref. 2.2.76, Section 5.1.2), this method is a pragmatic approach that has been used in PRA-related applications; it involves specifying the functional form of the prior population-variability distribution, and fitting the prior to available data, using classical techniques, for example, the maximum likelihood method. A discussion of the adequacy of the parametric empirical Bayes method for determining the population-variability distribution is given at the end of this section.

Applying the parametric empirical Bayes method requires first, to categorize the reliability data sources into two types: those that provide information on exposure data, (i.e., the number of failures that were recorded over an exposure time (in case of a failure rate)), or over a number of demands (in case of a failure probability), and those that do not provide such information. In the latter case, reliability estimates for a failure rate or failure probability are provided in the form of a mean or a median value, along with an uncertainty estimate, typically an error factor.

For each data source, the reliability information about a component's failure rate or failure probability is mathematically represented by its likelihood function. If exposure data are provided, the likelihood function takes the form of a Poisson distribution (for failure rates), or a binomial distribution (for failure probabilities) (Ref. 2.2.76, Section 4.2). When no exposure data is available, the reliability estimates for failure rates or failure probabilities are interpreted as expert opinion, for which an adequate representation of the likelihood function is a lognormal distribution ((Ref. 2.2.76, Section 4.4) and (Ref. 2.2.53, pp. 312, 314, and 315)).

The next step is to specify the form of the population-variability distribution. In its simplest form, the parametric empirical Bayes method only considers exposure data and employs distributions that are conjugate to the likelihood function (i.e., a gamma distribution if the likelihood is a Poisson distribution, and a beta distribution if the likelihood is binomial) (Ref. 2.2.11, Section 8.2.1), which have the advantage of resulting in relatively simpler calculations. This technique, however, is not applicable when both exposure data and expert opinion are to be taken into consideration, because no conjugate distribution exists in this situation. Following the approach of *The Combined Use of Data and Expert Estimates in Population Variability Analysis* (Ref. 2.2.53, Section 3.1), the population-variability distribution in this case is chosen to be lognormal. More generally, for consistency, the parametric empirical Bayes method is applied using the lognormal functional form for the population-variability distributions regardless of the type of reliability data available for the component considered (exposure data, expert opinion, or a combination of the two). In the rest of this section, the population-variability distribution in its lognormal form is noted $g(x, \nu, \tau)$, where x is the reliability parameter for the component (failure rate or failure probability), and ν and τ, the two unknowns to be determined, are respectively the mean and standard deviation of the normal distribution associated with the lognormal. The use of a lognormal distribution is appropriate for modeling the population-variability of failure rates and failure probabilities, provided in the latter case that any tail truncation above $x = 1$ has a negligible effect (Ref. 2.2.76, p. 99). The validity of this can by confirmed by selecting the failure probability with the highest mean and the most skewed lognormal distribution and calculating what the probability is of exceeding one. In Table C4-1 of Appendix C, PRV-FOD fits this profile, with a mean failure probability of 6.54E-03 and an error factor of 27.2. The probability that the distribution exceeds one is 2E-04. Stated equivalently, 99.98% of the values taken by the distribution are less than one. This confirms that the use of a truncated lognormal distribution to represent the probability distribution is appropriate.

To determine ν and τ, it is first necessary to express the likelihood for each data source as a function of ν and τ only, (i.e., unconditionally on x). This is done by integrating, over all possible values of x, the likelihood function evaluated at x, weighted by the probability of observing x, given ν and τ. For example, if the data source i indicates that r failures of a component occurred out of n demands, the associated likelihood function $L_i(\nu, \tau)$, unconditional on the failure probability x, is as follows in Equation 11:

$$L_i(\nu,\tau) = \int_0^1 Binom(x,r,n) \times g(x,\nu,\tau)dx \qquad \text{(Eq. 11)}$$

where $Binom(x,r,n)$ represents the binomial distribution evaluated for r failures out of n demands, given a failure probability equal to x, and $g(x,\nu,\tau)$ is defined as previously indicated. This equation is similar to that shown in "Bayesian Parameter Estimation in Probabilistic Risk Assessment" (Ref. 2.2.76, Equation 37). If the component reliability is expressed in terms of a failure rate and the data source provides exposure data, the binomial distribution in Equation 11 would be replaced by a Poisson distribution. If the data source provided expert opinion only (no exposure data), the binomial distribution in Equation 11 would be replaced by a lognormal distribution.

The maximum likelihood method is an acceptable method to determine ν and τ (Ref. 2.2.76, p. 101). The maximum likelihood estimators for ν and τ are obtained by maximizing the likelihood function for the entire set of data sources. Given the fact that the data sources are independent, the likelihood function is the product of the individual likelihood functions for each data source (Ref. 2.2.53, Equation 4). To find the maximum likelihood estimators for ν and τ, it is equivalent and computationally convenient to maximize the log-likelihood function, which is the sum of the logarithms of the likelihood function for each data source.

The calculation of ν and τ completely determines the population-variability distribution g for the reliability of a given active component. The associated parameters to be plugged into SAPHIRE are the mean and the error factor of the lognormal distribution g, which are calculated using the formulas given in NUREG/CR-6823 (Ref. 2.2.11, Section A.7.3). Specifically, the mean of the lognormal distribution is equal to $\exp(\nu + \tau^2/2)$ and the error factor is equal to $\exp(1.645 \times \tau)$. A discussion of the adequacy of the empirical Bayes method for the YMP analysis is found in Attachment C, Section C2.1.

An adjustment to the parametric empirical Bayes method was done in a few instances where the error factor of the calculated lognormal distribution was found to be excessive. In a synthetic examination of the failure rates of various components, "External Maintenance Rate Prediction and Design Concepts for High Reliability and Availability on Space Station Freedom" (Ref. 2.2.49, Figure 3) finds that electromechanical and mechanical components have, overall, a range of variation approximately between 2×10^{-8}/hr (5th percentile) and 6×10^{-5}/hr (95th percentile). Using the definition of the error factor given in NUREG/CR-6823

(Ref. 2.2.11, Section A.7.3), this corresponds to an error factor of $\sqrt{6 \cdot 10^{-5} / 2 \cdot 10^{-8}} = 55$. Therefore, in the PCSA, it is considered that lognormal distributions resulting from the empirical Bayes method that yield error factors with a value greater than 55, are too diffuse to adequately represent the population-variability distribution of a component. In such instances (i.e., the two cases in the entire PCSA database when the error factors from the Bayesian estimation were greater than 200), the lognormal distribution used to represent the population-variability is modified as follows. It has the same median as that predicted by the parametric empirical Bayes method, and its error factor is assigned a value of 55. The median is selected as the unvarying parameter because, contrary to the mean, it is not sensitive to the behavior of the tails of the distribution, and therefore is unaffected by the value taken by the error factor. Based on NUREG/CR-6823 (Ref. 2.2.11, Section A.7.3), the median is calculated as exp(ν), where ν is obtained by the maximum likelihood estimation.

A limitation of the parametric empirical Bayes method that prevented its use for all active components of the PCSA is that the calculated lognormal distribution can sometimes have a very small error factor (with a value around 1), corresponding to a distribution overly narrow to represent a population-variability distribution. As indicated in NUREG/CR-6823 (Ref. 2.2.11, p. 8-4), this situation can arise when the reliability data sources provide similar estimates for component reliability. The inadequacy of the parametric empirical Bayes method in such situations is made apparent by plotting the probability density function of the lognormal distribution, and comparing it with the likelihood functions associated with the reliability estimates of each data source. In the cases where the lognormal distribution does not approximately encompass the likelihood functions yielded by the data sources, it is not used to model the population-variability distribution. Instead, this distribution is modeled using the data source that yields the most diffuse likelihood using one of the two methods described in the next paragraph.

To be developed, a population-variability distribution requires at least two data sources, and therefore the previous method is not applicable when only one data source is available. In this case, the probability distribution for the reliability parameter of an active component is that yielded by the data source. For example, if the data source provides a mean and an error factor for the component reliability parameter, the probability distribution is modeled in SAPHIRE as a lognormal distribution with that mean, and that error factor. If the data source does not readily provide a probability distribution, but instead exposure data, i.e., a number of recorded failures over an exposure time for failure rates, or over a number of demands for failure probabilities, the probability distribution for the reliability parameter is developed through a Bayesian update using Jeffrey's noninformative prior distribution as indicated in NUREG/CR-6823 (Ref. 2.2.11, Section 6.2.2.5.2). This noninformative prior conveys little prior belief or information, thus allowing the data to speak for itself.

4.3.3.2 Dependent Events

Dependent events have long been recognized as a concern for those responsible for the safe design and operation of high-consequence facilities because these events tend to increase the probability of failure of multiple systems and components. Two failure events, A and B, are dependent when the probability of their coincidental occurrence is higher than expected if A and B were each an independent event. Dependent events occur from four dependence mechanisms: functional, spatial, environmental, and human:

1. **Functional dependence** is present when one component or system relies on another to supply vital functions. An example of a functional dependence in this analysis is electric power supply to HVAC. Functional dependence is explicitly modeled in the event tree and fault tree logic.

2. **Environmental dependence** is in play when system functionality relies on maintaining an environment within designed or qualified limits. Here, an example is material property change as a result of temperature change. Environmental effects are modeled in the system reliability analyses as modifications (e.g., multiplying factors) to system- and component-failure probabilities and are also included in the passive equipment failure analyses. External events such as earthquakes, lightning strikes, and high winds that can degrade multiple SSCs are modeled explicitly as initiating events and are discussed in other documents (Ref. 2.2.28 and Ref. 2.4.4).

3. **Spatial dependence** is at work when one SSC fails by virtue of close proximity to another. For example, during an earthquake one SSC may impact another because of close proximity. Another example is inadvertent fire suppression actuation which wets SSCs below it. Spatial dependences are identified by explicitly looking for them in the facility layout drawings. Inadvertent fire suppression is modeled explicitly in the event trees and fault trees.

4. **Human dependence** is present when a structure, system, component, or function fails because humans intervene inappropriately or failed to intervene. In the YMP, most human errors are associated with initiating events (inadvertent actuation) or are pre-initiator failures (failure to restore after maintenance). The PCSA includes an extensive human reliability analysis which is described later in this section, in Section 6.4 and in Attachment E. The results of the human reliability analysis (HRA) are integrated into the event tree and fault tree models for a complete characterization of event sequence frequency.

4.3.3.3 Common-Cause Failures

Common-cause failures (CCFs) can result from any of the dependence mechanisms described above. The term common-cause failure is widely employed to describe events in which the same cause degrades the function of two or more SSCs that are relied upon for redundant operations, either at the same time or within a short time relative to the overall component mission time. Because of their significance to overall SSC reliability when redundancy is employed, CCFs are a special class of dependent failures that are addressed in the PCSA.

Because CCFs are relatively uncommon, it is difficult to develop a statistically significant sample from monitoring only one system or facility, or even several systems. The development of CCF techniques and data, therefore, rely on a national data collection effort that monitors a large number of nuclear power systems. Typically, the fraction of component failures associated with common causes leading to multiple failures ranges between 1% and 10% (Ref. 2.2.48, Ref. 2.2.58, and Ref. 2.2.54). This fraction depends on the component; level of redundancy (e.g., two, three, or four); duty cycle; operating and environmental conditions; maintenance interventions; and testing protocol, among others. For example, equipment that is operated in cold standby mode (i.e., called to operate occasionally on demand) with a large amount of preventive maintenance intervention tends to have a higher fraction of CCFs than systems that continuously run.

It is not practical to explicitly identify all CCFs in a fault tree or event tree. Surveys of failure events in the nuclear industry have led to several parameter models. Of these, three are most commonly used: the Beta Factor method (Ref. 2.2.48), the Multiple Greek Letter method (Ref. 2.2.57), and the Alpha Factor method (Ref. 2.2.58). These methods do not require an explicit knowledge of the dependence failure mode.

The PCSA uses the Alpha Factor method (Ref. 2.2.58), which is summarized below. After identifying potential CCF events from the fault trees, appropriate alpha factors are identified according to the procedure described in NUREG/CR-5801 (Ref. 2.2.56). The general equations for estimating the probability of a CCF event in which *k* of *m* components fail are as follows in Equations 12, 13, and 14:

$$Q(k,m) = \frac{k}{\binom{m-1}{k-1}}\alpha_k Q_t$$

for staggered test (Eq. 12)

$$Q(k,m) = \frac{k}{\binom{m-1}{k-1}}\frac{\alpha_k}{\alpha_t}Q_t$$

for non-staggered test (Eq. 13)

where α_k denotes the alpha factor for size *k*, Q_t denotes the total failure probability, and:

$$\alpha_t = \sum_{k=1}^{m} k\alpha_k$$

(Eq. 14)

Generic alpha factors are used in the PCSA taken from NUREG/CR-5801 (Ref. 2.2.56). The process of applying these alpha factors is explained further in Attachment C, Section C3.

4.3.4 Human Reliability Analysis

Human interactions that are typically associated with the operation, test, calibration, or maintenance of an SSC (e.g., drops from a crane when using slings) are implicit in the empirical data. If this is the case, empirical data may be used, provided human errors that cause the SSC failures are explicitly enumerated and determined to be applicable to YMP operations. When this was the case in the PCSA, the appropriate method of Section 4.3.3.1 was applied. Otherwise, an HRA was performed, the methodology of which is summarized in this section. The HRA task is performed in a manner that implements the intent of the high-level requirements for HRA in *Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications* (Ref. 2.2.8) and incorporates the guidance in *Preclosure Safety Analysis – Human Reliability Analysis* (Ref. 2.2.70). It emphasizes a comprehensive qualitative analysis and uses applicable quantitative models.

The HRA task identifies, models, and quantifies HFEs postulated for YMP operations to assess the impact of human actions on event sequences modeled in the PCSA. YMP operations differ from those of traditional nuclear power plants, and the HRA reflects these differences. Appendix E.IV of Attachment E includes further discussion of these differences and how they influence the choice of methodology.

The overall steps to the PCSA HRA are identification of HFEs, preliminary analysis (screening), and detailed analysis. The HRA task ensures that the HFEs identified by the other tasks (e.g., HAZOP evaluation, MLD development): (1) are created on a basis that is consistent with the HRA techniques used, (2) are appropriately reincorporated into the PCSA (modeled HFEs derived from the previously mentioned PCSA methods), and (3) provide appropriate human error probabilities (HEPs) for all modeled HFEs. The HRA work scope largely depends on boundary conditions defined for it.

4.3.4.1 HRA Boundary Conditions

Unless specifically stated otherwise, the following general conditions and limitations are applied throughout the HRA task. The first two conditions always apply. The remaining conditions apply unless the HRA analyst determines that they are inappropriate. This judgment is made for each individual action considered:

1. Only HFEs made in the performance of assigned tasks are considered. Malevolent behaviors (e.g., deliberate acts of sabotage) are not considered in this task.

2. All personnel act in a manner they believe to be in the best interests of operations and safety. Any intentional deviation from standard operating procedures is made because employees believe their actions to be more efficient or because they believe the action as stated in the procedure to be unnecessary.

3. Since the YMP is currently in the design phase, facility-specific information and operating experience is generally not available. Instead, similar operations involving similar hazards and equipment are reviewed to establish surrogate operating experience to use in the qualitative analysis. Examples of reviewed information would include SNF handling at reactor sites having independent spent fuel storage and any other facilities whose primary function includes handling and disposal of very large containers of extremely hazardous material. Equipment design and operational characteristics at the GROA facilities, once they are built and operating (including crew structures, training, and interactions), are adequately represented by these currently operating facilities.

4. The YMP is initially operating under normal conditions and is designed to the highest quality human factor specifications. The level of operator stress is optimal unless the analyst determines that the human action in question cannot be accommodated in such a manner as to achieve optimal stress.

5. In performing the operations, the operator does not need to wear protective clothing unless it is an operation similar to those performed in other comparable facilities where protective clothing is required.

6. The tasks are performed by qualified personnel, such as operators, maintenance workers, or technicians. All personnel are certified in accordance with the training and certification program stipulated in the license. They are to be experienced and have functioned in their present positions for a sufficient amount of time to be proficient.

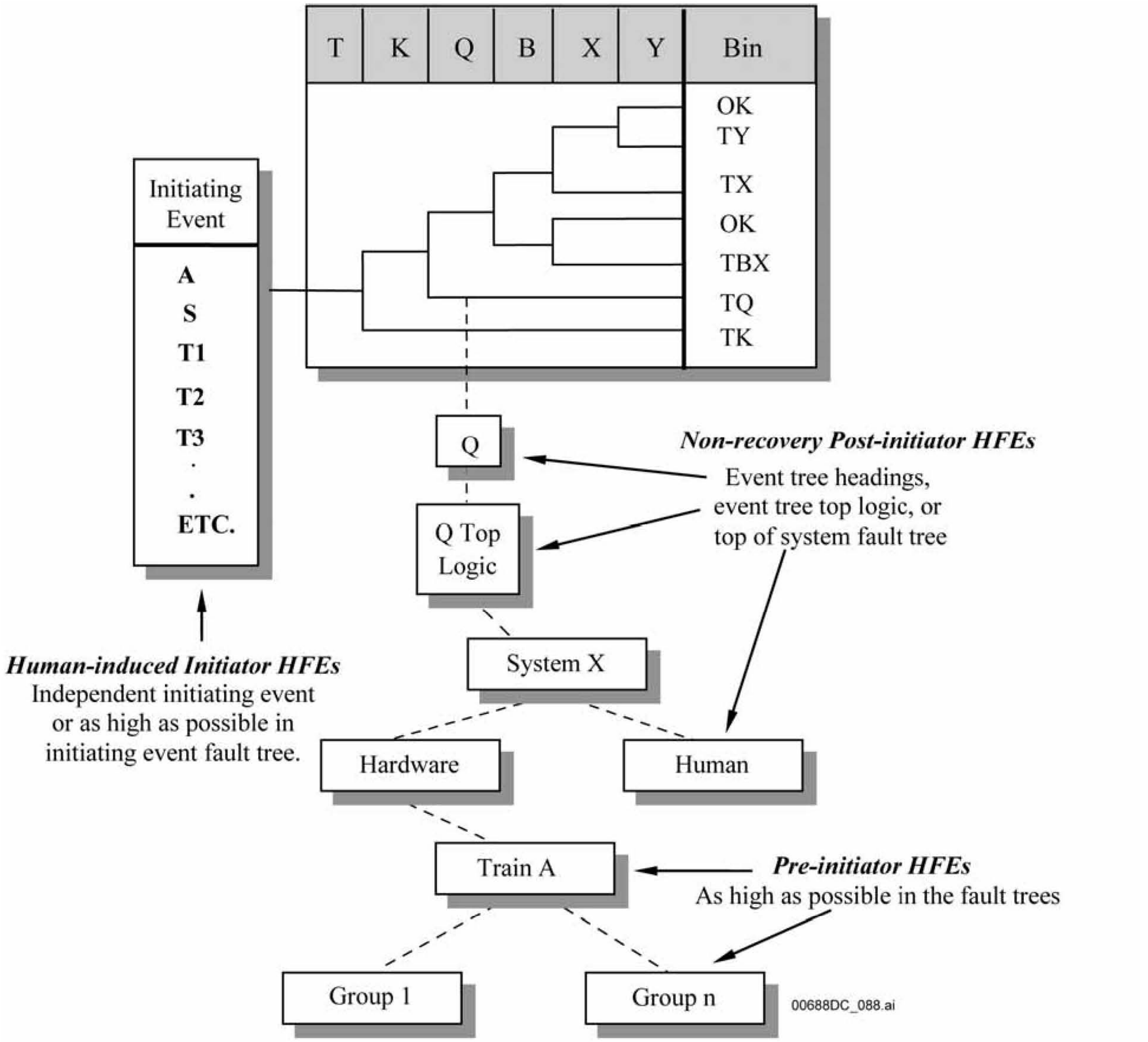
7. The environment inside each YMP facility is not adverse. The levels of illumination and sound and the provisions for physical comfort are optimal. Judgment is required to determine what constitutes optimal environmental conditions. The analyst makes this determination, and documents, as part of the assessment of performance influencing factors, when there is a belief that the action is likely to take place in a suboptimal environment. Regarding outdoor operations onsite, similar judgments must be made regarding optimal weather conditions.

8. While all personnel are trained to procedures, and procedures exist for all work required, the direct presence and use of procedures (including checklists) during operation is generally restricted to actions performed in the control room. Workers performing skill-of-craft operations do not carry written procedures on their person while performing their activities.

These factors are evaluated qualitatively for each situation being analyzed.

4.3.4.2 HRA Methodology

The HRA consists of several steps that follow the intent of ASME RA-S-2002, *Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications* (Ref. 2.2.8) and the process guidance provided in *Technical Basis and Implementation Guidelines for Technique for Human Event Analysis (ATHEANA)* NUREG-1624 (Ref. 2.2.67). The step descriptions are based on the ATHEANA documentation, with some passages taken essentially verbatim and others paraphrased to adapt material that is based on nuclear power plants to the YMP facilities. Additional information is available in the ATHEANA documentation (Ref. 2.2.67). Section 10.3 of NUREG-1624 (Ref. 2.2.67), provides an overview of the method for incorporating HFEs into a PRA. Figure 4.3-9 illustrates this integration method.



NOTE: HFE = human failure event.

Source: Original

Figure 4.3-9. Incorporation of Human Reliability Analysis within the PCSA

Step 1: Define the Scope of the Analysis—The objective of the YMP HRA is to provide a comprehensive qualitative assessment of the HFEs that can contribute to the facility's event sequences resulting in radiological release, criticality, or direct exposure. Any aspects of the work that provide a basis for bounding the analysis are identified in this step. In the case of the YMP, the scope is bounded by the design state of the facilities and equipment.

Step 2: Describe Base Case Scenarios—In this step, the base case scenarios are defined and characterized for the operations being evaluated. In general, there is one base case scenario for each operation included in the model. The base case scenario represents the most realistic description of expected facility, equipment, and operator behavior for the selected operation.

Step 3: Identify and Define HFEs of Concern—Possible HFEs and/or unsafe actions (i.e., actions inappropriately taken or actions not taken when needed) that result in a degraded state are generally identified and defined in this step. After HFEs are identified they must be classified to support subsequent steps in the process. The result of this identification process is a list of HFEs and a description of each HFE scenario, including system and equipment conditions and any resident or triggered human factor concerns (e.g., performance-shaping factors (PSFs)). This combination of conditions and human factor concerns then becomes the error-forcing context (EFC) for a specific HFE. As defined by ATHEANA (Ref. 2.2.67), an EFC is the situation that arises when particular combinations of PSFs and plant conditions create an environment in which unsafe actions are more likely to occur. Additions to and refinements of these initial EFCs are made during the preliminary and detailed analyses. The analyses performed in later steps (e.g., Steps 6 and 7) may identify the need to define additional HFEs or unsafe actions.

Step 4: Perform Preliminary Analysis and Identify HFEs for Detailed Analysis—The preliminary analysis is a type of screening analysis used to identify HFEs of concern. This type of analysis is commonly performed in HRA to conserve resources for those HFEs that are involved in the important event sequences. The preliminary quantification process consists of the following subtasks:

1. Identification of the initial scenario context
2. Identification of the key or driving factors of the scenario context
3. Generalization of the context by matching it with generic, contextually anchored rankings or ratings
4. Discussion and justification of the judgments made in subtask 3
5. Refinement of HFEs, associated contexts, and assigned HEPs
6. Determination of final preliminary HEP for HFE and associated context.

Once preliminary values have been assigned, the model is run, and HFEs are identified for a detailed analysis if (1) the HFE is a risk-driver for a dominant sequence, and (2) using the preliminary values, that sequence is above Category 1 or Category 2 according to the performance objectives in 10 CFR 63.111 (Ref. 2.3.2).

Step 5: Identify Potential Vulnerabilities—This information collection step defines the context for Step 6 in which scenarios that deviate from the base case are identified. In particular, analysts search for potential vulnerabilities in the operators' knowledge and information base for the initiating event or base case scenario(s) under study that might result in the HFEs and/or unsafe actions identified in Step 4. The knowledge and information base is taken in the context of the specific HFE being evaluated. It includes not only the internal state of knowledge of the operator (i.e., what the operator inherently knows), but also the state of the information provided (e.g., available instrumentation, plant equipment status). The HRA analysts rely on experience in other similar operations.

Step 6: Search for HFE Scenarios—In this step, the analyst must identify deviations from the base case scenario that are likely to result in risk-significant unsafe action(s). These deviations are referred to as HFE scenarios. The method for identifying HFE scenarios in the YMP HRA is stated in Step 3. This process continues throughout the event sequence development and quantification. The result is a description of HFE scenarios, including system and equipment conditions, along with any resident or triggered human factor concerns (e.g., PSFs). These combinations of conditions and human factor concerns then become the EFC for a specific HFE.

Step 7: Quantify Probabilities of HFEs—Detailed HRA quantification is performed for those HFEs that appear in dominant cut sets for event sequences that do not comply with 10 CFR 63.111 performance objectives (Ref. 2.3.2) after initial fault tree or event sequence quantification. The goal of the detailed analysis is to determine whether or not the preliminary HFE quantification is too conservative such that event sequences can be brought into compliance by a more realistic HRA. However, the detailed analysis may result in a requirement for additional design features or specification of a procedural control (Step 9) that reduces the likelihood of a given HFE in order to achieve compliance with 10 CRF 63.111 performance objectives (Ref. 2.3.2). The activities of a detailed HRA are as follows:

- Qualitative analysis (e.g., identification of PSFs, definitions of important characteristics of the given unsafe action, assessment of dependencies)
- Selection of a quantification model
- Quantification using the selected model
- Verification that HFE probabilities are appropriately updated in the PCSA.

The four quantification approaches that are in the PCSA, either alone or in combination, follow:

1. CREAM (Ref. 2.2.51)
2. HEART/NARA (Ref. 2.2.85)/(Ref. 2.2.37)
3. THERP (with some modifications) (Ref. 2.2.81)

When an applicable failure mode cannot be reasonably found in one of the above methods, then the following HRA method is used:

4. ATHEANA expert elicitation approach (Ref. 2.2.67).

The selection of a specific quantification method for the failure probability of an unsafe action(s) is based upon the characteristics of the HFE quantified. Appendix E.IV of Attachment E provides a discussion of why these specific methods were selected for quantification, as well as a discussion of why some methods, deemed appropriate for HRA of NPPs, are not suitable for application in the PCSA. It also gives some background about when a given method is applicable based on the focus and characteristics of the method.

Step 8: Incorporate HFEs into PCSA—After HFEs are identified, defined, and quantified, they must be reincorporated into the PCSA. Section 10.3 of NUREG-1624 (Ref. 2.2.67) provides an overview of the state-of-the-art method for performing this step in PRAs. The term reincorporated is used because some HFEs are identified within the fault tree and event tree analysis. All event sequences that contain multiple HFEs are examined for possible dependencies. Figure 4.3-9 shows how the different types of HFEs discussed previously are incorporated into the model based on their temporal phase, which determines where in the model each type of HFE is placed. More detailed discussion of how this is done is provided in Attachment E.

Step 9: Evaluation of HRA/PCSA Results and Iteration with Design—This last step in the HRA is performed after the entire PCSA is quantified. HFEs that ultimately prove to be important to categorization of event sequences are identified. Because the YMP design and operations were still evolving during the course of this analysis, they could be changed in response to this analysis. This iteration is particularly necessary when an event sequence is not in compliance with the performance objectives of 10 CFR 63.111 (Ref. 2.3.2) because the probability of a given HFE dominates the probability of that event sequence. In those cases, a design feature or procedural safety control could be added to reduce the probability or completely eliminate the HFE. An example of such iteration includes the interlocks that ensure that cask lids are securely grappled. The interlocks might have a bypass feature when a yoke is attached to a grapple. An operator might fail to void the bypass when attempting to grapple a heavy load. The design changed such that the bypass would automatically be voided (by an electromechanical interlock) as soon as a yoke is attached to a grapple.

4.3.4.3 Classification of HFEs

HFEs are classified to support the HRA preliminary analysis, selection of HRA quantification methods, and detailed quantification. A combination of four classification schemes is used in the YMP HRA. The first three schemes are familiar standards in HRA. The fourth scheme has its basis in behavioral science and has been used in some second-generation HRA methods. The four classification schemes are as follows:

1. The three temporal phases used in PRA modeling:

   A. Pre-initiator
   B. Human-induced initiator
   C. Post-initiator

2. Error modes:

   A. Errors of omission (EOOs)
   B. Errors of commission (EOCs)

3. Human failure types:

   A. Slips/lapses
   B. Mistakes

4. Informational processing failures:

   A. Monitoring and detection
   B. Situation awareness
   C. Response planning
   D. Response implementation.

These classification schemes are used in concert with each other. They are not mutually exclusive. The first three schemes have been standard PRA practice; additional information on these three schemes can be found in Section E5.1 of Attachment E. The fourth scheme is summarized below.

Assessment of HFEs can be guided by a model of higher-level cognitive activities, such as an information processing model. Several such models have been proposed and used in discussing pilot performance for aviation. The model that is used for the YMP HRA is based on the discussion in Chapter 4 of NUREG-1624 (Ref. 2.2.67) and consists of the following elements:

- Monitoring and detection—Both of these activities are involved with extracting information from the environment. Also, both are influenced by the characteristics of the environment and the person's knowledge and expectations. Monitoring that is driven by the characteristics of the environment is called data-driven monitoring. Monitoring initiated by a person's knowledge or expectations is called knowledge-driven monitoring. Detection can be defined as the onset of realization by operators that an abnormal event is happening.

- Situation awareness—This term is defined as the process by which operators construct an explanation to account for their observations. The result of this process is a mental model, called a situation model that represents the operator's understanding of the present situation and their expectations for future conditions and consequences.

- Response planning—This term is defined as the process by which operators decide on a course of action, given their awareness of a particular situation. Often (but not always) these actions are specified in procedures.

- Response implementation—This term is defined as the activities involved with physically carrying out the actions identified in response planning.

When there are short time frames for response and the possibility of severely challenging operating conditions (e.g., environmental conditions) exists, then failures in all information processing stages must be considered. Also, slips/lapses and mistakes are considered for each information processing stage. Response implementation failures are expected to dominate the pre-initiator failures that are modeled. Post-initiator failures and failures that initiate event sequences can occur for all information processing stages, although detection failures are likely to be important only for events requiring response in very short time frames.

4.3.5 Fire Analysis

Fire event sequence analysis consists of four parts:

1. Development of fire ignition frequencies for each location in the facility or operations area. These are all called fire initiating event frequencies.

2. Development of the fire severity in terms of both temperature and durations. This was discussed in Section 4.3.2.

3. Development of the conditional probability of fire damaging a cask, canister, or waste package target. This was also discussed in Section 4.3.2.

4. Development of and quantification of fire event sequence diagrams and event trees. Development of the ESDs and event trees was discussed in *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34). Quantification of fire event trees is conducted exactly like quantification of any other event tree and is described in Section 4.3, Section 4.3.1, and Section 4.3.7.

This section summarizes the method for the fire initiating event analysis performed as a part of the PCSA. The analysis was performed as part of an integrated analysis of internal fires in the surface and subsurface facilities. The full fire analysis and detail on the methods and data are documented in Attachment F to this volume. The fire analysis is subject to the boundary conditions described in the following section.

4.3.5.1 Boundary Conditions

The general boundary conditions used during the fire analysis are compatible with those described in Section 4.3.10. The principal boundary conditions for the fire analysis are listed below:

- Plant Operational State. Initial state of the facility is normal with each system operating within its limiting condition of operation limits.

- Number of Fire Events to Occur. The facility is analyzed to respond to one fire event at a given time. Additional fire events as a result of independent causes or of re-ignition once a fire is extinguished are bounded by the one fire event.

- Ignition Source Counting. Ignition sources are counted in accordance with applicable counting guidance contained in *Detailed Methodology*. Volume 2 of *EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities* (Ref. 2.2.45).

- Fire Cable and Circuit Failure Analysis. Unlike nuclear power plants, which depend on the continued operation of equipment to prevent fuel damage, the YMP facilities cease operating on loss of power or control. Therefore, fire damage in rooms that do not contain waste cannot result in an increased level of radiological exposure. See Section 6.0 for a more detailed explanation involving treatment of loss of electrical power.

- HVAC Fire Analysis. HVAC is not relied upon to mitigate potential releases associated with fire event sequences in recognition that a large amount of fire generated, non-radiological particulates could render the HVAC filters ineffective.

- No Other Simultaneous Initiating Events. The facility is analyzed to respond to one initiating event at a given time. Additional initiating events as a result of independent causes are bounded by the one initiating event.

- Data Collection Scope. The fire ignition data collection and analysis are performed for locations relevant to waste handling in the facilities.

- Component Failure Modes. The failure mode of a SSC affected by a fire is the most severe with respect to consequences. For example, the failure mode for a canister could be the overpressurization of a reduced strength canister.

- Component Failure Probability. Fires large enough to fail waste containment components will be large enough to fail all active components in the same room. Active components fail in a de-energized state for such fires.

4.3.5.2 Analysis Method

Nuclear power plant fire risk assessment techniques have limited applicability to facilities such as the RF or other facilities in the GROA. The general methodological basis of the PCSA fire analysis is the *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology* (Ref. 2.2.73). Chemical agent disposal facilities are similar to those in the GROA in that these facilities are handling and disposal facilities for highly hazardous materials. This is a "data based" approach in that it utilizes actual historical experience on fire ignition and fire propagation to determine fire initiating event frequencies. That approach has been adapted to utilize data applicable to the YMP waste handling facilities. To the extent applicable to a non-reactor facility, *EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities*. NUREG/CR-6850 Volumes 1 and 2 (Ref. 2.2.45 and Ref. 2.2.46) are also considered in the development of this analysis method. The method complies with the applicable requirements of *Fire PRA Methodology* (Ref. 2.2.5) that are relevant to a non-reactor facility. The steps in the analysis are summarized below and described in detail in Attachment F, Section F4:

A. Identification of initiating events. Current techniques in fire risk assessment for nuclear power plants focus on fire that can damage electrical and control circuits or impact other equipment that can compromise process and safety systems. This type of approach is not generally applicable to YMP because loss of electric power is a safe state except for the need for HVAC after a release of radionuclides. In general, when systems are affected by fire, they cease to function. While at a nuclear power plant this is of concern, as described in Section 6.0 for the YMP waste handling facilities, this means that fuel handling stops and initiating events capable of producing elevated levels of radioactivity are essentially unrealizable. The fire analysis, therefore, focused on the potential for a fire to directly affect the waste containers and cause a breach that would result in a release, rather than analyzing fires that would remove power from fuel handling systems. After a release of radionuclides, the HVAC system, with its HEPA filtration, aids in the abatement radioactivity that is released from buildings. However, the occurrence of fires tends to significantly reduce the effectiveness of HEPA filtration and the fire event sequence analysis, therefore, does not rely on this system. Consideration is given both to fires that start in rooms containing waste and fires that start in other rooms and propagate to where the waste is located. The four steps of this process are as follows:

1. Identify fire-rated barriers and designate fire zones. The facility is broken into fire zones based on the location of fire-rated barriers. The rating of the barriers is not significant to the methodology, so barriers of all ratings are considered. In order for a fire zone to exist, the penetrations, doorways, and ducts must also be limited to the perimeter of the zone. Note that a floor is always considered to be a fire barrier as long as it is solid. Zones are identified by a number, determined by the analyst, and will consist of one or more rooms.

2. Identify the rooms where waste can be present. Each room where waste can be present, even if only for a brief time, is listed. The first set of fire initiating events to be considered in the PCSA is fires that affect each of these rooms, but do not affect other rooms that could contain waste.

3. Define local initiating events. Fire ignition occurrences are identified for each room within a fire zone. The total occurrences of a fire within a room containing a waste form is composed of the occurrences of ignitions in that room plus the occurrences of ignitions in surrounding rooms, within the fire zone, which propagate across room boundaries to the room containing the waste form. The locations of fire initiating events were identified in the MLD (Ref. 2.2.34).

4. Define large fire initiating events. Traditional fire risk studies for nuclear power plants have tended to ignore large fires, arguing that the fire barriers in place will prevent such occurrences. However, actual observed historical data shows that large fires in buildings occur. Large fires are defined for this study as those that spread to encompass the entire building. This is recognized in the latest fire risk guidance from NRC and Electric Power Research Institute (EPRI) (Ref. 2.2.46) and (Ref. 2.2.45, Section 11.5.4) in which potential large fire initiating events are identified. The general approach is as follows:

    a) In the YMP facilities, waste containers, except during the short time they are being lifted by a canister transfer machine (CTM), are on the ground floor. Continuing with the focus on rooms that contain waste forms, large fires may be divided two ways. One is associated with fires that start on the ground floor and spread to the entire building. The other is a fire that starts anywhere else in the building.

    b) As a practical analysis technique, any fire that spreads out of a fire area is considered a large fire.

B. Quantification of fire ignition frequency. The quantification of initiating event frequency involves three steps. First, the overall frequency of fire ignition for the facility is determined, then that frequency is allocated to the individual room in the facility based on the number and types of ignition sources in the rooms. Types of ignition sources are characterized in general terms such as mechanical, electrical, or combustible liquid. Finally, propagation probabilities are applied to determine the overall frequency that a fire reaches the area of the waste. Quantification uses data from the following sources for equipment ignition frequencies and conditional probabilities of propagation:

1. *Utilisation of Statistics to Assess Fire Risks in Buildings* (Ref. 2.2.82)

2. *Summary & Overview. Volume 1 of EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities. EPRI-1011989 and NUREG/CR-6850* (Ref.2.2.46)

3. *Detailed Methodology. Volume 2 of EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities. EPRI TR-1011989 and NUREG/CR-6850* (Ref. 2.2.45)

4. *Fires in or at Industrial Chemical, Hazardous Chemical and Plastic Manufacturing Facilities, 1988-1997 Unallocated Annual Averages and Narratives* (Ref. 2.2.1)

5. *Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction: 1980 – 1998* (Ref. 2.2.2)

6. *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology* (Ref. 2.2.73).

C. Determine initiating event frequency. The definition of each initiating event includes the implicit condition that the fire actually threatens a target that contains radioactive material. Therefore, for each initiating event, the initiating event frequency considers two aspects: the fraction of time there is a waste container in the room, and the probability of a fire propagates to that waste container. The probability of the presence of a target waste form is the fraction of time that the waste form(s) is in the area affected by the fire; (e.g., for a room fire, it is the fraction of time a waste form is in the room). There are two types of propagation that are considered: propagation within a room and propagation between rooms.

1. Fire propagation within rooms. The question is whether the fire, which can ignite wherever there is an ignition source in the room, reaches the area within the room in which the waste is located. Equation 15 obtains:

$$f_{ier\text{-}i} = P_{wri}\,[f_i\,(FR_a + (FR_n \times (P_{pc} + P_{rc})) + (FR_f \times P_{rc}))] \qquad \text{(Eq. 15)}$$

where

$f_{ier\text{-}i}$ = frequency of fire affecting waste form, *i-th* room

P_{wri} = probability that a waste form is in the *i-th* room

f_i = frequency of ignition, *i-th* room

FR_a = fraction of ignition sources at the waste form

FR_n = fraction of ignition sources near the waste form

P_{pc} = conditional probability for fire confined to part of room of origin

FR_f = fraction of ignition sources far from the waste form

P_{rc} = conditional probability for fire confined to room of origin

The values for P_{wri}, P_{pc}, and P_{rc} in the previous equation were developed from the analysis performed by National Fire Protection Association (NFPA) (Ref. 2.2.2). The frequency f_i is the sum of frequencies of ignition of all ignition sources in the room. The fraction of ignition sources at, near, and far from the waste form was developed from equipment layout drawings such as:

a) *Receipt Facility Normal Electrical Room Equipment Layout* (Ref. 2.2.25)

b) *Receipt Facility General Arrangement Ground Floor Plan* (Ref. 2.2.24).

2. Fire propagation to large fire**.** The probability of a large fire (defined for this study as one that propagates beyond the fire area of origin) is developed from Equation 16:

$$f_{ief\text{-}fj\text{-}ri} = f_i \text{ x } P_{fc} \qquad \text{(Eq. 16)}$$

where

$f_{ief\text{-}fj\text{-}ri}$ = frequency of fire in zone j starting in room i

f_i = frequency of ignition, *i-th* room

P_{fc} = conditional probability for fire extending beyond the fire area of origin.

The probability of a fire extending beyond the fire area of origin is found from NFPA (Ref. 2.2.2).

The final initiating event frequency is determined by multiplying the frequency of the fire reaching the waste form (in occurrences per year) times the probability that a waste form is present (fraction of time per waste form) times 50 (years/ operating lifetime during the preclosure period). This yields the initiating event frequency for a fire of a specific severity affecting a waste form, per waste form processed, over the preclosure period. The remainder of the event sequence quantification follows Section 4.3.6.

4.3.6 Event Sequence Quantification

4.3.6.1 Overview of Quantification

Event sequences are represented by event trees and are quantified via the product of the initiating event frequency and the pivotal event probabilities. Event sequences that lead to a successful end state (designated as "OK") are not considered further. The result of quantification of an event sequence is expressed in terms of the number of occurrences over the preclosure period. This number is the product of the following factors:

1. The number of demands (sometimes called trials) or the time exposure interval of the operation or activity that gives rise to the event sequence. For example, this could be the total number of transfers of a cask in a facility preparation area.

2. The frequency of occurrence per demand or per time interval of the initiating event. For example, this could be the frequency of cask drop per transfer by a crane. Initiating event frequencies are developed either using fault trees or by direct application of industry-wide data, as explained in Section 4.3.2. Factors one and two are represented in the initiator event trees.

3. The conditional probability of each of the pivotal events of the event sequence, which appear in the associated system-response event tree. These probabilities are the results of a passive equipment failure analyses, fault tree analyses (e.g., HVAC), and direct probability input (e.g., moderator introduced), or judgment. For example, the conditional probability of cask failure given a drop from 12 ft or less is less than 1E-05.

SAPHIRE Version 7.26 (Ref. 2.2.40) (Section 4.2) is used as the integrating software for the Boolean reduction and quantification of event sequences. All fault trees and event trees are entered into or produced directly in SAPHIRE. All reliability information relevant to quantification is input into SAPHIRE. Following analyst input instructions or rules, SAPHIRE performs the following functions for this analysis:

- Following analyst instructions, links the initiator event tree with the appropriate system response event tree.

- Following analyst instructions, called rules, links the fault trees and direct pivotal event input probabilities that are involved in an event sequence.

- Performs the Boolean manipulations to obtain minimal cut sets.

- Combines the minimal cut sets of each event sequence and each end state.

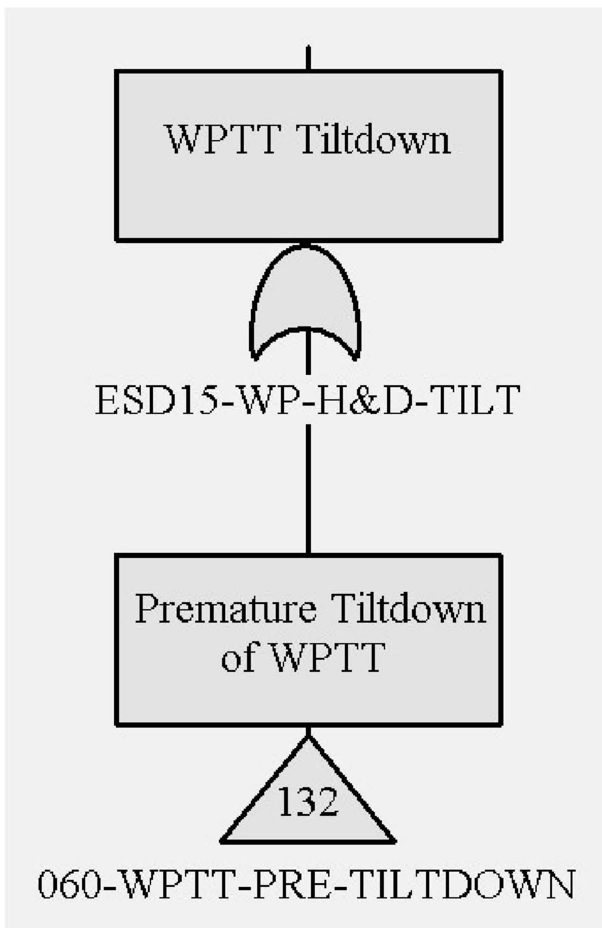
- Combines the minimal cut sets of each end state of all little bubbles to obtain the set of minimal cut sets of an end state for a big bubble initiating event.

- Obtains a point estimate number of occurrences of the minimal cut sets using the entered reliability information.

- Obtains the probability distributions of the minimal cut sets using the entered uncertainty information.

- Provides reports, as specified by the analyst, for each end state of each big bubble.

Development of analyst instructions, or rules, is facilitated by the following naming convention. The names identified in the initiating event fault trees are defined to be unique to the event tree. Fault trees are linked by development of a linking fault tree to transfer the appropriate fault tree to the event tree pivotal event or initiating event. Figure 4.3-10 shows an example of this. ESD15-WP-H&D-TILT is the unique identifier that is assigned to the initiating event tree to represent the initiating event for a premature WPTT tiltdown. The benefit to using this method is that many smaller, specific fault trees can be linked together into a single initiating or pivotal event, thereby reducing the work associated with development of event sequence specific fault trees.



NOTE: WPTT = waste package transfer trolley.

Source: Original

Figure 4.3-10. Transfer from Event Tree to Fault Tree

The frequency of each minimal cut set is the product of the frequency and conditional probabilities of the events that compose it. The frequency of each event sequence is a probabilistic sum of the frequencies of each minimal cut set.

SAPHIRE, developed by Idaho National Laboratory, stands for "Systems Analysis Programs for Hands-on Integrated Reliability Evaluations." It is 32-bit software that runs under Microsoft Windows. Features of SAPHIRE that help an analyst build and quantify a set of event trees and fault trees are as follows:

- A listing of where a basic event appears, including within cut sets. Conversely, the basic events that are not used are known and can be easily removed when it comes time to "clean" the database.

- Context-driven menu system that performs actions (report cut sets, view importance measures, display graphics, etc.) on objects such as fault trees, event trees, and event sequences.

Fault trees can be constructed and analyzed to obtain different measures of system unreliability. These system measures are:

- Overall initiating or pivotal event failure frequency
- Minimal cut sets size, number, and frequency
- Built in features include:

  - Generation, display, and storage of cut sets
  - Graphical editors (fault tree and event tree)
  - Database editors
  - Uncertainty analysis
  - Data Input/Output via ASCII text files (MAR-D)
  - Special seismic analysis capability.

SAPHIRE is equipped with two uncertainty propagation techniques: Monte Carlo and Latin Hypercube sampling. To take advantage of these sampling techniques, twelve uncertainty distributions are built such that the appropriate distribution may be selected. SAPHIRE contains a cross-referencing tool, which provides an overview of every place a basic event, gate, initiating, or pivotal event is used in the model.

4.3.6.2 Propagation of Uncertainties and Event Sequence Categorization with Uncertainties

The fundamental viewpoint of the PCSA is probabilistic in order to develop information suitable for the risk informed nature of 10 CFR Part 63 (Ref. 2.3.2). Any particular event sequence may or may not occur during any operating time interval, and the quantities of the parameters of the models may not be precisely known. Characterizing uncertainties and propagating these uncertainties through the event tree/fault tree model is an essential element of the PCSA. The PCSA includes both aleatory and epistemic uncertainties. Aleatory uncertainty refers to the inherent variation of a physical process over many similar trials or occurrences. For example, development of a fragility curve to obtain the probability of canister breach after a drop would involve investigating the natural variability of tensile strength of stainless steel. Epistemic uncertainty refers to our state of knowledge about an input parameter or model. Epistemic uncertainty is sometimes called reducible uncertainty because gathering more information can reduce the uncertainty. For example, the calculated uncertainty of a SSC failure rate developed from industry-wide data will be reduced when sufficient GROA specific operational information is included in a Bayesian analysis of the SSC failure rate.

Uncertainty in the value of any input parameter and the event sequence frequency is expressed by a probability distribution. Probability distribution is propagated through models using SAPHIRE. As described in Section 4.3.1, categorization is performed using the mean value of event sequences emanating from the big bubble in Figure 4.3-4. By the definition of the term, mean values are derived solely from probability distributions.

Using the screening criteria set out in 10 CFR 63.2 (Ref. 2.3.2), the categorization of an event sequence that is expected to occur m times over the preclosure period (where m is the mean or expected number of occurrences) is carried out as follows:

- A value of m greater than or equal to one places the corresponding event sequence into Category 1.

- A value of m less than one indicates that the corresponding event sequence is not expected to occur before permanent closure. To determine whether the event sequence is Category 2, its probability of occurrence over the preclosure period needs to be compared to 10^{-4}. A measure of the probability of occurrence of the event sequence over the preclosure period is given by a Poisson distribution that has a parameter taken equal to m. The probability, P, that the event sequence occurs at least one time before permanent closure is the complement to one that the event sequence occurs exactly zero times during the preclosure period. Using the Poisson distribution, $P = 1 - \exp(-m)$, a value of P greater than or equal to 10^{-4} implies that the value of m is greater than or equal to $-\ln(1 - P) = m$, which is numerically equal to 10^{-4}. Thus, a value of m greater than or equal to 10^{-4}, but less than one, implies the corresponding event sequence is a Category 2 event sequence.

- Event sequences that have a value of m less than 10^{-4} are designated as beyond Category 2.

Using either Monte Carlo or Latin Hypercube methods allows probability distributions to be arithmetically treated to obtain the probability distributions of minimal cut sets and the probability distributions of event sequences. The PCSA used Monte Carlo simulation with 10,000 trials and a standard seed so the results could be reproduced. The number of trials for final results was arrived at by increasing the number of trials until the median, mean, and 95th percentile were stable within the standard Monte Carlo error.

The adequacy of categorization of an event sequence is further investigated if its expected number of occurrences m over the preclosure period is close to a category threshold.

If m is greater than 0.2, but less than one, the event sequence, which a priori is Category 2, is reevaluated differently to determine if it should be recategorized as Category 1. Similarly, if m is greater than 2×10^{-5}, but less than 10^{-4}, the event sequence, which a priori is beyond Category 2, is reevaluated to determine if it should be recategorized as Category 2.

The reevaluation begins with calculating an alternative value of m, designated by m_a, based on an adjusted probability distribution for the number of occurrences of the event sequence under consideration. The possible distributions that are acceptable for such a purpose would essentially have the same central tendency, embodied in the median (i.e., the 50th percentile), but relatively more disparate tails, which are more sensitive to the shape of the individual distributions of the basic events that participate in the event sequence. Accordingly, the adjusted distribution is selected as a lognormal that has the same median M as that predicted by the Monte Carlo sampling. Also, to provide for a reasonable variability in the distribution, an error factor $EF = 10$ is used, which means that the 5th and 95th percentiles of the distribution are respectively lesser or greater than the median by a factor of 10.

If the calculated value of m_a is less than one, the alternative distribution confirms that the event sequence category is the same as that predicted by the original determination, i.e., Category 2. Similarly, if the calculated value of m_a is less than 10^{-4}, the alternative distribution confirms that the event sequence category is the same as that predicted by the original determination, i.e., beyond Category 2.

In contrast, if the calculated value of m_a is greater than one, the alternative distribution indicates that the event sequence is Category 1, instead of Category 2 found in the original determination. In such a case, the conflicting indications are resolved by conservatively assigning the event sequence to Category 1.

Similarly, if the calculated value of m_a is greater than 10^{-4}, the alternative distribution indicates that the event sequence is Category 2, instead of beyond Category 2 found in the original determination. In such a case, the conflicting indications are resolved by conservatively assigning the event sequence to Category 2.

The calculations carried out to quantify an event sequence are performed using the full precision of the individual probability estimates that are used in the event sequence. However, the categorization of the event sequence is based upon an expected number of occurrences over the preclosure period given with one significant digit.

4.3.7 Identification of ITS SSCs, Development of Nuclear Safety Design Bases, and Development of Procedural Safety Controls

4.3.7.1 Identification of ITS SSCs

ITS SSCs are subject to nuclear safety design bases that are established to ensure that safety functions and reliability factors applied in the event sequence analyses are explicitly defined in a manner that assures proper categorization of event sequences.

ITS is defined in 10 CFR 63.2 (Ref. 2.3.2) as:

> "*Important to safety*, with reference to structures, systems, and components, means those engineered features of the geologic repository operations area whose function is:
>
> (1) To provide reasonable assurance that high-level radioactive waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the requirements of § 63.111(b)(1) for Category 1 event sequences; or
>
> (2) To prevent or mitigate Category 2 event sequences that could result in radiological exposures exceeding the values specified at § 63.111(b)(2) to any individual located on or beyond any point on the boundary of the site."

Structures are defined as elements that provide support or enclosure such as buildings, free standing tanks, basins, dikes, and stacks. Systems are collections of components assembled to perform a function, such as HVAC, cranes, trolleys, and transporters. Components are items of equipment that taken in groups become systems such as pumps, valves, relays, piping, or elements of a larger array, such as digital controllers.

Implementation of the regulatory definition of ITS has produced the following specific criteria in the PCSA to classify SSCs:

> A SSC is classified as ITS if it appears in an event sequence and at least one of the following criteria apply:
>
> - The SSC is relied upon to reduce the frequency of an event sequence from Category 1 to Category 2.
> - The SSC is relied upon to reduce the frequency of an event sequence from Category 2 to beyond Category 2.
> - The SSC is relied upon to reduce the aggregated dose of Category 1 event sequences by reducing the event sequence mean frequency.
> - The SSC is relied upon to perform a dose mitigation or criticality control function.

A SSC is classified as ITS in order to assure safety function availability over the operating lifetime of the repository. The classification process involves the selection of the SSCs in the identified event sequences (including event sequences that involve nuclear criticality) that are relied upon to perform the identified safety functions such that the preclosure performance objectives of 10 CFR Part 63 (Ref. 2.3.2) are not exceeded. The ITS classification extends only to the attributes of the SSCs involved in providing the ITS function. If one or more components of a system are determined to be ITS, the system is identified as ITS, even though only a portion of the system may actually be relied upon to perform a nuclear safety function. However, the specific safety functions that cause the ITS classification are delineated.

Perturbations from normal operations, human errors in operations, human errors during maintenance (preventive or corrective), and equipment malfunctions may initiate Category 1 or Category 2 event sequences. The SSCs supporting normal operations (and not relied upon as described previously for event sequences) are identified as non-ITS. In addition, if an SSC (such as permanent shielding) is used solely to reduce normal operating radiation exposure, it is classified as non-ITS.

4.3.7.2 Development of Nuclear Safety Design Bases

Design bases are established for the ITS SSCs as described in 10 CFR 63.2 (Ref. 2.3.2):

> "Design bases means that information that identifies the specific functions to be performed by a structure, system, or component of a facility and the specific values or ranges of values chosen for controlling parameters as reference bounds for design. These values may be constraints derived from generally accepted "state-of-the-art" practices for achieving functional goals or requirements derived from analysis (based on calculation or experiments) of the effects of a postulated event under which a structure, system, or component must meet its functional goals..."

The safety functions for this analysis were developed from the applicable Category 1 and Category 2 event sequences for the SSCs that were classified as ITS. In general, the controlling parameters and values were grouped in, but were not limited to, the following five categories:

1. Mean frequency of SSC failure. It shall be demonstrated by analysis that the ITS SSC will have a mean frequency of failure (e.g., failure to operate, failure to breach), with consideration of uncertainties, less than or equal to the stated criterion value.

2. Mean frequency of seismic event-induced failure. It shall be demonstrated by analysis that the ITS SSC will have a mean frequency of a seismic event-induced failure (e.g., tipover, breach) of less than 1E-04 over the preclosure period, considering the full spectrum of seismic events less severe than that associated with a frequency of 1E-07/yr.

3. High confidence of low mean frequency of failure. It shall be demonstrated by analysis that the ITS SSC will have a high confidence of low mean frequency of failure associated with seismic events of less than or equal to the criterion value. The high confidence of low mean frequency of failure value is a function of uncertainty, expressed as β_c, which is the lognormal standard deviation of the SSC seismic fragility.

4. Preventive maintenance and/or inspection interval. The ITS SSCs shall be maintained or inspected to assure availability, at intervals not to exceed the criterion value.

5. Mean unavailability over time period. It shall be demonstrated by analysis that the ITS SSCs (e.g., HVAC and emergency electrical power) will have a mean unavailability over a period of a specified number of days, with consideration of uncertainties, of less than the criterion value.

These controlling parameters and values ensure that the ITS SSCs perform their identified safety functions such that 10 CFR Part 63 (Ref. 2.3.2) performance objectives are met. The controlling parameters and values include frequencies or probabilities in order to provide a direct link from the design requirements for categorization of event sequences. The PCSA will demonstrate that these controlling parameters and values are met by design of the respective ITS SSCs.

Table 6.9-1 in Section 6.9 presents a list of ITS SSCs, the nuclear safety design bases of the ITS SSCs, the actual value of the controlling parameter developed in this analysis, and a reference to that portion of the analysis (e.g., fault tree analysis), which demonstrates that the criterion is met.

4.3.7.3 Identification of Procedural Safety Controls

10 CFR 63.112(e) (Ref. 2.3.2) requires that the PCSA include an analysis that "identifies and describes the controls that are relied upon to limit or prevent potential event sequences or mitigate their consequences" and "identifies measures taken to ensure the availability of safety systems." This section describes the approach for specifying and analyzing the subset of procedural safety controls (PSCs) that are required to support the event sequence analysis and categorization.

The occurrence of an initiating or pivotal event is usually a combination of human errors and equipment malfunctions. A human reliability analysis is performed for the human errors. Those human actions that are relied upon to reduce the frequency of or mitigate the consequence of an event sequence are subject to procedural safety controls.

The approach for deriving PSCs from the event sequence analysis is outlined in the following:

1. Use event tree and supporting fault tree models for initiating events and pivotal events to identify HFEs.
2. Identify the types of PSCs necessary to support the HRA analysis for each of the HFEs. For example, provide clarifications about what is to be accomplished, time constraints, use of instrumentation, interlock and permissives that may back-up the human action.
3. Perform an event sequence analysis using screening HRA values. Identify the PSCs that appear to be needed to reduce the probability of or mitigate the severity of event sequences. The same criteria are used to identify ITS SSCs.
4. Work with the design and engineering organizations to add equipment features that will either eliminate the HFE or support crew and operators in the performance of the action. In effect, this entails development of design features that appear instead of a human action or under an AND gate with a human action.
5. Quantify event sequences again, identifying HFEs for which detailed HRA must be performed. The detailed HRA would lead to specific PSCs that are needed to reduce the frequency of event sequences or mitigate their consequences.

4.3.8 Event Sequence to Dose Relationship

Outputs of the event sequence analysis and categorization process include tabulations of event sequences by expected number of occurrences, end state, and waste form. The event sequences are sorted by Category 1, Category 2 and beyond Category 2. Summaries of the results are tabulated in Section 6.8 and Attachment G with the following information:

1. Event sequence designator. A unique designator is provided for each event sequence to permit cross-references between event sequence categorization and consequence and criticality analysis.

2. End state. One of the following is provided for each event sequence:

   A. DE-SHIELD-DEGRADE or DE-SHIELD-LOSS (Direct Exposure). Condition leading to potential exposure due to degradation of shielding provided by the cask or the aging overpack.

   B. RR-FILTERED (Radionuclide Release, Filtered). Condition leading to a potential release of radionuclide due to loss of waste form primary containment (e.g., cask with uncanistered commercial SNF or canister). However, the availability of the secondary confinement (structural and HVAC with HEPA filtration) provides mitigation of the consequences.

   C. RR-UNFILTERED (Radionuclide Release, Unfiltered). Condition leading to a potential release of radionuclide due to loss of waste form primary containment (e.g., cask with uncanistered commercial SNF or canister), and a breach in the secondary confinement boundary (e.g., no HEPA filtration to provide mitigation of the consequences or breach of the structural confinement).

   D. RR-FILTERED-ITC and RR-UNFILTERED-ITC (Radionuclide Release, Important to Criticality, Filtered or Unfiltered). Condition leading to a potential release of radionuclide due to loss of waste form primary containment (e.g., cask with uncanistered commercial SNF or canister) with or without HEPA filtration. In addition, the potential of exposing the unconfined waste form to moderator could result in conditions important to criticality. This characteristic of the end state is used by both the dose consequence analysts and the criticality analysts.

   E. ITC (Important to Criticality). This end state is not used for the RF because all potential criticality initiators are associated with a radiological release (i.e., end state RR-UNFILTERED-ITC).

3. General description of the event sequence. This is a high level description that will be explained by the other conditions described above. For example, "Filtered radionuclide release resulting from a drop from a crane that causes a breach of both sealed transportation cask and sealed TAD canister."

4. Material-at-risk. Identify and define the number of each waste form that contributes to the radioactivity or criticality hazard of the end state (e.g., number of TAD canisters, DPCs, uncanistered commercial SNF assemblies, etc., involved in the event sequence).

5. Expected number of occurrences. Provide the expected mean number of occurrences of the designated event sequences over the preclosure period and associated median and standard deviation.

6. The event sequence categorization. Provide the categorization of the designated event sequence and the basis for the categorization.

7. The bounding consequences. Provide the bounding consequence analysis cross-reference, as applicable, for each Category 1 or 2 event sequence to the bounding event number from the preclosure consequence analysis.

10 CFR 63.111 (Ref. 2.3.2) requires that the doses associated with Category 1 and Category 2 event sequences meet specific performance objectives. There are no performance objectives for beyond Category 2 event sequences. Dose consequences associated with each Category 1 and Category 2 event sequence are evaluated in preclosure consequence analyses, by comparison, to pre-analyzed release conditions (or dose categories) that are intended to characterize or bound the actual event sequences (Ref. 2.2.31). As such, the results of the event sequence analysis and categorization serve as inputs to the consequence analysis for assignment to dose categories.

4.3.9 Event Sequence to Criticality Relationship

The requirements for compliance with preclosure safety regulations are defined in 10 CFR 63.112 (Ref. 2.3.2). Particularly germane to criticality considerations, is the requirement in 10 CFR 63.112, Paragraph (e) and Subparagraph (e)(6). Paragraph (e) requires an analysis to identify the controls that are relied upon to limit or prevent potential event sequences or mitigate their consequences. This is a general requirement imposed on all event sequence analyses. Subparagraph (e)(6) specifically notes that the analyses should include consideration of "means to prevent and control criticality." The PCSA criticality analyses (Ref. 2.2.33) employ specialized methods that are beyond the scope of the present calculation. However, the event sequence development analyses inform the PCSA criticality analyses by identifying the event sequences and end states that may have a potential for criticality. As noted in Section 4.3, previously, some event sequence end states include the phrase "important to criticality." This indicates that the end state implies the potential for criticality and that a criticality investigation is indicated.

To determine the criticality potential for each waste form and associated facility and handling operations, criticality sensitivity calculations are performed. These calculations evaluate the impact on system reactivity of variations in each of the parameters important to criticality during the preclosure period, that is, waste form characteristics, reflection, interaction, neutron absorbers (fixed and soluble), geometry, and moderation. The criticality sensitivity calculations determine the sensitivity of the effective neutron multiplication factor (k_{eff}) to variations in any of these parameters as a function of the other parameters. These criticality calculations demonstrate that one of the following is true for each parameter:

- It is bounding (i.e., its analyzed value is greater than or equal to the design limit) or its effect on k_{eff} is bounded and does not need to be controlled. This is designated as a No in Table 4.3-1.

- It needs to be controlled if another parameter is not controlled (conditional control). This is designated as a Conditional in Table 4.3-1.

- It needs to be controlled because it is the primary criticality control parameter. This is designated as a Yes in Table 4.3-1.

The criticality control parameters analysis, which comprises the background calculations that led to Table 4.3-1, is presented in detail in the *Preclosure Criticality Safety Analysis* (Ref. 2.2.33). Event sequences that impact the criticality control parameters that have been established as needing to be controlled are identified, developed, quantified, and categorized. These event sequences are referred to as event sequences ITC. The following matrix elements, indicating the need for control, are treated in the current event sequence analysis:

- Conditional: needs to be controlled if moderator is present

- Conditional: needs to be controlled during a boron dilution accident

- Yes: moderation is the primary criticality control

- Yes: interaction for DOE standardized SNF canisters needs to be controlled.

Table 4.3-1. Criticality Control Parameter Summary

Operation / Parameter	Commercial SNF (Dry Operations)	Commercial SNF (WHF Pool and Fill Operations)	DOE SNF	HLW
Waste Form Characteristics	No[a]	No[a]	No[b]	No[c]
Moderation	Yes[d]	N/A	Yes[d]	No
Interaction	No	Conditional[g]	Yes[e]	No
Geometry	Conditional[f]	Conditional[g]	Conditional[f]	No
Fixed Neutron Absorbers	Conditional[f]	Conditional[g]	Conditional[f]	No
Soluble Neutron Absorber	N/A	Yes[h]	N/A	N/A
Reflection	No	No	No	No

NOTE: [a] The *Preclosure Criticality Safety Analysis* (Ref. 2.2.33) considers bounding waste form characteristics. Therefore, there is no potential for a waste form misload.
[b] The *Preclosure Criticality Safety Analysis* (Ref. 2.2.33) considers nine representative DOE SNF types. Because the analysis is for representative types and loading procedures for DOE standardized SNF canisters have not been established yet, consideration of waste form misloads is not appropriate.
[c] Criticality safety design control features are not necessary for HLW canisters because the concentration of fissile isotopes in an HLW canister is too low to have criticality potential.
[d] Moderation is the primary criticality control parameter
[e] Placing more than four DOE standardized SNF canisters outside the staging racks or a codisposal waste package needs to be controlled.
[f] Needs to be controlled only if moderator is present.
[g] Needs to be controlled only if the soluble boron concentration in the pool and transportation cask/DPC fill water is less than the minimum required concentration.
[h] Minimum required soluble boron concentration in the pool is 2500 mg/L boron enriched to 90 atom % ^{10}B.

DOE = U.S. Department of Energy; HLW = high-level radioactive waste; SNF = spent nuclear fuel; WHF = Wet Handling Facility.

Source: *Preclosure Criticality Safety Analysis* (Ref. 2.2.33, Table 6)

4.3.10 Boundary Conditions and Use of Engineering Judgment Within a Risk Informed Framework

4.3.10.1 Boundary Conditions

The initiating events considered in the PCSA define what could occur within the site GROA and are limited to those events that constitute a hazard to a waste form while it is present in the GROA. Initiating events include internal events occurring during waste handling operations conducted within the GROA and external events (e.g., seismic, wind energy, or flood water events) that impose a potential hazard to a waste form, waste handling systems, or personnel within the GROA. Such initiating events are included when developing event sequences for the PCSA. However, initiating events that are associated with conditions introduced in SSCs before they reach the site are not within the scope of the PCSA. The excluded from consideration offsite conditions include drops of casks, canisters, or fuel assemblies during loading at a reactor site; improper drying, closing, or inerting at the reactor site; rail or road accidents during transport; tornado or missile strikes on a transportation cask; or nonconformances introduced during cask or canister manufacture that result in a reduction of containment strength. Such potential precursors are subject to deterministic regulations (e.g., 10 CFR Part 50 (Ref. 2.3.1), 10 CFR Part 71 (Ref. 2.3.3), and 10 CFR Part 72 (Ref. 2.3.4)) and associated quality assurance programs. As a result of compliance to such regulations, the SSCs are deemed to pose no undue risk to health and safety. Although the analyses do not address quantitative probabilities to the aforementioned excluded precursors, it is clear that conservative design criteria and QA controls result in unlikely exposures to radiation.

Other boundary conditions used in the PCSA include:

- Plant operational state. Initial state of the facility is normal with each system operating within its vendor prescribed operating conditions.

- No other simultaneous initiating events. It is standard practice to not consider the occurrence of other initiating events (human-induced and naturally occurring) during the time span of an event sequence because (a) the probability of two simultaneous initiating events within the time window is small and, (b) each initiating event will cause operations in the waste handling facility to be terminated which further reduces the conditional probability of the occurrence of a second initiating event, given the first has occurred.

- Component failure modes. The failure mode of a SSC corresponds to that required to make the initiating or pivotal event occur.

- Fundamental to the basis for the use of industry-wide reliability parameters within the PCSA, such as failure rates, is the use of SSCs within the GROA that conform to NRC accepted consensus codes and standards, and other regulatory guidance.

4.3.10.2 Use of Engineering Judgment

10 CFR Part 63 (Ref. 2.3.2) is a risk-informed regulation rather than a risk-based regulation. The term risk-informed was defined by the NRC to recognize that a risk assessment can not always be performed using only quantitative modeling. Probabilistic analyses may be supplemented with expert judgment and opinion, based on engineering knowledge. Such practice is fundamental to the risk assessment technology used for the PCSA.

10 CFR Part 63 (Ref. 2.3.2) does not specify analytical methods for demonstrating performance, estimating the reliability of ITS SSCs (whether active or passive), or calculating uncertainty. Instead, the risk-informed and performance-based preclosure performance objectives in 10 CFR Part 63 (Ref. 2.3.2) provide the flexibility to develop a design, and demonstrate that it meets performance objectives for preclosure operations including the use of well established (discipline-specific) methodologies. As exemplified in the suite of risk-informed regulatory guides developed for 10 CFR Part 50 (Ref. 2.3.1) facilities (e.g., Regulatory Guide 1.174 (Ref. 2.2.72) and NUREG-0800 (Ref. 2.2.64, Section 19)), such methodologies use deterministic and probabilistic inputs and analysis insights. The range of well established techniques in the area of PRA, which is used in the PCSA, often relies on the use of engineering judgment and expert opinion (e.g., in development of seismic fragilities, human error probabilities, and the estimation of uncertainties).

As described in Section 4.3.3, for example, active SSC reliability parameters will be developed using a Bayesian approach; and the use of judgment in expressing prior state-of-knowledge is a well-recognized and accepted practice (Ref. 2.2.55, Ref. 2.2.6, Ref. 2.2.11, and Ref. 2.2.63).

The NRC issued HLWRS-ISG-02 (Ref. 2.2.69) to provide guidance for compliance to 10 CFR 63.111 and 112 (Ref. 2.3.2). This document states that "treatment of uncertainty in reliability estimates may depend on the risk-significance (or reliance) of a canister system in preventing or reducing the likelihood of event sequences." Furthermore, HLWRS-ISG-02 (Ref. 2.2.69) indicates that reliability estimates for high reliability SSCs may include the use of engineering judgment supported by sufficient technical basis; and empirical reliability analyses of a SSC could include values based on industry experience and judgment (Ref. 2.2.69).

In a risk-informed PCSA, therefore, the depth, rigor of quantitative analysis, and the use of judgment depends on the risk-significance of the event sequence. As such, decisions on the level of effort applied to various parts of the PCSA are made based on the contribution to the frequency of end states and the severity of such end states. An exhaustive analysis need not be performed to make this resource allocation. Accordingly, the PCSA analyst has flexibility in determining and estimating the reliability required for each SSC, at the system or component level, and in selecting approaches in estimating the reliability. The quantified reliability estimates used to reasonably screen out initiating events, support categorization, or screening of event sequences must be based on defensible and traceable technical analyses. The following summarizes the approaches where judgment is applied to varying degrees.

All facility safety analyses, whether or not risk-informed, take into account the physical conditions, dimensions, materials, human-machine interface, or other attributes such as operating conditions and environments to assess potential failure modes and event sequences. Such factors guide the assessment of what can happen, the likelihood, and the potential consequences. In many situations, it could be considered obvious that the probability of a particular exposure scenario is very small. In many cases, it is impractical or unnecessary to actually quantify the probability when a non-probabilistic engineering analysis provides sufficient assurance and insights that permit the event sequence to be either screened out, or demonstrated to be bounded by another event sequence. Examples of such are provided in Section 6.0.

When Empirical Information is not Available

There is generally no or very little empirical information for the failure of passive SSCs such as transportation casks and spent fuel storage canisters. Such failures are postulated in predictive safety and risk analyses and then the SSCs are designed to withstand the postulated drops, missile impacts, seismic shaking, abnormal temperatures and pressures, etc. While in service, few if any SSCs have been subjected to abnormal conditions that approach the postulated abnormal scenarios so there is virtually no historical data to call on.

Therefore, structural reliability analyses are used in the PCSA to develop analysis-based failure probabilities for the specific event sequences identified within the GROA. Uncertainties in the calculated stresses/strains and the capacity of the SSCs to withstand those demands include the use of judgment, based on standard nuclear industry practices for design, manufacturing, etc., under the deterministic NRC regulatory requirements of 10 CFR Part 50 (Ref. 2.3.1), 10 CFR

Part 71 (Ref. 2.3.3), or 10 CFR Part 72 (Ref. 2.3.4). It is standard practice to use the information basis associated with the consensus standard and regulatory requirement information as initial conditions of a risk-informed analysis. This approach is acceptable for the PCSA subject to the following:

1. The conditions associated with the consensus codes and standards and regulatory requirements are conservatively applicable to the GROA.

2. Equivalent quality assurance standards are applied at the GROA.

3. Operating processes are no more severe than those licensed under the aforementioned deterministic regulations.

Use of Empirical Reliability Information

In those cases where applicable, quantitative historical component reliability information is available, the PCSA followed Sections 4.3 including the application of judgment that is associated with Bayesian analysis. Similarly, as described in Sections 4.3.5, 4.3.6, and 4.3.7, historical data is applied in human reliability, fire, and flooding analyses with judgment-based adjustments as appropriate for the RF and GROA operating conditions.

Use of Qualitative Information When Reliability Information is not Available

In those cases where historical records of failures to support the PCSA are not available, qualitative information may be used to assign numerical failure probabilities and uncertainty. This approach is consistent with the Bayesian framework used in the PCSA, consistent with HLWRS-ISG-02 (Ref. 2.2.69), and involves the use of judgment in the estimation of reliability or failure probability values and their associated uncertainties. In these cases, the PCSA analyst may use judgment to determine probability and reliability values for components.

The following guidelines are used in the PCSA when it is necessary to use judgment to assess the probability of an event. The analyst will select a median at the point believed to be just as likely that the “true” value will lie above as below. Then, the highest probability value believed possible is conservatively assigned as a 95th percentile or error factor (i.e., the ratio of the 95th percentile to median), rather than a 99th or higher percentile, with a justification for the assignments. A lognormal distribution is used because it is appropriate for situations in which the result is a product of multiple uncertain factors or variables. This is consistent with the “A Central Limit Theorem for Latin Hypercube Sampling” (Ref. 2.2.71). The lower bound, as represented by the 5th percentile, is checked to ensure that the distribution developed using the median and 95th percentile does not cause the lower bound to generate values for the variable that are unrealistic compared to the knowledge held by the analyst.

In some cases, an upper and lower bound is defensible, but no information about a central tendency is available. A uniform distribution between the upper and lower bound is used in such cases.

Another way in which risk-informed judgment is applied to obtain an appropriate level of effort in the PCSA, involves a comparison of event sequences. For example, engineering judgment readily indicates that a 23-ft drop of a canister onto an unyielding surface would do more damage to the confinement boundary, than a collision of a canister with a wall at maximum crane speed (e.g., 40 ft per minute). A rigorous probabilistic structural analysis of the 23-ft drop is performed and these results may be conservatively applied to the relatively benign slow speed collision.

5. LIST OF ATTACHMENTS

		Number of Pages
Attachment A	Event Trees	82
Attachment B	System/Pivotal Event Analysis – Fault trees	360
Attachment C	Active Component Reliability Data Analysis	51
Attachment D	Passive Equipment Failure Analysis	92
Attachment E	Human Reliability Analysis	194
Attachment F	Fire Analysis	124
Attachment G	Event Sequence Quantification Summary Tables	2
Attachment H	SAPHIRE Model and Supporting Files	2 + CD

6. BODY OF ANALYSIS

The *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34), which describes the RF, its equipment, and its operations (Ref. 2.2.34, Section 6.1.2, Attachments A, and B), should be consulted in conjunction with the present analysis.

6.0 INITIATING EVENT SCREENING

The NRC's interim staff guidance for its evaluation of the level of information and reliability estimation related to the Yucca Mountain repository, *Interim Staff Guidance HLWRS-ISG-02, Preclosure Safety Analysis - Level of Information and Reliability Estimation* (Ref. 2.2.69, p. 3), states that there are multiple approaches that DOE could use to estimate the reliability of SSCs that contribute to initiating events or event sequence propagation (i.e., pivotal events), including the use of judgment. 10 CFR 63.102(f) (Ref. 2.3.2) provides that initiating events are to be considered for inclusion in the PCSA for determining event sequences only if they are reasonably based on the characteristics of the geologic setting and the human environment, and are consistent with the precedents adopted for nuclear facilities with comparable or higher risks to workers and the public.

This section provides screening arguments that eliminate extremely unlikely initiating events from further considerations. Screening of initiating events is a component of a risk-informed approach that allows attention to be concentrated on important contributors to risk. The screening process eliminates those potential initiators that are either incapable of initiating an event sequence having radiological consequences or are too improbable during the preclosure period to warrant further consideration. The screening arguments are based on either a qualitative or quantitative analysis documented under separate cover, or through engineering judgment based on considerations of site and design features documented herein.

Initiating events are screened out and are termed beyond Category 2 if they satisfy either of the following criteria:

- The initiating event has less than one chance in 10,000 of occurring during the preclosure period.
- The initiating event has less than one chance in 10,000 over the preclosure period of causing physical damage to a waste form that would result in the potential for radiation exposure or inadvertent criticality.

In some instances, initiating event screening analysis is based on engineering or expert judgment. Such judgment is based on applications of industry codes and standards, comparison to results of analyses for other similar event sequences that are included, or plausibility arguments based on the combinations of conditions that must be present to allow the initiating event to occur and the event sequence to propagate.

6.0.1 Boundary Conditions for Consideration of Initiating Events

6.0.1.1 General Statement of Boundary Conditions

Manufacturing, loading, and transportation of casks and canisters are subject to other regulations other than 10 CFR Part 63 (e.g., 10 CFR Part 50 (Ref. 2.3.1), 10 CFR Part 71 (Ref. 2.3.3), and 10 CFR Part 72 (Ref. 2.3.4)) and associated quality assurance programs. As a result of compliance with such regulations, the affected SSCs are deemed to provide reasonable assurance that the health and safety of the public are protected. However, if a potential precursor condition could result in an airborne release that could exceed the performance objectives for Category 1 or Category 2 event sequences, or a criticality condition, then a qualitative argument that the boundary condition is reasonable is provided. A potential initiating event that is outside of the boundary conditions but has been found to require a qualitative discussion is the failure to properly dry a SNF canister prior to sealing it and shipping it to the repository.

6.0.1.2 Specific Discussion of Receipt of Properly Dried SNF Canisters

Under the boundary conditions stated for this analysis, canisters shipped to the repository in transportation casks are received in the intended internally dry conditions. Shipments of SNF received at the repository, whatever their origin, are required to meet the requirements of 10 CFR Part 71 (Ref. 2.3.3). NUREG-1617 (Ref. 2.2.66) provides guidance for the NRC safety reviews of packages used in the transport of spent nuclear fuel under 10 CFR Part 71 (Ref. 2.3.3). The review guidance, NUREG-1617 (Ref. 2.2.66, Section 7.5.1.2), instructs reviewers that, at a minimum, the procedures described in the safety analysis report should ensure that:

> Methods to drain and dry the cask are described, the effectiveness of the proposed methods is discussed, and vacuum drying criteria are specified.

NUREG-1536 (Ref. 2.2.65, Chapter 8, Section V) refers to an acceptable process to evacuate water from SNF canisters. No more than about 0.43 gram-mole of water (about 8 grams) will be left in the canister if adequate vacuum drying is performed (Ref. 2.2.65). The following example is cited as providing adequate drying (Ref. 2.2.65, Chapter 8, Section V):

> The cask should be drained of as much water as practicable and evacuated to less than or equal to 4E-4 MPa (3.0 mm Hg or Torr). After evacuation, adequate moisture removal should be verified by maintaining a constant pressure over a period of about 30 minutes without vacuum pump operation. The cask is then backfilled with an inert gas (e.g., helium) for applicable pressure and leak testing.

If the pressure creeps back up to unacceptable level during the 30-minute evaluation time, or in cases where it is important to control oxidant concentrations or achieve needed process reliability improvements, a further step may be performed as follows (Ref. 2.2.65, Chapter 8, Section V).

> The cask is then re-evacuated and re-backfilled with inert gas before final closure. Care should be taken to preserve the purity of the cover gas and, after backfilling, cover gas purity should be verified by sampling.

The procedure described appears to ensure that very little water is left behind. However, the probability of undetected failure when performing the process is not addressed in the deterministic regulation 10 CFR Part 71 (Ref. 2.3.3) or in NUREG-1536 (Ref. 2.2.65). Indeed, there is no after-the-fact water or error detection method in NUREG-1536 or the regulation. Therefore, some unknown number of canisters may arrive in the GROA with more residual water than is expected with proper drying. Because the canisters are welded and are not required to provide for sampling the inside of the canister, nondestructive measurement of the residual water content would be difficult. The following discussion provides reasonable assurance that no significant risks are omitted from the analysis due to adoption of the boundary condition that canisters shipped to the repository in transportation casks are received in the intended internally dry conditions:

1. The YMP will be accepting, handling, and emplacing TAD canisters in a manner consistent with the specifications laid out in the TAD canister system performance specification (Ref. 2.2.41) which prescribes the use of consensus codes and standards along with design requirement associated with GROA specific event sequences.

2. **Criticality**—GROA operating processes are similar to those of nuclear power plant sites with respect to the use of cranes, and there are no processes or conditions that would exacerbate adverse effects associated with abnormal amounts of water retention. Event sequences involving drop and breach of an SNF canister are beyond Category 2 as shown in Section 6.8. To receive a license to transport SNF, 10 CFR 71.55 (Ref. 2.3.3) requires the licensee to demonstrate subcriticality given that "the fissile material is in the most reactive credible configuration consistent with the damaged condition of the package and the chemical and physical form of the contents" under the hypothetical accident conditions specified in 10 CFR 71.73 (Ref. 2.3.3). Drop events, which are unlikely to breach the canister, are also unlikely to impart sufficient energy to the fuel to reconfigure it so dramatically that criticality would be possible even if water is present. It is concluded that existing regulations that apply to the canister and transportation cask for transportation to the repository provide reasonable assurance that a criticality event sequence that depends on the presence of water inside the canister and reconfiguration of the fuel would not occur under conditions that could reasonably be achieved during handling at the repository.

3. **Hydrogen explosion or deflagration**—Radiation from SNF can generate radiolytic hydrogen and oxygen gas in a SNF canister if water is inadvertently left in the canister before it is sealed. Given a processing error that leaves enough residual water, the gas concentrations could conceivably reach levels where a deflagration event could occur. However, precautions taken at the generator sites are expected to make receipt of a canister that was improperly dried unlikely. In addition, an ignition source would be required for an explosion or deflagration to occur. High electrical conductivity of the metal canister would dissipate any high voltage electrical discharge (which is unlikely in any case) and preclude arcing within the canister. Normal handling operations do not subject the canisters to energetic impacts that could cause frictional sparking inside the canister. Therefore, a further unlikely event, such as a canister drop would have to occur to ignite the gas. Considering the combination of unlikely events that must occur, event sequences involving this combination of failures are judged to contribute

insignificantly to the frequency of the grouped event sequences of which they would be a part.

4. **Overpressurization due to residual water**—Given a processing error that leaves an excessive amount of residual water, the internal pressure due to vaporization of water could conceivably breach the canister. If sufficient water were to be left in the canister, overpressurization would occur within hours of the canister being welded closed. Therefore, overpressurization would occur while the canister is still in the supplier's possession and not in the GROA. Ambient environmental conditions in the GROA are similar to those that would be encountered by the canister while it is on the supplier's site and during transportation to the GROA. If there is not enough water to cause overpressurization before the canister reaches the GROA, then overpressurization would not occur in the GROA. Therefore, event sequences associated with this failure mode are considered to be physically unrealizable for loaded canisters that are received from offsite.

6.0.2 Screening of External Initiating Events

6.0.2.1 Initial Screening of External Initiating Events

External Events Hazards Screening Analysis (Ref. 2.2.28) identifies potential external initiating events at the repository for the preclosure period and screens a number of them from further evaluation based on severity or frequency considerations. The four questions that constitute the evaluation criteria for external events screening are:

1. Can the external event occur at the repository?

2. Can the external event occur at the repository with a frequency greater than 10^{-6}/yr, that is, have a 1 in 10,000 chance of occurring in the 100 year preclosure period?

3. Can the external event, severe enough to affect the repository and its operation, occur at the repository with a frequency greater than 10^{-6}/yr, that is, have a 1 in 10,000 chance of occurring in the 100 year preclosure period?

4. Can a release that results from the external event severe enough to affect the repository and its operations occur with a frequency greater than 10^{-6}/yr, that is, have a 1 in 10,000 chance of occurring in the 100 year preclosure period?

The screening criteria are applied for each of the external event categories listed in Table 6.0-1. Each external event category is evaluated separately with a definition and the required conditions for the external event to be present at the repository. Then the four questions are applied. Those external event categories that are not screened out are retained for further evaluation as initiating events in the event sequences for the preclosure safety analysis.

As noted in Table 6.0-1, the potential external initiating event categories that are retained for further evaluation are seismic activity and loss of power. Seismically induced event sequences are developed, categorized, and documented in a separate analysis (Ref. 2.4.4). Loss of offsite power (LOSP) is treated together with internal causes of power loss in Section 6.0.2.2.

Table 6.0-1. Retention Decisions from External Events Screening Analysis

External Event Category	Retention Decision. If Not Retained, Basis for Screening.
Seismic activity	**YES**. Retained for further analysis.
Nonseismic geologic activity	**NO**. Except for one of the subcategories, drift degradation, the external events in this category are not applicable to the site or do not occur at a rate that could affect the repository during the preclosure period. The chance of drift degradation severe enough to affect the repository and its operation over the preclosure period is less than 1/10,000.
Volcanic activity	**NO**. The chance of volcanic activity occurring at the repository over the preclosure period is less than 1/10,000.
High winds / tornadoes	**NO**. The chance of a high wind or tornado event severe enough to affect the repository and its operation occurring at the repository over the preclosure period is less than 1/10,000.
External floods	**NO**. The chance of a flood event severe enough to affect the repository and its operation occurring at the repository over the preclosure period is less than 1/10,000.
Lightning	**NO**. The chance of a lightning event severe enough to affect the repository and its operation occurring at the repository over the preclosure period is less than 1/10,000.
Loss of power event	**YES**. Retained for further analysis. See Section 6.0.2.2 for a screening analysis of loss of electrical power as an initiating event.
Loss of cooling capability event	**NO**. The primary requirements for cooling water at the Yucca Mountain site during the preclosure period are makeup water for the WHF pool and cooling of HVAC chilled water. The chance of a loss of cooling capability occurring at the repository over the preclosure period is less than 1/10,000.
Aircraft crash	**NO**. The chance of an accidental aircraft crash occurring at the repository over the preclosure period is less than 1/10,000.
Nearby industrial/military facility accidents	**NO**. The chance of an industrial or military facility accident occurring at the repository over the preclosure period is less than 1/10,000.
Onsite hazardous materials release	**NO**. The chance of an accident event sequence initiated by the release of onsite hazardous materials at the repository over the preclosure period is less than 1/10,000.
External fires	**NO**. The chance of an external fire severe enough to affect the repository and its operation occurring at the repository over the preclosure period is less than 1/10,000.
Extraterrestrial activity	**NO**. Extraterrestrial activity is defined as an external event involving objects outside the earth's atmosphere and enters the earth's atmosphere, survive the entry through the earth's atmosphere and strike the surface of the earth. Extraterrestrial activity include: meteorites, asteroids, comets, and satellites. The chance of an occurrence at the repository over the preclosure period is less than 1/10,000.

NOTE: The source document defines the categories.
HVAC = heating, ventilation, and air conditioning; WHF = Wet Handling Facility.

Source: Adapted from *External Events Hazards Screening Analysis* (Ref. 2.2.28, Sections 6 and 7).

6.0.2.2 Screening of Loss of Electrical Power as an Initiating Event

Loss of electrical power, whether caused by onsite or offsite failures, is expected to occur during the preclosure period. Conveyances, cranes, and CTMs that rely on electric power will stop upon loss of power, but are designed to hold loads indefinitely. A set of redundant emergency diesel generators and the associated ITS electrical distribution system would start upon LOSP in order to continue operation of the ITS HVAC confinement system.

LOSP is not shown as an initiating event in the event trees because, by itself, it does not cause mechanical handling equipment to malfunction in a way that causes a drop or other mechanical impact of a waste container. Therefore, load drop and LOSP may be treated as independent events. The following calculation demonstrates that a LOSP and coincident load drop is beyond Category 2.

The LOSP frequency is estimated at 3.6E-02/yr (Ref. 2.2.42, Table 3-8), with a failure to recover power within 24 hours of 1.8E-02 (Ref. 2.2.42, Table 4-1). Thus, during the 50-yr portion of the preclosure period in which waste handling operations are conducted, the expected number of LOSP events is:

LOSP # = 3.6E-02 / yr × 50 yr
= 1.8;

The initiating frequency of a LOSP lasting more than 24 hours would be:

LOSP-IE = 3.6E-02 / yr × (1.8E-02) × 50 yr
= 3.2E-02 / preclosure period

An independent load drop from a crane following a LOSP would probably be caused by crane holding and emergency brake failures or random hoist cable breaks (each CTM and crane uses multiple wire ropes) because no other movement induced failure modes have been identified. Crane brake failures are more frequent than wire rope breaks, and for this calculation, the brake failure rates are used to determine a load drop probability. Two failure modes for the brakes have been modeled: failure of the brake to set and failure of the brakes to hold for an extended period. As documented in Attachment C, Table C4-1, estimated crane brake failure rates are:

- Holding (pneumatic) brake (BRP-FOD & BRP-FOH): 5.0E-05 per demand (initial setting of the brake) and 8.4E-06 per hour (holding the load for the duration of the power loss)

- Emergency brake (BRK-FOD & BRP-FOH): 1.5E-06 per demand (initial setting of the brake) and 4.4E-06/hr (holding the load for the duration of the power loss).

The four components of LOSP and brake failures are:

1. Both the holding brake and emergency brake fail to set on a LOSP resulting in a load drop.

2. Holding brake fails to set at LOSP. Emergency brake sets at LOSP but fails to hold during an extended loss of power (720 hours) resulting in a load drop.

3. Emergency brake fails to set at LOSP. Holding brake sets at LOSP but fails to hold during an extended loss of power (720 hours) resulting in a load drop.

4. Both brakes set at LOSP but fail to hold during an extended loss of power (720 hours) resulting in a load drop.

The failure components described above are analogous to the failure modes of a two train system in standby where at least one train must successfully start and run for a specified mission time to prevent system failure.

The fourth component described above dominates probabilistically and its calculation is described below. The sum of the other three are more than two orders of magnitude lower.

The likelihood of an extended LOSP has been estimated by using the probability of a LOSP exceeding 24 hours, which is the longest non-recovery period identified in NUREG/CR-6890 (Ref. 2.2.42). The 720 hour period for which a brake holding failure has been modeled should provide ample time to either recover offsite power or for operators to implement an alternative means to safely lower any load. Provision for manual lowering of loads is provided in NOG-1 cranes (Ref. 2.2.10).

The probability of the fourth component described above – the combination of LOSP and load drop (brakes set but fail to hold over a 720 hour mission time) is:

LOSP-IE × Holding brake fails × Emergency brake fails =
= 3.2E-02 × (8.4E-06 × 720) × (4.4E-06 × 720)
= 6.1E-07

Thus, the LOSP load drop probability over the preclosure period is estimated to be 6E-07. This number of occurrences of the compound initiating event is much less than one chance in 10,000 (1E-4) during the preclosure period. Therefore, event sequences with LOSP and a coincident drop load as the initiating event are beyond Category 2.

The possibility of inadvertent direct exposure of workers due to a loss of electrical power is considered next. Canisters are always shielded during facility operations by a transportation cask, a canister preparation platform, concrete floors and walls, the CTM shield bell and shield skirt, the WPTT, facility shield doors, and the TEV shield compartment. Loss of electrical power to any of these simply stops operations while maintaining shielding. For example, inadvertent shield bell and shield door motion can not occur in the absence of electrical power. Therefore, direct exposure to workers owing to loss of electrical power is considered to be beyond Category 2.

It has been shown that loss of electrical power in conjunction with other failures is screened out as an initiating event. Nevertheless, this compound failure mode is included in the initiating and pivotal event fault trees as appropriate. For example, the hoist brake on the CTM requires electrical power to remain unengaged. A loss of power would cut power to the brake, leading to its automatic engagement. If the brake fails in conjunction with a loss of power in this scenario, a drop of the load could occur, initiating an event sequence. This failure scenario is included in the CTM fault tree. For the overhead cranes, the initiating event frequencies are based on industry-wide empirical data for cranes. The ITS HVAC system depends on continued electrical power and it is explicitly modeled in the fault tree for this pivotal event.

6.0.3 Screening of Internal Initiating Events

All facility safety analyses, whether risk-informed or not, take into account the physical conditions, dimensions, materials, human-machine interface, and other attributes such as operating conditions and environments, to assess potential failure modes and event sequences. Such accounting guides the assessment of what can happen, the likelihood, and the potential consequences. In many situations, it is obvious that the probability of a particular exposure scenario is very low. In many cases, it is impractical or unnecessary to actually quantify the probability when a non-probabilistic engineering analysis provides sufficient assurance and insights that permit the scenario to be either screened out or demonstrated to be bounded by another scenario.

Potential initiating events were qualitatively identified in the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34) for quantitative treatment in the present analysis. For completeness, some events were identified in the event sequence development analysis that are extremely unlikely or physically unrealizable and can reasonably be qualitatively screened from further consideration. A qualitative screening argument for certain internal initiating events is developed in the present analysis as documented in Table 6.0-2. The first column of Table 6.0-2 indicates the branch of the initiator event tree (where applicable) that pertains to the screened initiating event. Each branch of an initiator event tree represents an initiating event or an initiating event group that includes other similar initiating events and corresponds to a little bubble on an ESD (Ref. 2.2.34, Attachments F and G). Some of the initiating events that are addressed in Table 6.0-2 were implicitly screened out in the event sequence development analysis and for that reason there is no applicable event tree. The screening argument for internal flooding is presented in Section 6.0.4. The screened initiating events are assigned frequencies of zero in the quantification of the model.

Table 6.0-2. Bases for Screening Internal Initiating Events

Initiator Event Tree (Branch No.)	Initiating Event Description	Screening Basis
RF-ESD03-DPC (#2) (Figure A5-7) RF-ESD03-TAD (#2) (Figure A5-8)	Operator drops cask during cask preparation activities	The 20-ton auxiliary crane, rather than the 200-ton crane, is used in the lid-removal operation. Because the cask is not intentionally lifted in this step, dropping the cask would require a series of extraordinary human failures. For DPCs, a cask drop would require a series of human failures as follows: During lid removal, the crew must fail to remove some fraction of the lid bolts, fail to properly use the check list to verify bolt removal, and use the wrong crane (the 20-ton crane would be incapable of lifting the cask). The crane operator and at least two other crewmembers will be standing on the platform in direct view of the cask during lid removal and they all would have to fail to notice that the entire cask is being lifted before the bolts break. Therefore, event sequences associated with this initiating event are judged to contribute insignificantly to the frequency of the grouped event sequences of which they would be a part.

Table 6.0-2. Bases for Screening Internal Initiating Events (Continued)

Initiator Event Tree (Branch No.)	Initiating Event Description	Screening Basis
		For casks other than DPCs, the lid is not removed from the cask at this point. Therefore, no configuration that could result in a crane lifting the cask occurs for such casks. This initiating event, as it relates to casks other than DPC casks, is considered to be unrealizable.
RF-ESD04-DPC (#2) (Figure A5-9) RF-ESD04-TAD (#2) (Figure A5-11)	Structural damage to transportation cask due to impact from the crane hook or rigging while under the cask preparation platform	In this operation, the lid is unbolted and the lid lift fixture is attached. The cask is flush or recessed with respect to the cask preparation platform, and therefore cannot be impacted. Therefore, event sequences associated with these initiating events are considered to be physically unrealizable.
No applicable event trees	Conveyance carrying a waste form collides with a shield door, causing the door to dislodge from its supports and fall onto the waste form	The shield doors are designed to withstand collision of the conveyance into the door without dislodging from their supports such that the stress of all support mechanisms of the door stay below yield. Therefore, this initiating event is considered physically unrealizable.
RF-ESD06-DPC (#7) (Figure A5-14) RF-ESD06-TAD (#7) (Figure A5-16)	Canister dropped inside the shield bell (with CTM slide gate closed)	Drops within the shield bell have been subsumed within event sequences for drops from the operational lift height, and are not separately addressed. This is conservative because the drop height within the shield bell is less than the operational lift height.
RF-ESD06-DPC (#5) (Figure A5-14) RF-ESD06-TAD (#5) (Figure A5-16)	Side impact from a slide gate	Slide gate impacts during CTM transfer are included in the CTM fault tree as a cause of canister drop, rather than as an independent initiating event. In addition, the motors on the slide gates have insufficient power to significantly damage a canister. Branch #5 of the listed event trees covers side impact with the CTM shield bell due to CTM collision.
RF-ESD06-DPC (#2) (Figure A5-14)	Canister impact during lid removal by the CTM	This initiating event is not applicable to the event tree listed because the DPC lid is not removed by the CTM. Therefore, event sequences associated with this initiating event are considered to be physically unrealizable.
RF-ESD09 (#2) Figure A5-22	Rollover of horizontal cask transfer trailer carrying a transportation cask in the Transportation Cask Vestibule or Cask Preparation Room	For a truck trailer to roll over, its center of mass has to move laterally beyond the wheel base of the trailer. This could occur upon traversing a significantly uneven surface, running over a very large object, turning sharply at high speed or by jack knifing the trailer while backing up. There are no uneven surfaces in the Transportation Cask Vestibule/Annex or Cask Preparation Room. The area in question has a flat concrete surface. There are no objects that could be run over that could significantly shift the trailer's center of mass. Turning sharply at high speed or jack-knifing the trailer is not possible inside the building because the rooms are too narrow and the truck comes to a complete stop outside the closed entrance door prior to the door opening and the truck entering. Therefore, event sequences associated with this failure mode are considered to be physically unrealizable.
No applicable event trees	Internal flooding	Internal flooding as an initiating event is screened from further analysis in Section 6.0.4.

Table 6.0-2. Bases for Screening Internal Initiating Events (Continued)

Initiator Event Tree (Branch No.)	Initiating Event Description	Screening Basis
No applicable event trees	Canister dropped into the Loading Room with no aging overpack present	Dropping a canister through the port without a staged aging overpack below would require a series of human failures and mechanical failures that makes the initiating event unlikely. The design incorporates an interlock to prevent the opening of the port slide gate when the aging overpack is not present (Ref. 2.2.30). The combination of (a) failure to stage the aging overpack, (b) failure of more than one operator to notice that it not staged, (c) failure of the hardwired interlock, and (d) drop of the canister are required for such an initiating event to occur. Considering the combination of unlikely events that must occur to cause this initiating event, event sequences involving this combination of failures are judged to contribute insignificantly to the frequency of the grouped event sequences of which they would be a part.
No applicable event trees	Tipover of CTT	The CTT is designed to prevent tipover. (Ref. 2.2.21, Section 3.2). The size, weight, low center of gravity, and low speed of the CTT ensure that no tipover can occur. During cask preparation activities, the CTT is normally set on the floor inside the cask preparation platform. As such, tipover is not physically realizable during preparation activities. During transit, the CTT glides slowly on a cushion of air, an inch or less above the floor. If air pressure is lost, the CTT, with its load, settles to the floor. While the CTT is in transit, or after settling to the floor, any applied force from facility operations is incapable of tipping over the CTT. Due the slow travel of the CTT, a loss of air pressure or a collision with other equipment or a facility structure will not result in tipover. Therefore, tipover of the CTT is considered physically unrealizable for internal events. CTT tipover, however, is analyzed in the seismic event sequence and categorization analysis.
No applicable event trees	Explosion of site prime mover fuel tank	The fuel tank of the site prime mover has safety features that preclude fuel tank explosion. Therefore, this initiating event is considered physically unrealizable.

NOTE: Initiator event trees are provided in Attachment A in the figures cited. The branch numbers are shown in each figure under the column labeled "#". The branch numbers are shown in each figure under the column labeled "#". CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister.

Source: Original

6.0.4 Screening of Internal Flooding as an Initiating Event

By the definition of an event sequence, a flood inside a facility would be an initiating event if it led to a sequence of events that would either breach waste containers, causing a release, or caused elevated radiological exposure without a release (i.e., direct exposure of personnel). Internal floods, whether caused by random failure or earthquakes, emerge from two sources. The first is inadvertent actuation of the fire-suppression system. The second is failure of water-carrying pipes or valves associated with chilled water, hot water, potable water, or other water systems. Drains, channels and curbs are situated to remove water from these sources. However, the following discussion does not rely on these.

Transportation casks and canisters are not physically susceptible breach associated with water in the short-term. With extremely long exposure to water, corrosion may be a factor, but intervention to drain water from the buildings would prevent such exposure. Short-term breaches do not occur owing to exposure to water. Canisters are surrounded by transportation casks or aging overpacks. Transportation casks are elevated at all times at least five feet above the floor by railcar or CTT. A lifted canister or/and cask is higher than these minimum elevations. Therefore, water from fire suppression and other water systems is unlikely to attain a depth that would contact transportation casks or canisters. Of greater significance, however, is that the fuel is contained in canisters within an overpack nearly all the time and these containers do not fail from short-term exposure to flood water. In this context, short-term is a time period that is at least 30 days but less than the length of time in which significant corrosion may occur.

Water impingement on electrical equipment (e.g., motor control centers, motors, and switchgear cabinets) would ordinarily trigger circuit protection features that would open the circuit and cause a loss of electrical power (which is covered in Section 6.0.2.2). If a short circuit occurred as a result of water impingement, normal circuit protection features or overheating of the wires would subsequently open the affected circuit. In an extreme situation, an electrical fire might be started. Fires from all causes are covered in Section 6.5.

The possibility of inadvertent direct exposure of workers due to internal flooding is considered next. Direct exposure to workers during a flood would occur if shielding were disabled as a result of the flooding. Canisters are always shielded during facility operations by transportation casks, cask preparation platforms, concrete floors and walls, the CTM shield bell or shield skirt, or the unloading or loading room shield doors. Loss of electrical power to any of these simply stops operation, if any, without affecting the shielding. Flooding might also cause hot shorts in control boxes. However, hardwired interlocks between the CTM slide gate, shield bell skirt, and shield doors prevents such inadvertent motion. Therefore, internal flooding cannot initiate an event sequence that causes increased levels of radiological exposure to workers.

Moderator intrusion into canisters resulting from event sequences that might breach a waste container is treated quantitatively as described in the pivotal event descriptions of Section 6.2.

6.1 EVENT TREE ANALYSIS

The event trees that are quantified in this analysis were developed from ESDs in the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34, Attachments F and G). This section describes the use of SAPHIRE (Section 4.2) to model event sequences. The event trees are discussed and presented in Attachment A.

6.1.1 Event Tree Analysis Methods

6.1.1.1 Linked Event Trees and Fault Trees

As described in Section 4, the PCSA uses linked event trees with linked fault trees to calculate the frequency of occurrence of event sequences. The SAPHIRE computer program (Section 4.2) is used for this purpose. The event tree quantification is supported by fault tree analysis (FTA) (Section 6.2 and Attachment B), HRA (Section 6.4 and Attachment E), and PEFA (Section 6.3 and Attachment D). The YMP preclosure handling is performed using four kinds of buildings as summarized below:

1. The RF accepts DPC and TAD canisters and places them into aging overpacks, either destined for the aging pads or the CRCF.

2. The CRCF accepts all waste containers except those supplied by the Naval Nuclear Propulsion Program (NNPP) for placement in waste packages destined for emplacement in the repository emplacement drifts. Three CRCFs are currently considered.

3. The WHF accepts DPCs and transportation casks containing uncanistered commercial SNF, transfers the SNF to TAD canisters which are destined for the CRCF or the aging pads.

4. The Initial Handling Facility (IHF) accepts canisters from the NNPP and some canisters containing high-level radioactive waste for placement in waste packages destined for emplacement in the repository emplacement drifts.

Preclosure waste handling as modeled in the PCSA also includes TEV and Subsurface Operations. The TEV accepts waste packages from the CRCF and IHF and, by means of rail, transports and deposits it into its designated location in the emplacement drifts. All other extra-building transportation, low-level waste handling, and balance of plant is called Intra-Site Operations.

Event sequences are developed for each of the four building types, TEV and Subsurface Operations, and Intra-Site Operations. Because each type of waste container in the RF has different characteristics that manifest during event sequences, separate event sequences are developed for each type of waste container. As described in the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34), event sequences are also developed separately for each major group of waste handling processes by location within the building. Therefore, event sequences also distinguish among the various steps in waste handling.

As described in Section 4.3, event sequences result in one of the following end states:

1. "OK"
2. Direct Exposure, Degraded Shielding
3. Direct Exposure, Loss of Shielding
4. Radionuclide Release, Filtered (HVAC)
5. Radionuclide Release, Unfiltered (HVAC system is not operating)

6. Radionuclide Release, Filtered, Also Important to Criticality
7. Radionuclide Release, Unfiltered, Also Important to Criticality
8. Important to Criticality (not applicable to the RF).

Radionuclide release describes a condition where radioactive material has been released from the container creating a potential inhalation or ingestion hazard, accompanied by the potential for immersion in a radioactive plume and direct exposure.

The SAPHIRE computer program has advanced features that permit the analyst to control the inputs and conditions for quantifying linked event trees and fault trees. One feature is the use of "basic rules" by which the analyst tells the program how and when to link certain variations of fault trees and basic event data that describe a given initiating and pivotal event. This allows path dependent development of sequence minimal cut sets and probabilities.

The primary inputs to the program are the following:

- Event tree logic models
- Fault tree logic models for initiating and pivotal events
- Initiating event frequencies derived from waste-form throughputs and numbers of opportunities for initiating an event sequence
- Basic event data that provides failure rates for active and passive equipment and for HFEs. The basic event data also includes a probability distribution of uncertainty associated with each basic event. The event tree and fault tree logic models are linked to the basic event library.

Each basic event is characterized by a probability distribution. SAPHIRE's Monte Carlo sampling method is employed to propagate the uncertainties to obtain event sequence mean values and parameters of the underlying probability distribution such as variance. As described in Section 4.3.6, categorization is done on aggregated event sequences, whose resultant probability distributions are also obtained by Monte Carlo simulation. SAPHIRE accounts for the correlation between analogous basic events sharing the same reliability information, which ensures the spread of the probability distribution of the event sequences in which these basic events intervene is not underestimated.

6.1.1.2 Initiator, System-Response, and Self-Contained Event Trees

Event sequences are described and graphically depicted using one or two event trees depending on whether the ESD considered has one or more initiating events:

1. **Self-contained event trees.** Self-contained event trees are used when only one initiating event appears in the corresponding ESD (Ref. 2.2.34, Attachment F). An example is RF-ESD05-DPC, which is shown in Figure A5-12 in Attachment A. The feed on the left side of the event tree is an event that represents the frequency of the challenge to the successful operation of the process step represented in the event tree. In the example, the frequency of challenge is equal to the number of transportation

casks containing DPCs that are handled over the preclosure period. The initiating event is presented next, followed by the pivotal events. By convention, the description of each branching event is stated as a success. The branching under each event heading represents success by an upward branch and failure by a downward branch. If a given pivotal event cannot occur in a given sequence due to a prior pivotal event or is irrelevant to the sequence, it does not appear in the event sequence as illustrated in the corresponding ESD and no branching occurs in the event tree. Each pathway through a self-contained event tree terminates in an end state. End states that are labeled "OK" mean that the sequence of events does not result in one of the specifically identified undesired outcomes. "OK" often means that normal operation can continue. The undesired end states represent a release of airborne radioactivity, a direct exposure to radiation, or a potential criticality condition.

2. **Separate initiator and system-response event trees**. Separate event trees for initiating events and the system response are used when more than one initiating event appears in the corresponding ESD (Ref. 2.2.34, Attachment F). The initiator event tree decomposes a group of initiating events into the specific failure events that comprise the group. For example, an initiator event tree, RF-ESD01-DPC, is shown in Figure A5-2 in Attachment A, and the corresponding system response event tree, RESPONSE-TCASK1, is shown in Figure A5-3. The feed to the left side of the initiator event tree is an event that represents the frequency of challenge to the successful operation of the process step represented in the event tree. In the example, the frequency of challenge is equal to the number of transportation casks containing DPCs that are received during the preclosure period. _ event trees do not end at end states but transfer to a system response event tree. The models to be used for the initiating events associated with each initiator event tree are specified in SAPHIRE "basic rules," which are attached to the initiator event tree.

   System response event trees contain only pivotal events. In accordance with the basic rules that are written for a given initiator event tree, the SAPHIRE program links specific fault tree model or basic event to a given pivotal event. For example, the system response tree in Attachment A, Figure A5-3 shows the system response event tree RESPONSE-TCASK1. Because the conditional probability of each pivotal event may be specific to the initiating event for each event sequence, the same system response event tree is quantified by SAPHIRE as many times as there are initiating events in the initiator event tree. The models to be used for the pivotal events associated with each initiating event and system response event tree are specified in SAPHIRE basic rules, which are attached to the associated initiator event tree.

6.1.1.3 Summary of the Major Pivotal Events

A self-contained event tree or a system response event tree may include pivotal events concerning the success or failure of the transportation cask, canister, shielding properties, HEPA filtration availability, and moderator intrusion susceptibility. The pivotal events are summarized in Attachment A, Section A3.

Each of the specific failure events included in a self-contained or system-response event tree may be linked to a basic event or to the top event of a fault tree. Two kinds of fault trees are developed and represented in Attachment B. The first type represents equipment fault trees including HFEs that contribute directly to the specific pivotal or initiating event. The second type links initiating and pivotal events to these equipment fault trees (via transfer gates) and miscellaneous events. This second type is called linking or connector fault trees. The equipment fault tree models are, in turn, linked to basic event reliability information separately entered into SAPHIRE. Some of the pivotal events do not have associated fault trees because they are linked directly to probabilities in the reliability database entered into SAPHIRE. Section 6.2 provides more information about the reliability information developed for this analysis.

6.1.2 Waste Form Throughputs

Each initiator event tree and self-contained event tree begins with the container throughputs, that is, the numbers of waste form units (such as casks or canisters) to be handled over the life of the RF. The throughputs are identified in Table 6.1-1 and are drawn into the descriptions of specific event trees as needed. With the number of waste form units as a multiplier in the event tree and the initiating events specified as a probability per waste form unit, the value passed to the system response is the number of occurrences of the initiating event expected over the life of the facility.

Table 6.1-1. Waste Form Throughputs for the RF Over the Preclosure Period

Waste Form Unit	RF Throughput Over Preclosure Period	Comment
Transportation casks containing a TAD canister	6,978	One canister per cask
Transportation casks containing a DPC	346	One canister per cask
TAD canisters (44 BWR or 21 PWR SNF assemblies per canister)	6,978	Same as number of TAD canister casks
DPCs (64 BWR or 25 PWR SNF assemblies per canister)	346	Same as number of DPC casks
Aging overpack containing a TAD canister	6,978	One canister per aging overpack
Aging overpack containing a DPC	346	One canister per aging overpack
Transportation casks containing a TAD canister	6,978	One canister per cask

NOTE: BWR = boiling water reactor; DPC = dual-purpose canister; PWR = pressurized water reactor; RF = Receipt Facility; SNF = spent nuclear fuel; TAD = transportation, aging, and disposal.

Source: Waste Form Throughputs for Preclosure Safety Analysis, (Ref. 2.2.27, Table 4).

6.1.3 Guide to Event Trees

Event trees are located in Attachment A. Table 6.1-2 contains the crosswalk from the ESD (Ref. 2.2.34, Attachment F) to the initiating event tree and response tree figure location in Attachment A.

Table 6.1-2. Figure Locations for Initiating Event Trees and Response Trees

ESD#	ESD Title	IE Event Tree Name	IE Event Tree Location	Response Tree Name	Response Tree Location
RF-ESD-01	Event Sequences for Activities Associated with Receipt of Transportation Cask into Cask Preparation Room	RF-ESD01-DPC RF-ESD01-TAD	Figure A5-2 Figure A5-4	RESPONSE -TCASK1	Figure A5-3
RF-ESD-02	Event Sequences for Activities Associated with Removal of Impact Limiters, Cask Upending, and Transfer to CTT or Cask Transfer Trailer	RF-ESD02-DPC RF-ESD02-TAD	Figure A5-5 Figure A5-6	RESPONSE -TCASK1	Figure A5-3
RF-ESD-03	Event Sequences Associated with Unbolting and Lid Adapter Installation	RF-ESD03-DPC RF-ESD03-TAD	Figure A5-7 Figure A5-8	RESPONSE -TCASK1	Figure A5-3
RF-ESD-04	Event Sequences Associated with Transfer of a Cask on CTT from Cask Preparation Area to Cask Unloading Room	RF-ESD04-DPC RF-ESD04-TAD	Figure A5-9 Figure A5-11	RESPONSE -TCASK2	Figure A5-10
RF-ESD-05	Event Sequences Associated with a Transportation Cask on a CTT or Site Transporter Colliding with Lid Bolting Room or Cask Unloading Room Shield Doors	RF-ESD05-DPC RF-ESD05-TAD	Figure A5-12 Figure A5-13	N/A	N/A
RF-ESD-06	Event Sequences for Activities Associated with the Transfer of a Canister from Transportation Cask, to Aging Overpack with CTM	RF-ESD06-DPC RF-ESD06-TAD	Figure A5-14 Figure A5-16	RESPONSE - CANISTER1	Figure A5-15
RF-ESD-07	Event Sequences for Activities Associated with Assembly and Closure of an Aging Overpack	RF-ESD07-DPC RF-ESD07-TAD	Figure A5-17 Figure A5-19	RESPONSE -AO1	Figure A5-18
RF-ESD-08	Event Sequences for Activities Associated with the Exporting of an Aging Overpack from the RF	RF-ESD08-DPC RF-ESD08-TAD	Figure A5-20 Figure A5-21	RESPONSE -AO1	Figure A5-18
RF-ESD-09	Event Sequences for Activities Associated with Export of Horizontal Cask on Cask Transfer Trailer	RF-ESD09	Figure A5-22	RESPONSE -TCASK1	Figure A5-3
RF-ESD-10	Event Sequences for Activities Associated with Direct Exposure During DPC Handling Activities	RF-ESD10	Figure A5-23	N/A	N/A
RF-ESD-11	Event Sequences for Activities Associated with Direct Exposure During CTM Activities	RF-ESD11	Figure A5-24	N/A	N/A
RF-ESD-12	Event Sequences for a Fire Occurring in Receipt Facility	RF-ESD12-DPC RF-ESD12-TAD	Figure A5-25 Figure A5-27	RESPONSE -FIRE	Figure A5-26

NOTE: CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; N/A = not applicable.

Source: Attachment A, Table A5-1

6.2 ANALYSIS OF INITIATING AND PIVOTAL EVENTS

6.2.1 Approach to Analysis of Initiating and Pivotal Events for Linking to Event Sequence Quantification

Section 4.3.2 provides a brief introduction to the application of FTA for initiating and pivotal events, including an example fault tree. Many of the initiating events involve faults in complex machinery for which no historical data exists at the system level, an exception being historical data on load drops from cranes. Therefore, FTA is employed to map elements of equipment design and operational features to various failure modes of components down to a level of assembly, termed “basic events” for which historical data is available. Attachment B presents the fault tree logic and stand-alone quantifications.

Much of the equipment used in the RF is also used in other surface facilities and the Intra-Site Operations. Furthermore, a given system, such as the site transporter, may affect the event sequences for several operational nodes of the same facility or several kinds of waste forms, as it does for the RF. Therefore, the logic of the fault trees described in this section and Attachment B are linked to event trees where appropriate, via an intermediate top event name that is unique to the event sequence per the waste form involved and operational node. In this way, the logic structure of the system fault tree may be used over and over but, by virtue of the rules feature of SAPHIRE, the inputs to each fault tree can be tailored to fit the event sequence.

The fault trees are linked to the event trees via the initiating event tree rules file and the application of linking fault trees. The rules file specifies the names of the linking fault trees for initiating event and pivotal event fault trees to be substituted into the event tree top events during quantification. The rules files also specify the use of particular values for basic events and other probabilistic factors that affect the event sequence quantification. The linking fault trees have unique names for the facility and the operational nodes for each event tree. The linking fault trees are very simple, usually having a single top event that is an OR gate that connects to one of the system fault trees. This allows for application of unique top event probabilities to the different initiating events modeled in the initiating event tree.

Attachment B, Sections B1 to B8 presents the system fault trees. These sections describe the bases for the system fault trees and the quantification of their top events.

Attachment B, Section B9 presents the linking fault trees used in the RF analysis. The linking fault trees are self explanatory. No quantification is performed for the linking trees alone.

A top event occurs when one of the ITS success criterion for a given SSC fails to be achieved. At least one success criteria is defined for each system. Multiple success criterion are defined for systems that perform multiple safety functions in the RF.

Each of the top events for the initiating event fault trees represent the conditional probability that the top event will occur when the system is put into service. That is, the results of the FTA answer a question such as “what is the probability for each canister lift that the CTM drops the canister, given a lift?” The expected number of canister drop initiating events during the preclosure period is the product of the number of times a canister is lifted during the preclosure operations and the conditional probability of the top event. Such values for the expected number of canister drops are not developed directly, however. Instead, the initiating event tree in SAPHIRE links the various fault tree logic models to the canister, or other waste form, and the throughput values to generate the initial portions of event sequence cut sets that are subsequently processed as part of the solution of the complete event sequence that includes pivotal events.

By contrast, the top event for the confinement function of the HVAC represents the conditional probability that the confinement feature is not achieved for the required duration following an airborne release of radioactive material inside the RF. The quantification of the top event, as summarized in Section 6.2.2.7 and detailed in Attachment B, Section B7, is expressed as unavailability. The results provide insight into the reliability of the HVAC and its contribution to event sequence quantification. Again, the quantified top event is not used directly in the event sequence quantification. Instead, the fault tree logic for the HVAC is linked to event sequence analysis via SAPHIRE.

In general, each of the FTAs in Attachment B are developed to include both (1) HFEs, and (2) mechanical failures that result in the occurrence of the top event. The HFEs include postulated unintended operator actions that could potentially occur during the facility activity and, as applicable, hardware failures for those SSCs whose functions are to prevent the top event from occurring given the unintended operator action occurs (e.g., interlock). Mechanical failures typically involve random component failures (electrical, mechanical, etc.) and failures from the loss of a supporting system (e.g., loss of power).

For quantification of the probability of the top event, failure probabilities are developed for each basic event (hardware or HFE) and are used to compute the probability of each cut set. For component failure data that is expressed as "failures per hour," a "mission time" must be defined. In many instances in the FTA quantification, a mission time of one hour is used if this value is conservative. Where mission time is critical, appropriate times are justified and incorporated into the event sequence quantification. Hardware failure probabilities are taken from the reliability analysis data discussed in Sections 6.3. HFE probabilities are taken from the HFE analysis discussed in Section 6.4.

Uncertainties in the probabilities of basic events are included in the inputs to the SAPHIRE analysis. The uncertainties are propagated through the FTA to yield the uncertainty distribution of the top event.

Issues that are addressed in the fault trees, in addition to the mapping of the descriptions of the physical system into a fault tree logic diagram based on explicit effects of mechanical and hardware failures, include the following:

- Basic event data
- Common-cause and common mode failures such as failures induced by common training, maintenance practices, fabrication, common electrical supplies, etc.
- Support systems and subsystems such as filtering (HVAC HEPA filters), electrical, etc.
- System interactions

- HFEs
- Control logic malfunctions.

The following subsections provide summaries of the analyses detailed in Attachment B. For each fault tree, the following information is provided:

- Physical description
- Operation
- Control system
- System/pivotal event success criteria
- Mission time
- Fault tree results.

6.2.2 Summary of Fault Tree Analysis

6.2.2.1 Site Prime Mover Fault Tree Analysis

The FTA for the site prime mover (SPM) is detailed in Attachment B, Section B1. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B1 for sources of information on the physical and operational characteristics of the SPM.

6.2.2.1.1 Physical Description

The SPM is a diesel/electric self-propelled vehicle that is designed to move railcars or truck trailers loaded with transportation casks. The transport occurs for both the Intra-Site Operations and within the RF. A speed limiter is used on the SPM to ensure the maximum speed does not exceed nine miles per hour. Movement of the SPM with railcars (termed SPMRC) within the RF is limited to the Transportation Cask Vestibule and the Cask Preparation Room.

Retractable railroad wheels attached to the front and rear axles of the SPM are used for rail operations. The driving and braking power comes directly from the road tires, as they are in contact with the rails. A diesel engine provides the energy to operate the SPM outside the facilities. Inside, the SPM is electrically driven via an umbilical cord from the facility main electrical supply.

6.2.2.1.2 Operations

In-facility SPM operations begin after the SPM has positioned the railcar outside the RF. The SPM diesel engine is shut down and the outer door is opened. Facility power is connected to the SPM for all operations inside the facility. The operator connects the pendant controller or uses a remote (wireless) controller to move the SPM to push the railcar into the vestibule. The Transportation Cask Vestibule serves as an airlock for the facility, providing an environmental separation between the Cask Preparation Room and the outside environment. To maintain negative pressure within the facility, the vestibule has interlocked inner and outer access doors. Only one door can open at a time when moving equipment in or out.

In the event of loss of power, the SPM is designed to stop, retain control of the railcar and enter a locked mode where it remains until operator action is taken to return to normal operations.

6.2.2.1.3 Control System

A simplified block diagram of the functional components on the SPMRC is shown in Attachment B, Section B1, Figure B1.2-1.

The control system provides features for preventing initiating events:

- The SPM is designed to stop whenever, (1) commanded to stop, or (2) when there is a loss of power.
- The operator can stop the SPM by either commanding a "stop" from the start/stop button or by releasing the palm switch which initiates an emergency stop.
- At anytime there is a loss of power detected, the SPM will immediately stop all movement and enter into "lock mode" safe state. The SPM will remain in this locked mode until power is returned and the operator restarts the SPM.

6.2.2.1.4 System/Pivotal Event Success Criteria

Success criteria for the SPM are the following:

- Prevent SPMRC collisions
- Prevent SPMRC derailment.

Various design features are provided to achieve each of the success criteria. The failure to achieve each success criterion defines the top event of a fault tree for the SPM.

6.2.2.1.5 Mission Time

A nominal one-hour mission time is used to calculate the failure probability for components having a time-based failure rate. One hour is conservative because it does not require more than one hour to disconnect the SPM from the railcar and move it from the facility. Otherwise, failure-on-demand probabilities are used.

For railcar derailment, the probability is based on the distance traveled inside the RF, 0.04 miles, and industry data derailment rate of 1.18E-5 per mile traveled (Attachment C, Table C4-1, Item DER-FOM).

6.2.2.1.6 Fault Tree Results

The detailed description in Attachment B, Section B1 documents the application of basic event data, CCFs, and HRA.

The SPMRC has two credible failure scenarios:

- SPMRC collides with RF structures
- SPMRC derailment.

Each failure mode may occur with various waste forms that are received in the transportation casks.

Results of the analysis are summarized in Table 6.2-1.

Table 6.2-1. Summary of Top Event Quantification for the SPM

Top Event	Mean Probability	Standard Deviation
SPM collides with RF structures (DPC on RC)	4.3E-03	1.1E-2
SPMRC derailment (DPC on RC)	4.7E-7	8.8E-14

NOTE: DPC = dual-purpose canister; RC = railcar; RF = Receipt Facility; SPM = site prime mover.

Source: Attachment B, Section B1, Figures B1.4-1 and B1.4-6.

6.2.2.2 Cask Transfer Trolley Fault Tree Analysis

The FTA for the CTT is detailed in Attachment B, Section B2. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B2 for sources of information on the physical and operational characteristics of the CTT.

6.2.2.2.1 Physical Description

The CTT is an air powered machine that is used to transport various vertically oriented transportation casks from the Cask Preparation Room to the Cask Unloading Room. The trolley consists of a platform, a cask support assembly, a pedestal assembly, a seismic restraint system, and an air system.

The CTT will handle a number of different casks so several different pedestals are used to properly position the cask height. Each pedestal sub-component is designed for its respective cask to sit down in a “cavity.” In addition, the cask is restrained in the longitudinal and transverse directions by the cavity walls and restrained in the vertical down direction by the pedestal itself. This design also ensures the cask is positioned correctly. The trolley is positioned within a set tolerance under the cask port in the Cask Unloading Room using bumpers and stops that are bolted to the floor of the Cask Unloading Room and which are designed with bolts that would break to allow the CTT to slide during a seismic event.

In addition, the cask is restrained by two electric powered linkage systems that prevent side motions during a seismic event. Different cask diameters are handled by bolting unique interface clamps on the seismic restraints. When the restraint system is properly positioned next to the cask, two locking pins are pneumatically actuated to secure the position of the system. If the locking pins are not secured, the CTT will not be able to power up and move/levitate.

The facility compressed air supply inflates air casters beneath the trolley platform, which allow the CTT to rise above the steel floor. The platform mounted hose reel has an air-powered return, a ball valve shut-off, quick disconnect fittings, and a safety air fuse. A main "off/on" control valve and separate flow control/monitoring valves for each air bearing allow adjustment and verification of pressure/flow for each individual bearing. Interlocks for the air are provided to verify the main incoming pressure is not too high, and to verify that all bearings have sufficient air pressure.

End mounted turtle-style drive units that are 360-degrees steerable, are used to steer the CTT. Traction is produced by down-pressure on the wheels provided by a small air bag on each drive unit.

The CTT is evaluated for a collision with another object while carrying the cask. The speed of the drives, 10 feet per minute (ft/min), has been set so that the forces the cask experiences during a 10 ft/min collision is less than the forces the cask would experience during a seismic event. The speed is controlled in two ways. First, the electrical control system is designed to only give a proportional signal to the air valve that produces a speed of 0 to 10 ft/min. In the event this control system fails, a factory set mechanical throttle valve, in line with each motor drive, allows a maximum amount of air through at any time to prevent a "run-away" condition.

6.2.2.2.2 Operation

Initially, the CTT is located in the Cask Preparation Room with the battery fully charged, the seismic restraints retracted, and with no air or electrical power connected. Based on the next planned cask to be loaded onto the trolley, the corresponding pedestal components are installed into the base, and bumpers are bolted onto the seismic restraints and supports. The air hose is then connected to the CTT.

The overhead crane moves a cask onto the pedestal. With the cask still attached to the crane, the operator remotely operates the seismic restraints and secures the cask to the CTT. When the restraints are in place, the locking pins are remotely inserted pneumatically. With the cask secured to the CTT, the overhead crane is disengaged from the cask.

When the locking pins are inserted properly, an interlock allows the air bearings and drive motors to be operated. Once all preparations of the cask are complete, the CTT can be raised and moved to the Cask Unloading Room. Guides bolted to the floor insure that the CTT can only move forward and back, and will position the CTT so that the cask is directly below the transfer port. Once in position, the air pressure to the bearings is stopped and the CTT rests in position. The shield doors that separate the Cask Preparation Room from the Cask Unloading Room are then closed.

6.2.2.2.3 Control System

The control system is relay based and includes a pendant station as its operator interface.

No programmable logic controller (PLC) is used – all interlocks are hard wired. The pendant is a standard crane pendant that has all of the controls for the unit including:

- Deadman handle – operator must depress both handles to allow air to flow to the system so the CTT can levitate or move horizontally.
- Emergency-stop button on the pendant control and on the CTT.
- Clockwise/counterclockwise momentary switch to turn the drive units for horizontal movement. This rotational characteristic is used to move the CTT to storage or maintenance location after it leaves the Cask Preparation Room.
- Forward/reverse switch to determine direction of the drive units.
- Drive speed – variable speed control switch.
- Cask restraint – selector switch that actuates the motor to close the restraints and automatically engage the locking pin.

During normal operations, the controls operate off a battery system contained on the CTT. Only one operator is needed to drive the CTT since it only travels in one direction when it is carrying a cask.

The main air supply valve is a pilot operated solenoid valve that is fail safe (i.e., it is a spring valve that closes upon loss of electrical power or loss of air pressure). The air supply valve opens when the locking pins actuate the limit switches and the pendant deadman switches are actuated.

6.2.2.2.4 System/Pivotal Event Success Criteria

Success criteria for the CTT are the following:

- Ensure the CTT remains stationary with no spurious movement during transportation cask placement onto the CTT, transportation cask preparation, or during unloading.
- Prevent collisions while moving the CTT with cask from the Cask Preparation Room to the Cask Unloading Room.

Various design features are provided to achieve each of the success criteria. The failure to achieve each success criterion defines the top event of a fault tree for the CTT.

6.2.2.2.5 Mission time

In all cases a conservative mission time of one hour per cask transfer is used for each fault tree.

6.2.2.2.6 Fault Tree Results

The detailed analysis is presented in Attachment B, Section B2.

There are four fault trees associated with the CTT:

1. Spurious movement of the CTT in the Cask Preparation Room while loading a cask onto the CTT.

2. Spurious movement of the CTT in the Cask Preparation Room during unbolting and lid adapter installation.

3. Collision with an object or structure while moving a cask from the Cask Preparation Room to the Cask Unloading Room.

4. Spurious movement of the CTT in the Cask Unloading Room while unloading canisters from the CTT.

The results of the analysis are summarized in Table 6.2-2. Four fault trees were developed where the top events correspond to one of the scenarios listed above.

Table 6.2-2. Summary of Top Event Quantification for the CTT

Top Event	Mean Probability	Standard Deviation
Spurious movement of the CTT during cask loading	1.8E-9	4.0E-9
Spurious movement of the CTT during cask preparation	1.2E-4	1.2E-4
CTT collision into structure	9.8E-4	1.2E-3
Spurious movement during canister transfer	2.8E-14	1.1E-13

NOTE: CTT = cask transfer trolley.

Source: Attachment B, Section B2, Figures B2.4-1, B2.4-5, B2.4-8, B2.4-12

6.2.2.3 Shield Door and Slide Gate Fault Tree Analysis

The RF Cask Unloading Room and Loading Room each have a slide gate providing access to the Canister Transfer Room and a shield door providing access to either the Cask Preparation Room or the Lid Bolting Room. The shield doors and slide gates provide shielding during canister unloading and loading.

The FTA is detailed in Attachment B, Section B3. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification.

6.2.2.3.1 Physical Description

The Cask Unloading Room shield door is opened to allow cask-carrying equipment, such as the CTT, to enter the room. Once equipment is positioned properly in a Cask Unloading Room, the shield door may be shut in preparation for removing canisters from the cask. Once the shield door is shut, the slide gate may be opened, to allow the CTM to perform cask unloading operations. Similarly, the Loading Room shield door is opened to allow canister-carrying equipment, such as the site transporter, to enter the room. Once the site transporter is in place under the slide gate in the Loading Room, the shield door may be shut in preparation for loading the canister into an aging overpack. Once the shield door is shut, the slide gate may be opened, to allow the CTM to perform canister loading operations.

The shield doors consist of a pair of large heavy doors that close together. The doors are operated by individual motors that have over-torque sensors to prevent crushing of an object. Each door has two position sensors to indicate either a closed or open door and an obstruction sensor prevents the doors from closing on an object. The shield doors and slide gate are interlocked to prevent one another from opening if the other is open. The shield doors are opened and closed via a hand lever that must be enabled by an enable/disable switch. An emergency open switch exists, enabling the doors to be opened in case of an emergency situation.

Similar to the shield doors, the slide gates that separates the Cask Unloading and Loading Rooms from the CTM (located in the Canister Transfer Room above these rooms), consists of two gates that close together between the Cask Unloading/Loading Rooms and the Canister Transfer Room. The gates are operated by individual motors that also have over-torque sensors. Each gate has limit switches to indicate open or closed gates. A CTM skirt-in-place switch is interlocked to the slide gate to prevent the gates from opening without the CTM in place and a CTM in-place bypass hand switch exists for maintenance activities. Slide gate operation is controlled by a hand switch coupled with an enable/disable switch and shield door interlocks prevent the slide gate from opening when the shield door is open. Open/closed and CTM in-place indicators exist to assist operators in their activities.

6.2.2.3.2 Operation

The Cask Unloading Room shield door is opened to allow cask-carrying equipment, such as the SPM, to enter the room. Once equipment is positioned properly in the Cask Unloading Room, shield doors are shut in preparation for removing canisters from the cask. Once the shield doors are shut, the slide gate may be opened to allow the CTM to perform cask unloading operations. Loading of the aging overpack in the Loading Room is analogous to cask unloading operations. The slide gate may be opened to allow aging overpack loading access if the shield doors are closed. Once loading is complete and the slide gate is closed, the shield doors are opened to allow aging overpack removal.

6.2.2.3.3 Control System

The control systems have hard-wired interlocks for the following functions:

- Redundant hardwire interlocks prevent the shield door from opening while the slide gate is open.
- The shield door system will not have any test, maintenance, or other modes/settings that will allow bypass of interlocks.
- A single interlock prevents the slide gate from opening when the CTM skirt is not in place.

- An obstruction sensor is provided to detect objects between the shield doors and prevent door closure initiation.

- Motor over-torque sensors are provided to prevent shield doors from causing damage to casks in the event of closure on a conveyance.

- Shield doors and slide gates are equipped with redundant hardwire interlocks to prevent one another from opening when the other is open.

6.2.2.3.4 System/Pivotal Event Success Criteria

Success criteria for the shield door and slide gate are the following:

- Prevent inadvertent opening of shield door
- Prevent inadvertent opening of the slide gate
- Prevent concurrent opening of the shield door and slide gate when waste is present
- Prevent shield door closing on conveyance.

Various design features are provided to achieve each of the success criteria. The failure to achieve each success criterion defines the top event for a fault tree for the CTT.

6.2.2.3.5 Mission time

Most of the basic events in the fault tree models are "failure on demand" for equipment failures and "failure per operation" for HFEs. A mission time of one hour is used to calculate the probability of a spurious signal being sent due to PLC failure.

6.2.2.3.6 Fault Tree Results

The detailed analysis is presented in Attachment B, Section B3.

The slide gate and shield door system has three credible failure scenarios:

1. Inadvertent opening of the shield door

2. Inadvertent opening of the slide gate

3. Shield door closes on conveyance.

The results of the analysis are summarized in Table 6.2-3. Three fault trees were developed where the top events correspond to one of the scenarios listed above.

Table 6.2-3. Summary of Top Event Quantification for the Shield Doors and Slide Gate

Top Event	Mean Probability	Standard Deviation
Inadvertent Opening of the Shield Door	1.3E-7	2.1E-7
Inadvertent Opening of the Slide Gate	3.6E-9	9.8E-9
Shield Door Closes on Conveyance	1.9E-6	2.7E-6

Source: Attachment B, Section B3, Figures B3.4-1, B3.4-4, B3.4-7

6.2.2.4 Canister Transfer Machine Fault Tree Analysis

The FTA for the CTM is detailed in Attachment B, Section B4. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B4 for sources of information on the physical and operational characteristics of the CTM.

6.2.2.4.1 Physical Description and Functions

The CTM operates in the Canister Transfer Room of the RF. The function is to transfer waste canisters from a cask on a CTT to an aging overpack on a site transporter. The ports in the floor of the Canister Transfer Room provide access to the Cask Unloading Room and Loading Room and access to the canister staging areas.

The CTM is an overhead crane bridge with two trolleys. The first is a canister hoist trolley with a grapple attachment and hoisting capacity of 70 tons. The second is a shield bell trolley that supports the shield bell. The bottom end of the shield bell is attached to a larger chamber to accommodate cask lids. The CTM bottom plate assembly supports a thick motorized slide gate. The slide gate, when closed, provides bottom shielding of the canister once the canister is inside the shield bell. Around the perimeter of the bottom plate, a thick shield skirt is provided which can be raised and lowered to prevent lateral radiation shine during a canister transfer operation.

6.2.2.4.2 Operations

A typical CTM canister transfer operation is the transfer of a waste canister from a transportation cask to an aging overpack. For this operation, a loaded transportation cask, secured in the CTT, is positioned below the transfer port in the Cask Unloading Room. The cask lid is in place but unbolted. Similarly, an empty aging overpack secured by the site transporter is positioned under the adjacent transfer port in the Loading Room.

The CTM is moved to a position over the center of the port above the loaded cask. The shield skirt is lowered to rest on the floor, and the port slide gate is opened. The CTM slide gate is opened and the canister grapple is lowered through the shield bell to engage and lift the cask lid. The port slide gate is closed and the shield skirt is raised so the CTM can be moved to a cask lid staging area to set down the lid.

Once the lid is staged the CTM is moved back over the port above the loaded cask to align the canister grapple. The shield skirt is lowered, the port slide gate is opened, and the grapple is lowered to engage the canister lifting feature. The canister is raised into the shield bell. The

CTM slide gate and the port slide gate are closed and the shield skirt is raised so the CTM can be moved to the port above the empty aging overpack. The aging overpack loading operations are essentially the reverse of the cask unloading.

The CTM canister grapple is used for handling large diameter canisters such as TAD canisters and DPCs. These grapples are attached to the CTM canister grapple by positioning the CTM over a slide gate located in the Canister Transfer Room floor and lowering the CTM hoist until the CTM grapple is accessible in the room below.

The CTM is normally controlled from the facility operations room, but a local control station is also provided.

Generally, under off-normal conditions the CTM is not in operation. Following a LOSP, all power to the CTM motors (e.g., hoist, bridge, trolley, and bell trolley) is lost. If a transfer is underway when power is lost, all of the CTM motors stop and the hoist holding brake engages. Operations would be suspended until power is restored and the load can be safely moved. Under other off-normal conditions, transfer operations would be suspended and the CTM would remain idle.

6.2.2.4.3 Control System

Hard-wired interlocks are provided to:

- Prevent bridge and trolley movement when the shield bell skirt is lowered.
- Prevent raising the shield bell skirt when the slide gate is open.
- Prevent hoist movement unless the grapple is fully engaged or disengage.
- Stop the hoist and erase the lift command when a canister clears the shield bell slide gate.
- Stop a lift before upper lift heights are reached (two interlocks are provided for this function).
- Prevent opening of the port gate unless the shield bell skirt is lowered and in position.
- Prevent hoist movement unless the shield bell skirt is lowered.
- Prevent lifting of a load beyond the operational limit of the CTM (load cells).

Some of these interlocks can be bypassed during maintenance. The most significant of these interlocks that can be bypassed is the interlock between the shield skirt position and the position of the slide gate (The shield skirt cannot be raised unless the slide gate is closed or the maintenance bypass is engaged.). The design of the grapple interlock ensures that the bypass is voided when a canister is grappled.

Much of the operational controls are provided by non-ITS PLCs. Spurious or failed operation of the PLCs is in the FTA when such operation may contribute to a drop or collision event.

6.2.2.4.4 System/Pivotal Event Success Criteria

Success criteria for the CTM are the following:

- Prevent a canister drop from a height below the design basis height for canister damage from any cause during the lifting, lateral movement, and lowering portions of the canister transfer.
- Prevent a canister drop from above the canister design limit drop height from any cause during the lifting, lateral movement, and lowering portions of the canister transfer.
- Prevent a drop of any object onto the canister from any cause during the lifting, lateral movement, and lowering portions of the canister transfer.
- Prevent a collision between the canister and the shield bell or Canister Transfer Room floor from any cause during the lifting, lateral movement, and lowering portions of the canister transfer.
- Prevent CTM movement that could result in a shearing force being applied to the canister when the canister is being lifted and is between the first and second floors of the RF.

The failure to achieve each success criterion defines the top event for a fault tree for the CTM.

6.2.2.4.5 Mission Time

The mission time for the ITS CTM is set to one (1) hour.

6.2.2.4.6 Fault Tree Results

The analysis is detailed in Attachment B, Section B4.

There are four scenarios associated with the CTM that represent potential initiating events:

1. The CTM drops a canister from a height below the design basis height for canister damage (this includes canister drops within the shield bell once the bell slide gate has been closed and drops through the Canister Transfer Room ports to the loading/unloading areas that can occur before the bell slide gate is closed).
2. The CTM drops a canister from a height above the design basis height for canister damage.
3. The CTM drops an object onto a canister.

4. The CTM, while carrying a canister, moves in such a manner (spurious movements, exceeding bridge or trolley end of travel limits) as to cause an impact of the canister with the shield bell.

The results of the analysis are summarized in Table 6.2-4. Five fault trees were developed. The top events correspond to the four potential initiating events defined above.

Table 6.2-4. Summary of Top Event Quantification for the CTM

Top Event	Mean Probability	Standard Deviation
CTM drop all heights	1.4E-5	1.4E-5
CTM high drops from two blocking events	2.8E-8	1.4E-7
Drop of object onto cask	1.4E-5	1.2E-5
CTM collision	3.9E-6	2.7E-7
CTM shear	4.9E-9	9.6E-9

NOTE: CTM = canister transfer machine.

Source: Attachment B, Section B4, Figures B4.4-1, B4.4-16, B4.4-21, B4.4-35, and B4.4-41.

6.2.2.5 CASK TRACTOR AND CASK TRANSFER TRAILER FAULT TREE ANALYSIS

The FTA for the cask tractor and cask transfer trailer is detailed in Attachment B, Section B5. For the purposes of this analysis, the cask tractor and the cask transfer trailer are collectively called the HCTT. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B5 for sources of information on the physical and operational characteristics of the HCTT.

6.2.2.5.1 Physical Description and Functions

The HCTT consists of a tractor and a trailer. The tractor is a large, four-wheel drive diesel tractor designed specifically for pulling the cask transfer trailer. The tractor has redundant brakes in addition to having a fail-safe emergency brake. The trailer has independently mounted non-driven hydraulic pendular axles with a minimum of four tires per axle that will ensure the cask remains level during transportation across uneven terrain. In addition to the pendular axles, the trailer has three other hydraulic systems: (1) stabilizing jacks, (2) cask support skid and positioning system, and (3) hydraulic ram.

6.2.2.5.2 Operation

The casks involved in these operations are kept horizontal from unloading off the SPMRC to a cask stand and then to the HCTT for export to the Aging Facility. After the impact limiters have been removed from the transportation cask, the cask is lifted off the SPMRC using the sling lift and placed on the cask stand. Trunnions are installed on the cask. The cask is then lifted off of the cask stand using yoke fixtures on the crane. The cask is then placed on the HCTT and secured. The HCTT is then driven out of the RF.

6.2.2.5.3 Control System

Once the HCTT is properly positioned in the RF, the brakes on both the tractor and trailer are engaged. The brakes are spring applied with hydraulic release calipers. There is a backup system on the tractor consisting of a split master cylinder.

Stabilizing jacks provide vertical support during the loading and unloading of the cask on the HCTT.

6.2.2.5.4 System/Pivotal Event Success Criteria

Success criteria for the HCTT is the prevention of a collision with other vehicles, facility structures, or equipment.

Various design features are provided to achieve each of the success criteria. These include redundant braking systems in the tractor and parking brakes that fail safe. The failure to achieve each success criterion defines the top event for a fault tree for the HCTT.

6.2.2.5.5 Mission Times

A conservative mission time of one hour is used to account for the time it takes the HCTT, loaded with a transportation cask, to move from the Cask Preparation Room through the vestibule doors to outside the RF. Once outside, movement of the HCTT is addressed in the Intra-Site Operations analysis.

6.2.2.5.6 Fault Tree Results

The HCTT fault tree analysis is detailed in Attachment B, Section B5.

There is one fault tree associated with the HCTT that represents a potential initiating event: HCTT collision with other vehicles, RF facility structures, or equipment when loaded with a transportation cask.

The results of the analysis are summarized in Table 6.2-5.

Table 6.2-5. Summary of Top Event Quantification for the HCTT

Top Event	Mean Probability	Standard Deviation
HCTT Collision	4.9E-3	2.6E-2

NOTE: HCTT = cask tractor and cask transfer trailer.

Source: Attachment B, Section B5, Figure B5.4-1.

6.2.2.6 Site Transporter Fault Tree Analysis

The FTA for the site transporter is detailed in Attachment B, Section B6. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B6 for sources of information on the physical and operational characteristics of the site transporter.

6.2.2.6.1 Physical Description

The site transporter is a diesel/electric self-propelled tracked vehicle that is designed to transport a concrete and steel ventilated aging overpack. The transport occurs both within the Intra-Site and within the RF. The analysis described herein is limited to movement of the site transporter within the RF, which is limited to the Loading Room and the Lid Bolting Room.

The site transporter is a track driven vehicle with four synchronized tracks (two on each side). The components of the drive system (i.e., tumblers, idlers, rollers) are not included in this analysis since these components are not ITS. An integrated diesel powered electric generator provides the electricity to operate the site transporter outside the facility building. Inside the facility buildings the site transporter is electrically driven via an umbilical cable from the facility main electrical supply.

A rear fork assembly and a pair of support arms are used to lift and lower the cask. The rear forks are inserted in two rectangular slots near the base of aging overpack. Casks are carried in a vertical orientation with the lid at the top. Access to the top of the casks is unobstructed.

A passive restraint system provides stabilization during cask movement. These restraints are brought into contact with the cask after it has been raised to the desire height. A pin is inserted into each of the three restraint arms to keep the restraint in place should there be a failure of the electromechanical assembly. The pins also serve as an interlock that prevents movement of a loaded site transporter without the restraints being properly installed.

6.2.2.6.2 Control System

There are two modes of control provided on the site transporter. Operators can control every operation on the site transporter with either a remote (wireless) controller or through a pendant connected to the site transporter. All safety interlocks and controls of the site transporter are hard wired between the specific relays, drives, circuit breakers, and other electrical equipment. No PLC or computer is used to control the machine.

6.2.2.6.3 Normal Operations

The site transporter operator lines up the front opening of the site transporter to envelop the aging overpack and positions the rear fork down and in-line with the rectangular lifting slots near the bottom of the aging overpack and moves the site transporter forward until the aging overpack is centered in the interior of the site transporter.

The rear forks are raised to contact the bottom of the lift slots but do not attempt to lift the cask at this time. The operator and interlocks (torque and/or position) are incorporated to prevent lifting with the rear forms only.

The operator initiates the lift support arm's interface sequence with the rear forks and cask to prepare for lifting. After the operator and machine's switches have confirmed that the rear forks and lift support are properly aligned with one another, the lift sequence is initiated. The control system will sequence the lift motors so all screws operate together.

When the lift has been completed, the operator performs the final positioning of the upper restraint arms and inserts a pin in each arm. When the pins are properly installed, the site transporter can move.

The operator trails behind the site transporter during movement using the remote control to drive the site transporter to the desired location. At the facility, the operator stops the site transporter outside the Site Transporter Vestibule, turns off the diesel generator, and attaches an electric power cable.

Once inside the building, the operator positions the site transporter in the Loading Room. During the various movements inside the RF, the operator disengages the restraint arms for lower and lift operations at the various stations. Each time, the operator removes or replaces the pins from the restraint arms, as appropriate. The movement interlock is engaged when the pins are removed. For example, once inside the Loading Room, the pins will be inserted, the restraints will be engaged, the aging overpack raised from the floor, and the umbilical cord attached. At the completion of the loading, the site transporter is moved out of the Loading Room into the Lid Bolting Room for completing the lid bolting.

6.2.2.6.4 System/Pivotal Event Success Criteria

Success criteria for the site transporter are the following:

- Prevent a collision of the site transporter with objects, structures, or shield doors.
- Prevent runaway situations.
- Prevent site transporter movements in the wrong direction.
- Prevent a rollover of the site transporter.
- Prevent spurious site transporter movements.
- Prevent a load drop during lift/lower or transport operations.

Various design features are provided to achieve each of the success criteria. The failure to achieve each success criterion defines the top event for a fault tree for the site transporter.

6.2.2.6.5 Mission Time

For quantification of the site transporter fault trees in Attachment B, Section B6, a mission time of one hour per cask transfer is used.

6.2.2.6.6 Fault Tree Results

There are four basic site transporter fault trees developed for the RF. The scenarios represented and the variations by these fault trees are the following:

1. Site transporter collides with RF structures:

   A. Importing aging overpack to Loading Room.
   B. Transfer from Loading Room to Lid Bolting Room.
   C. Exporting aging overpack from Lid Bolting Room.

2. Site transporter load drop during lift/lower.

3. Site transporter tipover.

4. Site transporter spurious movement.

The results of the analysis are summarized in Table 6.2-6 for the seven fault trees.

Table 6.2-6. Summary of Top Event Quantification for the Site Transporter

Top Event	Mean Probability	Standard Deviation
ST collision in RF	4.6E-3	1.4E-2
ST load drop during lift/lower	3.8E-8	8.9E-8
ST rollover	2.3E-6	1.9E-6
ST spurious movement	2.0E-13	4.4E-13

NOTE: RF = Receipt Facility; ST = site transporter.

Source: Attachment B, Section B6, Figure B6.4-1, B6.4-6, B6.4-20, B6.4-23

6.2.2.7 HVAC FAULT TREE ANALYSIS

The FTA for the HVAC is detailed in Attachment B, Section B7. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B7 for sources of information on the physical and operational characteristics of the HVAC system.

6.2.2.7.1 HVAC Description and Function

The ITS HVAC is a two (2) train system of identical components. One train is always operational and one train is in standby mode. This system is not configured to run both trains at the same time without bypassing control circuitry. This off-normal situation is not addressed in this analysis.

In the RF, the Train A HVAC equipment is located on the opposite end of the building from Train B HVAC equipment. Each HVAC train exhausts air through separate discharge ducts into the atmosphere. Although these trains are interconnected through interior duct work, the trains are independent. A back-draft damper is used on each train to ensure there is no airflow from the atmosphere back through the standby train.

This HVAC system is composed of four subsystems:

1. A series of dampers are used to control pressure, flow, as well as flow direction in the system.

2. Three HEPA filters, each consisting of one medium efficiency roughing filter (60-90% efficiency), two high efficiency filters for particulate removal in air (99.97% efficiency), and a mister/demister for maintaining proper humidity levels.

3. One exhaust fan with a rated capacity of 40,500 cfm and an exhaust fan motor rated at 200 hp.

4. Control circuitry with logic contained in an erasable programmable read-only memory located in the adjustable speed drive (ASD) controller used for controlling the speed of the operating fan and on fault detection, and for off-nominal conditions, shutting down the operating train and transmitting signals to the standby system to start.

6.2.2.7.2 Success Criteria

One success criterion is defined for the each of independent Trains, A and B, for providing the HVAC confinement function: maintain negative differential pressure in the RF for the specified mission time.

The respective trains of the ITS portions of the HVAC are identical. Various design features are provided to achieve each of the success criteria for the respective trains and for the combined system.

The FTA for the HVAC includes separate analyses for the respective trains. The failure to achieve the success criterion defines the top event for the fault tree for each train of the HVAC.

6.2.2.7.3 Mission Time

The mission time for the HVAC system is 720 hours (Attachment B, Section B7). However, the mission time for the backup system has been taken as half of the active system (i.e., 360 hours). This is to account for the difference in failure rates between active and passive systems.

6.2.2.7.4 Fault Tree Results

The top event in this fault tree is "Delta pressure not maintained in RF." This is defined as the inability of the ITS HVAC system to maintain proper delta pressure within the facility. The system failure probability and standard deviation, including failure of electrical power are as follows:

- The mean HVAC system probability of failure, including loss of electrical power is 3.8E-02

- The standard deviation is 9.4E-02.

These results are presented in Attachment B, Section B7, Figure B7.4-1

6.2.2.8 AC Power Fault Tree Analysis

The FTA for the AC power system is detailed in Attachment B, Section B8. The following is a summary of the design, operations, success criteria, and results of the fault tree quantification. See Attachment B, Section B8 for sources of information on the physical and operational characteristics of the AC power system.

6.2.2.8.1 System Description

The ITS AC power system supplies power to the ITS systems (for example, the HVAC systems). The ITS power system consists of two elements; those used during normal operations and those used during off-normal conditions. During normal operations AC power is supplied from one of two offsite 138kV offsite power lines through the 138kV to 13.8kV switchyard and then through the plant AC power distribution system to the various facilities throughout the site. Off-normal conditions for the distribution of AC power occur during a LOSP.

A LOSP may be the result of problems on the power grid, or may be the result of failures within the plant AC power systems. Under these conditions, the AC power source for the RF ITS equipment is two onsite ITS diesel generators. Power is supplied to ITS loads via the same onsite AC power distribution system that is used during normal operation. Each ITS diesel generator supplies power to one Train (A or B) of ITS systems. Each diesel generator, its associate support systems, and the power distribution system are independent and electrically isolated from the other ITS diesel generator, its support systems, and power distribution system.

The ITS loads within the RF are powered via two ITS 480V load centers and two ITS 480V motor control centers (MCC) located within separate areas of the RF. Each division of the AC power supply from the diesel generator switchgears to the RF passes through a 13.8kV to 480V transformer.

The ITS onsite power portion of the ITS power supply system is intended to provide back-up power to selected buildings and operations in the event of a main transmission power loss (a LOSP). The primary components in each division include an ITS diesel generator, support systems for the diesel generator, and a load sequencer. Both ITS diesel generators are located in the Emergency Diesel Generator Facility (EDGF). Each is sized to provide sufficient 13.8kV power to support all ITS loads of one division in six facilities (i.e., three CRCFs, the WHF, the RF, and the EDGF).

The ITS diesel generator starts upon detection of an undervoltage condition via an undervoltage relay of the 13.8kV ITS switchgear. Each ITS diesel generator is equipped with a complete independent set of support systems including HVAC systems, uninterruptible and DC power systems, a fuel oil system, diesel generator start subsystem, diesel generator cooling subsystem, and lube oil subsystem.

The load sequencer controls sequence of events that occur after a LOSP and the ITS diesel generator start. Upon a LOSP the load sequencer opens the RF ITS load center feed breaker. After the diesel generator starts and reaches rated capacity, the load sequence connects the ITS diesel generator to the 13.8kV ITS switchgear and then reconnects the RF loads.

6.2.2.8.2 Operations

Under normal operating conditions, AC power is supplied from two 138kV offsite power lines. Power is passed through the 138kV to 13.8kV switchyard to the two independent 13.8kV ITS switchgear. From here, power is transmitted via separate lines to a 13.8kV to 480V transformer supporting Trains A and B of the RF. Power to individual ITS components within each facility is provided via 480V load centers and MCCs (one of each for Train A and one of each for Train B in each facility) powered through these transformers.

During a LOSP, both ITS diesel generators are required to start and accept loads in a timely manner. Upon a LOSP, the onsite power distribution system supporting ITS loads is disconnected from the switchyard; a circuit breaker between the 13.8kV ITS switchgear and the switchyard 13.8kV switchgear in each train automatically opens. Both ITS diesel generators start automatically and are connected to the 13.8kV ITS switchgear when the connecting breaker is closed by the load sequencer. The load sequencer then reconnects the RF loads to the 13.8kV ITS switchgear. Both diesel generators continue to supply AC power until normal power is restored.

Environmental systems are provided to maintain the temperature in the various EDGF rooms and RF ITS electrical rooms within acceptable levels.

6.2.2.8.3 Control System

The ITS diesel generator starts upon detection of an undervoltage condition via an undervoltage relay of the 13.8kV ITS switchgear. The 13.8kV ITS switchgears are isolated from the main switchyard upon a loss of power in the switchyard. The loads in the RF are shed upon a loss of power indication.

A load sequencer controls the loading of the ITS diesel generator onto the 13.8kV ITS switchgear upon the ITS diesel generator reaching rated output. The same load sequencer controls reloading the RF loads onto the AC power system.

6.2.2.8.4 System/Pivotal Event Success Criteria

Success criterion for the AC power system is defined in terms of its support function for the ITS HVAC confinement function. The AC power system must operate in support of the HVAC system for as long as necessary to successfully provide confinement after the potential release of radioactive material inside the RF. There are two independent trains of HVAC and each of these must be supported by an independent AC power system. Therefore, the following success criteria apply to the respective AC power supply trains:

- Provide AC power from either the normal offsite power lines or from the ITS diesel generator (DG A) to the HVAC train powered through RF ITS Load Center A and ITS MCC A1 for the mission time of 720 hours.

- Provide AC power from either the normal offsite power lines or from the ITS diesel generator (DG B) to the HVAC train powered through RF ITS Load Center B and ITS MCC B1 for the mission time of 720 hours.

The respective trains of the ITS portions of the AC power system are essentially identical. Various design features are provided to achieve each of the success criteria for the respective trains.

The FTA for the AC power system includes separate analyses for the respective trains. The failure to achieve the success criterion defines the top event for the fault tree for each train of the AC power system.

6.2.2.8.5 Mission Time

The mission time for the ITS AC power system is the same as for the HVAC system, 720 hours.

6.2.2.8.6 Fault Tree Results

Two fault trees are developed for the AC power system, one for Train A and one for Train B. The respective top events are:

- "Loss of AC power at ITS Load Center A for the RF," defined as a failure of the normal and ITS on-site power supplies to provide power to ITS Load Center A1.
- "Loss of AC power at ITS Load Center B for the RF," defined as a failure of the normal and ITS on-site power supplies to provide power to ITS Load Center B.

The results are essentially the same for either train:

- The mean probability of failure or either train value is 3.2E-02
- The standard deviation is 7.8E-02.

These results are presented in Attachment B, Section B8, Figures B8.4-1 and B8.4-3.

6.2.2.9 Potential Moderator Sources

6.2.2.9.1 Internal Floods

Internal floods are potential sources of moderator addition into a canister associated with pivotal events in the event sequences included in Section 6.1. Moderator addition into a canister can occur following a breach of the canister and a subsequent internal flood. The internal flooding analysis considers all waste handling facilities.

During most of its handling at the repository, a canister is surrounded by at least one other barrier to water intrusion: a transportation cask, a transportation cask within a CTT, an aging overpack, a waste package, a waste package within a WPTT, or a waste package within a TEV.

Each facility is equipped with a normally dry, double-preaction sprinkler system in areas where waste forms are handled ((Ref. 2.2.16), (Ref. 2.2.29), (Ref. 2.2.23), and (Ref. 2.2.36)). Such systems, which require both actuation of smoke and flame detectors to allow the preaction valve to open and heat actuation of a fusible link sprinkler head to initiate suppression, have a very low frequency of spurious operation. A 30-day period from the occurrence of the canister breach to the time definitive action can be taken to prevent introduction of water into the canister is reasonable and is the same as the period used to assess dose for a radiological release. The spurious actuation frequency over a 30 day mission time after a breach is calculated below.

An estimate of the probability of spurious actuation is developed using a simplified screening model that addressed the following cut sets that result in actuation:

- Spurious preaction valve opens before canister breach × failure of a sprinkler head during post-breach mission time (30 days).

- Failure of a sprinkler head during building evacuation × water left in dry piping after last test (1st quarter following annual test).

The frequency of sprinkler failure is estimated using an individual sprinkler head failure frequency of 1.6E-6/yr (Ref. 2.2.13, Table 1), the estimated number of sprinklers (1 per 130 ft^2 based on NFPA 13 (Ref. 2.2.59, Table 8.6.2.2.1(b)) and the applicable area (Ref. 2.2.20). For example, the area of CRCF Waste Package Loadout Room 1015 is listed as 7,470 ft^2 (Ref. 2.2.20). At 130 ft^2/sprinkler, 58 sprinklers are estimated. The failure of any sprinkler in the room is then estimated to be 58 × 1.6E-6/yr × 1/8760 hrs/yr, or 1.1E-8/hr.

The frequency of preaction valve spurious open is estimated using the solenoid valve spurious open data in Section 6.3 of 8.1E-07/hr. This is reasonable because a solenoid valve must open to relieve the air pressure from the diaphragm which keeps the valve closed.

The value of the first cut set is (1.6E-6/yr × 1/8760 hr/yr × 720 h) × (8.1E-7/hr × 720 h) = 8E-11/sprinkler head. The second cut set is more significant: 0.025 (human error screening value) × (1.6E-6/yr × 1/8760 hr/yr × 720 h) = 3E-9/sprinkler head.

Applying the sum of these values, 3E-9/sprinkler head, to the number of sprinklers calculated for the waste handling areas of the four facilities results in the following estimates of the probability of spurious sprinkler actuation found in Table 6.2-7.

Table 6.2-7. Probability of Spurious Sprinkler Actuation

Facility	Waste Handling Area (ft2) [a]	Number of Sprinkler Heads	Probability of Spurious Actuation in 30 day Period in Waste Handling Areas
CRCF(ea)	42,000	330	1E-6
IHF	30,000	240	9E-7
RF	19,000	150	5E-7
WHF	28,000	215	6E-7

NOTE: [a] CRCF area based on room numbers 1005E, 1016-1026, 2004,2007, 2007A, and 2007B; IHF area based on room numbers 1001-1003, 1006-1008, 1011,1012, 1026, and 2004; RF area based on room numbers 1013, 1015, 1016, 1017, 1017A, and 2007; WHF area based on room numbers 1007-1010, 1016, 2004, 2006, and 2008. CRCF = Canister Receipt and Closure Facility, IHF = Initial Handling Facility, RF = Receipt Facility, WHF = Wet Handling Facility.

Source: Original

Piping carrying water is present in the waste form handling areas of the CRCF, IHF and WHF. Piping lengths in these areas of the CRCF and WHF are below 100 feet per facility. For the IHF, approximately 6,800 feet of piping runs no closer than 60 feet of the cask unbolting area (Ref. 2.2.83). Even the length of piping in the IHF has little impact post-breach, as the probability of a pipe crack or rupture in a 30 day period following a potential breach is less than 2.0E-3. There is no wet piping in the waste form handling areas of the RF (Ref. 2.2.83).

The probability of a pipe crack in a 30 day period was estimated using the pipe leak data from NUREG/CR-6928 (Ref. 2.2.43, Table 5-1). Piping leaks and large break rates applicable to non-service water applications are used in the analysis. These values are considered appropriate for repository systems because of the conditioning applied to the fluids in the systems will be that typical of the commercial nuclear power plant:

External leak small (1 to 50 gpm): Leak rate = 2.5E-10 $hr^{-1}ft^{-1}$

External leak large (> 50 gpm): Leak rate = 2.5E-11 $hr^{-1}ft^{-1}$

Multiplying the sum of the small and large crack frequencies (2.8E-10 $hr^{-1}ft^{-1}$) by the length of piping in the waste handling areas of each facility, and the number of hours in a 30 day period (720 hr), a conditional probability of water leakage in all waste handling areas given a breach is approximated as follows:

CRCF = 2.8E-10 $hr^{-1}ft^{-1} \times$ 100 ft $\times$ 720 h = 2.0E-05

IHF < 2.8E-10 $hr^{-1}ft^{-1} \times$ 6,800 ft $\times$ 720 h = 1.4E-03

WHF = 2.8E-10 $hr^{-1}ft^{-1} \times$ 75 ft $\times$ 720 h = 1.5E-05

RF = 2.8E-10 $hr^{-1}ft^{-1} \times$ 0 ft $\times$ 720 h = 0.

It is appropriate to use the waste handling area piping lengths because they are separated by concrete walls from the non-waste handling areas of buildings.

The above applies to event sequences that do not involve fires as an initiating event. During fire initiating event sequences, fire suppression would actuate in the locations sufficiently heated by the fire. The fire initiating event analysis is described in Section 6.5, and the conditional probability of canister failure owing to fires is described in Section 6.3. The analysis is performed without the salutary effects of fire suppression in order to demonstrate large margins of safety during fire event sequences. Furthermore, the location of each fire is analyzed as around the outer shell of the overpack that surrounds the canister, which neither accounts for the CTT or WPTT enclosures that surround the overpack nor the elevated position of the canisters with respect to a fire on the floor. The frequency of containment breach due to fire is significantly overestimated because of this conservative approach.

For fires that occur in locations that contain canisters sealed within bolted transportation casks, the fire location will be floor level and the transportation casks rise as much as 20 feet above the floor. Casks are relatively thick walled compared to canisters and sustain a relatively small internal pressurization when compared to canisters. Therefore, if a fire is large enough, it will fail the internal canister first, as indicated in Attachment D. This will cause the bolted and sealed cask to bear the overpressure that is inside the canister. The cask bolts might act as elastic springs allowing the top to break the seal and relieve the internal pressure. This would be a mechanism that prevents cask breach. However, a hot fire may result in sufficient loss of strength of the bottom portion of the stainless steel cask such that it breaches. If failure occurs because of bolt stretching the cask lid remains on top of the cask preventing fire suppression water from entering. Commercial DPCs and TAD canisters will require at least 100 liters of water to enter the canister if optimally distributed among the fuel rods (Ref. 2.2.33). Casks are raised above the floor. They lay on top of railcars, are lifted from there by cranes, sit inside a CTT, or lay sideways on a pallet. They are at least five feet from the floor. If the bottom portion of the canister breaches, there is no physical mechanism for this much water to enter the cask and then the canister, remain as water (not boil off), and optimally mix with the fuel rods.

This latter situation also applies to canisters sealed within a welded waste package. The waste package sits inside a WPTT or is inside a TEV. In the former case it is more than three feet from the floor (Ref. 2.2.17) and in the latter case about one foot from the floor (Ref. 2.2.18). In the latter case, however, the TEV offers an additional layer of protection against fires. In addition, it is physically unrealistic for a sufficient amount of available fire suppression water to cause 100 liters to leak into a breached canister, but not extinguish the fire or at least reduce the severity of the fire such that a breach would not occur.

For a canister inside of an open transportation cask or waste package, the orientation of these is always vertical, and the cask and waste package are always elevated above the floor where the fire occurs. The occurrence of a fire of sufficient severity will fail the canister first as described above. An open transportation cask or waste package might allow fire suppression water to spray in from the top. The building configuration, however, precludes this occurrence. The cask lids are removed while in the upload cell below the CTM. The cask and waste package ports are above the casks and waste package. There is no fire suppression piping spanning the ports because the ports must be kept clear in order to perform lift and load operations. In the Waste Package Positioning Room and welding area, the lid is on the waste package and fire suppression piping can not be above an open waste package because of the welding machine. In the cutting cell in which a cask is open (WHF only), there can be no fire suppression piping above an open cask because of the cutting equipment.

Upon failure of the canister inside the cask, the cask will not be susceptible to pressurization failures as above. Instead, water can only enter in a cask (or waste package) if the cask body melts through. Fires capable of melting stainless steel or Alloy 22, however, have an occurrence frequency within the waste handling facilities of less than 1E-05 over the preclosure period (Attachment D). Thus, breach of the cask or waste package in a manner that would allow water to enter the canister is essentially not physically realizable.

When a canister is being lifted, transferred inside the shield bell, and lowered, it is not inside an outer cask. However, fires can not be severe enough to breach a canister while being moved, as described in more detail in Attachment D. Water intrusion, therefore, is not physically realizable for this situation.

It is concluded that moderator entry into breached canisters during fire event sequences is not physically realizable because of a combination of physical mechanisms, building and equipment configuration, and overpack material properties. Furthermore, the existence of water from fire suppression is inconsistent with the fire analyses performed to obtain the probability of containment failure owing to fire. If fire suppression were indeed available, the probabilities of canister breach would be far lower. However, in order to complete an event sequence quantification, the conditional probability of moderator entry into a canister after canister breach during a fire initiating event sequence is assessed as *extremely unlikely* and assigned a lognormal distribution with a median of 0.001 and an error factor of 10. This yields a mean value of 3E-03. The large error factor is assigned because of the potential of human error to defeat some of the reasons that water will not enter the cask or waste package (e.g., neglecting to place a lid on the waste package just before a severe fire). These assignments are consistent with the methodology on the use of judgment provided in Section 4.3.10.

6.2.2.9.2 Lubricating Fluid

Another source of moderation is lubricating fluid in cranes. Crane lube oil is of limited quantity (<150 gallons) and housed in a welded gear box with a leak pan below it capable of capturing the entire gearbox fluid inventory. An estimate of the leakage rate through the gear box and drip pan is found by multiplying the gear case motor failure frequency (all modes) of 0.88E-06 per hour (Ref. 2.2.38, p. 2-104 and Section 6.3) by 0.5, over the 50 years by the conditional probability of oil pan failure. A loss of lubrication would fail the crane operation and also be detected by oil pressure indicators. The conditional probability of oil pan failure may be estimated by analogy to receiver tank leakage during the interval between gearbox failure and detection. The interval is conservatively estimated to be 30 days. The all modes failure rate of a receiver tank is 0.34 E-06 per hour (Ref. 2.2.38, p. 2-213). Using an exposure interval of 50 years (which represents the operating life of the surface facilities), the conditional probability of lubricating fluid entering a breached canister would be less than:

> 0.88E-06/hr × 50 yrs × 8,760 hr/yr × 0.34E-06/hr × 720 hr/30days = 9.4E-05/ over the preclosure period.

This probability is overstated because, (1) it does not account for inspections during the operating period of the facility, and (2) it does not account for the conditional probability that lubricating fluid can find its way into a breached canister. Therefore, lubricating fluid is eliminated as a potential moderator.

6.3 DATA UTILIZATION

6.3.1 Active Component Reliability Data

The fault tree models described in Section 6.2 include random failures of active mechanical equipment as basic events. In order to numerically solve these models, estimates of the likelihood of failure of these equipment basic events are needed. The active component reliability estimates are developed by gathering and reviewing industry-wide data, and applying Bayesian combinatorial methods to develop mean values and uncertainty bounds that best represented the range of the industry-wide information.

6.3.1.1 Industry-wide Reliability Data for Active Components

While data from the facility being studied are the preferred source of equipment failure rate information, it is common in a safety analysis for information from other facilities in the same industry to be used when facility-specific data is sparse or unavailable. Because the YMP is a one-of-kind facility and has no operating history, it is necessary to develop the required data from the experience of other nuclear and non-nuclear equipment operations. Industry-wide data sources are documents containing industrial or military experience on component performance. These sources are from previous safety/risk analyses and reliability studies performed nationally or internationally and also standards or published handbooks. For the YMP PCSA, a database is constructed using a library of industry-wide data sources of reliability data from nuclear power plants, equipment used by the military, chemical processing plants, and other facilities. The sources used are listed in Attachment C, Section C1.2.

The data source scope has to be sufficiently broad to cover a reasonable number of the equipment types modeled, yet with enough depth to ensure that the subject matter is appropriately addressed. For example, a separate source might be used for electronics data versus mechanical data, so long as the detail and the applicability of the information provided justify its use. Lastly, the quality of the data source is considered to be a measure of the source's credibility. Higher quality data sources are based on equipment failures documented by a facility's maintenance records. Lower quality sources use either abbreviated accounts of the failure event and resulting repair activity, or do not allow the user to trace back to actual failure events. Every effort is made in this analysis to use the highest quality data source available for each active component type and failure mode.

A potential disadvantage of using industry-wide data is that a source may provide failure rates that are not realistic because the industry-wide source environment, either physical or operational, may not correlate to the facility modeled. Part of the PCSA active component reliability analysis effort, therefore, is to evaluate the similarity between the YMP operating environment and that represented in each data source to ensure data appropriateness. This evaluation process is described in Section C1.2.

Given the fact that the YMP will be a relatively unique facility (although portions will be similar to the spent fuel handling and storage areas of commercial nuclear plants), the data development perspective is to collect as much relevant failure estimate information as possible to cover the spectrum of equipment operational experience. It is reasonable to expect that the YMP equipment would fall within this spectrum (Section 3.2.1). The scope of the sources selected for this data set is therefore deliberately broad to take advantage of the combined experience of many facilities, not a single plant. It is then intended to provide a combined estimate that reflects as best as possible the uncertainty ranges of the individual estimates. This ensures that the data are not skewed towards the possibly atypical behavior of one particular plant, industry or operating environment. The combinatorial process, utilizing Bayes' Theorem, is discussed in the following subsection.

Among the active components whose reliability is quantified with industry-wide data are the 200-ton cranes, jib cranes, canister maneuvering cranes, and the spent fuel transfer machine (SFTM). The SFTM is not used in the RF; however it is being discussed in this section for completeness. The rationale for using such data for these estimates is that a significant amount of crane experience exists within the commercial nuclear power industry and other applications, and that this experience can be used to bound the anticipated crane performance at YMP. Furthermore, the repository is expected to have training for crane operators and maintenance programs similar to those of nuclear power plants. Crane and SFTM handling incidents that result in a drop are included in the drop probability regardless of cause; they may be caused by equipment failures (including failures in the yokes and grapples), human error, or some combination of the two.

Every attempt was made to find more than one data source for each component type and failure mode combination (TYP-FM), although multiple sources are not always available for a specific piece of equipment. When data was extracted from several sources, it was combined using Bayesian estimation (as described further below), and compared by plotting the individual and combined distributions. However, the comparison process often resulted in one source being selected as most representative of the TYP-FM. Ultimately, 53% of the TYP-FMs were quantified with one data source, 8% with two data sources, 8% with three data sources and 31% with four or more data sources.

6.3.1.2 Application of Bayes' Theorem to PCSA Database

The application of industry-wide data sources introduces uncertainty in the input parameters used in basic events and, ultimately, the quantification of probabilities of event sequences. Uncertainty is a probabilistic concept that is inversely proportional to the amount of knowledge, with less knowledge implying more uncertainty. Bayes' theorem is a common method of mathematically expressing a decrease in uncertainty gained by an increase in knowledge (for example, knowledge about failure frequency gained by in-field experience).

There are several approaches for applying Bayes' theorem to data management and combining data sources, as described in NUREG/CR-6823 (Ref. 2.2.11). For the PCSA, the method known as "parametric empirical Bayes" is primarily used. This permits a variety of different sources to be statistically combined and compared, whether the inputs are expressed as the number of failures and exposure time or demands, or as means and lognormal error factors.

A typical application of Bayes' theorem is illustrated as follows. A failure rate for a given component is needed for a fault tree, e.g., a fan motor in the HVAC system. There is no absolute value for the failure rate, but there are several data sources for the same kind of fan and/or similar fans that may exhibit considerable variability for many reasons. Applying any or all of the available data to the YMP introduces uncertainty in the analysis of the reliability of the HVAC system. Bayes' theorem provides a mechanism for systematically treating the uncertainty and applying available data sources using the following steps:

1. Initially, estimate the failure rate to be within some range with a probability distribution. This is termed the "prior" probability of having a certain value of the failure rate that expresses the state of knowledge before any new information is applied.

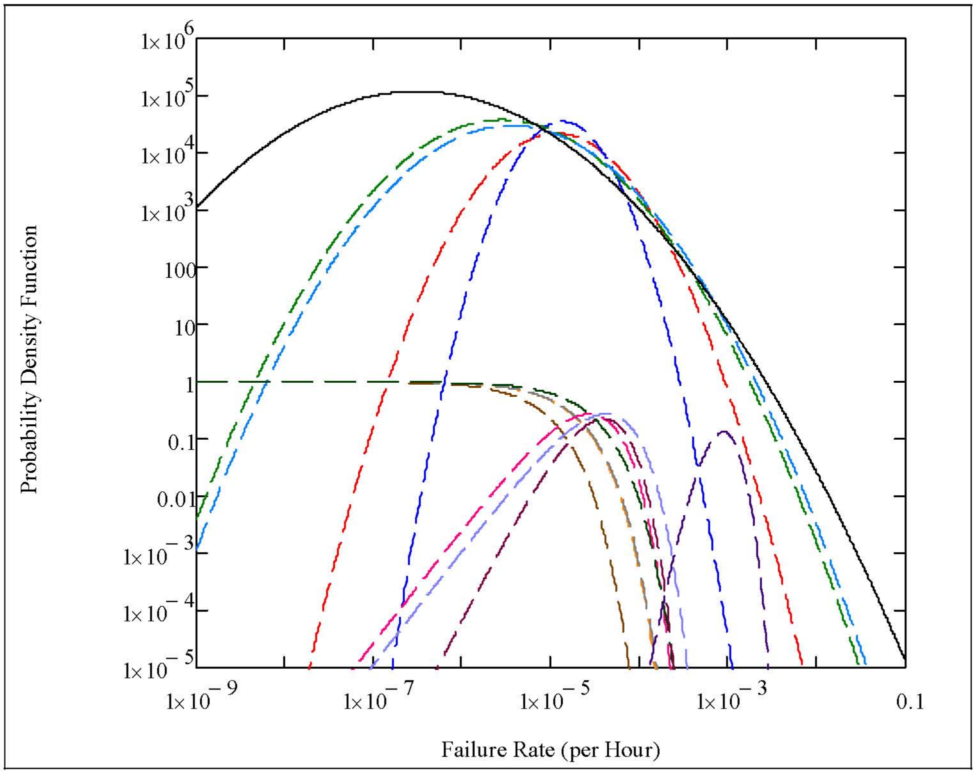
2. Characterize the test information, or evidence, in the form of a likelihood function that expresses the probability of observing the number of failures in the given number of trials if the failure rate is a certain value. The evidence comprises observations or test results on the number of failure events that occur over a certain exposure, operational, or test duration.

3. Update the probability distribution for the failure rate based on the new body of evidence.

The likelihood function is defined by the analyst in accordance with the kind of evidence. For time-based failure data, a Poisson model is used for the likelihood function. For demand-based failure data, a binomial model is used. The mathematical expression for applying Bayes' theorem to data analysis is described in Attachment C, Section C2.

For the analysis presented herein, MathCAD is used to calculate the population-variability (prior) distributions of active components. As described in Attachment C, Section C2.1, the method of "The Combined Use of Data and Expert Estimates in Population Variability Analysis" (Ref. 2.2.53, pp. 311–321) is used as the basis example for the combinations performed. In this method, the population-variability distribution of the failure rate is approximated by a lognormal distribution whose unknown parameters, v and τ, respectively the mean and standard deviation of the associated normal distribution, are determined. Calculating v and τ involves calculating the likelihood function associated with the reliability information in each data source. For a data source providing a failure rate point estimate, the likelihood function is a lognormal distribution, function of the failure rate x, and characterized by its median value and associated error factor. For a data source providing exposure data (given in the form of a number n of recorded failures over an exposure time t), the likelihood function is a Poisson distribution, expressing the probability that n failures are observed when the expected number of failures is x times t.

The maximum likelihood method is used to calculate v and τ. This involves maximizing the likelihood function for the entire set of data sources. This likelihood function is the product of the individual likelihood function for each data source because the data sources are independent from each other. It is equivalent and computationally convenient to find the maximum likelihood estimators for v and τ by using the sum of the log-likelihood (logarithm of the likelihood) of each data source. As a result, the likelihood functions from the individual data sources and a population-variability probability density function for the combination are produced and plotted for comparison, as in the example shown as Figure 6.3-1.



Source: Attachment C, Figure C2.1-1

Figure 6.3-1. Likelihood Functions from Data Sources (Dashed Lines) and Population-Variability Probability Density Function (Solid Line)

If only a single data source is considered applicable to a given TYP-FM combination and if the data source provides a mean and an error factor for the component reliability parameter, the probability distribution is modeled in SAPHIRE as a lognormal distribution with that mean and that error factor. However, if the data source does not readily provide a probability distribution, but instead exposure data, (i.e., a number of recorded failures over an exposure time for failure rates or over a number of demands for failure probabilities), the probability distribution for the reliability parameter is developed through a Bayesian update using Jeffrey's noninformative prior distribution (i.e., gamma for time-related failure modes and beta for demand based failure modes).

Example implementations of the methods used for these cases are provided in Attachment C.

6.3.1.3 Common-Cause Failure Data

Dependent failures are modeled in event tree and fault tree logic models. When possible, potential dependent failures are modeled explicitly via the logic models. For example, failure of the HVAC system is explicitly dependent upon failure in the electrical supply system that is modeled in the fault trees. Similarly, the effects of erroneous calibration or other human failure events can be explicitly included in the system fault tree models and the basic event probabilities considered during the HRA. Otherwise, potential dependencies known as CCFs are included in fault tree logic, but their probabilities are quantified by an implicit, parametric method. Therefore, another subtask of the active component reliability data analysis is to estimate common-cause failure probabilities.

Surveys of failure events in the nuclear industry have led to several parameter models. Of these, three are most commonly used: the Beta Factor method (Ref. 2.2.48), the Multiple Greek Letter method (Ref. 2.2.57), and the alpha factor method (Ref. 2.2.58). In a parametric model, the probability of two or more components failing by a CCF is estimated by use of the equations provided in Section 4.3.3.3.

For the PCSA, common-cause failure rates or probabilities are estimated using the alpha factor method (Ref. 2.2.58) because it is a method that includes a self-consistent means for development of uncertainities.

The data analysis reported in NUREG/CR-5485 (Ref. 2.2.58) consisted of:

1. Identifying the number of redundant components in each subsystem being reported, (e.g., two, three, or four (termed the CCF group size)).

2. Partitioning the total number of reported failure events for a given component into the number of components that failed together, (i.e., one component at a time, two components at a time, and so on up to failure of all components in a given CCF group).

3. Calculating the alpha factor for a given component type to provide a basis for estimating the probability of CCFs involving two, three, etc., or all components (see equation in Attachment C, Section C3).

4. Performing statistical analysis and curve fitting to define the mean and uncertainty range for alpha factors for various CCF group sizes up to eight.

The data analysis also produces prior distributions for the alpha factors. The results are the mean alpha factors and uncertainty bounds, reported in NUREG/CR-5485 (Ref. 2.2.58, Table 5-11) and reproduced in Attachment C, Table C3-1.

These alpha-factors values are used for failure-on-demand events (e.g., pump failure to start) and by using the alpha factor divided by two for failure-to-operate events (e.g., pump fails to run). For example, for a two-out-of-two failure on demand event, the mean alpha factor of 0.047 (shown in the far right column of Table C3-1 associated with α_2) was multiplied by the mean failure probability for the appropriate component type and failure mode (from Table C4-1) to yield the common-cause failure probability.

6.3.1.4 Input To SAPHIRE Models

Since the primary active component reliability data task objective is to support the quantification of fault tree models developed in SAPHIRE by the system analysts, the output data has to conform to the format appropriate for input to the SAPHIRE code.

SAPHIRE provides template data to the fault tree models in the form of three input comma delimited files:

- .BEA – attributes to assign information to the proper SAPHIRE fields
- .BED – descriptions of the component type name and failure mode
- .BEI – information on the failure rate or probability estimates and distributions used.

Demonstration files for the .BEA, .BED and .BEI template data files provided with SAPHIRE were originally used to construct the PCSA template data files to ensure the proper formatting of the data for use by the fault tree models. In general, the .BEA file provides attribute designators for the code to implement such that the template data is properly assigned to the appropriate fields in SAPHIRE. The .BED file allows description information to be entered and linked to the template data name or designator (which in the PCSA case was the TYP-FM coding). Examples of descriptions used for the PCSA template data were, clutch failed to operate, relay spurious operation, position sensor fails on demand, and wire rope breaks. The .BEI file contains the actual active component reliability parameters, namely the mean value and uncertainty parameter, either the lognormal error factor, or the shape parameter of the Beta or Gamma distributions.

Geometric means of the input parameters from the data sources are initially used as screening values for each TYP-FM and are entered into the .BEI file, along with a default Error Factor of 10. Once the Bayesian combination process is completed for all of the TYP-FM combinations, mean and uncertainty parameter information are entered into the .BEI files, and tested in SAPHIRE before being distributed to the systems analysts.

The template data is utilized by the fault tree models by being imported into SAPHIRE using the MAR-D portion of the SAPHIRE code, then by using the modify event feature to link the template data to each basic event in the fault tree. This permits each active component of the same type and failure mode to utilize the same failure estimate and uncertainty information, based on the results of the data investigation and Bayesian combination process.

Attachment C, Section C4, presents a more thorough discussion of the active component reliability data development process, as well as a table of the template data that is imported into SAPHIRE.

6.3.1.5 Summary of Active Component Reliability Data in RF Analysis

Table 6.3-1 summarizes the active component reliability data used in each basic event of the RF models. Development of this table is discussed in detail in Attachment C, Section C4. Mission times are discussed in Section 6.2.

Table 6.3-1. Active Component Reliability Data Summary

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-#EEE-##52-B5-C52-FOD	Circuit Breaker (AC) Fails on Demand	2.24E-03	2.24E-03	
200-#EEE-BATB5-1-FAN-FTR	Fan (Motor-Driven) Fails to Run	5.18E-03	7.21E-05	72
200-#EEE-BATB5-2-FAN-FTR	Fan (Motor-Driven) Fails to Run	5.18E-03	7.21E-05	72
200-#EEE-BATB5CL-FAN-CCF	Fan (Motor-Driven) Fails to Run	2.45E-04	7.21E-05	3
200-#EEE-ITSBATB-BAT-FOD	Battery No Output Given Challenge	8.20E-03	8.20E-03	
200-#EEE-LDCNTRA-BUA-FOH	RF ITS Load Center A Fails	4.39E-04	6.10E-07	720
200-#EEE-LDCNTRA-C52-FOD	ITS Load Center A feed breaker Fails to Reclose	2.24E-03	2.24E-03	
200-#EEE-LDCNTRA-C52-SPO	Load Center A Feed Circuit Breaker Spurious Operation	3.82E-03	5.31E-06	720
200-#EEE-LDCNTRB-BUA-FOH	RF ITS Load Center B Fails	4.39E-04	6.10E-07	720
200-#EEE-LDCNTRB-C52-FOD	13.8 ITS SWGR to RF LC B Circuit Breaker Fails on Demand	2.24E-03	2.24E-03	
200-#EEE-LDCNTRB-C52-SPO	RF Load Center Circuit Breaker (AC) Spur Op	3.82E-03	5.31E-06	720
200-#EEE-LDCNTRS-C52-CCF	Common cause failure of the ITS Load Center feed breakers to reclose	1.05E-04	1.05E-04	
200-#EEE-MCC0001-C52-SPO	RF ITS MCC 0001 Feed Breaker Spurious Operation	3.82E-03	5.31E-06	720
200-#EEE-MCC0001-MCC-FOH	RF ITS MCC 00001 Fails	5.38E-03	7.49E-06	720
200-#EEE-MCC0002-C52-SPO	RF MCC-00002 Feed Breaker Spurious Operation	3.82E-03	5.31E-06	720
200-#EEE-MCC0002-MCC-FOH	RF ITS MCC00002 Failure	5.38E-03	7.49E-06	720
200-#EEE-RFITS-A-XMR-CCF	RF ITS Transformer train A CCF	4.92E-06	2.91E-07	34
200-#EEE-RFITS-A-XMR-FOH	RF ITS Transformer Train B Failure	2.10E-04	2.91E-07	720
200-#EEE-RFITS-B-XMR-FOH	RF ITS Transformer Train B Failure	2.10E-04	2.91E-07	720
200--DRUM001-DM--FOD	CTM Drum Failure on Demand	4.00E-08	4.00E-08	
200-CR---IEL001--IEL-FOD	Interlock A From Slide Gate Fails	2.75E-05	2.75E-05	
200-CR---IEL001-IEL-FOD	Skirt Interlock Failure	2.75E-05	2.75E-05	1
200-CR---IEL002--IEL-FOD	Interlock B From Slide Gate Fails	2.75E-05	2.75E-05	
200-CR---IELCCF--IEL-CCF	Common Cause Failure of Interlocks From Slide Gate	1.29E-06	1.29E-06	
200-CR--IEL001--IEL-FOD	Interlock A From Slide Gate Fails	2.75E-05	2.75E-05	
200-CR--IEL002--IEL-FOD	Interlock B From Slide Gate Fails	2.75E-05	2.75E-05	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-CR--IELCCF-IEL-CCF	Common Cause Failure of Interlocks from Slide Gate	1.29E-06	1.29E-06	1
200-CR--PLC001--PLC-SPO	Inadvertent Signal Sent due to PLC Failure	3.65E-07	3.65E-07	
200-CR-PLC001-PLC-SPO	Inadvertent Signal sent due to PLC Failure	3.65E-07	3.65E-07	
200-CRN-HSTTRLMO-MOE-FSO	Crane Hoist Motor (Electric) Fails to Shut Off	1.35E-08	1.35E-08	
200-CRN-PLC0101--PLC-SPO	Crane Bridge Motor PLC Spurious Operation	3.65E-07	3.65E-07	
200-CRN2-2-BLOCK-CRN-TBK	200 Ton Crane Two Block Drop	4.41E-07	4.41E-07	
200-CRN2-2BLKDON-CRN-TBK	200 Ton Crane Two Block Drop	4.41E-07	4.41E-07	
200-CRN2-DROPDPC-CRN-DRP	200 Ton Crane Drop	3.21E-05	3.21E-05	
200-CRN2-DROPDPC-CRS-DRP	200 Ton Crane Sling Drop	1.21E-04	1.21E-04	
200-CRN2-DROPON--CRN-DRP	200 Ton Crane Drop	3.21E-05	3.21E-05	
200-CRN2-DROPTAD-CRN-DRP	200 Ton Crane Drop	3.21E-05	3.21E-05	
200-CRNBRIDGMTR-MOE-FSO	Crane Bridge Motor (Electric) Fails to Shut Off	1.35E-08	1.35E-08	
200-CRNDRPONDPC-CRN-DRP	200 Ton Crane Drop	3.21E-05	3.21E-05	
200-CRWT-ATB1001-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.54E-05	7.54E-05	
200-CRWT-ATB1011-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.54E-05	7.54E-05	
200-CRWT-ATB2002-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.54E-05	7.54E-05	
200-CRWT-ATB222-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.54E-05	7.54E-05	
200-CRWT-ATD0002-AT-FOH	ST D-Axis Electrical Actuator #2 Fails Lift/Lower	7.54E-05	7.54E-05	
200-CRWT-ATD001-AT-FOH	ST D-Axis Electrical Actuator #1 Fails Lift/Lower	7.54E-05	7.54E-05	
200-CRWT-ATD03-AT-FOH	ST D Axis Electrical Actuactor #1 Movement Fails	7.54E-05	7.54E-05	
200-CRWT-ATD04-AT-FOH	ST D-Axis Electrical Actuactor #2 Movement Fails	7.54E-05	7.54E-05	
200-CRWT-ATP002-AT-FOH	ST P-Axis Electrical Failure During Movement	7.54E-05	7.54E-05	
200-CRWT-ATR10002-AT-FOH	ST R-Axis Electrical Actuator #1 Fails Movement	7.54E-05	7.54E-05	
200-CRWT-ATR2004-AT-FOH	ST R-Axis electrical Actuator #2 Fails Movement	7.54E-05	7.54E-05	
200-CRWT-BEA#1-BEA-BRK	Boom#1 Fails During Cask Movement	2.40E-08	2.40E-08	
200-CRWT-BEA22-BEA-BRK	Boom#2 Fails During Cask Lift	2.40E-08	2.40E-08	
200-CRWT-BEAB202-BEA-BRK	Boom#2 Fails During Cask Movement	2.40E-08	2.40E-08	
200-CRWT-BEAD003-BEA-BRK	ST D-Axis Actuactor Structual Arm #2 Failure Movement	2.40E-08	2.40E-08	
200-CRWT-BEAD006-BEA-BRK	ST D-Axis Actuactor Structual Arm #1 Failure Movement	2.40E-08	2.40E-08	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-CRWT-BEAP02-BEA-BRK	ST P-Axis Mechanical Failure During Movement	2.40E-08	2.40E-08	
200-CRWT-BEAR103-BEA-BRK	ST R-Axis Actuator Structural Arm #1 Failure Movement	2.40E-08	2.40E-08	
200-CRWT-BEAR204-BEA-BRK	ST R-Axis Actuator Structural Arm #2 Failure Movement	2.40E-08	2.40E-08	
200-CRWT-BRK001--BRK-FOD	Tractor Brake A Fails	1.46E-06	1.46E-06	
200-CRWT-BRK002--BRK-FOD	Tractor Brake B Fails	1.46E-06	1.46E-06	
200-CRWT-BRK003--BRK-FOD	Trailer Brakes Fail	1.46E-06	1.46E-06	1
200-CRWT-BRKCCF--BRK-CCF	CCF of Both Tractor Brakes	6.86E-08	6.86E-08	1
200-CRWT-CBP0000-CBP-OPC	Electrical Power Dist Cable Failure on ST	9.13E-08	9.13E-08	
200-CRWT-CON0000-CON-FOH	Electrical Power Dist Connectors Fail on ST	7.14E-05	7.14E-05	
200-CRWT-CTSHC000-CT-SPO	Spurious Command to Raise/Lower AO or STC	2.27E-05	2.27E-05	
200-CRWT-DROP11-BEA-BRK	Boom#1 Fails During Cask Lift	2.40E-08	2.40E-08	
200-CRWT-ECP0000-ECP-FOH	ST Restraint Arms Position Selector Fails	1.79E-06	1.79E-06	
200-CRWT-ELEC-MOE-FOD	ST Electric Motor Failure	6.00E-05	6.00E-05	
200-CRWT-IEL0001-IEL-FOH	Restraint System Interlock Failure	3.43E-05	3.43E-05	
200-CRWT-LC000011-LC-FOD	ST Lift/Lower Selector Level Fails	6.25E-04	6.25E-04	
200-CRWT-LPATH--ATH--CCF	CCF of Pendular Axle Hydrualics During Load/Unload	8.38E-05		
200-CRWT-LPATH1--ATH-FOH	Pendular Axle Hydraulic 1 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH2--ATH-FOH	Pendular Axle Hydraulic 2 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH3--ATH-FOH	Pendular Axle Hydraulic 3 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH4--ATH-FOH	Pendular Axle Hydraulic 4 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH5--ATH-FOH	Pendular Axle Hydraulic 5 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH6--ATH-FOH	Pendular Axle Hydraulic 6 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH7--ATH-FOH	Pendular Axle Hydraulic 7 Failure	1.78E-03	8.91E-04	2
200-CRWT-LPATH8--ATH-FOH	Pendular Axle Hydraulic 8 Failure	1.78E-03	8.91E-04	2
200-CRWT-LSJATH--ATH-CCF	CCF of Stabalizing Jacks	8.38E-05		
200-CRWT-LSJATH1-ATH-FOH	Stabalizing Jack 1 Failure	1.78E-03	8.91E-04	2
200-CRWT-LSJATH2-ATH-FOH	Stabilizing Jack 2 Failure	1.78E-03	8.91E-04	2
200-CRWT-LSJATH3-ATH-FOH	Stabilizing Jack 3 Failure	1.78E-03	8.91E-04	2
200-CRWT-LSJATH4-ATH-FOH	Stabilizing Jack 4 Failure	1.78E-03	8.91E-04	2

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-CRWT-LVRD01-LVR-FOH	ST D-Axis Actuactor Structual Arm #1 Failure	2.10E-06	2.10E-06	
200-CRWT-LVRD02-LVR-FOH	ST D-Axis Actuactor Structual Arm #2 Failure	2.10E-06	2.10E-06	
200-CRWT-PIND004-PIN-BRK	ST D-Axis Actuactor Pin #2 Failure Movement	2.12E-09	2.12E-09	
200-CRWT-PIND005-PIN-BRK	ST D-Axis Actuactor Pin #1 Failure Movement	2.12E-09	2.12E-09	
200-CRWT-PINP04-PIN-BRK	ST P-Axis Pin failure During Movement	2.12E-09	2.12E-09	
200-CRWT-PINR103-PIN-BRK	ST R-Axis Mechanical Pin #1 Failure During Movement	2.12E-09	2.12E-09	
200-CRWT-PINR202-PIN-BRK	ST R-Axis Mechanical Pin #2 Failure During Movement	2.12E-09	2.12E-09	
200-CRWT-SJKB011-SJK-FOH	Screw Lift on Boom #1 Fails	8.14E-06	8.14E-06	
200-CRWT-SJKB101-SJK-FOH	Screw Lift on Boom #1 Fails	8.14E-06	8.14E-06	
200-CRWT-SJKB202-SJK-FOH	Screw Lift on Boom #2 Fails	8.14E-06	8.14E-06	
200-CRWT-SJKB22-SJK-FOH	Screw Lift on Boom #2 Fails	8.14E-06	8.14E-06	
200-CRWT-TRCT-STEER-FAIL	Tractor Steering System Failure	1.84E-5	1.84E-5	
200-CRWT-TRD0001-TRD-FOH	Front Portside Track Failure	5.89E-07	5.89E-07	
200-CRWT-TRD0002-TRD-FOH	Rear Portside Track Failure	5.89E-07	5.89E-07	
200-CRWT-TRD0003-TRD-FOH	Front Starboard Track Failure	5.89E-07	5.89E-07	
200-CRWT-TRD0004-TRD-FOH	Rear Starboard Track Failure	5.89E-07	5.89E-07	
200-CRWT-ZSD00005-ZS-FOD	ST D-Axis Position Switch Failure Movement	2.93E-04	2.93E-04	
200-CRWT-ZSD0006-ZS-FOD	ST D-Axis Position Switch Failure Lift/Lower	2.93E-04	2.93E-04	
200-CRWT-ZSP00003-ZS-FOD	ST P-Axis Position Switch Failure During Movement	2.93E-04	2.93E-04	
200-CRWT-ZSR00005-ZS-FOD	ST R-Axis Position Switch Failure Movement	2.93E-04	2.93E-04	
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	2.93E-04	2.93E-04	
200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1.38E-05	1.38E-05	
200-CTM--330121--ZS--FOD	CTM Hoist First Upper Limit Switch 0121 Failure on Demand	2.93E-04	2.93E-04	
200-CTM--330122--ZS--FOD	CTM Final Hoist Upper Limit Switch 0122 Failure on Demand	2.93E-04	2.93E-04	
200-CTM--CBL0001-CBL-FOD	CTM Hoist Wire rope Breaks	2.00E-06	2.00E-06	1
200-CTM--CBL0002-CBL-FOD	CTM Hoist Wire rope Breaks	2.00E-06	2.00E-06	1
200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	9.40E-08	9.40E-08	
200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1.15E-06	1.15E-06	
200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1.15E-06	1.15E-06	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-CTM--HOISTMT-MOE-FTR	CTM Hoist Motor (Electric) Fails to Run	6.50E-06	6.50E-06	1
200-CTM--HOLDBRK-BRK-FOD	Brake Failure on Demand	1.46E-06	1.46E-06	
200-CTM--HOLDBRK-BRK-FOH	CTM Holding Brake (Electric) Failure to hold	3.52E-05	4.40E-06	8
200-CTM--IMEC125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	2.75E-05	2.75E-05	
200-CTM--IMEC125-ZS-FOD	CTM Load Cell Limit Switch Failure on Demand	2.93E-04	2.93E-04	
200-CTM--LOWERBL-BLK-FOD	CTM Lower Sheaves Failure on Demand	1.15E-06	1.15E-06	
200-CTM--MISSPOOL-DM-MSP	CTM Mis-spool events pool event	6.86E-07	6.86E-07	
200-CTM--OVERSP--ZS--FOD	CTM Hoist motor speed Limit Switch Failure on Demand	2.93E-04	2.93E-04	
200-CTM--PORTGT1-MOE-SPO	Spurious port gate1 motor operation	6.74E-07	6.74E-07	1
200-CTM--PORTGT1-PLC-SPO	Port Gage PCL Spurious Operation	3.65E-07	3.65E-07	
200-CTM--PORTGT2-MOE-SPO	Port Gate Motor (Electric) Spurious Operation	6.74E-07	6.74E-07	1
200-CTM--PORTGT2-PLC-SPO	Port Gage PCL Spurious Operation	3.65E-07	3.65E-07	
200-CTM--SLIDEGT-MOE-SPO	CTM Slide Gate Motor (Electric) Spurious Operation	6.74E-07	6.74E-07	1
200-CTM--SLIDEGT-PLC-SPO	CTM Slide Gate PLC Spurious Operation	3.65E-07	3.65E-07	
200-CTM--SLIDGT2-IEL-FOD	CTM Slide Gate Interlock Failure	2.75E-05	2.75E-05	
200-CTM--TROLLY-MOE-SPO	CTM Trolley Motor (Electric) Spurious Operation	6.74E-07	6.74E-07	1
200-CTM--UPPERBL-BLK-FOD	CTM Upper Sheaves failure	1.15E-06	1.15E-06	
200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.99E-03	3.99E-03	
200-CTM--WTSW125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	2.75E-05	2.75E-05	
200-CTM--WTSW125-ZS--FOD	CTM Load Cell Limit Switch Failure on Demand	2.93E-04	2.93E-04	
200-CTM--YS01129-ZS--FOD	CTM Drum Brake control circuit Limit Switch 1129 Failure	2.93E-04	2.93E-04	
200-CTM--ZSH0111-ZS--SPO	CTM grapple engaged Limit Switch Spurious Operation	1.28E-06	1.28E-06	1
200-CTM-ASD0122#-CTL-FOD	CTM Hoist ASD Controller fails	2.03E-03	2.03E-03	8
200-CTM-BIDGMTR-#TL-FOH	CTM Bridge motor Torque limiter Failure	2.86E-02	8.05E-05	360
200-CTM-BRDGEMTR-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation	6.74E-07	6.74E-07	
200-CTM-BREDGMTR-#CT-FOD	CTM Hand Held Radio Remote Controller Fails	4.00E-06	4.00E-06	
200-CTM-BRIDGETR-#PR-FOH	CTM Bridge Passive restraint (end stops) Failure	1.95E-06	4.45E-10	4380
200-CTM-BRIDGETR-MOE-FSO	CTM Bridge motor fails to stop	1.35E-08	1.35E-08	1
200-CTM-BRIDGMTR-IEL-FOD	CTM Shield Skirt-Bridge motor Interlock Failure	2.75E-05	2.75E-05	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-CTM-BRIDGMTS-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation -shear	3.37E-08	6.74E-07	0.1
200-CTM-DRTM-CT-FOD	CTM Drive Train Protection and Fail Det. Controller Failure	4.00E-06	4.00E-06	
200-CTM-DRUMBRK-BRP-FOH	CTM Drum Brake (Pneumatic) Failure to Hold	6.70E-05	8.38E-06	8
200-CTM-HOISTMTR-MOE-FSO	CTM Hoist Motor (Electric) Fails to Shut Off	1.35E-08	1.35E-08	1
200-CTM-HSTTRLLS-MOE-SPO	CTM Hoist Trolley Motor (Electric) Spurious Operation m- shear	3.37E-08	6.74E-07	0.1
200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motorTorque limiter Failure	2.86E-02	8.05E-05	360
200-CTM-HSTTRLLY-IEL-FOD	CTM shield skirt Hoist Trolley motor Interlock Failure	2.75E-05	2.75E-05	
200-CTM-HSTTRLLY-MOE-SPO	Motor (Electric) Spurious Operation	6.74E-07	6.74E-07	1
200-CTM-OPSENSOR-SRX-FOH	Canister above CTM slide gate optical sensor fails	4.70E-06	4.70E-06	1
200-CTM-PLC0101S-PLC-SPO	CTM Bridge Motor PLC Spurious Operation - shear	3.65E-07	3.65E-07	
200-CTM-PLC0102S-PLC-SPO	CTM Shield Bell Trolley PLC Spurious Operation -shear	3.65E-07	3.65E-07	
200-CTM-PLC0103S-PLC-SPO	CTM Hoist Trolley PLC Spurious Operation -shear	3.65E-07	3.65E-07	
200-CTM-SBELTRLS-MOE-SPO	Motor (Electric) Spurious Operation	6.74E-08	6.74E-07	0.1
200-CTM-SBELTRLY-#TL-FOH	CTM Shield Bell MotorTorque limiter Failure	2.86E-02	8.05E-05	360
200-CTM-SBELTRLY-IEL-FOD	CTM Shield Bell Trolley Interlock Failure	2.75E-05	2.75E-05	
200-CTM-SBELTRLY-MOE-SPO	Motor (Electric) Spurious Operation	6.74E-07	6.74E-07	
200-CTM-SKRTCTCT-SRP-FOD	CTM Skirt floor contact sensors fail	3.99E-03	3.99E-03	
200-CTM-SLIDGT2-SRX-FOD	CTM slide Gate Position Sensor Fails on Demand	1.10E-03	1.10E-03	
200-CTM-TROLLEYT-MOE-FSO	CTM Trolley motor fails to stop	1.35E-08	1.35E-08	1
200-CTM-TROLLYTR-#PR-FOH	CTM Trolley end run stops Failure	1.95E-06	4.45E-10	4380
200-CTM-TROLYCNT-#HC-FOD	CTM trolley motor hand controller fails	1.74E-03	1.74E-03	
200-CTM-ZSL0111-ZS--SPO	CTM Grapple engaged Limit Switch Spurious Operation	1.28E-06	1.28E-06	
200-CTT--CT001---CT--SPO	On-Board Controller Initiates Spurious Signal	2.27E-05	2.27E-05	
200-CTT--DSW000--ESC-CCF	Common Cause Failure of Deadman Switches	1.18E-05	1.18E-05	
200-CTT--DSW001--ESC-FOD	Deadman Switch #1 Fails Closed	2.50E-04	2.50E-04	
200-CTT--DSW002--ESC-FOD	Deadman Switch #2 Fails Closed	2.50E-04	2.50E-04	
200-CTT--HC001---HC--SPO	Hand Held Controller Initiates Spurious Signal	5.23E-07	5.23E-07	
200-CTT--SV301---SV--SPO	Solenoid Valve Spurious Operation	4.09E-07	4.09E-07	
200-CTT--ZS301---ZS--FOD	Pin Limit Switch #1 Fails	2.93E-04	2.93E-04	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-CTT--ZS302---ZS--FOD	Pin Limit Switch #2 Fails	2.93E-04	2.93E-04	
200-CTT-FWDREVM1-SV-FOH	Failure of SV Providing Fwd/Rev to Motor 1	4.87E-05	4.87E-05	
200-CTT-FWDREVM2-SV-FOH	Failure of SV Providing Fwd/Rev to Motor 2	4.87E-05	4.87E-05	
200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.09E-07	4.09E-07	
200-CTT-SV401-SV-FOH	Failure of Air Supply Solenoid Valve for Air Bags	4.87E-05	4.87E-05	
200-CTT-SVROTM1-SV-FOH	Failure of SV Providing Rotation to Motor 1	4.87E-05	4.87E-05	
200-CTT-SVROTM2-SV-FOH	Failure of SV Providing Rotation to Motor 2	4.87E-05	4.87E-05	
200-CTT-ZS301-SW-CCF	Common Cause Failure of Limit Switches	1.38E-05	1.38E-05	
200-DRUMBRK-BRP-FOH	CTM Drum Brake (Pneumatic) Failure on Demand	8.38E-06	8.38E-06	
200-FL---SC001---SC--FOH	Forklift Speed Control Fails	1.28E-04	1.28E-04	
200-FL---SC006---SC--FOH	Forklift Speed Control Fails	1.28E-04	1.28E-04	1
200-HTC--HC021---HC--FOD	Remote Stop Control Transmits Wrong Instruction	1.74E-03	1.74E-03	
200-HTC--SV601---SV--FOD	Main Air Supply Valve Fails on Demand	6.28E-04	6.28E-04	
200-HTC--SV602---SV--FOD	Solenoid Valve Fails to Close	6.28E-04	6.28E-04	
200-HTTCOLLIDE---G65-FOH	Speed Limiter Fails	1.16E-05	1.16E-05	
200-PORTSLIDEGTE-IEL-FOD	Port Slide Gate Interlock Fails	2.75E-05	2.75E-05	
200-SD---PLC001--PLC-SPO	Spurious Signal from PLC Closes Door	3.65E-07	3.65E-07	
200-SD---SRU001--SRU-FOH	Ultrasonic Obstruction Sensor Fails	2.16E-03	9.62E-05	
200-SD---TL000---TL--CCF	Common Cause Failure of Over Torque Sensors	6.80E-04	3.78E-06	
200-SD---TL001---TL--FOH	Motor #1 Over Torque Sensor Fails	1.44E-02	8.05E-05	
200-SD---TL002---TL--FOH	Motor #1 Over Torque Sensor Fails	1.44E-02	8.05E-05	
200-SLDGATE-IEL-FOD	Slide gate interlock fails	2.75E-05	2.75E-05	
200-SPMRC-BRP000-BRP-FOD	Brake (Pneumatic) Failure on Demand PMRC Fails to Stop on Loss of Power	5.02E-05	5.02E-05	
200-SPMRC-BRP001-BRP-FOD	SPMRC Brake (Pneumatic) Failure on Demand	5.02E-05	5.02E-05	
200-SPMRC-CBP001-CBP-OPC	Power Cable to SPMRC - Open Circuit	9.13E-08	9.13E-08	
200-SPMRC-CBP001-CBP-SHC	SPMRC Power Cable - Short Circuit	1.88E-08	1.88E-08	
200-SPMRC-CPL00-CPL-FOH	Railcar Automatic Coupler System Fails	1.91E-06	1.91E-06	
200-SPMRC-CT000--CT--FOD	SPMRC Primary Stop Switch Fails	4.00E-06	4.00E-06	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-SPMRC-CT0001-CT-FOD	On-Board Controller Fails to Respond	4.00E-06	4.00E-06	
200-SPMRC-CT002--CT--FOH	Pendant Direction Controller Fails	6.88E-05	6.88E-05	
200-SPMRC-CT003-CT-SPO	On-Board Controller Initiates Spurious Signal	2.27E-05	2.27E-05	
200-SPMRC-DERIL-PER-MILE (DER-FOH)	Derailment of a Rail Car per Mile	1.18E-05	1.18E-05	
200-SPMRC-G65000-G65-FOH	SPMRC Speed Control (Govenor) Fails	1.16E-05	1.16E-05	
200-SPMRC-HC001--HC--SPO	Spurious Command from Pendant Controller	5.23E-07	5.23E-07	
200-SPMRC-HC001-HC--FOD	Pendant Control Transmits Wrong Signal	1.74E-03	1.74E-03	
200-SPMRC-IEL011-IEL-FOD	Failure of Mobile Platform Anti-Coll Interlock	2.75E-05	2.75E-05	
200-SPMRC-MOE000-MOE-FSO	PMRC Lock Mode State Fails on Loss of Power	1.35E-08	1.35E-08	
200-SPMRC-SC021--SC--FOH	Speed Controller on SPMRC Pendant Fails	1.28E-04	1.28E-04	
200-SPMRC-SEL021-SEL-FOH	Speed Selector on SPMRC Pendant Fails	4.16E-06	4.16E-06	
200-SPMRC-STU001-STU-FOH	SPMRC End Stops Fail	2.11E-04	4.81E-08	4380
200-ST---BRK001--BRK-FOD	ST Fails to Stop on Loss of Power	1.46E-06	1.46E-06	
200-ST---CBP004-CBP--OPC	ST Power Cable - Open Circuit	9.13E-08	9.13E-08	
200-ST---CBP004-CBP--SHC	ST Power Cable Short Circuit	1.88E-08	1.88E-08	
200-ST---CT000---CT--FOD	ST Primary Stop Switch Fails	4.00E-06	4.00E-06	
200-ST---CT002---CT--FOH	Direction Controller Fails	6.88E-05	6.88E-05	
200-ST---HC000--HC--SPO	Spurious Commands from Remote Control	5.23E-07	5.23E-07	
200-ST---HC001--HC--FOD	Remote Control Transmits Wrong Signal	1.74E-03	1.74E-03	
200-ST---HC002---HC--SPO	Spurious Command to Lift/Lower AO or STC	5.23E-07	5.23E-07	
200-ST---MOE000--MOE-FSO	ST Lock Mode State Fails on Loss of Power	1.35E-08	1.35E-08	
200-ST---MOE021--MOE-FSO	Drive System on Primary Propulsion Fails	1.35E-08	1.35E-08	
200-ST---SC002--SC--FOH	Speed Control on ST Pendant Control Fails	1.28E-04	1.28E-04	
200-ST---SC021---SC--FOH	Speed Controller on ST Pendant Fails	1.28E-04	1.28E-04	
200-ST---SC021---SC--SPO	On-Board Controller Initiates Spurious Signal	3.20E-05	3.20E-05	
200-ST---SEL021--SEL-FOH	Speed Selector on ST Pendant Fails	4.16E-06	4.16E-06	
200-ST-MOE0001-MOE-FSO	ST Lock Mode State Fails on Loss of Power	1.35E-08	1.35E-08	
200-ST-SC021-SC-SPO	On Board Controller Initiates Spurious Signals	3.20E-05	3.20E-05	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-TILTFRAME-CSC-FOH	Cask tilting frame fails	4.81E-08	4.81E-08	
200-VCOO-SFAN001-FAN-FTR	Supply Fan #1 for RF Fails	5.06E-02	7.21E-05	720
200-VCOO-SFAN002-FAN-FTR	Supply Fan #2 for CRCF Fails	5.06E-02	7.21E-05	720
200-VCT0-AHU0001-AHU-FTR	RF ITS Elec AHU 00001 Fails to run	2.65E-03	3.68E-06	720
200-VCT0-AHU0001-CTL-FOD	RF ITS Elec AHU 00001 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-AHU0002-AHU-FTR	RF ITS ELec AHU 00002 Fails to Run	2.65E-03	3.68E-06	720
200-VCT0-AHU0002-CTL-FOD	RF ITS Elec AHU 00002 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-AHU0002-FAN-FTS	RF ITS Elec AHU 00002 Fails to Start	2.02E-03	2.02E-03	
200-VCT0-AHU0003-AHU-FTR	RF ITS Elec AHU 00003 Fails to run	2.65E-03	3.68E-06	720
200-VCT0-AHU0003-CTL-FOD	RF ITS Elec AHU 00003 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-AHU0004-AHU-FTR	RF ITS ELec AHU 00004 Fails to Run	2.65E-03	3.68E-06	720
200-VCT0-AHU0004-CTL-FOD	RF ITS Elec AHU 00004 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-AHU0004-FAN-FTS	RF ITS Elec AHU 00004 Fails to Start	2.02E-03	2.02E-03	
200-VCT0-AHU0103-AHU-CCR	CCF of the running RF ITS Elec AHUs to continue to run	6.20E-05	6.20E-05	
200-VCT0-AHU0202-AHU-CCR	CCF of standby RF ITS Elec AHUs to start/run	1.60E-04	1.60E-04	
200-VCT0-EXH-009-CTL-FOD	RF ITS Elec Exh fan 00005 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-EXH-009-FAN-FTR	RF ITS Elec Exhaust Fan 00005 Fails to Run	5.06E-02	7.21E-05	720
200-VCT0-EXH-010-CTL-FOD	RF ITS Elec Exh Fan 0006 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-EXH-010-FAN-FTR	RF ITS Elec Exh. Fan 0010 Fails to Run	5.06E-02	7.21E-05	720
200-VCT0-EXH-010-FAN-FTS	RF ITS Elec Exh fan 00006 Fails to Start	2.02E-03	2.02E-03	
200-VCT0-EXH-011-CTL-FOD	RF ITS Elec Exh fan 00007 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-EXH-011-FAN-FTR	RF ITS Elec Exhaust Fan 00007 Fails to Run	5.06E-02	7.21E-05	720
200-VCT0-EXH-012-CTL-FOD	RF ITS Elec Exh Fan 0008 Controller Fails	2.03E-03	2.03E-03	
200-VCT0-EXH-012-FAN-FTR	RF ITS Elec. Exh Fan 00012 Fails to Run	5.06E-02	7.21E-05	720
200-VCT0-EXH-012-FAN-FTS	RF ITS Elec Exh fan 00008 Fails to Start	2.02E-03	2.02E-03	720
200-VCT0-EXH0911-FAN-CCR	CCF of running Exh fans for RF ITS Elec.	1.20E-03	1.20E-03	
200-VCT0-EXH1012-FAN-CCF	CCF to start/run: standby Exh fans for the RF ITS Elec	1.30E-03	1.30E-03	
200-VCTO--B---FAN-FTS	Train B Fan Fails to Start	2.02E-03	2.02E-03	360
200-VCTO-DMP000A-DMP-FRO	Manual Damper for Train A Fails	6.03E-05	8.38E-08	720

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-VCTO-DMP000B-DMP-FRO	Manual Damper for Train B Fails	3.02E-05	8.38E-08	360
200-VCTO-DMP001A-DMP-FRO	Manual damper Input to Exhaust Fan A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMP001B-DMP-FRO	Manual damper Input to Exhaust Fan B Fails	3.02E-05	8.38E-08	360
200-VCTO-DMPA05I-DMP-FRO	Manual Damper #05 input Train A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMPA05O-DMP-FRO	Manual Damper #05 Output Train A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMPA06I-DMP-FRO	Manual Damper #06 Input Train A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMPA06O-DMP-FRO	Manual Damper #06 Output Train A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMPA07I-DMP-FRO	Manual Damper #07 in Train A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMPA07O-DMP-FRO	Manual Damper #07 Output Train A Fails	6.03E-05	8.38E-08	720
200-VCTO-DMPB08I-DMP-FRO	Manual Damper #08 input Train B Fails	3.02E-05	8.38E-08	360
200-VCTO-DMPB08O-DMP-FRO	Manual Damper #08 Output Train A Fails	3.02E-05	8.38E-08	360
200-VCTO-DMPB09I-DMP-FRO	Manual Damper #09 input Train A Fails	3.02E-05	8.38E-08	360
200-VCTO-DMPB09O-DMP-FRO	Manual Damper #09 Output Train A Fails	3.02E-05	8.38E-08	360
200-VCTO-DMPB10I-DMP-FRO	Manual Damper #10 Input in Train B Fails	3.02E-05	8.38E-08	360
200-VCTO-DMPB10O-DMP-FRO	Manual Damper #10 Output Train A Fails	3.02E-05	8.38E-08	360
200-VCTO-DTC0A-DTC-RUP	Duct Fails between HEPA and Exhaust Fan (10 feet)	2.68E-03	3.72E-06	720
200-VCTO-DTC0B-DTC-RUP	Duct Fails between HEPA and Exhaust Fan (10 feet)	1.34E-03	3.72E-06	360
200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.06E-02	7.21E-05	720
200-VCTO-FAN00B-FAN-FTR	Exhaust Fan in Train B Fails	2.56E-02	7.21E-05	360
200-VCTO-FAN00B-FAN-FTS	Exhaust Fan in Train B Fails to Start	2.02E-03	2.02E-03	
200-VCTO-FANA-PRM-FOH	Speed Control Exhaust Fan Train A Fails to maintain Delta P	5.38E-07	5.38E-07	
200-VCTO-FANB-PRM-FOH	Speed Control Exhaust Fan Train B Fails to maintain Delta P	1.94E-04	5.38E-07	360
200-VCTO-FSLAB0-SRF-FOH	Low Flow Train A Sensor Failure	7.70E-04	1.07E-06	720
200-VCTO-HEPA-CCF	Common Cause Failure of HEPA filters (2 of 3)	1.45E-04	2.01E-07	720
200-VCTO-HEPA05-DMS-FOH	Moisture Separator/Demister HEPA 05 Fails	6.55E-03	9.12E-06	720
200-VCTO-HEPA06-DMS-FOH	Moisture Separator/Demister HEPA 06 Fails	6.55E-03	9.12E-06	720
200-VCTO-HEPA07-DMS-FOH	Moisture Separator/Demister HEPA 07 Fails	6.55E-03	9.12E-06	720
200-VCTO-HEPA0A5-HEP-LEK	HEPA #05 Train A Leaks	2.16E-03	3.00E-06	720
200-VCTO-HEPAA05-HEP-LEK	HEPA #05 Train A Leaks	3.00E-06	3.00E-06	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
200-VCTO-HEPAA05-HEP-PLG	HEPA #A05 Train A Plugged	3.07E-03	4.27E-06	720
200-VCTO-HEPAA06-DMS-FOH	Moisture Separator/Demister HEPA 06 Fails	6.55E-03	9.12E-06	720
200-VCTO-HEPAA06-HEP-LEK	HEPA #06 Train A Leaks	2.16E-03	3.00E-06	720
200-VCTO-HEPAA06-HEP-PLG	HEPA #A10 Train A Plugged	3.07E-03	4.27E-06	720
200-VCTO-HEPAA07-HEP-LEK	HEPA #07 Train A Leaks	2.16E-03	3.00E-06	720
200-VCTO-HEPAA07-HEP-PLG	HEPA #A07 Train A Plugged	3.07E-03	4.27E-06	720
200-VCTO-HEPAB-CCF	Common Cause Failure of HEPA filters (2 of 3)	7.24E-05		
200-VCTO-HEPAB08-DMS-FOH	Moisture Separator/Demister HEPA 08 Fails	3.28E-03	9.12E-06	360
200-VCTO-HEPAB08-HEP-LEK	HEPA #B12 Train B Leaks	1.08E-03	3.00E-06	360
200-VCTO-HEPAB08-HEP-PLG	HEPA #B08 Train B Plugged	1.54E-03	4.27E-06	360
200-VCTO-HEPAB09-DMS-FOH	Moisture Separator/Demister HEPA 09 Fails	3.28E-03	9.12E-06	360
200-VCTO-HEPAB09-HEP-LEK	HEPA #B09 Train B Leaks	1.08E-03	3.00E-06	360
200-VCTO-HEPAB09-HEP-PLG	HEPA #B09 Train B Plugged	1.54E-03	4.27E-06	360
200-VCTO-HEPAB10-DMS-FOH	Moisture Separator/Demister HEPA 10 Fails	3.28E-03	9.12E-06	360
200-VCTO-HEPAB10-HEP-LEK	HEPA #B10 Train B Leaks	1.08E-03	3.00E-06	360
200-VCTO-HEPAB10-HEP-PLG	HEPA #B10 Train B Plugged	1.54E-03	4.27E-06	360
200-VCTO-IEL0001-IEL-FOD	RF Door Interlock Failure	2.75E-05	2.75E-05	
200-VCTO-PDSLA0B-SRP-FOD	Pressure Differential Train A Switch Fails	3.99E-03	3.99E-03	720
200-VCTO-TDMP00A-DTM-FOH	Damper (Tornado) Failure	1.61E-02	2.26E-05	720
200-VCTO-TDMP00B-DTM-FOD	Tornado damper Train B Fails On Demand	8.71E-04	8.71E-04	
200-VCTO-TDMP00B-DTM-FOH	Tornado damper Train B Fails	8.10E-03	2.26E-05	360
200-VCTO-TRAINB-MAINT	Train B HVAC is Off-Line for Maintenance	2.74E-03	2.74E-03	
200-VCTO-UDMP000-UDM-FOH	Backdraft Damper for Train B exhaust Fails	8.10E-03	2.26E-05	360
200CTM-PLC0101#-PLC-SPO	CTM Bridge Motor PLC Spurious Operation	3.65E-07	3.65E-07	
200CTM-PLC0102#-PLC-SPO	CTM Shield Bell Trolley PLC Spurious Operation	3.65E-07	3.65E-07	
200CTM-PLC0103#-PCL-SPO	CTM Hoist Trolley PLC Spurious Operation	3.65E-07	3.65E-07	
26D-##EG-DAYTNKA-TKF-FOH	ITS DG A Day Tank (00002A) Fails	1.58E-04	4.40E-07	360
26D-##EG-DAYTNKB-TKF-FOH	ITS DG B Day fuel tank fails	1.58E-04	4.40E-07	360
26D-##EG-FLITLKA-IEL-FOD	ITS DG A fuel transfer pumps Interlock Failure	2.75E-05	2.75E-05	

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
26D-##EG-FLITLKB-IEL-FOD	ITS DG B fuel transfer pumps Interlock Failure	2.75E-05	2.75E-05	
26D-##EG-FTP1DGA-PMD-FTR	ITS DG A Fuel Transfer Pump Fails to Run	1.23E-02	3.45E-05	360
26D-##EG-FTP1DGA-PMD-FTS	ITS DG A Fuel Pump 1A Fails to Start	2.50E-03	2.50E-03	
26D-##EG-FTP1DGB-PMD-FTR	ITS DG B Fuel Transfer Pump 1 (Motor Driven) Fails to Run	1.23E-02	3.45E-05	360
26D-##EG-FTP1DGB-PMD-FTS	ITS DG B Fuel Transfer Pump 1 (Motor Driven) Fails to Start	2.50E-03	2.50E-03	
26D-##EG-FTP2DGA-PMD-FTR	ITS DG A Fuel Transfer Pump 2A Fails to Run	1.23E-02	3.45E-05	360
26D-##EG-FTP2DGA-PMD-FTS	ITS DG A Fuel Transfer pump 2A Fails to Start	2.50E-03	2.50E-03	
26D-##EG-FTP2DGB-PMD-FTR	ITS DG B Fuel Transfer Pump 2 (Motor Driven) Fails to Run	1.23E-02	3.45E-05	360
26D-##EG-FTP2DGB-PMD-FTS	ITS DG B Fuel Transfer Pump 2 (Motor Driven) Fails to Start on Demand	2.50E-03	2.50E-03	
26D-##EG-FULPMPA-PMD-CCR	Common cause failure of ITS DG A fuel pumps to run	2.90E-04	2.90E-04	
26D-##EG-FULPMPA-PMD-CCS	Common cause failure of ITS DG A fuel pumps to start	1.20E-04	1.20E-04	
26D-##EG-FULPMPB-PMD-CCR	Common cause failure of ITS DG B fuel pumps to run	2.90E-04	2.90E-04	
26D-##EG-FULPMPB-PMD-CCS	Common cause failure of ITS DG B fuel pumps to start	1.20E-04	1.20E-04	
26D-##EG-HVACFN1-FAN-FTR	ITS DG B room Fan 1 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-##EG-HVACFN1-FAN-FTS	ITS DG B room Fan (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-##EG-HVACFN2-FAN-FTR	ITs DG B room Fan 2 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-##EG-HVACFN2-FAN-FTS	ITS DG B Room Fan (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-##EG-HVACFN3-FAN-FTR	ITS DG B room Fan 3 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-##EG-HVACFN3-FAN-FTS	ITS DG B Room Fan 3 (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-##EG-HVACFN4-FAN-FTR	ITS DG B Fan 4 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-##EG-HVACFN4-FAN-FTS	ITS DG B Room Fan 4 (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-##EG-STRTDGA-C72-SPO	ITS Switchgear A Battery Circuit Breaker (DC) Spur Op	3.85E-04	1.07E-06	360
26D-##EG-STRTDGB-C72-SPO	13.8kV ITS SWGR Battery B Circuit Breaker (DC) Spur Op	3.85E-04	1.07E-06	360
26D-##EG-WKTNK_A-TKF-FOH	ITS DG A Bulk Fuel Tank (00001A) Fails	1.58E-04	4.40E-07	360
26D-##EG-WKTNK_B-TKF-FOH	ITS DG B Bulk Fuel Tank Fails	1.58E-04	4.40E-07	360
26D-##EGBATCHRGA-BYC-FOH	ITS Switchgear A Battery: Battery Charger failure	1.28E-03	7.60E-06	168
26D-##EGBATCHRGB-BYC-FOH	ITS DG B Battery Charger failure	1.28E-03	7.60E-06	168
26D-#EEE-SWGRDGA-BUA-FOH	13.8kV ITS Switchgear A Failure	4.39E-04	6.10E-07	720

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
26D-#EEE-SWGRDGB-AHU-FTR	EDGF Switchgear Room Air Handling Unit Failure to Run	2.65E-03	3.68E-06	720
26D-#EEE-SWGRDGB-BUA-FOH	13.8kV ITS Switchgear B Bus Failure	4.39E-04	6.10E-07	720
26D-#EEESWGRDGA-AHU-FTR	13.8kV ITS Switchgear room Air Handling Unit Fails	2.65E-03	3.68E-06	720
26D-#EEG-HVACFA1-FAN-FTR	ITS DG A room Fan 1 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-#EEG-HVACFA1-FAN-FTS	ITS DG A room Fan 1 (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-#EEG-HVACFA2-FAN-FTR	ITS DG A room Fan 2 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-#EEG-HVACFA2-FAN-FTS	ITS DG A room Fan 2 (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-#EEG-HVACFA3-FAN-FTR	ITS DG A room Fan 3 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-#EEG-HVACFA3-FAN-FTS	ITS DG A room Fan 3 (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-#EEG-HVACFA4-FAN-FTR	ITS DG A room Fan 4 (Motor-Driven) Fails to Run	2.56E-02	7.21E-05	360
26D-#EEG-HVACFA4-FAN-FTS	ITS DG A room Fan 4 (Motor-Driven) Fails to Start	2.02E-03	2.02E-03	
26D-#EEU-208_DGA-BUD-FOH	ITS DC Panel A DC Bus Failure	8.64E-05	2.40E-07	360
26D-#EEU-208_DGB-BUD-FOH	DC Bus Failure	8.64E-05	2.40E-07	360
26D-#EEY-DGALOAD-C52-FOD	DG A Load Breaker (AC) Fails to Close	2.24E-03	2.24E-03	
26D-#EEY-DGBLOAD-C52-FOD	ITS DG B Load Breaker (AC) Fails to Close	2.24E-03	2.24E-03	
26D-#EEY-DGLOADS-C52-CCF	Common cause failure of ITS DG Load Breakers to close	1.05E-04	1.05E-04	
26D-#EEY-ITS-DGB-#DG-FTS	Diesel Generator Fails to Start	8.38E-03	8.38E-03	
26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.70E-01	4.08E-03	360
26D-#EEY-ITSDG-A-#DG-FTS	Diesel Generator Fails to Start	8.38E-03	8.38E-03	
26D-#EEY-ITSDGAB-#DG-CCR	CCF ITS DG A & B Fail to Run	1.80E-02		
26D-#EEY-ITSDGAB-#DG-CCS	CCF DG A and B to Start	3.90E-04	3.90E-04	
26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.70E-01	4.08E-03	360
26D-#EEY-OB-SWGA-C52-FOD	13.8kV ITS SWGR feed breaker (AC) Fails to open	2.24E-03	2.24E-03	
26D-#EEY-OB-SWGA-C52-SPO	13.8kV ITS SWGR A feed Breaker Spurious Operation	3.82E-03	5.31E-06	720
26D-#EEY-OB-SWGB-C52-FOD	Circuit Breaker (AC) Fails on Demand	2.24E-03	2.24E-03	
26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.82E-03	5.31E-06	720
26D-#EEY-OB-SWGS-C52-CCF	Common cause failure of 13.8kV ITS SWGR feed breakers to open	1.04E-04	1.04E-04	
26D-#EG-BATTERYB-BTR-FOD	ITS SWGR Control Battery B No Output	8.20E-03	8.20E-03	
26D-#EG-LCKOUTRL-RLY-FTP	13.8kV ITS Switchgear Feed breaker lock out relay fails to Open CB	3.15E-03	8.77E-06	360

Table 6.3-1. Active Component Reliability Data Summary (Continued)

Basic Event Name	Basic Event Description	Basic Event Mean Probability[a]	Mean Failure Rate[a]	Mission Time (Hours)
26D-#EG-LDSQNCRB-SEQ-FOD	ITS DG B load sequencer fails	2.67E-03	2.67E-03	
26D-#EG-LOCKOUTB-RLY-FTP	13.8 ITS SWGR Lockout Relay (Power) Fails to Open CB	3.15E-03	8.77E-06	360
26D-#EGLDSQNCRA-SEQ-FOD	DG A Load Sequencer Fails	2.67E-03	2.67E-03	
26D-EG-BATTERYA-BTR-FOD	ITS Switchgear A Battery No Output Given Challenge	8.20E-03	8.20E-03	
27A-#EEE-BUS2DGA-C52-SPO	13.8kV Open Bus 2 ITS Load Breaker Spurious Operation	3.82E-03	5.31E-06	720
27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.82E-03	5.31E-06	720
27A-#EEN-OPENBS2-BUA-FOH	13.8kV Open Bus 2 Bus Failure	4.39E-04	6.10E-07	720
27A-#EEN-OPENBS4-BUA-FOH	13.8kV Open Bus 4 Bus Failure	4.39E-04	6.10E-07	720
27A-#EEN-OPNBS1A-SWP-SPO	13.8kV Open Bus 2 to ITS Div A Electric Power Switch Spur. Xfer	1.12E-04	1.55E-07	720
27A-#EEN-OPNBS3B-SWP-SPO	13.8kV Open Bus 4 to ITS B Electric Power Switch Spur Xfer	1.12E-04	1.55E-07	720

NOTE: [a]Although the values in this table are shown to a precision of three significant figures, the values are not known to that level of precision. The values in Attachment C may show fewer significant figures. Such differences are not meaningful in the context of this analysis because the corresponding uncertainties (which are accounted for in the analysis) are much greater than differences due to rounding.

AC = alternating current; AHU =;air-handling unit; CCF = common-cause failure; CRCF = Canister Receipt and Closure Facility; CTM = canister transfer machine; DG = diesel generator; HEPA = high-efficiency particulate air; ITS = important to safety; MCC = motor control center; PLC = programmable logic controller; RF = Receipt Facility; SFP = spent fuel pool; SPMRC = site prime mover railcar; SPMTT = site prime mover truck trailer; ST = site transporter; SV = solenoid valve ; WP = waste package; WPTT = waste package transfer trolley.

Source: Attachment C, Section C4.

6.3.2 Passive Equipment Failure Analysis

Many event sequences described in Section 6.1 include pivotal events that arise from loss of integrity of a passive component, namely one of the aging overpacks, casks or canisters that contain a radioactive waste form. Such pivotal events involve (1) loss of containment of radioactive material that prevents airborne releases, or (2) LOS effectiveness. Both types of pivotal events may be caused by failure modes caused by either physical impact to the container or by thermal energy transferred to the container. This section summarizes the results of the passive failure analyses detailed in Attachment D that yield the conditional probability of loss of containment or LOS.

6.3.2.1 Probability of Loss of Containment

An overview of the methodology for calculating the probability of failure of passive equipment from drops and impact loads is presented in Section 4.3.2.2. Consistent with HLWRS-ISG-02 (Ref. 2.2.69), the methodology essentially consists of comparing the demand upon the equipment to a capacity curve. The probability of failure is the value of the cumulative distribution function for the capacity curve, evaluated at the demand upon the container. More detailed discussion is presented in Attachment D. The methodology is applicable to all of the waste containers that are processed in the RF, including transportation casks, aging overpacks, and canisters. As described in Section 4.3.2.2, the condition at which a passive component is said to fail depends on the success criteria defined for the component in the RF operation. Passive components are designed and manufactured to ensure that the success criteria are met in normal operating conditions and with margin, to ensure that the success criteria are also met when subjected to abnormal loads, including those expected during event sequences. The design margins, and in some cases materials, may be dictated by the code and standards applied to a given type of container as characterized by tensile elongation data for impact loads and by strength at temperature data for thermal loads.

As described in Sections 4.3.2.2, the probability of a passive failure is often based on consideration of variability (uncertainty) in the applied load, and the variability in the strength (resistance) of the component. The variability in the physical and thermal loading are derived from the systems analysis that defines the probabilities of physical or thermal loads of a given magnitude in a given event sequence. Such conditions arise from the event sequence analysis described in Section 6.1. For the analysis of the effects of fires on waste containers, probability distributions were developed for both the load and the response. For drops and impacts, however, an event sequence analysis is used to define conservative conditions for the load rather than deal with possible ranges of such parameters. Therefore, the calculation of the probability of passive failures is based on the response or resistance characteristics of the container, given the conservative point value for the drop or impact load defined for a given event sequence.

6.3.2.2 Probability of Loss of Containment for Drops and Impacts

Calculation of the probability of failure of the various containers is based on the variability in the strength (resistance) of the container as derived from tests and structural analysis, including Finite Element Analysis (FEA), detailed in Attachment D. Loss of containment probability analysis has been evaluated for various containers by three different studies:

- *Seismic and Structural Container Analyses for the PCSA* (Ref. 2.2.35)
- *Structural Analysis Results of the DOE SNF Canisters Subjected to the 23-Foot Vertical Repository Drop Event to Support Probabilistic Risk Evaluations.* EDF-NSNF-085 (Ref. 2.2.78) and *Qualitative Analysis of the Standardized DOE SNF Canister for Specific Canister-on-Canister Drop Events at the Repository.* EDF-NSNF-087 (Ref. 2.2.79)
- *Naval Long Waste Package Vertical Impact on Emplacement Pallet and Invert* (Ref.2.2.22).

All analyses have applied essentially the same methods that include FEA to determine the structural response of the various canisters and casks to drop and impact loads, developing a fragility function for the material used in the respective container, and using the calculated responses (strains) with the fragility function to derive the probability of container breach.

Failure probabilities for drops are summarized in Table 6.3-2. Conservative representations of drop height are defined for operations with each type of container. Sometimes more than one conservative drop height is specified, for example, for normal height crane lifts and two-block height crane lifts. Lawrence Livermore National Laboratory (LLNL) predicts failure probabilities of $<1.0 \times 10^{-8}$ for most of the events (Ref. 2.2.35). If a probability for the event sequence is less than 1×10^{-8}, additional conservatism is incorporated in the PCSA by using a failure probability of 1.0×10^{-5}, which are termed "LLNL, adjusted". This additional conservatism is added to account for, (a) future evolutions of cask and canister designs, and (b) uncertainties, such as undetected material defects, undetected manufacturing deviations, and undetected damage associated with handling before the container reaches the repository, which are not included in the tensile elongation data.

LLNL calculates strains by modeling representative casks, aging overpacks, and canisters that encompass TAD canisters, naval SNF canisters, and a variety of DPCs with the dynamic finite element code, LS-DYNA (Ref. 2.2.35). For these canisters, only flat-bottom drops are considered to model transfers by a CTM. This is justified because these canisters fit sufficiently tightly within the CTM and potential dropped canisters are guided by the canister guide sleeve of the CTM to remain in a vertical position.

INL calculates strains by modeling DOE SNF and MCOs with the static finite element code, ABAQUS (Ref. 2.2.78). The structural evaluations consider off-vertical drops. In such cases, the deformation of the waste form container is greater on the localized area of impact than for a flat-bottom drop, and will therefore yield a greater calculated probability of breach.

Probability of failure is conservatively calculated by comparing the peak strain to the cumulative distribution function derived from tensile strain to failure test data reported in the literature, representing aleatory uncertainty associated with the variability of test coupon data.

BSC FEA analysis used LS-DYNA to model waste packages. Alloy 22 is not stainless steel but a nickel-based alloy, and the most appropriate metric for probability of failure is a cumulative distribution function over extended toughness fraction (Attachment D, Section D1.4). The probability of failure is calculated using the peak toughness index over the waste package, which is a measure of the alloy's energy absorbing capability.

Table 6.3-2. Failure Probabilities Due to Drops and Other Impacts

	Drop Height (ft)	Failure Probability	Note
Representative transportation cask[a]	**13.1**	**1.0×10^{-5}**	**4 degrees from vertical, LLNL, adjusted, no impact limiters**
	6	**1.0×10^{-5}**	**3 degrees from horizontal, LLNL, adjusted, no impact limiters**
	Slapdown after 13.1 foot drop	**1.0×10^{-5}**	**LLNL, adjusted, no impact limiters**
Representative canister	**32.5[b]**	**1.0×10^{-5}**	**Flat bottomed, LLNL, adjusted**
DOE standardized 24-in or 18-in canister	**23**	**1.0×10^{-5}**	**3 degrees from vertical, LLNL, adjusted using INL FEA**
Aging overpack	**3**	**1.0×10^{-5}**	**LLNL, adjusted**
MCO canister	**23**	**9.0×10^{-2}**	**LLNL using INL FEA**
HLW canister	**30**	**6.7×10^{-2}**	**Bayesian interpretation of test data, 0 failures in 13 drops.**

NOTE: [a] Also applies to shielded transfer casks used on-site and horizontal transfer casks. Although shielded transfer casks are not used in the RF, they are mentioned here for completeness.
[b]For transfers by the CTM, this drop height is greater than the maximum drop height (except for CTM transfers in the IHF)
BSC = Bechtel SAIC; DOE = U.S. Department of Energy; FEA=finite element analysis; HLW = high-level radioactive waste; INL = Idaho National Laboratory; LLNL = Lawrence Livermore National Laboratory; MCO = multicanister overpack.

Source: Attachment D

Containment failure probabilities due to other physical impact conditions, equivalent to drops, are listed in Table 6.3-3. These probabilities were modeled by LLNL using FEA, resulting in prediction of failure probabilities of $<1.0 \times 10^{-8}$. Again, additional conservatism was incorporated by using a failure probability of 1.0×10^{-5} for most of these events. The side impact event was not adjusted from the LLNL result of $< 1.0 \times 10^{-8}$ because of the very low velocities involved. A comparison of the strains induced by drops and slow speed, side impacts indicates significantly lower strains for the low velocity impacts.

Table 6.3-3. Failure Probabilities Due to Miscellaneous Events

Event	Failure Probability	Note

Derail	**1.0×10^{-5}**	**LLNL, adjusted, analogous to 6', 3° from horizontal**
Rollover	**1.0×10^{-5}**	**LLNL, adjusted, analogous to 6', 3° from horizontal**
Drop on	**1.0×10^{-5}**	**LLNL, adjusted** **10-metric-ton load onto container**
Tip over	**1.0×10^{-5}**	**LLNL, adjusted, analogous to** **13.1-foot drop plus slap-down**
Side impact from collision with rigid surface	**1.0×10^{-8}**	**Or value for low speed collision, whichever is greater (Table 6.3-4)** **Crane moving 20 ft/min**
Tilt down/up	**1.0×10^{-5}**	**LLNL, adjusted; Bounded by slap-down**

NOTE: LLNL = Lawrence Livermore National Laboratory.

Source: Attachment D.

Table 6.3-4 shows failure probabilities for various collision events for various containers as a function of impact speed. For each of the events, the collision speed, whether in miles per hour (mph) or feet per minute (fpm) is converted to feet per second (fps), then to an equivalent drop height in feet. The drop heights are very small compared with the drop heights for the modeled situations summarized in Table 6.3-2. The damage to a container, expressed in terms of strain, is roughly proportional to the impact energy, which is proportional to the drop height, as is readily seen from the following:

> Energy from drop = $mgh \propto Fs$ and $F \propto mg$, therefore, $s \propto h$, where s = strain, F = local force on container from drop, m = mass of container, h = drop height, and g = acceleration of gravity.

For drop heights other than those for the modeled situations presented in Table D3.4-1, failure probabilities can be estimated by shifting capacity curve to match the conservative failure probabilities listed in Table D3.4-1. The mean failure drop height, H_m, is found so that the probability of failure, P, is the value listed in Table D3.4-1 for the drop height, H_d, listed in Table D3.4-1.

$$P = \int_{-\infty}^{x} N(t)\,dt \quad and \quad x = \frac{H_d / H_m - 1}{COV} \qquad \text{(Eq. 17)}$$

where

P = Probability of failure for container dropped from height H_d

N(t) = Standard normal distribution with mean of zero and standard deviation of one

t = Variable of integration

H_d = Modeled drop height for which the failure probability has been determined

H_m = Median failure drop height of the failure drop height distribution such that the failure probability at the modeled drop height, H_d, is P

COV = Coefficient of variation = ratio of standard deviation to mean for strain capacity distribution, applied here to stress capacity or true tensile strength

The probabilities of failure for the collision cases listed in Table 6.3-4 are then determined using the above formula with H_m determined above and with H_d being the drop height corresponding to the collision speed as listed in Table 6.3-4.

Two-blocking events are also included in Table 6.3-4. The failure probabilities of these events are shown in *PEFA Chart.xls* included in Attachment H. The CTM, which lifts canisters, is designed such that drops from the height associated with two-blocking is very low probability and no higher than drops from normal operation. The design features that ensure this are: slide gate closure and two levels of shut-off switches as the normal lift height is exceeded, and a tension relief device that prevents over tensioning of hoist cables if the two-block height is reached. Transportation cask handling cranes are also equipped with the shut-off switches and the tension relief device.

During transfers by a CTM, a shear-type structural challenge was identified as a potential initiating event. This challenge would be caused, for example, by the spurious movement of the CTT from which the canister is extracted, before the canister is fully lifted inside the CTM shield bell. A bounding value of one is selected for the probability of failure of the transferred canister. This conservative estimate is used because the structural response of a canister to a shear-type structural challenge was not evaluated and its probability cannot be inferred from comparison with other structural challenges to the canister.

Table 6.3-4. Failure Probabilities for Collision Events and Two-Blocking

				Failure Probabilities for Various Container Types				
Collision Scenario	Speed	Velocity (ft/sec)	Equivalent Drop Height (ft)a	Transportation Cask	Canister	Waste Package	MCO	High-Level Radioactive Waste
Railcar	**2.5 (mph)**	**3.67**	**0.21**	**1.00E-08**				
Truck trailer	**2.5 (mph)**	**3.67**	**0.21**	**1.00E-08**				
Crane	**20 (ft/min)**	**0.33**	**0.00**	**1.00E-08**				
CTT	**10 (ft/min)**	**0.17**	**0.00**	**1.00E-08**	**1.00E-08**		**1.00E-08**	**1.00E-08**
ST	**2.5 (mph)**	**3.67**	**0.21**		**1.00E-08**		**1.00E-08**	**1.00E-08**
WPTT	**40 (ft/min)**	**0.67**	**0.01**		**1.00E-08**	**1.00E-08**	**1.00E-08**	**1.00E-08**
WP (in TEV)	**1.7 (mph)**	**2.49**	**0.10**			**1.00E-08**		
CTM	**20 (ft/min)**	**0.33**	**0.00**		**1.00E-08**		**1.00E-08**	**1.00E-08**
CTM	**40 (ft/min)**	**0.67**	**0.01**		**1.00E-08**		**1.00E-08**	**1.00E-08**
Two blocking				**1.00E-05**	**1.00E-05**	**NA**	**1.00E+00**	**6.70E-02**

NOTE: [a]**Values that are less than 0.005 are reported as 0.00.**
CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; DSTD = DOE standardized canister; ft = feet; MCO = multicanister overpack; min = minutes; mph = miles per hour; sec = seconds; ST = site transporter; TAD = transportation, aging, and disposal; TEV = transport and emplacement vehicle; WP =waste package; WPTT = waste package transfer trolley.

Source: Original

6.3.2.3 Probability of Canister Failure in a Fire

In addition to passive equipment failures as a result of structural loads, passive failures can also occur as a result of thermal loads such as exposure to fires or abnormal environmental conditions, for example, loss of HVAC cooling. The PCSA evaluates the probability of loss of containment (breach) due to a fire for several types of waste form containers, including: transportation casks containing uncanistered SNF assemblies, and canisters representative of TAD canisters, DPCs, DOE standardized canisters, HLW canisters, and naval SNF canisters.

The methods for analyzing thermally-induced passive failures are discussed in Section 4.3.2.2, and detailed in Attachment D. In summary, the probability of failure of a waste form container as a result of a fire is evaluated by comparing the demand upon a container (which represents the thermal challenges of the fire vis-à-vis the container), with the capacity of the container (which represents the variability in the temperature at which failure would occur). The demand upon the container is controlled by the fire duration and temperature, because these factors control the amount of energy that the fire could transfer to the container.

In response to a fire, the temperature of the waste form container under consideration increases as a function of the fire duration. The maximum temperature is calculated using a heat transfer model that is simplified to allow a probabilistic analysis to be performed that accounts for the variability of key parameters. The model accounts for radiative and convective heat transfers from the fire, and also for the decay heat from the waste form inside a container. The temperature evolution of waste form containers is analyzed based on a simplified geometry with a wall thickness that, for the range of waste form containers of interest in the PCSA, is representative or conservatively small. Specifically, two characteristic canister wall thicknesses are modeled: 0.5 inches, characteristic of some DPCs and other waste canisters; and 1.0 inches, the anticipated thickness of TAD canisters and naval SNF canisters. The wall thickness of a container is an important parameter that governs both container heating and failure. Other conservative and realistic modeling approaches are introduced in the heat transfer model, as appropriate. For example, fires are conservatively considered to engulf a container, regardless of the fact that a fire at the GROA may simply be in the same room as a container. When handled, TAD canisters, DPCs, DOE standardized canisters, HLW canisters and naval SNF canisters are enclosed within another SSC, for example a transportation cask, the shielded bell of a canister transfer machine, or a waste package. Therefore, a fire does not directly impinge on such canisters. In contrast, the external surface of a transportation cask containing uncanistered SNF may be impinged upon directly by the flames of the fire.

Accounting for the uncertainty of the key parameters of the fires and the heat transfer model, the maximum temperature reached by a waste form container, which represents the demand upon the container due to a fire, is characterized with a probability distribution. The distribution is obtained through Monte Carlo simulations.

To determine whether the temperature reached by a waste form container is sufficient to cause the container to fail, the fire fragility distribution curve for the container is evaluated. In the PCSA, this curve is expressed as the probability of breach of the container as a function of its temperature. Two failure modes are considered for a container that is subjected to a thermal challenge: creep-induced failure and limit load failure. Creep, the plastic deformation that takes place when a material is held at high temperature for an extended period under tensile load, is possible for long duration fires. Limit load failure corresponds to situations where the load exerted on a material exceeds its structural strength. This failure mode is considered because the strength of a container decreases as its temperature increases. The variability of the key parameters that can lead to a creep-induced failure or limit load failure is modeled with probability distributions. Monte Carlo simulations are then carried out to produce the fire fragility distribution curve for a container.

The probability of a waste form container losing its containment function as a result of a fire is calculated by running numerous Monte Carlo simulations in which the temperature reached by the container, sampled from the probability distribution representing the demand on the container, is compared to the sampled failure temperature from the fragility curve. The model counts the simulation result as a failure if the container temperature exceeds the failure temperature. Statistics based upon the number of recorded failures in the total number of simulations are used to estimate the mean of the canister failure probability.

Table 6.3-5 shows the calculated mean and standard deviation for the failure probability of a canister in the following configurations: a canister in a transportation cask, a canister in a waste package, and a canister in a shielded bell.

Table 6.3-5. Summary of Canister Failure Probabilities in Fire

Configuration[b]	Failure Probability	
	Mean	**Standard Deviation**
Thin-Walled[c] Canister in a Waste Package[a]	3.2×10^{-4}	5.7×10^{-5}
Thick-Walled[c] Canister in a Waste Package[a]	1.0×10^{-4}	2.2×10^{-5}
Thin-Walled Canister in a Transportation Cask	2.0×10^{-6}	1.4×10^{-6}
Thick-Walled Canister in a Transportation Cask	1.0×10^{-6}	1.0×10^{-6}
Thin-Walled Canister in a Shielded Bell	1.4×10^{-4}	2.6×10^{-5}
Thick-Walled Canister in a Shielded Bell	9.0×10^{-5}	$1.7 \times 10^{-5-}$

NOTE: [a] For the 5-DHLW/DOE SNF waste package, this probability applies only to the DOE HLW canisters located on the periphery of the waste package. The DOE SNF canister in the center of the waste package would not be heated appreciably by the fire.
[b] Configurations not addressed in this table include, any canister in a waste package that is inside the transfer trolley or any canister inside an aging overpack. In these configurations, the canister is protected from the fire by the massive steel transfer trolley or by the massive concrete overpack. Calculations have shown that the temperatures experienced by the canister in these configurations are well below the canister failure temperature, so that failures for these configurations can be screened. For conservatism, a screening conditional probability of 1×10^{-6} could be used.
[c] Naval SNF canisters are modeled as thick walled. Other canisters are modeled as thin walled.

Source: Attachment D, Table D2.1-9.

Note that no failure probability is provided for a bare canister configuration. The reason for this is that the canister is outside of a waste package or cask for only a short time. During that time, the canister is usually inside the shielded bell of the CTM. The preceding analysis addressed a fire outside the shielded bell. When in that configuration, the canister is shielded from the direct effects of the fire. A fire inside the shielded bell, which could directly heat the canister, is not considered to be credible for two reasons. First, the hydraulic fluid used in the CTM equipment is non-flammable and no other combustible material could be present inside the bell to cause a fire. Second, the annular gap between the canister and the bell is only 3 inches wide, but is approximately 27 feet long. Given this configuration, it is unlikely that there would be sufficient inflow of air to sustain a large fire that could heat a significant portion of the canister wall. There may be sufficient inflow to sustain a localized fire, but such a fire would not be adequate to heat the canister to failure.

The canister is also outside of a cask, waste package, or shielded bell as it is being moved from a cask into the shielded bell or from the shielded bell into a waste package. The time during which the canister would be in this configuration is extremely short, a matter of minutes, so a fire that occurs during this time is extremely unlikely. In addition, because the gap between the top of the waste package or cask and ceiling of the transfer cell is generally much shorter than the height of the canister, only a small portion of the canister surface would be exposed to the fire. Furthermore, this exposure would only be for the short time that the canister was in motion.

In addition, monolithic borosilicate glasses incorporating HLW do not appear to have the potential to release any significant amount of non-volatile radionuclides. These materials would have been heated to temperatures exceeding those anticipated for most fire situation during formation and are not anticipated to undergo any chemical change under fire conditions (Ref. 2.2.9, Section II).

For these reasons, failure of a bare canister was not considered credible and is not explicitly modeled in the PCSA.

6.3.2.4 Probability of Loss of Containment from Heatup

In addition to fire-related passive failures, the PCSA considered other passive equipment failures due to abnormal thermal conditions. The thermal event of greatest concern for the surface facilities is loss of HVAC cooling. If HVAC cooling is lost, the ambient temperature in the facility will increase. This increase would be particularly significant for relatively small enclosures such as the transfer cells.

A series of bounding calculations was performed to determine the maximum temperature that could be reached by a canister following loss of HVAC cooling (Ref. 2.2.14). These calculations consider a range of decay heat levels and a loss of cooling for 30 days, which is consistent with NUREG-0800 (Ref. 2.2.64, Section 9.2.5). These analyses indicate that the canister temperature would remain well below 500°C (773°K) (Ref. 2.2.14). This temperature is hundreds of degrees below the temperature at which the canister would fail (Attachment D, Figures D2.1-4 and D2.1-5). For that reason, canister failure due to a loss of HVAC is physically unrealizable and considered beyond Category 2.

6.3.2.5 Probability of Loss/Degradation of Shielding

Loss or degradation of shielding probabilities are summarized in Table 6.3-6.

Shielding of a waste form that is being transported inside the GROA is accomplished by several types of shielded containers, including: transportation casks, shielded transfer casks, aging overpacks, shielded components of a WPTT, and shielded components of a TEV. In addition to a shielding function, sealed transportation casks and shielded transfer casks exert a containment function.

A structural challenge may cause shielding degradation or shielding loss. Loss of shielding occurs when an SSC fails in a manner that leaves a direct path for radiation to stream, for example, as a result of a breach. Degradation of shielding occurs when a shielding SSC is not breached but its shielding function is degraded. In the PCSA, a shielding degradation probability after a structural challenge is derived for those transportation casks that employ lead for shielding. Finite-element analyses on the behavior of transportation casks subjected to impacts associated with various collision speeds, reported in NUREG/CR-6672 (Ref. 2.2.80), indicate that lead slumping after an end impact could result in a reduction of shielding; transportation casks without lead are not susceptible to such shielding degradation. This information is used in Attachment D to derive the shielding degradation probability of a transportation cask at drop heights characteristic of crane operations. The distribution is developed for impacts on surfaces made of concrete, which compare to the surfaces onto which drops could occur at the GROA. No impact limiter is relied upon to limit the severity of the impact. Conservatively, the distribution is applied to transportation casks and also shielded transfer casks, regardless of whether or not they use lead for shielding. Thus, for containers that have both a containment and shielding function, the PCSA considers a probability of containment failure (which is considered to result in a concurrent loss of shielding), and also a probability of shielding degradation (which is associated with those structural challenges that are not sufficiently severe to cause loss of containment). Table 6.3-6 displays the resulting shielding degradation probabilities for transportation casks and shielded transfer casks after a structural challenge. Given that there is significant conservatism in the calculation of strain and the uncertainty associated with the fragility (strength), the resulting estimates include uncertainties and are considered conservative.

Shielding loss is also considered to potentially affect an aging overpack subjected to a structural challenge, if the waste form container inside does not breach. Given the robustness of aging overpacks, a shielding loss after a 3-ft drop height is calculated to have a probability of 5×10^{-6} per aging overpack impact, based upon the judgment that this probability may be conservatively related to but lower than the probability of breach of an unprotected waste form container inside the aging overpack (Attachment D). If the structural challenge is sufficiently severe to cause the loss of containment (breach) of the waste form container inside the aging overpack, the loss of the aging overpack shielding function is considered guaranteed to occur.

A CTM provides shielding with the shield bell, shield skirt, and associated slide gates. Also, the CTM is surrounded by shield walls and doors, which are unaffected by structural challenges resulting from internal random initiating events. Therefore, such challenges leave the shielding function intact.

A WPTT that transports a waste package is considered to lose its shielding function if it is subjected to a structural challenge sufficiently severe to cause the breach of the sealed waste package, or, when the waste package is not yet sealed, the breach of one or more canisters inside, as applicable. Conversely, if the structural challenge is not sufficiently severe to cause a canister or waste package breach, it is postulated to also be sufficiently mild to leave the shielding function intact.

Similarly, a TEV that transports a waste package is considered to lose its shielding function if it is subjected to a structural challenge sufficiently severe to cause the breach of the waste package. Conversely, if the structural challenge is not sufficiently severe to cause a waste package breach, it is postulated to also be sufficiently mild to leave the shielding function of the TEV intact.

The PCSA treats the degradation or loss of shielding of an SSC due to a thermal challenge as described in the following paragraphs:

If the thermal challenge causes the loss of containment (breach) of a canister, the SSC that provides shielding and in which the canister is enclosed is considered to have lost its shielding capability. The SSC providing shielding may be, for example, a WPTT. A transportation cask containing uncanistered SNF is also considered to have lost its shielding if it has lost its containment function.

If the thermal challenge is not sufficiently severe to cause a loss of containment function, it is nevertheless postulated that it will cause shielding loss of the transportation cask, shielded transfer cask, canister transfer machine, cask transfer trolley, waste package transfer trolley, or TEV affected by the thermal challenge and in which the waste form container is enclosed. This is because the neutron shield on these SSCs is made of a polymer which is not anticipated to withstand a fire without failing. Note, however, that the degradation of gamma shielding of these SSCs is unlikely to be affected by a credible fire. Although credible fires could result in the lead melting in a lead-sandwich transportation cask, there is no way to displace the lead, unless the fire is accompanied by a puncture or rupture of the outer steel wall of the cask. Preliminary calculations were unable to disprove the possibility of hydraulic failure of the steel encasing due to the thermal expansion of molten lead, so loss of gamma shielding for steel-lead-steel transportation casks engulfed in fire is postulated. Conservatively, in the PCSA, transportation casks and shielded transfer casks are postulated to lose their shielding function with a probability of one, regardless of whether or not they use lead for shielding.

Aging overpacks made of concrete are not anticipated to lose their shielding function as a consequence of a fire because the type of concrete used for aging overpacks is not sensitive to spallation. In addition, it is likely that the aging overpacks will have an outer steel liner. For these reasons, a loss of aging overpack shielding in a fire has been screened from consideration in the PCSA.

Table 6.3-6. Probabilities of Degradation or Loss of Shielding

	Probability	Note
Sealed Transportation cask and shielded transfer casks shielding degradation after structural challenge	**1×10^{-5}**	**Attachment D, Section D3.4.**
Aging overpack shielding loss after structural challenge	**5×10^{-6}**	**Attachment D, Section D3.4**
CTM shielding loss after structural challenge	**0**	**Structural challenges sufficiently mild to leave the shielding function intact**
WPTT shielding loss after structural challenge	**0**	**Structural challenges sufficiently mild to leave the shielding function intact**
TEV shielding loss (shield end)	**0**	**Structural challenges sufficiently mild to leave the shielding function intact**
Shielding loss by fire for waste forms in transportation casks or shielded transfer casks	**1**	**Lead shielding could potential expand and degrade. This probability is conservatively applied to transportation casks and STCs that do not use lead for shielding.**
Shielding loss by fire for aging overpacks, CTM shield bell, and WPTT shielding	**0**	**Type of concrete used for aging overpacks is not sensitive to spallation; Uranium used in CTM shield bell and WPTT shielding does not lose its shielding function as a result of a fire.**

NOTE: CTM = canister transfer machine; STC = shielded transfer cask; TEV = transport and emplacement vehicle; WPTT = waste package transfer trolley.

Source: Attachment D, Table D3.4-1.

6.3.2.6 Probability of Other Fire-Related Passive Failures

In addition to the canisters, other passive equipment could fail as a result of a fire. For the PCSA, only failures that would result in a radionuclide release or radiation exposure are considered.

6.3.2.7 Application to Event Sequence Models

Table 6.3-7 summarizes passive failure events needed for the event sequence modeling. The values are either specifically developed in Attachment D, or are values from bounding events. Probabilities for some events were obtained by extrapolation from developed probabilities as described in this section or in Attachment D. The derivation of all passive failure probabilities is described in Attachment D and shown in PEFA Chart.xls included in Attachment H.

It should be noted that Table 6.3-7 addresses all passive event failures for the various waste form configurations. Table 6.3-8 identifies the specific passive failure basic events used in event sequence modeling and quantification for the RF. The probability of each basic event is based on one of the values presented in Tables 6.3-2 through 6.3-7.

Table 6.3-7. Summary of Passive Event Failure Probabilities

	10 T dropped on container	Container vertical drop from normal operating height	Container 30-foot vertical drop	Container 45-foot vertical drop	6-foot Horizontal Drop, Rollover	2.5 mph Flat side impact/ collision	2.5 mph Localized side impact/ collision	9 mph Flat side impact/ collision	2.5 mph end-to-end Collision	9 mph end-to-end Collision	Slapdown (bounds tipover)	Thin-Walled Canister Fire	Thick-Walled Canister Fire
Loss of Containment													
Canister in Transport Cask	1.E-05	1.E-05	1.E-05	N/A	1.E-05	1.E-08	1.E-08	1.E-08	1.E-08	1.E-08	1.E-05	2.E-06	1.E-06
Transport Cask with Bare Fuel	1.E-05	1.E-05	1.E-05	N/A	1.E-05	1.E-08	1.E-08	1.E-08	1.E-08	1.E-08	1.E-05	5.E-02[1]	6.E-03[2]
Canister	1.E-05	1.E-05	1.E-05	N/A	N/A	N/A	N/A	N/A	N/A	N/A	1.E-05	N/A	N/A
Waste Package	1.E-05	N/A	N/A	N/A	1.E-05	1.E-08	N/A	1.E-08	1.E-05	1.E-05	no challenge	3.E-04	1.E-04
Bare MCO	N/A	1.E-01	~ 1	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Bare DOE Standard Canister	1.E-05	1.E-05	1.E-03	N/A	N/A	N/A	N/A	N/A	1.E-05	1.E-05	N/A	N/A	N/A
Bare High Level Waste Canister	N/A	3.E-02	7.E-02	~ 1	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Canister in Shield Bell	N/A	1.E-05	N/A	N/A	N/A	1.E-08	N/A	N/A	N/A	N/A	N/A	1.E-04	9.E-05
Canister in AO	1.E-05	1.E-05	N/A	N/A	N/A	1.E-08	1.E-08	1.E-08	N/A	N/A	1.E-05	1.E-06	1.E-06
Loss of Shielding													
Transport Cask	1.E-05	1.E-05	1.E-05	N/A	1.E-05	1.E-08	1.E-08	1.E-08	1.E-08	1.E-08	1.E-05	~ 1	~ 1
Aging Overpack	1.E-05	5.E-06	N/A	N/A	N/A	1.E-05	1.E-05	1.E-05	1.E-05	1.E-05	1.E-05	~ 0	~ 0
TEV, CTM, WPTT	No challenge	no challenge	N/A	N/A	no challenge	no challenge	N/A	no challenge	no challenge	no challenge	no challenge	~ 0	~ 0

NOTE: 1 Truck cask
2 Rail cask
3. Represents passive event failure probabilities for a drop of a HLW canister onto another HLW canister.
N/A = not applicable, no scenarios identified.

Source: Attachment D

Table 6.3-8. Passive Failure Basic Events used in RF Event Sequence Analysis

Basic Event Name	Basic Event Description	BE Value	Condition
Passive Failures from Mechanical Events			
CAN-FAIL-SD-IMPACT	Canister fails due to collision	1.00E-08	2.5-mph flat side impact/collision with canister in TC
CAN-IN-AO-DROP	Canister Failure from miscellaneous impacts	1.00E-05	AO container drop
CAN-IN-AO-DROPON	Canister Failure from Drop, Drop On, Roll or Tip	1.00E-05	10 T dropped on container
CAN-IN-AO-IMPACT	Canister Failure from miscellaneous impacts	1.00E-08	2.5-mph Localized side impact/collision
CAN-IN-AO-ROLLOVER	Canister Failure from miscellaneous impacts	1.00E-05	AO container drop
CAN-IN-AO-TIP	Canister Failure from miscellaneous impacts	1.00E-05	Slapdown (bounds tipover)
CANISTER-FAIL-CTM-2BLOCK	Canister Failure due to CTM 2 Block Drop	1.00E-05	30 ft Canister drop
DPC_FAIL_IN_TC	Canister Failure	1.00E+00	DPC fails given transportation cask fails
DPC-CAN-IN-AO-COLL	Canister Failure from Collision	1.00E-08	2.5-mph Flat side impact/collision with canister in AO
DPC-FAIL-CTM-IMPACT	Canister Failure	1.00E-08	2.5-mph Flat side impact/collision with canister in CTM
DPC-FAIL-NO-CASK	Canister Failure	1.00E-05	Canister drop or 10 ton dropped on canister in CTM
DPC-FAIL-NO-CASK-IMP	Canister Failure	1.00E-08	2.5-mph flat side impact/collision with canister in TC
DPC-FAIL-SPURMOVE	Canister Failure	1.00E+00	Spurious movement of CTT or ST during unloading or unloading a canister
TAD_FAIL_IN_TC	Canister Failure	1.00E+00	TAD fails given transportation cask fails
TAD-CAN-IN-AO-COLL	Canister Failure from ST Collision	1.00E-08	2.5-mph Flat side impact/collision with canister in AO
TAD-FAIL-CTM-IMPACT	Canister Failure	1.00E-08	2.5-mph Flat side impact/collision with canister in Shielded Bell
TAD-FAIL-NO-CASK	Canister Failure	1.00E-05	Canister drop or 10 ton dropped on canister in CTM
TAD-FAIL-NO-CASK-IMP	Canister Failure	1.00E-08	2.5-mph flat side impact/collision with canister in TC
TAD-FAIL-SPURMOVE	Canister Failure	1.00E+00	Spurious movement of CTT or ST during unloading or unloading a canister

Table 6.3-8. Passive Failure Basic Events used in RF Event Sequence Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Condition
Passive Failures from Mechanical Events			
TCASK	Transportation Cask Fails	1.00E-08	2.5-mph flat side impact/collision with canister in TC
TCASK-2BLOCK	Cask Failure due to 2 Block Drop	1.00E-05	30 ft Drop
TCASK-FAIL-COLL	Transportation Cask Fails	1.00E-08	2.5-mph flat side impact/collision with canister in TC on HCTT
TCASK-FAIL-ROLLOVER	TC fails due to rollover	1.00E-05	6-foot horizontal drop, rollover with canister in TC on HCTT
TCASK-MISC-DROP	TC Fails from Drop	1.00E-05	TC drop during handling and transfer to CTT
TCASK-MISC-DROPON	TC fails due load drop onto cask	1.00E-05	10 ton dropped on container during handling and transfer to CTT
TCASK-MISC-IMP	TC fails from side Impacts	1.00E-08	2.5-mph Localized side impact/collision during handling and transfer to CTT
TCASK-SPURMOVE	TC Fails due to Spurious Movement	1.00E-08	2.5-mph flat side impact/collision to TC
TCASK-TIPOVER	Transportation Cask Fails due Tipover	1.00E-05	Slapdown (bounds tipover)
Shielding Failures			
CTM-SHIELDING	CTM shielding fails	0.00E+00	Loss of CTM shielding during CTM handling activities
TCASK-SHIELDING-DROP	Transportation Cask Shielding Fails	1.00E-05	Loss of cask shielding from 15 ft drop during handling and transfer to CTT
TCASK-SHIELDING-IMP	Transportation Cask Shielding Fails	1.00E-08	2.5-mph flat side impact/collision to TC
TCASK-SHIELDING	Transportation Cask Shielding Fails	1.00E-05	6-foot horizontal drop, rollover with canister in TC on HCTT
TCASK-SHIELDING-2BLK	TC shielding fails from two block drop	1.00E-05	Two block drop of TC during cask handling and movement to CTT

NOTE: AO = aging overpack; CTM = canister transfer machine; DPC = dual-purpose canister; TAD = transportation, aging and disposal; TC = transportation cask.

Source: Original

6.3.3 Miscellaneous Data

Split fractions for specific fire scenarios are derived from the exposure frequencies detailed in Section 6.5 and Attachment F. Table 6.3-9 identifies the frequency associated with a waste type in a specific configuration and location with or without diesel fuel present.

Table 6.3-10 provides details on how specific residence time fractions were developed for the IHF fire event sequence analysis. The formulas use the index notation in Table 6.3-9.

Data that is not defined as Active Component Reliability Data (Section 6.3.1) or Passive Equipment Failure Data (Section 6.3.2), but are used in the reliability analysis for this facility are listed in the following Table 6.3-11.

Table 6.3-9. Fire Analysis for Wastes Types in Specific Configuration

Location	Mean Fire Initiation Frequency		Container Type or Location
	DPC	TAD	
Localized fire			
Vestibule/Lid Bolting Room (Diesel Present)	8.1E-07	8.1E-07	AO
Loading Room (Diesel Present)	3.5E-07	3.5E-07	AO
Vestibule/Preparation Area (Diesel Present)	1.9E-06	4.6E-07	TC
Preparation Area (No Diesel Present)	1.2E-05	3.1E-06	TC
Preparation Area	2.1E-06	9.1E-07	TC
Cask Unloading Room	3.9E-07	3.9E-07	TC
Transfer Room	1.1E-07	1.1E-07	CTM
Large Fire			
Large Fire Threatens TC/TAD (No Diesel)	---	1.1E-05	TC
Large Fire Threatens TC/TAD or TC/DPC, Diesel Present	8.6E-07	8.6E-07	TC
Large Fire Threatens TC/DPC, No Diesel	1.6E-05	---	TC
Large Fire Threatens TC/DPC, No Diesel	1.2E-05	---	TC
Large Fire Threatens TC/DPC, Diesel Present	1.8E-06	---	TC
Large Fire Threatens TC/DPC, No Diesel	1.1E-05	---	TC
Large Fire Totals For Waste Forms in Various Containers			
Large Fire Threatens Waste Form in TC	4.2E-05	1.2E-05	TC
Large Fire Threatens Waste Form in CTM	4.9E-07	4.9E-07	CTM
Large Fire Threatens Waste Form in AO, Diesel Present	6.1E-06	6.1E-06	AO
Total for Large Fire Threatens Waste Form in RF	4.9E-05	1.9E-05	

NOTE: AO = aging overpack; CTM = canister transfer machine; DPC = dual-purpose canister; RF = Receipt Facility; TAD = transportation, aging and disposal canister; TC = transportation cask.

Source: Table 6.5-4

Table 6.3-10. Split Fractions for Waste Types in Various Configurations

	Mean	Split Fraction
TAD Calculation		
Large Fire Threatens TAD in TC	1.2E-05	6.5E-01
Large Fire Threatens TAD in CTM	4.9E-07	2.6E-02
Large Fire Threatens TAD in AO	6.1E-06	3.3E-01
Total	1.9E-05	
DPC Calculation		
Large Fire Threatens DPC in TC	4.2E-05	8.6E-01
Large Fire Threatens DPC in CTM	4.9E-07	1.0E-02
Large Fire Threatens DPC in AO	6.1E-06	1.3E-01
Total	4.9E-05	

NOTE: AO = aging overpack; CTM = canister transfer machine; DPC = dual-purpose canister; RF = Receipt Facility; TAD = transportation, aging and disposal canister.

Source: Original

Table 6.3-11. Miscellaneous Data Used In the Reliability Analysis

Basic Event Name	Basic Event Description	BE Value	Bases	References
200-#EEE-LDCNTRA-BUA-MTN	ITS Load Center Train A OOS for Maintenance	1.025E-004	Probability equipment will be in maintenance over preclosure period as determined by HRA.	Section 6.4
200-#EEE-LDCNTRA-BUA-ROE	Failure to Restore ITS Load Center Train A post maint	1.025E-005	Probability equipment will not be restored following maintenance over preclosure period as determined by HRA.	Section 6.4
200-#EEE-LDCNTRB-BUA-MTN	ITS Load Center Train B OOS for Maintenance	1.025E-004	Probability equipment will be in maintenance over preclosure period as determined by HRA.	Section 6.4
200-#EEE-LDCNTRB-BUA-ROE	Failure to Restore ITS Load Center Train B post maint	1.025E-005	Probability equipment will not be restored following maintenance over preclosure period as determined by HRA.	Section 6.4
200-CR-CASK-UNLOADING	Canister is Exposed During Mid-Unloading	1.000E+000	Probability that canister will be partially unshielded during unloading and loading operations	Section 6.4
200-CSKPREPLIFTNUMBER	Number of object Lifts for Cask Prep	1.000E+000	Total number of lifts by 200-ton crane during transportation cask preparation.	Section 6.4
200-CTMOBJLIFTNUMBERD	Number of objects lifted by CTM during DPC canister transfer	1.000E+000	Number of lifts required by the CTM to transfer a DPC	Section 6.4

Table 6.3-11. Miscellaneous Data Used In the Reliability Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Bases	References
200-CTMOBJLIFTNUMBERT	Number of objects lifted by CTM during TAD canister transfer	1.000E+000	Number of lifts required by the CTM to transfer a TAD	Section 6.4
200-DPCPREPLIFTNUMBER	Number of object Lifts for DPC Prep	3.000E+000	There are three crane lifts associated with the preparation of the DPC in the Cask Preparation Area. Therefore, a value of 3 is assigned to this basic event.	Section 6.4
200-EXCESSIVE-WIND-SPEED	Sustained Wind Exceeds 40 mph & Gust to 90 mph	4.700E-003	Sustained wind with speed exceeding 40 mph and gust to 90 mph has an estimated frequency of 5.7E-02 per yr and with a mission time of 720 hours, the probability of such an occurrence is 4.7E-3.	Ref. 2.2.26
200-FIRE-SUPPRESSION	Inadvertent Actuation of the Fire suppression System	5.000E-007	Fire suppression system inadvertently activates during normal IHF operations (no fire)	Section 6.2.2.9
200-LIFTS-PER-DPC-CAN	Number of Lifts per DPC Canister	1.000E+000	HRA determination of the number of lifts associated with DPC canisters in CTM.	Section 6.4
200-LIFTS-PER-TAD-CAN	Number of Lifts per TAD Canister	1.000E+000	HRA determination of the number of lifts associated with TAD canisters in CTM.	Section 6.4
200-MODERATOR-IN-FIRE	Water Moderator Enters Cask	1.000E+000	Conservative estimate of probability of water entering a cask from fire suppression during a fire	N/A
200-OIL-MODERATOR	Oil Moderator Sources in RF (Gear Boxes)	9.000E-005	Section 6.0	Section 6.0
200-PWR-LOSS	Loss of Site Power	4.100E-006	Commercial power reliability requirement	N/A
200-SPMRC-MILES-IN-RF	Miles Traveled in RF	4.000E-002	(Site) prime mover travel distance on rails inside the RF.	Ref. 2.2.24
200-TRANSCTTLIFTNUMBER	Number of Crane Lifts	3.000E+000	Total number of crane lifts.	
200-TRANSNSCTTLIFTNUMBER	Number of Crane Lifts	1.000E+000	Total number of lifts by the 200-ton crane during transfer of a TC from conveyance to preparation station.	Section 6.4
200-TRANSSTANDLIFTNUMBER	Crane Lifts with sling lift	2.000E+000	Number of lifts performed by sling lift.	Section 6.4

Table 6.3-11. Miscellaneous Data Used In the Reliability Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Bases	References
200-UPENDOBJLIFTNUMBER	Number of object lifts	3.000E+000	Number of crane lifts performed during upending TC in Cask Preparation Area.	Section 6.4
200-VCOO-NITS-PWR-FAILS	Non-ITS Power Failure to RF Supply Fan	2.991E-003	Commercial power reliability requirement	N/A
200-VCTO-CONTDOORS-OPEN	Vestibule Doors Open receipt or Export from RF	1.000E+000	House event set to true to account for the probability that a vestibule door is open at the time of release.	N/A
200-VCTO-DRS0000-DRS-OPN	Vestibule Door Open During Receipt/Export	1.600E-004	Probability that vestibule doors are open over preclousure period as determined by HRA	Section 6.4
26D-#EEY-ITSDG-A-#DG-MTN	ITS DG A OOS Maintenance	1.950E-003	Probability equipment will be in maintenance over preclosure period as determined by HRA.	Section 6.4
26D-#EEY-ITSDG-A-#DG-RSS	Failure to properly return ITS DG A to service	1.950E-004	Probability equipment will not be restored following maintenance over preclosure period as determined by HRA.	Section 6.4
26D-#EEY-ITSDG-B-#DG-MTN	ITS DG B OOS Maintenance	1.950E-003	Probability equipment will be in maintenance over preclosure period as determined by HRA.	Section 6.4
26D-#EEY-ITSDG-B-#DG-RSS	Failure to properly restore ITS DG-B to service	1.950E-004	Probability equipment will not be restored following maintenance over preclosure period as determined by HRA.	Section 6.4
CELL-DOOR	Door remains on tracks and does not fall onto CTT/ST	1.000E+000	Value used in analysis	
DPC	Number of DPCs	3.460E+002	Total number of DPCs received at RF over preclosure period.	Ref. 2.2.27
DPCS	Number of DPCs processed through the RF during preclosure period	3.460E+002	Total number of DPCs received at RF over preclosure period.	Ref. 2.2.27
DPCS-TADS	Number of DPCs & TADs processed through the RF during preclosure period	7.324E+003	Total number of DPCs and TADs processed at RF over preclosure period.	Ref. 2.2.27
LOSP	Loss of offsite power	2.990E-003	Commercial power reliability requirement	N/A

Table 6.3-11. Miscellaneous Data Used In the Reliability Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Bases	References
LOSP-4	Failure of Off Site Power	4.100E-006	Commercial power reliability requirement	N/A
TAD	Number of TADs	6.976E+003	Total number of TADs received at RF over preclosure period.	Ref. 2.2.27
TADS	Number of TADs processed through the RF during preclosure period	6.976E+003	Total number of TADs received at RF over preclosure period.	Ref. 2.2.27
ESD12-DFIRE-IN-PREP-DPC	TC with DPC in vestibule/prep area threatened by diesel fire	1.850E-006	Localized Fire Threatens a TC containing a DPC in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-9
ESD12-DFIRE-IN-PREP-TAD	TC with TAD in vestibule/prep area threatened by diesel fire	4.600E-007	Localized Fire Threatens a TC containing a TAD in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-9
ESD12-DPC-IN-LG-FIRE	DPC threatened by large fire	4.830E-005	A large fire threatens a container with a DPC in the facility. Variations in container type failure probabilities are accounted for by assigning split fractions.	Section 6.3, Table 6.3-9 and 6.3-10
ESD12-FIRE-CTM-DPC	Fire in transfer area threatens DPC	1.100E-007	Localized Fire Threatens a TC containing a DPC in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-9
ESD12-FIRE-CTM-TAD	Fire in transfer area threatens TAD	1.100E-007	Localized Fire Threatens a TC containing a TAD in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-BOLT-DPC	DPC threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a DPC in the Lid Bolting Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-BOLT-TAD	TAD threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a TAD in the Lid Bolting Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-LOAD-DPC	DPC threatened by fire in loading room	3.500E-007	Localized Fire Threatens a TC containing a DPC in the Loading Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-9

Table 6.3-11. Miscellaneous Data Used In the Reliability Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Bases	References
ESD12-FIRE-IN-LOAD-TAD	TAD threatened by fire in loading room	3.500E-007	Localized Fire Threatens a TC containing a TAD in the Loading Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-PREP-DPC	DPC in TC threatened by fire in prep area	1.200E-005	Localized Fire Threatens a TC containing a DPC in the Preparation Area with diesel present	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-PREP-TAD	TAD in TC threatened by fire in prep area	3.100E-006	Localized Fire Threatens a TC containing a TAD in the Preparation Area with diesel present	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-PREPCT-DPC	DPC in TC threatened by fire in prep area	2.100E-006	Localized Fire Threatens a TC containing a DPC in the Preparation Area	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-PREPCT-TAD	TAD in TC threatened by fire in prep area	9.100E-007	Localized Fire Threatens a TC containing a TAD in the Preparation Area	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-UNLD-DPC	DPC threatened by fire in unloading room	4.000E-007	Localized Fire Threatens a TC containing a DPC in the Unloading Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-9
ESD12-FIRE-IN-UNLD-TAD	TAD threatened by fire in unloading room	3.900E-007	Localized Fire Threatens a TC containing a TAD in the Unloading Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-9
ESD12-TAD-IN-LG-FIRE	TAD threatened by large fire	1.850E-005	A large fire threatens a container with a TAD in the facility. Variations in container type failure probabilities are accounted for by assigning split fractions.	Section 6.3, Table 6.3-9 and 6.3-10

NOTE: CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; HRA = human reliability analysis; IHF = Initial Handling Facility; ITS = important to safety; RF = Receipt Facility; ST = site transporter; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

6.4 HUMAN RELIABILITY ANALYSIS

The PCSA has emphasized human reliability analysis because the waste handling processes include substantial interactions between equipment and operating personnel. If there are human interactions that are typically associated with the operation, testing, calibration, or maintenance of a certain type of SSC (e.g., drops from a crane when using slings) and this SSC has been treated using industry-wide data per Attachment C, then human failure events may be implicit in the reliability data. The analyst is tasked with determining whether that is the case. Otherwise, the analyst includes explicit identification, qualitative modeling, and quantification of HFEs, as described in this section. The methodology applied is provided in Section 4.3.4, and the detailed description of the HRA is presented in Attachment E.

6.4.1 HRA Scope

The scope of the HRA is established in order to focus the analysis on the issues pertinent to the goals of the overall PCSA. Thus, the scope is as follows:

1. HFEs are only considered if they contribute to a scenario that has the potential to result in a release of radioactivity, a criticality event, or a radiation exposure to workers. Such scenarios may include the need for mitigation of radionuclides, for example, provided by the confinement HVAC system.

2. Pursuant to the above, the following types of HFEs are excluded:

   A. HFEs resulting in standard industrial injuries (e.g., falls)

   B. HFEs resulting in the release of hazardous nonradioactive materials, regardless of amount

   C. HFEs resulting solely in delays to or losses of process availability, capacity, or efficiency.

3. The identification of HFEs is restricted to those areas of the facility that handle waste forms and only during the times that waste forms are being handled (e.g., HFEs are not identified for the Cask Preparation Room during the export of empty transportation casks).

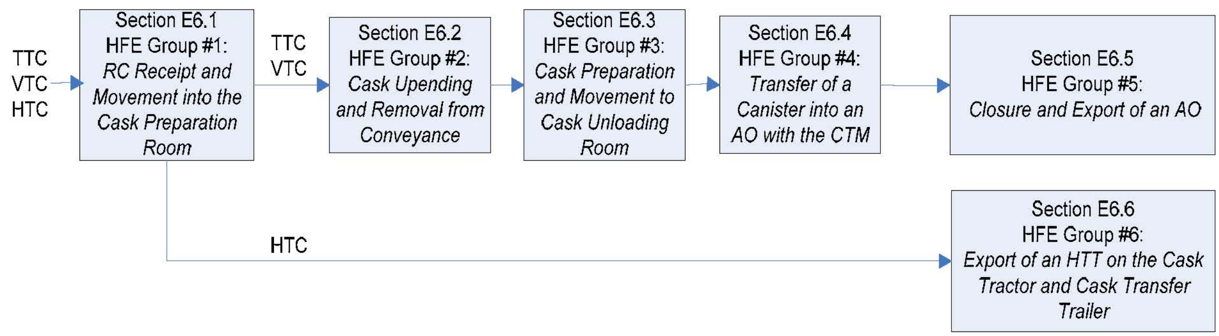
4. The exception to #3 is that system-level HFEs are considered for support systems (e.g., electrical power for confinement HVAC) when those HFEs could result in a loss of a safety function related to the occurrence or consequences associated with the events specified in #1.

5. Post-initiator recovery actions (as defined in Section E5.1.1.1) are not credited in the analysis; therefore HFEs associated with them are not considered.

6. In accordance with Section 4.3.10.1 (on boundary conditions of the PCSA), initiating events associated with conditions introduced in SSCs before they reach the site are not, by definition of 10 CFR 63.2 (Ref. 2.3.2), within the scope of the PCSA nor, by extension, within the scope of the HRA.

6.4.2 Base Case Scenarios

The first step in this analysis is to describe the RF operations in sufficient detail such that the human reliability analysts can identify specific deviations that would lead to a radiation release, a direct exposure, or a criticality event. To do this, the RF operations were broken into six separate operational steps, as depicted in Figure 6.4-1.



NOTE: AO = aging overpack; CTM = canister transfer machine; HFE =human failure event; HTC = a transportation cask that is never upended; RC = railcar; TTC = a transportation cask that is upended using a tilt frame; VTC = a transportation cask that is upended on a railcar.

Source: Original

Figure 6.4-1. RF Operations

The base case scenario for each HFE group represents a realistic description of expected facility, equipment, and operator behavior for the selected operation. These scenarios are created from discussions between the human reliability analysts, other PCSA analysts, and personnel from engineering and operations. In addition to a detailed description of the operation itself, these base case scenarios include a brief description of the initial conditions and relevant equipment features (e.g., interlocks). The relationship between these HFE groups and the corresponding PFD nodes and ESDs are mapped in Attachment E, Table E6.0-1.

6.4.3 Identification of Human Failure Events

There are many possible human errors that could occur at YMP the effects of which might be significant to safety. Human errors, based upon the three temporal phases used in PRA modeling, are categorized as follows:

- Pre-initiator HFEs
- Human-induced initiator HFEs
- Post-initiator HFEs[1]:
  - Non-recovery
  - Recovery.

Each of these types of HFEs is defined in Attachment E, Section E5.1.1.1. The PCSA model was developed and quantified with pre-initiator and human-induced initiator HFEs included in the model. The safety philosophy of waste handling operations is that an operator need not take any action after an initiating event and there are no actions identified that could exacerbate the consequences of an initiating event. This stems from the definitions and modeling of initiating events and subsequent pivotal events as described in Section 6.1 and Attachment A. All initiating events are proximal causes of either radionuclide release or direct exposure to

[1] Terminology common to nuclear power plants refer to post-initiator non-recovery events as Type C events and recovery events as Type CR events.

personnel. With respect to the latter, personnel evacuation was not considered in reducing the frequency of direct exposure but personnel action could cause an initiating event. With respect to the former, pivotal events address containment integrity, confinement availability, shielding integrity, and moderator availability that have no post-initiator human interactions. Containment and shielding integrity are associated only with the physical robustness of the waste containers. Confinement availability is associated with a continuously operating HVAC and the status of equipment confinement doors. Human interactions for HVAC are pre-initiator. Human actions for shielding are associated the with the initiator phase. Moreover, recovery post-initiator HFEs were not identified and not relied upon to reduce event sequence frequency. Thus, the focus of the HRA task is to support the other PCSA tasks to identify these two HFE phases.

Pre-Initiator HFEs

Pre-initiators are identified by the system analysts when modeling fault trees during the system analysis task. Special attention is paid to the possibility that an error can be repeated in similar redundant components or trains, leading to a human CCF.

Human-Induced Initiator HFEs

Human-induced initiator HFEs are identified through an iterative process whereby the human reliability analysts, in conjunction with other PCSA analysts and engineering and operations personnel, meet and discuss the design and operations of the facility and the SSCs in order to appropriately model the human interface. This iterative process began with the HAZOP evaluation, the MLD and event sequence development, and the event tree and fault tree modeling, and it culminated in the preliminary analysis and incorporation of HFEs into the model. Included in this process is an extensive information collection process where industry data for potential vulnerabilities and HFE scenarios are reviewed. The following sources were examined:

- *A Survey of Crane Operating Experience at U.S. Nuclear Power Plants from 1968 – 2002*, NUREG-1774 (Ref. 2.2.52)
- *Control of Heavy Loads at Nuclear Power Plants*, NUREG-0612 (Ref. 2.2.62)
- Naval Facilities Engineering Command Internet Web Site, Navy Crane Center (NCC). The database includes the following information:
  - NCC Quarterly Reports ("Crane Corner") 2001 through 2007
  - NCC Fiscal Year 2006 Crane Safety Reports (covers fiscal year 2001 through 2006)
  - NCC Fiscal Year 2006 Audit Report.
- DOE Occurrence Reporting and Processing System (ORPS) Internet Web Site, Operational Experience Summaries (2002 through 2007) (http://www.hss.energy.gov/CSA/analysis/orps/orps.html)

- Institute of Nuclear Power Operations (INPO) database (https://www.inpo.org). The INPO database contains the following information:
  - Licensee event reports
  - Equipment Performance and Information Exchange System
  - Nuclear Plant Reliability Data System.
- *Savannah River Site Human Error Data Base Development for Nonreactor Nuclear Facilities (U)* (Ref. 2.2.12)
- All Scientech/Licensing Information Service (LIS) data on ISFSI events (1994 through 2007) and Dry Storage Information Forum (New Orleans, LA, May 2-3, 2001). This database includes the following information:
  - Inspection reports
  - Trip reports
  - Letters, etc.

HFEs identified include both EOOs and EOCs.

The result of this identification process is a list of HFEs and a description of each HFE scenario, including system and equipment conditions and any resident or triggered human factor concerns (e.g., PSFs). This combination of conditions and human factors concerns then becomes the EFC for a specific HFE. Additions and refinements to these initial EFCs are made during the preliminary and detailed analyses.

6.4.4 Preliminary Analysis

A preliminary analysis is performed to allow HRA resources for the detailed analyses to be focused on only the most risk-significant HFEs. The preliminary analysis includes verification of the validity of HFEs included in the initial PCSA model, assignment of conservative HEPs to all HFEs and verification of those probabilities. The actual quantification of preliminary values is a six-step process that is described in detail in Appendix E.III of Attachment E. Once the preliminary probabilities are assigned, the PCSA model is quantified (initial quantification) to determine which HFEs require a detailed quantification. HFEs are identified for a detailed analysis if (1) the HFE is a risk-driver for a dominant sequence, and (2) using the preliminary values, an aggregated event sequence is above Category 1 or Category 2 according to 10 CFR 63.111 (Ref. 2.3.2) performance objectives.

In cases where HFEs are completely mitigated by hardware (i.e., interlocks), the HFE is generally assigned a value of 1.0 unless otherwise noted, and the hardware is modeled explicitly in the fault tree.

6.4.5 Detailed Analysis

Once preliminary values have been assigned, the model is run, and HFEs are identified for a detailed analysis if (1) the HFE is a risk-driver for a dominant sequence, and (2) using the preliminary values, that sequence is Category 1 or Category 2. A dominant sequence is one that does not meet the performance objectives according to the performance objectives in 10 CFR 63.111 (Ref. 2.3.2). The objective of a detailed analysis is to develop a more realistic HRA and identify design features to be added that will provide compliance with the aforementioned regulation. Many of the important to safety features of Section 6.9 were identified during the HRA. The remaining HFEs retain their assigned preliminary values. For the preliminary analysis, many of the HFEs are modeled in a simplified form in the event trees and fault trees; although, for the preliminary analysis, each action is separated as much as possible for the detailed analysis. This separation is done to ensure that the detailed analysis is thorough and that the relationship between the system functionality and operations crew is transparent. First an HFE is broken down into the various scenarios that lead to the failure. Then, each scenario is further broken down into specific required actions and their applicable procedures, along with the systems and components that must be operated during performance of each action. Each action in each scenario has its own unique context, dependencies, and set of PSFs, and each is quantified independently. The failure probabilities for these unsafe actions are quantified by the HRA method appropriate to the HFE, its classification (e.g., errors of commission (EOC), errors of omission (EOO), observation error, execution error), and the context. For this analysis, several HRA methods were considered, and the following four methods were selected (Appendix E.IV of Attachment E provides a discussion of the selection process):

- CREAM (Ref. 2.2.51)
- HEART/NARA (Ref. 2.2.85)/(Ref. 2.2.37)
- THERP with some modifications (Ref. 2.2.81)
- ATHEANA's expert elicitation approach (Ref. 2.2.67).

For the preliminary analysis, HFEs are modeled at a high level where several subtasks are combined into a single task so that explicit consideration of dependencies between subtasks is eliminated. For a detailed assessment, where the various actions that constitute an HFE are explicitly quantified, dependencies are also explicitly addressed using the basic formulae in Table 6.4-1 from the THERP method (Ref. 2.2.81), where N is the independently derived HEP.

Table 6.4-1. Formulae for Addressing HFE Dependencies

Level of Dependence	Zero	Low	Medium	High	Complete
Conditional Probability	N	$\frac{1 + 19N}{20}$	$\frac{1 + 6N}{7}$	$\frac{1 + N}{2}$	1.0

Source: Modified from *Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications.* NUREG/CR-1278 (Ref. 2.2.81), Table 20-17, p. 20–33.

After estimates for HFE probabilities are generated, these results are reviewed by the HRA team and, in some cases, by knowledgeable operations personnel, as a "sanity check." Principally, such checks are used, for example, to compare the probabilities of different HFEs and determine whether or not these probabilities are consistent with the judgment of experts regarding the associated operator actions. A review of this type is particularly important for HFE probabilities that are generated using data from the THERP method (Ref. 2.2.81) since it is difficult to identify all important PSFs that are appropriate for repository operations. In addition, the HFE probability estimates are reviewed to ensure that they do not exceed the lower limit of credible human performance as defined by NARA (Ref. 2.2.37). HFE probabilities produced in this HRA are mean values; uncertainties are accounted for by applying an error factor to the mean value of the overall HFE according to the guidelines presented in Section E3.4 of Attachment E.

6.4.6 Human Failure Event Probabilities used in RF Event Sequences Analysis

The results of the HRA are the HFE probabilities used in the event tree and fault tree quantification process, which are listed in Table 6.4-2.

Table 6.4-2. Human Failure Event Probability Summary

Basic Event Name	HFE Description	ESD	HFE Group	Basic Event Mean Probability	Error Factor	Type of Analysis
200-#EEE-LDCNTRA-BUA-ROE	Operator fails to restore Load Center Train-A post maintenance	Electrical	OA	1.03E-05	10	Preliminary
200-#EEE-LDCNTRA-BUA-ROE	Operator fails to restore Load Center Train-B post maintenance	Electrical	OA	1.03E-05	10	Preliminary
26D-#EEY-ITSDG-A-#DG-RSS	Operator fails to restore Diesel Generator A to service	Electrical	OA	1.95E-04	10	Preliminary
26D-#EEY-ITSDG-B-#DG-RSS	Operator fails to restore Diesel Generator B to service	Electrical	OA	1.95E-04	10	Preliminary

Table 6.4-2. Human Failure Event Probability Summary (Continued)

Basic Event Name	HFE Description	ESD	HFE Group	Basic Event Mean Probability	Error Factor	Type of Analysis
200-Liddisplace1-HFI-NOD	Operator inadvertently displaces cask lid during platform activities	10	3, 5	N/A[b]	N/A	Omitted from analysis
200-OpAOImpact01-HFI-NOW	Operator causes AO impact during AO closure	7	5	3.00E-03	5	Preliminary
200-OpCaskDrop01-HFI-NOD	Operator drops cask during cask preparation activities	3	3	N/A[b]	N/A	Omitted from analysis
200-OpClCTMGate1-HFI-NOD	Operator inappropriately closes slide or port gate during vertical canister movement and continues lifting	6	4	1.00E-03	5	Preliminary
200-OpCollide001-HFI-NOD	Operator causes low-speed collision of auxiliary vehicle with RC, HCTT, CTT, or TTC	2	2, 6	3.00E-03	5	Preliminary
200-OpCTCollide1-HFI-NOD	Operator causes low-speed collision of auxiliary vehicle with CTT	3, 7	3, 5	3.00E-03	5	Preliminary
200-OpCTCollide2-HFI-NOD	Operator causes low-speed collision of CTT with SSC during transfer from preparation station to Unloading Room	4	3	1.00E-03	5	Preliminary
060-OpCTMDirExp1-HFI-NOD	Operator causes direct exposure during CTM activities (second floor)	11	4	8E−06	10	Detailed
200-OpCTMDrInt01-HFI-COD	Operator lifts object or canister too high with CTM (two-block)	6	4	1.0	N/A	Preliminary
200-OpCTMdrop001-HFI-COD	Operator drops object onto canister during CTM operations	6	4	4.00E-07	10	Detailed
200-OpCTMdrop002-HFI-COD	Operator drops canister during CTM operations	6	4	5.00E-07	10	Detailed
200-OpCTMImpact1-HFI-COD	Operator moves the CTM while canister or object is below or between levels	6	4	4.00E-08	10	Detailed

Table 6.4-2. Human Failure Event Probability Summary (Continued)

Basic Event Name	HFE Description	ESD	HFE Group	Basic Event Mean Probability	Error Factor	Type of Analysis
200-OpCTMImpact2-HFI-COD	Operator causes canister impact with lid during CTM operations (TAD canister)	6	4	N/A[b]	N/A	Omitted from analysis
200-OpCTMImpact5-HFI-COD	Operator causes canister impact with SSC during CTM operations	6	4	1.0	N/A	Preliminary
200-OpCTTImpact1-HFI-NOD	Operator causes an impact between cask and SSC due to crane operations	3	3	3.00E-03	5	Preliminary
200-OpDirExpose1-HFI-NOD	Operator causes direct exposure during CTM activities (first floor)	11	4	1.00E-01	3	Preliminary
200-OpDirExpose2-HFI-NOD	Operator causes direct exposure during CTM activities (transfer into an AO)	11	4	1.00E-04	10	Preliminary
200-OpDPCShield1-HFI-NOW	Operator causes loss of shielding while installing DPC lift fixture	10	3	4.00E-04	10	Detailed
200-OpFailRstInt-HFI-NOM	Operator fails to restore interlock after maintenance	11	4	1.00E-02	3	Preliminary
200-OpFailSG-HFI-NOD	Operator fails to close the CTM slide gate moving CTM with canister inside bell (direct exposure)	11	4	1.00E-03	5	Preliminary
200-OpFailStop-HFI-NOD	Operator fails to stop ST if tread fails	8	5	1.0	N/A	Preliminary
200-OpFLCollide1-HFI-NOD	Operator causes high speed collision of auxiliary vehicle with RC, HTC, ST, CTT or TTC	2, 3, 7, 9	2, 6, 3, 5	1.0	N/A	Preliminary
200-OpHTCollide1-HFI-NOD	Operator causes low speed collision between HCTT and facility SSCs	9	6	3.00E-03	5	Preliminary
200-OpHTIntCol01-HFI-NOD	Operator causes high speed collision between HCTT and facility SSCs	9	6	1.0	N/A	Preliminary

Table 6.4-2. Human Failure Event Probability Summary (Continued)

Basic Event Name	HFE Description	ESD	HFE Group	Basic Event Mean Probability	Error Factor	Type of Analysis
200-OpImpact0000-HFI-NOD	Operator causes impact of cask during transfer of CTT into the Cask Unloading Room or ST out of Loading Room	4, 7	3, 5	N/A[b]	N/A	Omitted from analysis
200-OpLoadDrop-HFI-NOD	Operator causes ST to drop AO	8	5	N/A	N/A	Preliminary
200-OpNoDiscoAir-HFI-NOD	Operator Causes Spurious Movement of the CTT while Canister is Being Unloaded	6	4	1.00E-03	5	Preliminary
200-OpNoUnBolt00-HFI-NOD	Operator fails to fully unbolt the cask lid before moving CTT into the Cask Unloading Room (TAD canister)	6	4	1.00E-03	5	Preliminary
200-OpNoUnBoltDP-HFI-NOD	Operator fails to fully unbolt the cask lid before moving CTT into the Cask Unloading Room (DPC)	6	4	N/A[b]	N/A	Omitted from Analysis
200-OpNoUnplugST-HFI-NOD	Operator causes spurious movement of the ST while canister is being loaded	6	4	1.00E-03	5	Preliminary
200-OpRCCollide1-HFI-NOD	Operator causes low-speed collision between RC and facility SSCs	1	1	3.00E-03	5	Preliminary
200-OpRCIntCol01-HFI-NOD	Operator causes high-speed collision between RC and facility SSCs	1	1	1.0	N/A	Preliminary
200-OpRCIntCol02-HFI-NOD	Operator causes MAP to collide into RC	1	1	1.0	N/A	Preliminary
200-OpSDClose001-HFI-NOD	Operator closes shield door on conveyance	5	OA (1, 3, 5, 6)	1.0	N/A	Preliminary
200-OpSpurMove01-HFI-NOD	Operator causes spurious movement of CTT or ST during preparation or closure	2, 3, 7	2, 3, 5, 6	1.00E-04	10	Preliminary

Table 6.4-2. Human Failure Event Probability Summary (Continued)

Basic Event Name	HFE Description	ESD	HFE Group	Basic Event Mean Probability	Error Factor	Type of Analysis
200-OpSTCollide1-HFI-NOD	Operator causes low-speed collision of ST with SSC while moving to the Lid Bolting Room	7	5	3.00E-03	5	Preliminary
200-OpSTCollide2-HFI-NOD	Operator causes low-speed collision of ST with SSC while exporting the ST	8	5	3.00E-03	5	Preliminary
200-OpTCImpact01-HFI-NOD	Operator causes an impact between cask and SSC during upending and removal	2	2, 6	3.00E-03	5	Preliminary
200-OpTipover001-HFI-NOD	Operator causes cask to tip over during cask upending and removal	2	2, 6	1.00E-04	10	Preliminary
200-OpTipover002-HFI-NOD	Operator causes cask to tip over during cask preparation activities	3	3	1.00E-04	10	Preliminary
200-OpTipOver003-HFI-NOD	Operator causes tipover of ST	7	5	1.00E-04	10	Preliminary
200-OpTipOver3-HFI-NOD	Operator causes tipover of CTT during movement to the Cask Unloading Room	4	3	N/A[b]	N/A	Omitted from analysis
200-VCTO-DR00001-HFI-NOD	Operators open two or more vestibule doors in RF	HVAC	OA	1.00E-02	3	Preliminary
200-VCTO-HEPALK-HFI-NOD	Operator fails to notice HEPA filter leak in Train A	HVAC	OA	1.0	N/A	Preliminary
200-VCTO-HFIA000-HFI-NOM	Human error exhaust fan switch wrong position	HVAC	OA	1.00E-01	3	Preliminary
Crane Drops (drop of cask or object onto cask)	Operator drops cask or drops object onto cask during crane operations	2, 3	OA (2, 3, 6)	N/A[a]	N/A	Historical data
Drop of object on AO	Operator drops heavy object on AO during AO closure	N/A	5	N/A[b]	N/A	Omitted from analysis
Gas Sampling	Operator improperly performs gas sampling	N/A	3	N/A[b]	N/A	Omitted from analysis

Table 6.4-2. Human Failure Event Probability Summary (Continued)

Basic Event Name	HFE Description	ESD	HFE Group	Basic Event Mean Probability	Error Factor	Type of Analysis
Load too Heavy	Operator causes drop of cask by attempting to lift a load that is too heavy for the crane	OA	OA (2, 3, 6)	N/A[b]	N/A	Omitted from analysis
Moderator	Operator introduces moderator into a moderator-controlled area of the RF	OA	OA	N/A[b]	N/A	Omitted from analysis
RC Derailment	Operator causes the RC to derail	1	1	N/A[a]	N/A	Historical data
Spurious Movement of CTT or ST during CTM Activities	Operator causes spurious movement of the CTT or ST during canister loading or unloading	6	4	N/A[b]	N/A	Omitted from analysis
ST Rollover	Operator causes rollover of ST during AO export	8	5	N/A[b]	N/A	Omitted from analysis
200-HCTT-Roll	Operator causes rollover of HCTT	9	6	N/A[b]	N/A	Omitted from analysis

NOTE: [a]Historical data was used to produce a probability of crane drops; this historical data is not included as part of the HRA, but is addressed in Attachment C, Section C1.3.
[b] These HFEs were initially identified, but omitted from analysis for various reasons, including a design change precluding the human failure, or the failure would require a series of unsafe actions in combination with mechanical failures, such that the event is no longer credible. See the appropriate HFE group in Attachment E for a case-by-case justification for these omissions.
AO = aging overpack; CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; ESD = event sequence diagram; HCTT = cask tractor and cask transfer trailer; HFE = human failure event; HTC = a transportation cask that is never upended; HVAC = heating, ventilation, and air conditioning; MAP = mobile access platform; N/A = not applicable; OA = over arching (applies to multiple HFE groups, see Section E6.0.2); RC = railcar; SSC = structure, system, or component; SSCs = structures, systems, and components; ST = site transporter; TAD = transportation, aging, and disposal; TTC = a transportation cask that is upended using a tilt frame.

Source: Original

6.5 FIRE INITIATING EVENTS

Attachment F of this document describes the work scope, definitions and terms, method, and results for the fire analysis performed as a part of the PCSA. The internal events of the PCSA model were evaluated with respect to fire initiating events and modified as necessary to address fire-induced failures that lead to exposures. The list of fire-induced failures included in the model were evaluated as to fire vulnerability, and fragility analyses were conducted as needed (Section 6.3.2 and Attachment D).

Fire initiating event frequencies were calculated for each initiating event identified for the RF. Section F5 of Attachment F details the analysis performed to determine these frequencies, using the methodology described in Section F4 of Attachment F.

6.5.1 Input to Initiating Events

Room and building areas, ignition frequencies, ignition source distributions, propagation probabilities, and residence fractions are the set of calculated values which contribute to calculating initiating event frequencies.

Room dimensions (Section F5.2.1 and F5.4of Attachment F) are utilized to determine individual room areas and the total building area. The area of the RF is utilized to evaluate the building ignition frequency. From methodology and equations presented in Section F4.3.1 of Attachment F, the building ignition frequency over the 50-year facility operation period of 2.6, is obtained for the RF. The results of this portion of the analysis are summarized in Table 6.5-1.

As discussed in Section F4.3.2.1 of Attachment F, an industrial building fire can begin as the result of numerous types of ignition sources, which are grouped into nine categories:

1. Electrical equipment
2. HVAC equipment
3. Mechanical process equipment
4. Heat-generating process equipment
5. Torches, welders, and burners
6. Internal combustion engines
7. Office and kitchen equipment
8. Portable and special equipment
9. No equipment involved.

Table 6.5-1. Room Areas and Total Ignition Frequency

Room	Area (m^2)	Room	Area (m^2)	Room	Area (m^2)	Room	Area (m^2)
1001	167	1020	237	1207	68	2002D	87
1002	368	1020A	22	1208	51	2002E	182
1003A	40	1021	191	1209	54	2002F	60
1003B	76	1021A	349	1210	57	2002G	17
1003C	53	1021B	12	1211	35	2003	334
1003D	140	1022	51	1212	39	2004	259
1003E	133	1023	54	1212A	7	2005	333
1003F	67	1025	56	1213	13	2006	296
1003G	45	1026	40	1214	13	2007	1,444
1004	261	1027	30	1215	30	2008	267
1004A	99	1028	75	1216	16	2009	308
1005	235	1028A	51	1217	38	2010	334
1005A	20	1029	42	1218	21	2011	259
1011	98	1030	30	1219	21	2012	333
1012	296	1031	32	1220	32	2022	54
1013	175	1200	8	1221	48	2023	54
1014	141	1201A	47	1222	4	2025	55

Table 6.5-1. Room Areas and Total Ignition Frequency (Continued)

Room	Area (m^2)	Room	Area (m^2)	Room	Area (m^2)	Room	Area (m^2)
1015	156	1201B	100	1223	34	2026	40
1016	126	1202	21	1224	73	2027	38
1017/1017A	1,993	1203	46	2001	167	2029	42
1018	256	1204	35	2002A	69	3001	24
1018A	51	1205	8	2002B	132	3026	40
1019	265	1206	36	2002C	17	3029	42
1019A	70						
Total Area (sq-m)					12,842		
Ignition Frequency (per sq-m/yr)					4.05E-06		
Ignition Frequency (per yr)					5.20E-02		
Ignition Frequency (50 years - preclosure period)					2.60E+00		

NOTE: m = meter; sq = square; yr = year.

Source: Table F5.2-1 of Attachment F.

Each category has a fraction representing the probability that, given an ignition, that category is the source of the ignition. These fractions are combined with the number of units in each category to determine the ignition frequency per ignition source. Uncertainty distributions have been applied to the ignition frequencies, and contribute to the resulting distribution for fire initiating event frequencies. The number of ignition sources in each category is further divided by location into specific rooms. Each piece of equipment in a category is defined as one ignition source, with some exceptions:

- MCCs, load centers, and equipment racks contribute an ignition source for each active vertical cabinet.
- An ignition source is counted for each motor over 5 hp for all equipment with motors.
- A welding ignition source is counted for each hour of operation expected per year.
- The ignition sources for mobile equipment are split between the rooms the equipment occupies in proportion to the amount of time the equipment will spend in each room.
- An ignition source is counted for every square meter in the room for the no equipment involved category.

The distribution and determination of ignition sources is further discussed in Section F5.4 of Attachment F, and summarized in Table 6.5-2. For the purposes of the summary, the "no equipment involved" category and the "heat-generating process equipment" category have been left out of Table 6.5-2. This was done because the values in the "no equipment involved" category are exactly equal to the square meters for each room (Table 6.5-1) and because there is no equipment for any of the facilities that falls under the "heat-generating process equipment" category (Section F5.4.4, Attachment F).

Table 6.5-2. Ignition Source Category and Room-by-Room Population

Room	Electrical	HVAC	Mechanical Equipment	Torches, welders, burners	Internal combustion engines	Office/ kitchen equipment	Portable Equipment
1001					7		
1002			3		59		
1004		4		5			4
1004A		4					2
1005	23	2					
1005A	1						
1012				5			
1013			2		34		
1014			4	5			
1015			2.03				1
1017/1017A			8.97	400	35		4
1018	80			5			2
1018A	2						
1019		4					4
1019A		4					2
1020	23	2					2
1020A	1						
1021			1		33		
1021A		2	2		32		
1028			1				
1207						1	
1208	6					1	
1209						2	
1210						2	
1212						1	
1218						1	
1219						1	
1220						1	
1223			1				
2003		2		5			2
2004		1					2
2005				5			
2006		6					2
2007			7				1
2008		2					2
2009		1					2

Table 6.5-2. Ignition Source Category and Room-by-Room Population (Continued)

Room	Electrical	HVAC	Mechanical Equipment	Torches, welders, burners	Internal combustion engines	Office/ kitchen equipment	Portable Equipment
2010		2		5			2
2011							2
2012	21			5			
TOTAL	**157**	**36**	**32**	**440**	**200**	**10**	**36**

NOTE: HVAC = heating, ventilation, and air conditioning.

Source: Table F5.5-1 of Attachment F.

Propagation probabilities (Section F5.6, Attachment F) are utilized in the analysis to define the probability of a fire spreading to various points specifically identified as areas in which a waste form may be vulnerable. Uncertainty distributions have been applied to the propagation probabilities, and contribute to the resulting distribution for fire initiating even frequencies.

Residence fractions (Section F5.7.1, Attachment F) developed from process throughputs define the length of time (in minutes), a waste form will be vulnerable in a particular area of the building and in a particular configuration. The minutes are converted to the fraction of time the vulnerability is present over the 50-year preclosure surface operation period, and are summarized in Table 6.5-3.

Table 6.5-3. Residence Fractions

Initiating Event	Residence Fraction
Waste Form in AO in Vestibule/Lid Bolting Room (Diesel)	
TAD or DPC in AO (incl. TTC & VTC) in Vestibule/Lid Bolting Room (Diesel Present)	1.2E-05
Waste Form in AO in Loading Room (Diesel)	
TAD or TC/DPC in AO (incl. TTC & VTC) in Loading Room (Diesel Present)	3.3E-06
Waste Form in Vestibule/Preparation Area (Diesel)	
TC/TAD on railcar in Vestibule/Preparation Area w/ SPM (Diesel Present)	2.1E-06
TC/DPC (TTC) on railcar in Vestibule/Preparation Area w/ SPM (Diesel Present)	2.1E-06
TC/DPC (VTC) in Vestibule/Preparation Area w/ SPM (Diesel Present)	2.1E-06
TC/DPC (HTC) in Vestibule/Preparation Area w/ SPM/truck (Diesel Present)	4.3E-06
Waste Form in Preparation Area (No Diesel)	
TC/TAD on railcar in Preparation Area (No Diesel Present)	1.6E-05
TC/DPC on railcar (TTC) in Preparation Area (No Diesel Present)	2.4E-05
TC/DPC on railcar (VTC) in Preparation Area (No Diesel Present)	1.3E-05
TC/DPC (HTC) on railcar in Preparation Area (No Diesel Present)	2.7E-05
Waste Form in Preparation Area	
TC/TAD on CTT in Preparation Area	6.4E-06
TC/DPC on CTT (VTC, incl. TTC) in Preparation Area	1.5E-05
Waste Form in Cask Unloading Room	

Table 6.5-3. Residence Fractions (Continued)

Initiating Event	Residence Fraction
TC/TAD on CTT in Cask Unloading Room	3.5E-06
TC/DPC (TTC) on CTT in Cask Unloading Room	1.8E-06
TC/DPC (VTC) in Cask Unloading Room	1.8E-06
Waste Form in Transfer Room	
TAD or DPC (including TTC & VTC) in Transfer Room	1.2E-06
TC/TAD or TC/DPC (TTC & VTC) (Diesel Present)	2.1E-06
TC/TAD (No Diesel)	2.6E-05
TAD or DPC (TTC & VTC) in CTM	1.2E-06
TAD or DPC (TTC & VTC) in AO (Diesel Present)	1.5E-05
TC/DPC (TTC) in CTM (No Diesel)	4.0E-05
TC/DPC (VTC) (No Diesel)	3.0E-05
TC/DPC (HTC) (Diesel Present)	4.3E-06
TC/DPC (HTC) (No Diesel)	2.7E-05

NOTE: AO = aging overpack; CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; HTC=transportation cask in the horizontal position; SPM = site prime mover; TAD = transportation, aging, and disposal canister; TC = transportation cask; TTC= transportation cask in the tilted position; VTC = transportation cask in the vertical position; WP = waste package.

Source: Tables F5.7-1, F5.7-2, F5.7-3, and F5.7-6 of Attachment F.

6.5.2 Initiating Event Frequencies

The results of the fire initiating event analysis are the fire initiating event frequencies and their associated distributions presented in Table 6.5-4. The frequencies represent the probability over the length of the preclosure surface operation period that a fire will threaten the stated waste container in the stated location. Initiating event frequencies are divided into two types of calculations, localized fires and large fires, and are calculated for all locations associated with waste handling operations and locations from which a fire can spread to a waste handling operational location. (In Attachment F, these locations are sometimes called vulnerabilities.) Calculations performed to obtain the initiating event are detailed in Section F5.7 of Attachment F.

Uncertainty distributions are utilized in the contribution to initiating event frequency calculations to account for statistical uncertainty in the data. Uncertainty distributions utilized for this analysis are lognormal distribution and normal distribution. Both distributions can be accurately represented by a mean and 50% value. The mean and median can be inputs to calculate the error factor (EF). The 97.5% value is also provided, and is a figure that represents a point at which only 2.5% of all possible outcomes will vary from the mean more significantly. Three uncertainty distributions were developed for this analysis, details for which are in Appendices II and III of Attachment F.

Monte Carlo simulations are performed to determine the mean, median, standard deviation, variance, minimum, and maximum values of each of the initiating event frequencies based on the variance of the contributing data. To accomplish this, the Microsoft Excel add-on package, Crystal Ball, is used (Section F5.8). This software requires input of two parameters (e.g., in the lognormal case, 50% and 97.5% values). Crystal Ball software allows probability distributions to be combined per formulas or equations representing initiating event frequency inputs entered into Excel. The software randomly selects a value from the possibilities defined by the distribution. Ten thousand Monte Carlo trials are performed.

Crystal Ball is run for all of the initiating events, the complete output of which is available in Appendix VI of Attachment F. In addition to showing the initiating event frequency distribution, the full output also shows the input distribution for the parameters that are varied, which match the distributions developed and documented in Appendices II and III of Attachment F.

Table 6.5-5 provides the fire analysis data for the basic events in this model.

Table 6.5-4. Results from Monte Carlo Simulation of Initiating Event Frequency Distributions

Initiating Event	Equipment	Mean	Median	97.5% Value	Error Factor	Type
Localized Fire Threatens Waste Form in AO in Vestibule/Lid Bolting Room (Diesel Present)	Site Transporter					
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Vestibule/Lid Bolting Room (Diesel Present)		8.1E-07	7.3E-07	1.80E-6	2.1	Lognormal
Localized Fire Threatens Waste Form in AO in Loading Room (Diesel Present)	Site Transporter					
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Loading Room (Diesel Present)		3.5E-07	3.2E-07	7.9E-07	2.0	Lognormal
Localized Fire Threatens Waste Form in Vestibule/Preparation Area (Diesel Present)	Site Prime Mover					
Localized Fire Threatens TC/TAD in Vestibule/Preparation Area (Diesel Present)		4.6E-07	4.2E-07	1.0E-06	2.0	Lognormal
Localized Fire Threatens TC/DPC (TTC) in Vestibule/Preparation Area (Diesel Present)		4.6E-07	4.2E-07	1.0E-06	2.0	Lognormal
Localized Fire Threatens TC/DPC (VTC) in Vestibule/Preparation Area (Diesel Present)		4.6E-07	4.2E-07	1.0E-06	2.0	Lognormal
Localized Fire Threatens TC/DPC (HTC) in Vestibule/Preparation Area (Diesel Present)		9.3E-07	8.3E-07	2.1E-06	2.2	Lognormal
Localized Fire Threatens Waste Form in Preparation Area	Railcar					
Localized Fire Threatens TC/TAD in Preparation Area (No Diesel Present)		3.1E-06	2.8E-06	6.9E-06	2.1	Lognormal
Localized Fire Threatens TC/DPC (TTC) in Preparation Area (No Diesel Present)		4.5E-06	4.0E-06	1.0E-05	2.2	Lognormal
Localized Fire Threatens TC/DPC (VTC) in Preparation Area (No Diesel Present)		2.5E-06	2.2E-06	5.5E-06	2.3	Lognormal
Localized Fire Threatens TC/DPC (HTC) in Preparation Area (No Diesel Present)		5.0E-06	4.5E-06	1.1E-05	2.1	Lognormal
Localized Fire Threatens Waste Form in Preparation Area	Cask Transfer Trolley					
Localized Fire Threatens TC/TAD in Preparation Area		9.1E-07	8.1E-07	2.1E-06	2.2	Lognormal
Localized Fire Threatens TC/DPC (VTC, including TTC) in Preparation Area		2.1E-06	1.9E-06	4.8E-06	2.1	Lognormal

Table 6.5-4. Results from Monte Carlo Simulation of Initiating Event Frequency Distributions (Continued)

Initiating Event	Equipment	Mean	Median	97.5% Value	Error Factor	Type
Localized Fire Threatens Waste Form in Cask Unloading Room	Cask Transfer Trolley					
Localized Fire Threatens TC/TAD in Cask Unloading Room		3.9E-07	3.5E-07	8.7E-07	2.1	Lognormal
Localized Fire Threatens TC/DPC (TTC) in Cask Unloading Room		2.0E-07	1.8E-07	4.4E-07	2.1	Lognormal
Localized Fire Threatens TC/DPC (VTC) in Cask Unloading Room		2.0E-07	1.8E-07	4.4E-07	2.1	Lognormal
Localized Fire Threatens Waste Form in Transfer Room	Canister Transfer Machine					
Localized Fire Threatens TAD or DPC (including TTC & VTC) in Transfer Room		1.1E-07	9.9E-08	2.5E-07	2.1	Lognormal
Large Fire Threatens TC/TAD or TC/DPC (TTC & VTC) (Diesel Present)		8.6E-07	7.6E-07	2.0E-06	2.3	Lognormal
Large Fire Threatens TC/TAD (No Diesel)		1.1E-05	9.5E-06	2.5E-05	2.4	Lognormal
Large Fire Threatens TAD or DPC (TTC & VTC) in CTM		4.9E-07	4.4E-07	1.1E-06	2.1	Lognormal
Large Fire Threatens TAD or DPC (TTC & VTC) in AO (Diesel Present)		6.1E-06	5.5E-06	1.4E-05	2.1	Lognormal
Large Fire Threatens TC/DPC (TTC) (No Diesel)		1.6E-05	1.5E-05	3.8E-05	1.8	Lognormal
Large Fire Threatens TC/DPC (VTC) (No Diesel)		1.2E-05	1.1E-05	2.9E-05	2.0	Lognormal
Large Fire Threatens TC/DPC (HTC) (Diesel Present)		1.8E-06	1.6E-06	4.1E-06	2.2	Lognormal
Large Fire Threatens TC/DPC (HTC) (No Diesel)		1.1E-05	9.8E-06	2.6E-05	2.2	Lognormal

NOTE: AO = aging overpack; CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; EF = error factor; HTC = transportation cask in horizontal position; TAD = transportation, aging, and disposal canister; TC = transportation cask; TTC = transportation cask in tilting position; VTC = transportation cask in vertical position.

Source: Table F5.7-7 of Attachment F.

Table 6.5-5. Basic Events Data Associated with Fire Analysis

Basic Event Name	Basic Event Description	BE Value	Bases	References
ESD12-DFIRE-IN-PREP-DPC	TC with DPC in vestibule/prep area threatened by diesel fire	1.850E-006	Localized Fire Threatens a TC containing a DPC in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-DFIRE-IN-PREP-TAD	TC with TAD in vestibule/prep area threatened by diesel fire	4.600E-007	Localized Fire Threatens a TC containing a TAD in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-DPC-IN-LG-FIRE	DPC threatened by large fire	4.830E-005	A large fire threatens a container with a DPC in the facility. Variations in container type failure probabilities are accounted for by assigning split fractions.	Section 6.3, Table 6.3-10 and 6.3-11
ESD12-FIRE-CTM-DPC	Fire in transfer area threatens DPC	1.100E-007	Localized Fire Threatens a TC containing a DPC in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-FIRE-CTM-TAD	Fire in transfer area threatens TAD	1.100E-007	Localized Fire Threatens a TC containing a TAD in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-BOLT-DPC	DPC threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a DPC in the Lid Bolting Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-BOLT-TAD	TAD threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a TAD in the Lid Bolting Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
Basic Event Name	Basic Event Description	BE Value	Bases	References
ESD12-FIRE-IN-LOAD-DPC	DPC threatened by fire in loading room	3.500E-007	Localized Fire Threatens a TC containing a DPC in the Loading Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-LOAD-TAD	TAD threatened by fire in loading room	3.500E-007	Localized Fire Threatens a TC containing a TAD in the Loading Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-PREP-DPC	DPC in TC threatened by fire in prep area	1.200E-005	Localized Fire Threatens a TC containing a DPC in the Preparation Area with diesel present	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-PREP-TAD	TAD in TC threatened by fire in prep area	3.100E-006	Localized Fire Threatens a TC containing a TAD in the Preparation Area with diesel present	Section 6.3, Table 6.3-10

Table 6.5-5. Basic Events Data Associated with Fire Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Bases	References
ESD12-DFIRE-IN-PREP-DPC	TC with DPC in vestibule/prep area threatened by diesel fire	1.850E-006	Localized Fire Threatens a TC containing a DPC in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-DFIRE-IN-PREP-TAD	TC with TAD in vestibule/prep area threatened by diesel fire	4.600E-007	Localized Fire Threatens a TC containing a TAD in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-DPC-IN-LG-FIRE	DPC threatened by large fire	4.830E-005	A large fire threatens a container with a DPC in the facility. Variations in container type failure probabilities are accounted for by assigning split fractions.	Section 6.3, Table 6.3-10 and 6.3-11
ESD12-FIRE-CTM-DPC	Fire in transfer area threatens DPC	1.100E-007	Localized Fire Threatens a TC containing a DPC in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-FIRE-CTM-TAD	Fire in transfer area threatens TAD	1.100E-007	Localized Fire Threatens a TC containing a TAD in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-BOLT-DPC	DPC threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a DPC in the Lid Bolting Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-BOLT-TAD	TAD threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a TAD in the Lid Bolting Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-PREPCT-DPC	DPC in TC threatened by fire in prep area	2.100E-006	Localized Fire Threatens a TC containing a DPC in the Preparation Area	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-PREPCT-TAD	TAD in TC threatened by fire in prep area	9.100E-007	Localized Fire Threatens a TC containing a TAD in the Preparation Area	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-UNLD-DPC	DPC threatened by fire in unloading room	4.000E-007	Localized Fire Threatens a TC containing a DPC in the Unloading Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-UNLD-TAD	TAD threatened by fire in unloading room	3.900E-007	Localized Fire Threatens a TC containing a TAD in the Unloading Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
Basic Event Name	Basic Event Description	BE Value	Bases	References
ESD12-TAD-IN-LG-FIRE	TAD threatened by large fire	1.850E-005	A large fire threatens a container with a TAD in the facility. Variations in container type failure probabilities are accounted for by assigning split fractions.	Section 6.3, Table 6.3-10 and 6.3-11

Table 6.5-5. Basic Events Data Associated with Fire Analysis (Continued)

Basic Event Name	Basic Event Description	BE Value	Bases	References
ESD12-DFIRE-IN-PREP-DPC	TC with DPC in vestibule/prep area threatened by diesel fire	1.850E-006	Localized Fire Threatens a TC containing a DPC in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-DFIRE-IN-PREP-TAD	TC with TAD in vestibule/prep area threatened by diesel fire	4.600E-007	Localized Fire Threatens a TC containing a TAD in the Vestibule/Preparation Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-DPC-IN-LG-FIRE	DPC threatened by large fire	4.830E-005	A large fire threatens a container with a DPC in the facility. Variations in container type failure probabilities are accounted for by assigning split fractions.	Section 6.3, Table 6.3-10 and 6.3-11
ESD12-FIRE-CTM-DPC	Fire in transfer area threatens DPC	1.100E-007	Localized Fire Threatens a TC containing a DPC in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-FIRE-CTM-TAD	Fire in transfer area threatens TAD	1.100E-007	Localized Fire Threatens a TC containing a TAD in the Transfer Area when diesel is present,	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-BOLT-DPC	DPC threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a DPC in the Lid Bolting Room , diesel is present in the Site Transporter	Section 6.3, Table 6.3-10
ESD12-FIRE-IN-BOLT-TAD	TAD threatened by fire in lid bolting room	8.100E-007	Localized Fire Threatens a TC containing a TAD in the Lid Bolting Room, diesel is present in the Site Transporter	Section 6.3, Table 6.3-10

NOTE: DPC = dual-purpose canister; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

6.6 NOT IN USE

6.7 EVENT SEQUENCE FREQUENCY RESULTS

This section provides the results of the event sequence quantification as produced from the SAPHIRE (Section 4.2) analyses. Quantification of an event sequence consists of calculating its number of occurrences over the 50-year preclosure period by combining the frequency of a single initiating event with the conditional probabilities of pivotal events that comprise the sequence. The quantification results are presented as an expression of the mean and median number of occurrences of each event sequence over the preclosure period, and the standard deviation as a measure of uncertainty. Section 6.8 describes the process for aggregation of similar event sequences to permit categorization as Category 1, Category 2, or beyond Category 2 event sequences.

The section presents a summary of how the quantification is performed by linking event trees, fault trees, and basic event input parameters. The discussion includes the rationale for truncating low values and the analysis of uncertainties.

The results include a summary of all event sequences that are quantified and a table summarizing the results of the final quantification (found in Attachment G).

6.7.1 Process for Event Sequence Quantification

Internal event sequences that are based on the event trees presented in Section 6.1 and fault trees presented in Section 6.2 are quantified using SAPHIRE (Section 4.2). In SAPHIRE, the quantification of an event sequence is always labeled as a “frequency” in the output formats.

The event sequence quantification methodology is presented in Section 4.3.6. An event sequence frequency is the product of several factors, as follows (with examples):

- The number of times the operation or activity that gives rise to the event sequence is performed over the preclosure period, for example, the total number of transfers of a TAD canister by a CTM in the RF over the preclosure period. In SAPHIRE, this number is entered in the first event of the initiator event tree from which the event sequence arises or in the first event of the system-response event tree if no initiator event tree exists.

- The probability of occurrence of the initiating event for the event sequence is considered. Continuing with the previous example, this could be the probability of dropping a TAD canister during its transfer by the CTM, or the probability of occurrence of a fire that could affect the TAD canister during its transfer by the CTM. The initiating event probability is modeled in SAPHIRE with a fault tree or with a basic event. In an initiator event tree, this probability is assigned on the branch associated with that initiating event, through the use of SAPHIRE rules (i.e., textual logic instructions that determine which fault tree or basic event is to be used). If no initiator event tree exists, this probability is entered in the second event of the system-response event tree.

- The conditional probability of each of the pivotal events of the event sequence, which appears in the system-response event tree. The pivotal event may represent a passive failure such as the breach of the containment boundary of the TAD canister or an active system failure such as the unavailability of the HVAC system. The conditional event probabilities of pivotal events are linked to the event sequence in SAPHIRE through the linkage to basic events in a fault tree that represents the pivotal event. The selection of pivotal event models and the associated basic event values may be determined by SAPHIRE rules.

Uncertainties in input parameters such as throughput rates, equipment failure rates, passive failure probabilities, and human failure events used to calculate basic event probabilities are propagated through the fault tree and event sequence logic to quantify the uncertainty in the event sequence quantification.

To quantify an event sequence, SAPHIRE first establishes the logic of the event sequence (i.e., the combination of individual successes and failures of pivotal events after the initiating event). SAPHIRE then links together the fault trees that support the initiating event and the pivotal events and uses Boolean logic to identify dependencies between the initiating event and the pivotal events and between pivotal events. SAPHIRE finally develops minimal cut sets for the event sequence considered. A minimal cut set for an event sequence is a Boolean reduced combination of a set of basic events that, if it occurs, will cause the event sequence to occur. The event sequence frequency is calculated as the sum of frequencies of the cut sets. For computational efficiency, minimal cut sets that have a frequency less than a cutoff value of 10^{-12} are not calculated by SAPHIRE. Such minimal cut sets are insignificant contributors to the number of occurrences of the event sequence over the preclosure period. This value is considered sufficient to ensure that all significant contributors are identified because it would require the sum of 1×10^{8} cut sets with a probability of occurrence of 1×10^{-12} over the preclosure period to reach the Category 2 threshold frequency of 1×10^{-4} over the preclosure period.

As an illustration of the above process, the quantification of the event sequence initiated by a drop of a TAD canister during a transfer in the RF, followed by the breach of the canister, the subsequent failure of the HVAC confinement to perform its confinement and filtering function over its mission time, but no moderator entry into the canister, is outlined in the following paragraphs.

The event sequence that leads to an unfiltered radionuclide release which is not important to criticality starts with an initiator event tree that depicts the number of TAD canisters that are transferred by the CTM in the RF over the preclosure period. Based on *Waste Form Throughputs for Preclosure Safety Analysis* (Ref. 2.2.27, Table 4), there are 6,978 such transfers. Next, the branch on the initiator event tree that deals with the drop of a canister is selected. In practice, this is done by SAPHIRE through the use of rules, which are assigned to the event called "INIT-EVENT," the fault tree whose top event models the probability of a TAD canister drop. Multiplying the number of TAD canister transfers by the probability of a drop yields the number of occurrences, over the preclosure period, of the initiating event for the event sequence considered.

SAPHIRE continues the construction of event sequence logic via a transfer to the system-response event tree which provides the basis for quantifying the rest of the event sequence through the use of the pivotal events described in Section 6.1 and Attachment B. First, the breach of the canister, given its drop, is evaluated under the pivotal event called "CANISTER". SAPHIRE rules are used to ensure that the probability assigned to this pivotal event pertains to the waste form considered in this event sequence–a TAD canister. The next pivotal event that appears in the system-response event tree is called "SHIELDING". This pivotal event has a probability of one (1), indicating that a loss of shielding is considered to occur if the canister breaches. This modeling conforms to the approach taken in the PCSA, where event sequences that lead to a radionuclide release also embed direct exposure of personnel to radiation that could result from a loss of shielding. The next pivotal event is called "CONFINEMENT." This event models the failure of HVAC to maintain confinement and perform filtering of the radionuclide release. This pivotal event is quantified with a fault tree. The mission time for the system is 720 hrs (i.e., 30 days). Finally, the last pivotal event is called "MODERATOR." This event models moderator intrusion into the breached canister. In the event sequence analyzed, no moderator entry occurs, that is, the success branch is followed.

Two fault trees appear in this example event sequence: one models the drop of the canister and the other models the loss of the HVAC system. These fault trees are linked by SAPHIRE and a Boolean reduction is applied to identify dependencies (such as a loss of power, which is a contributor to both a load drop by the CTM and the loss of the HVAC system), and remove nonminimal cut sets.

The SAPHIRE event sequence quantification report includes the number of occurrences of each cut set that contributes to an event sequence and the summation over the cut set to yield a number of occurrences of the event sequence over the preclosure period. The internal processes of SAPHIRE provides quantification of cut sets that represent combinations of basic events from respective initiating event trees and pivotal event tress. The summation over such cut sets represents the cumulative frequency of an initiating event (e.g., drop), containment (e.g., canister) breach, confinement unavailability, and moderator availability.

As noted, uncertainties in input parameters are propagated through the fault tree and event sequence logic to quantify the uncertainty in the event sequence quantification. The uncertainty analysis uses the Monte Carlo method that is built into SAPHIRE. Each event sequence was analyzed using 10,000 trials. The number of trials is considered sufficient to ensure accurate results for the distribution parameters.

6.7.2 Event Sequence Quantification Summary

Table G-1 of Attachment G presents the result of the event sequence quantification. Table G-1 summarizes the results of the final quantification and lists the following elements: (1) event tree from which the sequence is generated, (2) SAPHIRE event sequence designator (ID), (3) initiating event description, (4) event sequence logic, (5) event sequence end state, (6) event sequence mean value, (7) event sequence median value, and (8) event sequence variance.

6.8 EVENT SEQUENCE GROUPING AND CATEGORIZATION

An aggregation grouping process is applied prior to a categorization of event sequences as was described in Section 4.3.1. It is appropriate for purposes of categorization to add the frequencies of event sequences that are derived from the same ESD that elicits the same combination of failure and success of pivotal events, and have the same end state. This is termed final event sequence quantification, discussed in Section 6.8.1, and the results give the final frequency of occurrence. Using the final frequency of occurrence, the event sequences are categorized according to the definition of Category 1 and Category 2 event sequences given in 10 CFR 63.2 (Ref. 2.3.2). Dose consequences for Category 1 and Category 2 event sequences are subject to the performance objectives of 10 CFR 63.111 (Ref. 2.3.2), which is performed in *Preclosure Consequence Analyses* (Ref. 2.2.31). Event sequences with a frequency of occurrence less than one chance in 10,000 of occurring before permanent closure of the repository are designated as beyond Category 2 event sequences and are not analyzed for dose consequences.

Rather than calculate dose consequences for each Category 2 event sequence identified in the categorization process, dose consequences are performed for a set of bounding events that encompass the end states and material at risk for event sequences. Therefore, dose consequences are determined for a representative set of postulated Category 2 event sequences, identified in Table 6.8-1 (Ref. 2.2.31, Table 2 and Section 7). Once event sequence categorization is complete, Category 2 event sequences are cross referenced with the bounding event number given in Table 6.8-1, thus assuring that Category 2 event sequences have been evaluated for dose consequences and compared to the 10 CFR 63.111 (Ref. 2.3.2) performance objectives.

Table 6.8-1. Bounding Category 2 Event Sequences

Bounding Event Number	Affected Waste Form	Description of End State	Material At Risk
2-01	LLWF inventory and HEPA filters	Seismic event resulting in LLWF collapse and failure of HEPA filters and ductwork in other facilities.	HEPA filters LLWF inventory
2-02*	HLW canister in transportation cask	Breach of sealed HLW canisters in a sealed transportation cask	5 HLW canisters
2-03*	HLW canister	Breach of sealed HLW canisters in an unsealed waste package	5 HLW canisters
2-04*	HLW canister	Breach of sealed HLW canister during transfer (one drops onto another)	2 HLW canisters
2-05*	Uncanistered commercial SNF in transportation cask	Breach of uncanistered commercial SNF in a sealed truck transportation cask in air	4 PWR or 9 BWR commercial SNF
2-06*	Uncanistered commercial SNF in pool	Breach of uncanistered commercial SNF in an unsealed truck transportation cask in pool	4 PWR or 9 BWR commercial SNF
2-07	DPC in air	Breach of a sealed DPC in air	36 PWR or 74 BWR commercial SNF
2-08*	DPC in pool	Breach of commercial SNF in unsealed DPC in pool	36 PWR or 74 BWR commercial SNF
2-09	TAD canister in air	Breach of a sealed TAD canister in air within facility	21 PWR or 44 BWR commercial SNF
2-10*	TAD canister in pool	Breach of commercial SNF in unsealed TAD canister in pool	21 PWR or 44 BWR commercial SNF

Table 6.8-1. Bounding Category 2 Event Sequences (Continued)

Bounding Event Number	Affected Waste Form	Description of End State	Material At Risk
2-11*	Uncanistered commercial SNF	Breach of uncanistered commercial SNF assembly in pool (one drops onto another)	2 PWR or 2 BWR commercial SNF
2-12*	Uncanistered commercial SNF	Breach of uncanistered commercial SNF in pool	1 PWR or 1 BWR commercial SNF
2-13*	Combustible and noncombustible LLW	Fire involving LLWF inventory	Combustible and noncombustible inventory
2-14*	Uncanistered commercial SNF in truck transportation cask	Breach of a sealed truck transportation cask due to a fire	4 PWR or 9 BWR commercial SNF

NOTE: BWR = boiling water reactor; DPC = dual-purpose canister; HEPA = high-efficiency particulate air; HLW = high-level radioactive waste; LLWF = Low-Level Waste Facility; PWR = pressurized water reactor; SNF = spent nuclear fuel; TAD = transportation, aging and disposal. Items marked with an asterisk (*) are not applicable to the RF.

Source: *Preclosure Consequence Analyses* (Ref. 2.2.31, Table 2)

6.8.1 Event Sequence Grouping and Final Quantification

Event sequences are modeled to represent the GROA operations and SSCs. Accordingly, an event sequence is unique to a given operational activity in a given operational area, which is depicted in an ESD. When more than one initiating event (for example, the drop, collision, or other structural challenges that could affect the canister) share the same ESD (and therefore elicit the same pivotal events and the same end states), it may be necessary to quantify the event sequence for each initiating event individually because the conditional probabilities of the pivotal events depend on the specific initiating event. In such cases, the frequencies of event sequences that are represented in the same ESD, having the same path through the event tree, and have the same end state are added together, thus comprising an event sequence grouping.

For example, an ESD may show event sequences that could occur during the transfer of a canister from one container to another by the CTM in the RF. More than one initiating event (for example, the drop, collision, or other structural challenges that could affect the canister) may share the same ESD (and therefore elicit the same pivotal events and the same end states), but give rise to event sequences that are quantified for each initiating event because the conditional probabilities of their pivotal events depend on the specific initiating event.

By contrast, some ESDs indicate a single initiating event. Such initiating events may be composites of several individual initiating events, but because the conditional probabilities of pivotal events and the end states are the same for each of the constituents, the initiators are grouped before the event sequence quantification.

In the PCSA, event-sequence grouping is performed for a given waste form configuration at the ESD level. The waste forms configurations considered are as follows. Note that not all waste container configurations are applicable to the RF:

- Waste package (not applicable to the RF)

- Naval SNF canister, by itself or in a transportation cask (not applicable to RF)
- HLW canister, by itself or in a transportation cask (not applicable to the RF)
- DOE standardized canister, containing DOE owned SNF, by itself or in a transportation cask (not applicable to the RF)
- MCO, by itself or in a transportation cask (not applicable to the RF)
- TAD canister, by itself, in a transportation cask, or in an aging overpack
- DPC, by itself, in a transportation cask, or an aging overpack
- Transportation cask containing bare SNF assemblies (not applicable to RF)
- SNF assembly (handled in the pool of the WHF and not applicable to RF)
- Low-level waste (not applicable to RF).

In SAPHIRE (Section 4.2), the grouping of event sequences is carried out using textual instructions, designated as partitioning rules. Partitioning rules gather into a single end state the minimal cut sets from the relevant individual event sequences that need to be grouped together, and further apply a Boolean reduction to ensure that nonminimal cut sets are removed. The event sequence frequencies from this step comprise the final event sequence quantification.

An illustration of the grouping of event sequences is described in the following. The potential structural challenges to a given canister during its transfer by the CTM in the RF are partitioned among seven different initiating events such as canister drop, collision, drop of a heavy load on the canister, etc. The event sequences involving the canister are quantified separately seven times, once for each initiating event. After an initiating event, the event sequences that elicit the same system response and lead to the same end state (i.e., those event sequences that follow the same path on the system-response event tree) are grouped together for purposes of categorization. Thus, the seven individual event sequences initiated by a TAD canister drop, collision, etc., that eventually result in a specific end state, for example, a filtered (i.e., mitigated) radionuclide release, are grouped together for the purposes of categorization as a single aggregated event sequence with a unique name termed the “event sequence group ID”. Since there are five different end states that can lead to exposure of personnel to radiation (i.e., result in an end state other than “OK”), there are five aggregated event sequences involving the TAD canister, each having a unique name. The frequency of each of the five aggregated event sequences represents the sum of frequencies of the seven individual event sequences.

The uncertainties in the grouped event sequences are generated by SAPHIRE as described in Section 6.7. The logic of the grouped event sequences is applied to recalculate the output probability distribution from the input parameters such as throughput rates, equipment failure rates, passive failure probabilities, and HFEs used to calculate basic event probabilities. These probability distributions are propagated through the fault tree and event sequence logic to quantify the uncertainty in the event sequence quantification.

6.8.2 Event Sequence Categorization

Based on the resultant frequency of occurrence, the event sequences are categorized as Category 1 or Category 2, per the definitions in 10 CFR 63.2 (Ref. 2.3.2), or beyond Category 2. The categorization is done on the basis of the expected number of occurrences of each event sequence during the preclosure period. For purposes of this discussion, the expected number of occurrences of a given event sequence over the preclosure period is represented by the quantity m.

Some event sequences are not directly dependent on the duration of the preclosure period. For example, the expected number of occurrences of TAD canister drops in the RF over the preclosure period is essentially controlled, among other things, by the number of TAD canisters and the number of lifts of these canisters. The duration of the preclosure period is not directly relevant for this event sequence, but is implicitly built into the operations. In contrast, for other event sequences, time is a direct input. For example, seismically induced event sequences are evaluated over a period of time. In such cases, event sequences are evaluated and categorized for the time during which they are relevant.

Using the parameter m for a given event sequence, categorization is performed using the screening criteria set out in 10 CFR 63.2 (Ref. 2.3.2), as follows:

- Those event sequences that are expected to occur one or more times before permanent closure of the GROA are referred to as Category 1 event sequences (Ref. 2.3.2). Thus, a value of m greater than or equal to one means the event sequence is a Category 1 event sequence.

- Other event sequences that have at least one chance in 10,000 of occurring before permanent closure are referred to as Category 2 event sequences (Ref. 2.3.2). Thus, a value of m less than one but greater than or equal to 10^{-4}, means the event sequence is a Category 2 event sequence.

- A measure of the probability of occurrence of the event sequence over the preclosure period is given by a Poisson distribution that has a parameter taken equal to m. The probability, P, that the event sequence occurs at least one time before permanent closure is the complement to one that the event sequence occurs exactly zero times during the preclosure period. Using the Poisson distribution, $P = 1 - \exp(-m)$ (Ref. 2.2.11, p. A-13). A value of P greater than or equal to 10^{-4} implies the value of m is greater than or equal to $-\ln(1 - P) = -\ln(1 - 10^{-4})$, which is approximately equal to 10^{-4}. Thus, a value of m greater than or equal to 10^{-4}, but less than one, implies the corresponding event sequence is a Category 2 event sequence.

- Event sequences that have a value of m less than 10^{-4} are designated as beyond Category 2.

An uncertainty analysis is performed on m to determine the main characteristics of its associated probability distribution, specifically the mean, 50th percentile (i.e., the median), and the standard deviation. The uncertainty analysis is performed in SAPHIRE using Monte Carlo with 10,000 samples as described in Section 4.3.6.2.

The calculations carried out to quantify an event sequence are performed using the full precision of the individual probability estimates that are used in the event sequence. However, the categorization of event sequences is based upon the expected number of occurrences over the preclosure period with one significant digit.

6.8.3 Final Event Sequence Quantification Summary

Initially, the results of the SAPHIRE event sequence gathering and quantification process are reported in a single table of all event sequences for the RF (Attachment G, Table G-2). Following the final categorization, the event sequences for the respective Category 2 (Table 6.8-3) and beyond Category 2 (Attachment G, Table G-3) are tabulated separately. There are no Category 1 (Table 6.8-2) events for the RF. As desired, other sorting may be performed. For example, event sequences that have end states important to criticality are tabulated separately (Attachment G, Table G-4). The format of the table headings and content are the same for each table as follows:

1. Event sequence group ID – assigned during the grouping process in SAPHIRE
2. End state – taken from the event tree
3. Event sequence description – narrative to describe the initiating event(s) and pivotal events that are involved
4. Material at risk – describes the quantity and type of waste form involved
5. Mean event sequence frequency (number of occurrences over the preclosure period)
6. Median event sequence frequency (number of occurrences over the preclosure period)
7. Standard deviation of the event sequence frequency (number of occurrences over the preclosure period)
8. Event sequence category – declaration of Category 1, Category 2, or Beyond Category 2
9. Basis for categorization (e.g., categorization by mean frequency or from sensitivity study for mean frequencies near a threshold, as described in Section 4.3.6.2)
10. Consequence analysis – cross-reference to the bounding event number in the dose consequence analysis (Table 6.8-1) (Ref. 2.2.31, Table 2 and Section 7).

The event sequences involving the breach of a TAD canister or a DPC are beyond Category 2 in the RF, regardless of whether or not the HVAC system is capable to fulfill its confinement and filtering function. This demonstrates that this system is not required for maintaining these event sequences in their final categorization.

Table 6.8-2. Category 1 Final Event Sequences Summary

Event Sequence Group ID	End State	Description	Material-At-Risk	Mean	Median	Std Dev	Event Sequence. Cat.	Basis for Categorization	Consequence Analysis
None									

Source: Original

Table 6.8-3. Category 2 Final Event Sequences Summary

Event Sequence Group ID	End State	Description	Material-At-Risk[4]	Mean[3]	Median[3]	Std Dev[3]	Event Sequence Cat.	Basis for Categorization	Consequence Analysis[1]
ESD12-TAD-SEQ2-DEL	Direct exposure, loss of shielding	This event sequence represents a thermal challenge to a TAD canister in a transportation cask, due to a fire, resulting in a direct exposure from loss of shielding. In this sequence the canister remains intact, and the shielding fails.	1 TAD canister	2.E-01	2.E-01	1.E-01	Category 2	Mean of distribution for number of occurrences of event sequence near a category threshold. Categorization confirmed by alternative distribution	N/A[2]
ESD10-SEQ2-DEL	Direct exposure, loss of shielding	This event sequence represents a direct exposure during preparation activities of a transportation cask containing a DPC. In this sequence there are no pivotal events.	1 DPC	1.E-01	1.E-01	1.E-01	Category 2	Mean of distribution for number of occurrences of event sequence	N/A[2]
ESD11-SEQ2-DEL	Direct exposure, loss of shielding	This event sequence represents a temporary loss of shielding during CTM operations, while a DPC or a TAD canister is being transferred. In this sequence there are no pivotal events.	1 DPC or 1 TAD canister	7.E-02	3.E-02	1.E-01	Category 2	Mean of distribution for number of occurrences of event sequence	N/A[2]
ESD12-DPC-SEQ2-DEL	Direct exposure, loss of shielding	This event sequence represents a thermal challenge to a DPC in a transportation cask, due to a fire, resulting in a direct exposure from loss of shielding. In this sequence the canister remains intact, and the shielding fails.	1 DPC	2.E-02	2.E-02	8.E-03	Category 2	Mean of distribution for number of occurrences of event sequence	N/A[2]

Table 6.8-3. Category 2 Final Event Sequences Summary

Event Sequence Group ID	End State	Description	Material-At-Risk[4]	Mean[3]	Median[3]	Std Dev[3]	Event Sequence Cat.	Basis for Categorization	Consequence Analysis[1]
ESD07-TAD-SEQ2-DEL	Direct exposure, loss of shielding	This event sequence represents a structural challenge to a TAD canister in an aging overpack, during aging overpack assembly and closure, resulting in a direct exposure from loss of shielding. In this sequence the canister remains intact, and the shielding fails.	1 TAD canister	8.E-04	6.E-04	1.E-03	Category 2	Mean of distribution for number of occurrences of event sequence	N/A[2]
ESD01-TAD-SEQ2-DED	Direct exposure, degradation of shielding	This event sequence represents a structural challenge to a TAD canister inside a transportation cask, during receipt activities, resulting in a direct exposure from degradation of shielding. In this sequence the transportation cask containment function remains intact, and the shielding fails.	1 TAD canister	3.E-04	2.E-04	1.E-03	Category 2	Mean of distribution for number of occurrences of event sequence	N/A[2]

Table 6.8-3. Category 2 Final Event Sequences Summary

Event Sequence Group ID	End State	Description	Material-At-Risk[4]	Mean[3]	Median[3]	Std Dev[3]	Event Sequence Cat.	Basis for Categorization	Consequence Analysis[1]
ESD08-TAD-SEQ2-DEL	Direct exposure, loss of shielding	This event sequence represents a structural challenge to a TAD canister in an aging overpack, during export activities, resulting in a direct exposure from loss of shielding. In this sequence the canister remains intact, and the shielding fails.	1 TAD canister	3.E-04	2.E-04	1.E-03	Category 2	Mean of distribution for number of occurrences of event sequence	N/A[2]

NOTE: [1]The bounding event number provided in this column identifies the bounding Category 2 event sequence identified in Table 6.8-1 from the *Preclosure Consequence Analyses* (Ref. 2.2.31, Table 2) that results in dose consequences that bound the event sequence under consideration.

[2]Because of the great distances to the locations of the offsite receptors, doses to members of the public from direct radiation after a Category 2 event sequence are reduced by more than 13 orders of magnitude to insignificant levels (Ref. 2.2.31, *GROA External Dose Rate Calculation*).

[3]The mean, median, and standard deviation displayed are for the number of occurrences, over the preclosure period, of the event sequence under consideration.

[4]The material at risk is, as relevant, based upon the nominal capacity of the waste form container involved in the event sequence under consideration, or accounts for the specific operation covered by the event sequence.

CTM = canister transfer machine; DPC = dual-purpose canister; ST = site transporter; TAD = transportation, aging, and disposal; TC = transportation cask.

Source: Original

6.9 IMPORTANT TO SAFETY STRUCTURES, SYSTEMS, AND COMPONENTS AND PROCEDURAL SAFETY CONTROL REQUIREMENTS

The results of the PCSA are used to define design bases for repository SSCs to prevent or mitigate event sequences that could lead to the release of radioactive material and/or result in radiological exposure of workers or the public. Potential releases of radioactive material are minimized to ensure resulting worker and public exposures to radiation are below the limits established by 10 CFR 63.111 (Ref. 2.3.2). This strategy requires using prevention features in the repository design wherever reasonable. This strategy is implemented by performing the PCSA as an integral part of the design process in a manner consistent with a performance-based, risk-informed philosophy. This integral design approach ensures the ITS design features and operational controls are selected in a manner that ensures safety while minimizing design and operational complexity through the use of proven technology. Using this strategy, design rules are developed to provide guidance on the safety classification of SSCs. The following information is developed in order to implement this strategy:

- Essential safety functions needed to ensure worker and public safety
- SSCs relied upon to ensure essential safety functions
- Design criteria that will ensure that the essential safety functions will be performed with a high degree of reliability and margin of safety
- Administrative and procedural safety controls that, in conjunction with the repository design ensure operations are conducted within the limits of the PCSAs.

Section 6.9.1 identifies ITS SSCs and Section 6.9.2 identifies the procedural safety controls.

6.9.1 Important to Safety Structures, Systems, and Components

Table 6.9-1 contains the nuclear safety design bases for the RF ITS SSCs. The first three columns identify the ITS system or facility, subsystem and component. The fourth column identifies the safety function relied upon in the event sequence analysis. The fifth column provides the characteristics of the safety function (i.e., controlling parameter or value) that is demonstrated to occur or exist in the design. The sixth column provides an event sequence in which the safety function and the characteristic is relied upon. The seventh column provides the source, usually a fault tree, for the controlling parameter or value.

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
Aging (AP)	Aging Handling/ Cask Transfer	Site Transporter (170-HAT0-MEQ-00001)	Protect against[c] spurious movement	1. The mean probability of spurious movement of the site transporter while the canister is being lifted or lowered shall be less than or 1×10^{-9} per transfer.	RF-ESD06-TAD (Seq. 5-4)	200-ST-SPURMOVE
			Limit speed	2. The speed of the site transporter shall be limited to 2.5 mph.	RF-ESD07-TAD (Seq. 3-3)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.
			Preclude a cask breach due to explosion	3. The site transporter fuel tank shall preclude fuel tank explosions.	Initiating event does not require further analysis[b]	Table 6.0-2.
			Reduce severity of a drop	4. The site transporter shall preclude a vertical drop of an aging overpack from a height greater than 3 ft measured from the equipment base.	RF-ESD07-TAD (Seq. 3-3)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	**Controlling Parameters and Values**		
		Cask Tractor (for use with the cask transfer trailer) (170-HAT0-HEQ-00001)	Reduce severity of collision	5. The speed of the site transporter shall be limited to 2.5 mph.	RF-ESD09 (Seq. 3-3)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.
			Preclude a cask breach due to explosion	6. The cask tractor fuel tank shall preclude fuel tank explosions.	Initiating event does not require further analysis[b]	Section 6.0
		Cask Transfer Trailer (for use with transportation casks and horizontal shielded transfer casks (HSTCs) (PWR DPC: [170-HAT0-TRLY-00001]) (BWR DPC: [170-HAT0-TRLY-00002])	Preclude a cask breach due to explosion	7. The cask transfer trailer fuel tank shall preclude fuel tank explosions.	Initiating event does not require further analysis[b]	Section 6.0
			Reduce severity of a drop	8. The cask transfer trailer shall preclude dropping a horizontally oriented transportation cask or HSTC from a height greater than 6 ft.	RF-ESD09 (Seq. 2-4)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
			Preclude puncture of a cask	9. The cask transfer trailer shall preclude puncture of a transportation cask or HSTC due to collision.	Initiating event does not require further analysis[b]	Section 6.0
	Aging Handling/ Aging Overpack	Aging Overpack (TAD: [170-HAC0-ENCL-00003]) (Vertical DPC: [170-HAC0-ENCL-00002])	Protect against [c] direct exposure to personnel	10. The mean conditional probability of loss of shielding of the aging overpack resulting from an impact or collision shall be less than or equal to 1 x 10-5 per impact.	RF-EDS07-TAD (Seq. 3-2)	AO_SHIELDING
				11. The mean conditional probability of loss of shielding of the aging overpack resulting from a drop shall be less than or equal to 1 x 10-5 per drop.	RF-ESD08-TAD (Seq. 4-2)	AO_SHIELDING
DOE and Commercial Waste Package System	Canistered Spent Nuclear Fuel	DPC (analyzed as a representative canister)	Provide containment	12. The mean conditional probability of breach of a canister resulting from a drop of the canister shall be less than or equal to 1 × 10-5 per drop.	RF-ESD06-DPC (Seq. 3-3)	DPC-FAIL-NO-CASK
				13. The mean conditional probability of breach of a canister resulting from a drop of a load onto the canister shall be less than or equal to 1 × 10-5 per drop.	RF-ESD07-DPC (Seq. 2-3	CAN-IN-AO-DROPON

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
				14. The speed of the site transporter shall be limited to 2.5 mph.	RF-ESD01-DPC (Seq. 3-4)	TCASK
				15. The mean conditional probability of breach of a canister contained within a cask resulting from the spectrum of firesd shall be less than or equal to 2 × 10-6 per fire event.	RF-ESD12-DPC (Seq. 5-3)	CANISTER-FIRE-TC
DOE and Commercial Waste Package System (continued)	Canistered Spent Nuclear Fuel (continued)	DPC (analyzed as a representative canister) (continued)	Provide containment (continued)	16. The mean conditional probability of breach of a canister contained within an aging overpack resulting from the spectrum of fires shall be less than or equal to 1 × 10-6 per fire event.	RF-ESD12-DPC (Seq. 2-3)	CANISTER-FIRE-AO
				17. The mean conditional probability of breach of a canister located within the CTM Shield Bell resulting from the spectrum of fires shall be less than or equal to 1 × 10-4 per fire event.	RF-ESD12-DPC (Seq. 9-3)	CANISTER-FIRE

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases: Safety Function	Nuclear Safety Design Bases: Controlling Parameters and Values	Representative Event Sequence (Sequence Number)	Source
		TAD Canister (analyzed as a representative canister)	Provide containment	18. The mean conditional probability of breach of a canister resulting from a drop of the canister shall be less than or equal to 1 × 10-5 per drop.	RF-ESD06-TAD (Seq. 3-3)	TAD-FAIL-NO-CASK
				19. The mean conditional probability of breach of a canister resulting from a drop of a load onto the canister shall be less than or equal to 1 × 10-5 per drop.	RF-ESD6-TAD (Seq. 6-3)	TAD-FAIL-NO-CASK
				20. The mean conditional probability of breach of a canister resulting from a side impact or collision shall be less than or equal to 1 × 10-8 per impact.	RF-ESD01-TAD (Seq. 3-4)	TCASK
				21. The mean conditional probability of breach of a canister contained within a cask resulting from the spectrum of fires shall be less than or equal to 2 × 10-6 per fire event.	RF-ESD12-TAD (Seq. 4-3)	CANISTER-FIRE-TC

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
				22. The mean conditional probability of breach of a canister located within the aging overpack resulting from the spectrum of fires shall be less than or equal to 1 × 10-6 per fire event.	RF-ESD12-TAD (Seq. 2-3)	CANISTER-FIRE-AO
				23. The mean conditional probability of breach of a canister located within the CTM Shield Bell resulting from the spectrum of fires shall be less than or equal to 1 × 10-4 per fire event.	RF-ESD12-TAD (Seq. 9-3)	CANISTER-FIRE
Mechanical Handling System	Cask Handling	Transportation Cask	Provide containment	24. The mean conditional probability of breach of a canister in a sealed cask resulting from a drop shall be less than or equal to 1 × 10-5 per drop.	RF-ESD06-TAD (Seq. 3-3)	TAD-FAIL-NO-CASK
				25. The mean probability of breach of a canister in a sealed cask resulting from a drop of a load onto the cask shall be less than or equal to 1 × 10-5 per drop.	RF-ESD06-TAD (Seq. 6-3)	TAD-FAIL-NO-CASK
				26. The mean conditional probability of breach of a canister in a sealed cask resulting from a side impact or collision shall be less than or equal to 1 × 10-8 per impact.	RF-ESD06-TAD (Seq. 5-3)	TAD-FAIL-CTM-IMPACT

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
			Protect against[c] direct exposure to personnel	27. The mean conditional probability of loss of cask gamma shielding resulting from a drop of a cask shall be less than or equal to 1 × 10-8 per drop.	RF-ESD02-TAD (Seq. 3-2)	TCASK-SHIELDING-IMP
				28. The mean conditional probability of loss of cask gamma shielding resulting from a drop of a load onto a cask shall be less than or equal to 1E-5 per impact.	RF-ESD03-TAD (Seq. 5-2)	TCASK-SHIELDING-DROP
				29. The mean conditional probability of loss of cask gamma shielding of a cask resulting from a collision or side impact to a cask shall be less than or equal to 1E-8 per impact.	RF-ESD04-TAD (Seq. 3-2)	TCASK-SHIELDING-IMP
		Site Prime Mover	Limit speed	30. The speed of the site prime mover shall be limited to 9 mph.	RF-ESD01-TAD (Seq. 3-4)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
			Preclude fuel tank explosion	31. The fuel tank of a site prime mover that enters the facility shall preclude fuel tank explosions. .	Initiating event does not require further analysis[b]	Table 6.0-2
		Cask Handling Yoke (200-HM00-BEAM-00001)	Protect against[c] drop	32. The cask handling yoke is an integral part of the load-bearing path. See cask handling crane requirements.	See cask handling crane requirements	See “Cask Handling Crane” requirements.
		Cask Handling Crane; 200-ton (200-HM00-CRN-00001	Protect against[c] drop	33. The mean probability of dropping a loaded cask from less than the two-block height resulting from the failure of any piece of equipment within the load-bearing path shall be less than or equal to 3E-5 per transfer with the cask yoke or 1E-4 per transfer with a sling.	RF-ESD02-TAD (Seq. 2-4) (yoke) RF-ESD02-DPC (Seq. 2-4) (sling)	200-CRN2-DROPTAD-CRN-DRP 200-CRN2-DROPDPC-CRS-DRP
			Protect against[c] drop	34. The mean probability of dropping a loaded cask from a two-block height resulting from the failure of a piece of equipment within the load-bearing path shall be less than or equal to 4 × 10-7 per transfer.	RF-ESD02-TAD (Seq. 7-4) (yoke) RF-ESD02-DPC (Seq. 7-4) (sling)	200-CRN2-2-BLOCK-CRN-TBK
			Limit drop height	35. The height of a two-block drop shall not exceed 30 feet from bottom of shortest cask to the floor.	RF-ESD02-TAD (Seq. 7-4)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
			Protect against[c] drop of a load onto a transportation cask	36. The mean probability of dropping a load onto a loaded cask or its contents shall be less than or equal to 9 × 10-5 per cask handled.	RF-ESD02-TAD (Seq. 6-4)	ESD2-TAD-DROPON
			Limit speed	37. The speed of the trolley and bridge shall be limited to 20 ft./min.	RF-ESD02-TAD (Seq. 4-4)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7. (2.5 mi/hr, from Table 6.3-7, equals 220 ft/min, which bounds 20 ft/min.)
		Cask Transfer Trolley (CTT) (including pedestal and seismic restraints) (Trolley: 200-HM00-TRLY-00001) (Pedestal: 200-HM00-PED-00001)	Limit speed	38. The speed of the CTT shall be limited to 2.5 mph.	RF-ESD04-TAD (Seq. 3-4)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.
			Protect against spurious movement	39. The mean probability of spurious movement of the CTT while a canister is being lifted by the CTM shall be less than or equal to 1×10-9 per transfer.	RF-ESD06-TAD (Seq. 4-3)	200-CTT-SPUR-MOVE

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
	Cask Handling/ Cask Receipt	Horizontal Lifting Beam (200-HMC0-BEAM-00001)	Protect against[c] drop	40. The horizontal lifting beam is an integral part of the load-bearing path. See cask handling crane requirements.	See cask handling crane requirements	See Cask Handling Crane requirements
		Cask Lid Lifting Grapples (DPC) (200-HMH0-HEQ-00008)	Protect against[c] drop of a load onto a canister	41. The cask lid lifting grapple is an integral part of the load-bearing path. See cask handling crane requirements.	See cask handling crane requirements	See Cask Handling Crane requirements
	Cask Handling/Cask Preparation	Rail Cask Lid Adapters (200-HMH0-HEQ-00002)	Protect against[c] drop	42. The rail cask lid adapters are an integral part of the load-bearing path. See cask handling crane requirements.	See cask handling crane requirements	See Cask Handling Crane requirements
		DPC Lid Adapter (200-HMH0-HEQ-00001)	Protect against[c] drop of a DPC	43. The DPC lid adapter is an integral part of the load-bearing path. See canister transfer machine requirements.	See canister transfer machine requirements	See Cask Handling Crane requirements
	Waste Transfer/ Canister Transfer	CTM (200-HTC0-FHM-00001)	Protect against[c] drop	44. The mean probability of dropping a canister from below the two-block height due to the failure of a piece of equipment within the load-bearing path shall be less than or equal to 1 × 10-5 per transfer for the CTM.	RF-ESD06-TAD (Seq. 3-3)	TAD-FAIL-NO-CASK
			Protect against[c] drop	45. The mean probability of drop of a canister from the two-block height due to the failure of a piece of equipment within the load-bearing path shall be less than or equal to 3× 10-8per transfer.	RF-ESD06-TAD (Seq. 8-3)	CANISTER-FAIL-CTM-2BLOCK

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
			Limit drop height	46. The height of a two-block drop shall not exceed 40 feet from the bottom of any canister to the cavity floor of the cask or aging overpack.	RF-ESD06-TAD (Seq. 8-3)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7.
			Protect against[c] drop of a load onto a canister	47. The mean probability of dropping a load onto a canister shall be less than or equal to 1 × 10-5 per transfer.	RF-ESD06-TAD (Seq. 6-3)	TAD-FAIL-NO-CASK
			Protect against[c] spurious movement	48. The mean probability of a spurious movement of the CTM while a canister is being lifted or lowered shall be less than or equal to 5 × 10-9 per transfer for the CTM.	RF-ESD06-TAD (Seq. 4-3)	ESD6-TAD-SPUR
			Limit Speed	49. The speed of the CTM trolley and bridge shall be limited to 20 ft/min.	RF-ESD06-TAD (Seq. 5-4)	This parameter limits the conditional probability of cask breach given a collision to the appropriate value from Table 6.3-7. (2.5 mph, from Table 6.3-7, equals 220 ft/min, which bounds 20 ft/min.)
			Preclude non-flat bottom drop of a DPC or TAD canister	50. The CTM shall preclude non-flat-bottom drops of DPCs and TADs.	Initiating event does not require further analysis[b]	Table 6.0-2

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases		Representative Event Sequence (Sequence Number)	Source
			Safety Function	Controlling Parameters and Values		
			Protect against[c] direct exposure to personnel	51. The mean probability of inadvertent radiation streaming to workers resulting from the inadvertent opening of the CTM slide gate, the inadvertent raising of the CTM shield skirt, or an inadvertent motion of the CTM away from a port shall be less than or equal to 1 × 10-8 per transfer.	RF-ESD06-TAD (Seq. 4-2)	200-SLD-GTE-OPN-INADVERT
			Preclude canister breach	52. Closure of the CTM slide gate shall be incapable of breaching a canister.	Initiating event does not require further analysis[b]	Table 6.0-2
		CTM Grapples (200-HTC0-HEQ-00001)	Protect against[c] canister drop	53. The CTM grapple is an integral part of the load-bearing path See canister transfer machine requirements.	See canister transfer machine requirements	See Canister Transfer Machine requirements
Receipt Facility	Receipt Facility (RF)	Shield Doors (including anchorages) and equipment confinement doors	Protect against direct exposure of personnel	54. Equipment and personnel shield doors shall have a mean probability of inadvertent opening of less than or equal to 1 × 10-7 per waste container handled.	RF-ESD011 (Seq. 2)	200-SHLD-DR-OPN-INADVERT
			Preclude collapse onto waste containers	55. An equipment shield door falling onto a waste container as a result of impact from a conveyance shall be precluded.	Initiating event does not require further analysis[b]	Table 6.0-2
		Cask Port Slide Gate (200-HTC0-HTCH-00001)	Protect against[c] dropping a canister due to a spurious closure of the slide gate	56. The mean probability of a canister drop resulting from a spurious closure of the slide gate shall be less than or equal to 5 × 10-6 per transfer.	RF-ESD06-TAD (Seq. 3-3)	GATE-36-58

Table 6.9-1. Preclosure Nuclear Safety Design Bases for RF ITS SSCs (Continued)

System or Facility (System Code)	Subsystem or Function (as Applicable)[a]	Component[a]	Nuclear Safety Design Bases: Safety Function	Nuclear Safety Design Bases: Controlling Parameters and Values	Representative Event Sequence (Sequence Number)	Source
			Protect against[c] direct exposure to personnel	57. The mean probability of occurrence of an inadvertent opening of a slide gate shall be less than or equal to 4 × 10-9 per transfer.	RF-ESD11 (Seq. 2)	200-SLD-GTE-OPN-INADVERT
			Preclude canister breach	58. Closure of the slide gate shall be incapable of breaching a canister.	Initiating event does not require further analysis[b]	Table 6.0-2
		Aging Overpack Port Slide Gate (200-HTC0-HTCH-00002)	Protect against[c] dropping a canister due to a spurious closure of the slide gate	59. The mean probability of a canister drop resulting from a spurious closure of the slide gate shall be less than or equal to 5 × 10-6 per transfer.	RF-ESD06-TAD (Seq. 3-3)	GATE-36-58
			Protect against[c] direct exposure to personnel	60. The mean probability of occurrence of an inadvertent opening of a slide gate shall be less than or equal to 4 × 10-9 per transfer.	RF-ESD11 (Seq. 2)	200-SLD-GTE-OPN-INADVERT
			Preclude canister breach	61. Closure of the slide gate shall be incapable of breaching a canister.	Initiating event does not require further analysis[b]	Table 6.0-2

NOTE:
a. Reference to all SSCs in this table, unless otherwise noted, is associated with operations involving the handling/processing/transfer of SNF/HLW
b. Design requirement is applied to reduce the frequency of any event sequence that could result in damage to a waste container to the beyond category 2 frequency range
c. 'Protect against' in this table means either 'reduce the probability of' or 'reduce the frequency of'.
d. The term "spectrum of fires" refers to the variations in the intensity and duration of the fire that are considered along with conditions that control the rate of heat transfer to the container (Attachment D, Section D2.1)

CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; HSTC = horizontal shielded transfer cask; ITS = important to safety; RF = Receipt Facility; SNF = spent nuclear fuel; SSC = structure, system, or component; TAD = transport, aging, and disposal

Source: Original

6.9.2 Procedural Safety Controls

Procedural safety controls (PSCs) are the controls that are relied upon to limit or prevent potential event sequences or mitigate their consequences. For this analysis, all PSCs were derived to reduce the initiating event sequence to an acceptable level.

Table 6.9-2 lists the PSCs that are required to support the event sequence analysis and categorization. The event sequence column identifies a representative event sequence that relies upon the PSC.

Table 6.9-2. Summary of Procedural Safety Controls for the Receipt Facility

Item	SSC	Procedural Safety Controls	Basis for Selection	Representative Event Sequence
1	CTT	The CTT is deflated during loading of cask onto trolley, cask preparation activities, and during canister unloading or loading activities.	This control limits the probability of spurious movement of the CTT and resulting collision or tipover.	RF-ESD06-TAD (Seq. 6-3)
2	ST	The ST is turned off during, AO bolting and unbolting, and canister unloading or loading activities.	This control limits the probability of spurious movement of the ST and resulting collision or tipover.	RF-ESD06-TAD (Seq. 6-3)
3	ITS SSCs	The amount of time that a waste form container spends in each process area or in a given process operation, including total residence time in a facility, is periodically compared against the average exposure times used in the PCSA. Additionally, component failures per demand and component failures per time period are compared against the PCSA. Significant deviations will be analyzed for risk significance.	PCSA uses exposure/residence times and reliability data to calculate the probability of an initiating event, or the probability of seismic induced failures that lead to an event sequence. This control ensures that the average exposure times and reliability data are maintained consistent with those analyzed in the PCSA.	Applies to all event sequence and fault tree quantification that uses data from Attachment C. Also applies to fire analysis per Section 4.3 and Attachment E.
4	Cask Preparation Platform	Transportation cask lid bolts are independently verified to have been removed prior to moving the cask from the cask preparation area to the unloading room.	This control prevents the CTM from attempting to remove the cask lid with bolts still in place resulting in failure of the bolts and possible drop of the lid or cask.	RF-ESD06-TAD (Seq. 3-3)
5	CTM Port Slide Gates	At completion of a canister transfer operation, the port slide gates are verified to be closed	While the CTM is being used to perform transfer operations, the Operational Radiation Protection Program provides the necessary controls to ensure that workers are not present with the slide gates open. This control limits the probability of workers receiving a direct exposure by entering the transfer room with the CTM away from a port with a waste form container present and the slide gate open.	RF-ESD11 (Seq. 2)

Table 6.9-2. Summary of Procedural Safety Controls for the Receipt Facility (Continued)

Item	SSC	Procedural Safety Controls	Basis for Selection	Representative Event Sequence
6	CTM	Prior to lifting or lowering a DPC or TAD canister, the CTM guide sleeve is to be verified to have been lowered.	This control limits the probability that a DPC or TAD canister is not in a vertical orientation during transfer such that any potential drops would be flat bottom drops.	RF-ESD06-TAD (Seq. 3-3)
7	Radiation Controlled Areas	Personnel will not enter radiation controlled areas without proper authorization from the control room. Under normal operating conditions, personnel will never enter radiation controlled areas when radiation lights are on outside the room.	To limit the probability of operators receiving a direct exposure by inadvertently entering a high radiation area.	All waste forms in: RF-ESD11

NOTE: AO = aging overpack; CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; ST = site transporter; TAD = transportation, aging, and disposal.

Source: Original

7. RESULTS AND CONCLUSIONS

This analysis report on the RF and its predecessor companion report, *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34), are part of the PCSA for the GROA that supports the license application. In combination, these documents identify, evaluate, quantify, and categorize event sequences for the GROA facilities and operations. They are part of a collection of analysis reports that encompass all waste handling activities and facilities of the GROA from initial operations to the end of the preclosure period. Probabilistic risk assessment techniques derived from both nuclear power plant and aerospace methods are used to perform the analyses to comply with the risk-informed aspects of 10 CFR 63.111 and 63.112 (Ref. 2.3.2) and to be responsive to the acceptance criteria articulated in the *Yucca Mountain Review Plan, Final Report,* NUREG-1804 (Ref. 2.2.68). The identification and development of the event sequences is limited to those that might lead to direct radiation exposure of workers or onsite members of the public, radiological releases that may affect workers or the public (onsite and offsite), and nuclear criticality.

The results of the analysis are discussed and presented in the logical progression through Section 6 of this document and are not reiterated here. Instead, only key points are highlighted. For the ungrouped event sequence results and the complete grouped event sequence summaries, electronic files are provided due to the large size of hard copy versions (refer to Attachments G and H). In addition, although the results from the SAPHIRE model are used and presented in Section 6 and Attachment B, the model itself is difficult to completely represent in paper form. Therefore, these outputs are also provided electronically (refer to Attachment H). Table 7-1 describes the results and indicates the location within this analysis for each result provided.

Table 7-1. Key to Results

Result	Description	Cross Reference
Grouping of event sequences	Grouping of event sequences and description of event sequence groups	Table G-1
Quantification of event sequences	Calculation of probability distributions for the numbers of occurrences of internal event sequence groups over the preclosure period	Table G-2
Categorization of event sequences	Assignment of frequency categories Category 1, Category 2, or beyond Category 2 to internal event sequence groups based on mean numbers of occurrences	Table 6.8-2 Table 6.8-3 Table G-3
Designation of structures, systems, and components as important to safety	Identification of SSCs that are relied on in the quantification of internal event sequences for prevention or mitigation	Table 6.9-1
Statement of nuclear safety design bases	List of nuclear safety design bases for SSCs that are relied on in the quantification of internal event sequences for prevention or mitigation	Table 6.9-1
Statement of procedural safety controls	List of procedural safety controls that are relied on in the quantification of internal event sequences for prevention or mitigation	Table 6.9-2

NOTE: SSCs = structures, systems, and components.

Source: Original

Summary of Event Sequences

The analysis concludes that there are no Category 1 event sequences and 7 Category 2 event sequences. Table 7-2 gives the number of Category 2 event sequences by end state for each waste form.

Table 7-2. Summary of Category 2 Event Sequences

End State	Description	Canister Types		
		DPC	TAD	TAD or DPC[a]
DE-SHIELD-DEGRADE	Direct exposure due to degradation of shielding	None	1	None
DE-SHIELD-LOSS	Direct exposure due to loss of shielding	2	3	1
RR-UNFILTERED	Radionuclide release, unfiltered	None	None	None
RR-FILTERED	Radionuclide release, filtered	None		None
RR-UNFILTERED-ITC	Radionuclide release, unfiltered, also important to criticality	None	None	None
RR-FILTERED-ITC	Radionuclide release, filtered, also important to criticality	None	None	None
ITC	Important to criticality	None	None	None

NOTES: [a]The event sequences counted here are not specific to canister type.
DPC = dual-purpose canister; TAD = transportation, aging, and disposal canister.

Source: Original

Summary of Conservatisms

It should be noted that the event sequence identification and categorization were conducted with conservatisms that increase confidence in the results. These conservatisms include those listed below:

1. Fire frequency and damage analyses are performed without relying on fire suppression. This increases the calculated frequency of large fires and also increases the duration and peak temperature of fires, thereby significantly increasing the calculated probability of waste container failure.

2. If a fire is calculated to propagate out of the initiating location fire zone, the entire building is considered to be involved in the fire.

3. In the PEFA for thermal and fire scenarios, conservatism is built into the boundary conditions, which consider the fire as occurring next to the waste containers instead of only a fraction of the fire occurrence being near the waste form. A fire closer to the target will lead to a higher target failure probability than a fire located further away. By considering all fires to be next to the waste forms, the thermal PEFA yields higher waste form failure probabilities than is likely.

4. For event sequences in which a cask containing a canister is subjected to a drop, slapdown, or in which a load is dropped onto the cask, the calculated containment failure probability pertains to the canister inside without regard to the integrity of the cask. That is, cask containment is not relied upon to reduce probability of containment failure.

5. The structural PEFA uses a conservative failure probability of 1E-5, whereas the actual PEFA assessment indicates values of less than 1E-8 failure probabilities (Table D1.2-7 of Attachment D). This conservatism provides event sequence quantification results orders of magnitude higher than what they would be if the actual PEFA assessment values are used.

6. The event sequence development for shielding degradation of transportation casks caused by an impact event considers all casks as if they contained lead gamma shielding that could slump. However, not all transportation casks received at the GROA will be leaded casks. Because non-leaded casks are not affected by this degraded shielding condition, the introduction of this conservatism increases the event sequence quantification value.

7. The structural analyses for drops and collisions of canisters or casks model a rigid, unyielding surface as the target.

8. The structural analysis for drops of loads onto casks or canisters uses a rigid unyielding object for the dropped load.

9. The probabilities of event sequences involving drops of casks and canisters represent a drop height of up to 40 ft for casks and 45 ft for bare canisters. This is much higher than the normal operational lift height but is applied for all lower drop heights. Lower drop heights would result in less structural challenge to casks and canisters.

10. When a canister is inside a waste package, failure of the waste package is considered to fail containment. That is, the canister is not relied upon to reduce the probability of containment failure.

11. Transportation casks are analyzed without impact limiters even for those event sequences in which impact limiters would be attached.

12. The speed limitation of crane and conveyances within facilities to 20 ft/min and 2.5 mph, respectively, is set to ensure no breach of casks or canisters. The probability of breach at such speeds is calculated to be less than 1E-08 per impact. Speeds could be considerably larger without changing the categorizations of event sequences.

13. The reliability evaluation of the ITS HVAC system, which provides confinement of radioactive material releases following a breach of a waste container, is based a mission time of 720 hrs (30 days). The use of this mission time in the analysis leads to a requirement that the emergency diesel generators provide power to the HVAC for 720 hrs following a release. The analysis does not account for the high likelihood of recovering offsite power within the mission time. Recovery of offsite power would reduce the length of time that the diesel generators would be required to run and would thereby reduce the calculated unavailability of the diesel generators. This conservative consideration leads to a lower ITS HVAC availability than is realistically expected.

14. The human reliability analysis screening values used for human failure events are typically one or more orders of magnitude higher than values that would be obtained through detailed analysis.

15. The probability of failure associated with the structural analysis of mechanical impact loads to casks and canisters is conservatively based on the maximum effective plastic strain of any brick (i.e., finite element mesh) in the modeled structure rather than on evidence of through-wall cracking.

16. Categorization of event sequences is based on the highest category after application of a conservative adjustment to account for the uncertainty in the calculated uncertainties.

17. To preserve flexibility in the conduct of operations, the throughput analysis (Ref. 2.2.27) embeds multiple and bounding waste handling scenarios in the throughput numbers. For example, it considers that all TAD canisters and DPCs could transit through the RF on their way to the Aging Facility. In fact, the capability to transfer of TAD canisters and DPCs from transportation casks to aging overpacks is shared between the RF and the CRCF. As a result, the allocated numbers for both facilities are higher than is realistically expected. Including this conservatism in the analysis yields calculated event sequence frequencies that are higher than is realistically expected.

ATTACHMENT A
EVENT TREES

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ATTACHMENT A
EVENT TREES

A1 INTRODUCTION

This attachment presents event trees that are derived from the ESDs in Attachment F of the *Receipt Facility Event Sequence Development Analysis* (Ref. 2.2.34). All initiator event trees and system response event trees are located at the end of this attachment. Refer to Table A5-1 for the figure locations of specific event and response trees. The event trees are presented in Figures A5-2 through A5-27 according to ordering rules of hierarchy in SAPHIRE. The first rule is that event trees are presented in ESD order. For example, the event trees associated with RF-ESD-01 appear first, and those associated with RF-ESD-02 appear after that, and so on. The second rule is that the first initiator event tree associated with the ESD appears first and the system response event trees are placed immediately following the first initiator event tree followed by the remaining initiator event trees for the ESD. For example, the first initiator event tree (RF-ESD01-DPC) associated with the first ESD (RF-ESD-01) is the first event tree figure. Then the system response event tree (RESPONSE-TCASK1) appears, followed by the remaining initiator event trees for the ESD (RF-ESD01-TAD). The same kind of ordering is done for each group in turn.

A2 READER'S GUIDE TO THE EVENT TREE DESCRIPTIONS

The following sections are organized by ESD. The event trees that correspond to each ESD are presented as follows:

1. The event trees for the waste forms covered are briefly described and listed (initiator and system response event trees or self contained event trees, as applicable).

2. The initiating events are described and listed. The listing is provided as a table that includes the assignments of fault trees or basic events to the initiating events. The assignments are made in SAPHIRE using basic rules or by fault tree construction. The goal of the initiating event table is to provide a link to the underlying system fault tree (Section 6.2 and Attachment B) or basic event (Section 6.3 and Attachment C). In a few cases, the assignment is not straightforward and a supplemental fault tree provides a link to the system fault tree or basic event level (Attachment B). Note that the initiating event frequencies are defined on a per-unit-handled basis. Thus, when the initiating event frequencies are multiplied by the number of units handled over the preclosure period, the result is an initiating event frequency over the preclosure period.

3. The system response event tree that corresponds to the initiator event tree or the system response for a self-contained event tree is covered as follows. Each pivotal event used in an event tree is listed in the event tree description section and summarized in Section A3. Each pivotal event is accompanied by a table that provides a link between the name given to the pivotal event in the event tree and the associated system fault tree or basic event. The goal of the pivotal event table is to provide a link to the underlying system fault tree (Section 6.2) or basic event (Section 6.3). In a few

cases, the assignment is not straightforward and a supplemental fault tree provides a link to the system fault tree or basic event level.

A3 SUMMARY OF THE MAJOR PIVOTAL EVENT TYPES

A self-contained event tree or a system response event tree may include pivotal events of following types:

CELL-DOOR. This pivotal event represents the success or failure of the shield door to not fail and damage waste forms.

TRANSCASK. This pivotal event represents the success or failure of the transportation cask to contain radioactive material after the impact caused by the initiating event. The failure of this pivotal event leads to the loss of the cask's containment function. The failure probability for this pivotal event is determined by PEFA, and is given in Table 6.3-4 in Section 6.3.2. In accordance with a simplifying approximation, the same failure probability is used for all casks for the various initiating events.

CANISTER. This pivotal event represents the success or failure of the canister to contain radioactive material after the impact caused by the initiating event. Failure of a containment pivotal event means that a release could occur if the canister containment barrier is breached (along with the cask or waste package containment, as applicable). In accordance with a simplifying approximation, the conditional probability of canister breach given cask breach is taken to be 1.

SHIELDING. Failure of a shielding pivotal event means that a direct exposure could occur. Casks, some canisters, the cask transfer machine shield bell, and the aging overpack include integral shields that could be pierced or degraded in some impact events. In addition, a breach of a container's seal can also result in a loss of shielding. Thus, this pivotal event represents the success or failure of the shielding function of the cask, canister, or aging overpack after the impact caused by the initiating event. Failure of shielding in this instance refers to an unspecified degree of shielding degradation due to the impact.

Loss of shielding is also a consequence of loss of containment (e.g., failure of the cask or canister). The response trees of Section A5 indicate shielding loss only in the event containment is not breached. If containment is breached shielding loss occurs along with a radiation release in the form of particulate mass which has significantly greater consequence than shine from a shielding loss.

CONFINEMENT. This pivotal event represents the success or failure of the HVAC system in continuing to provide HEPA filtration (radiological confinement) after the initiating event. Success of the pivotal event requires the facility structural integrity as well as the functioning of equipment associated with the HVAC system. Failure results in a potential airborne release that is not mitigated by the HEPA filtration system.

MODERATOR. This pivotal event represents the conditional probability of introducing liquid moderator (water or crane gearbox lubricating oil) into a breached canister, given that a breached canister is present. The conditional probability of failure (introduction of liquid moderator) is the same for all waste forms and all initiating events. Failure of a moderator pivotal event results in an end state that may be susceptible to nuclear criticality. The opportunity for criticality also depends on other pivotal events (e.g., loss of containment, which may allow liquid moderator into a breached canister) and physical properties of the waste form.

Each of the specific failure events included in a self-contained or system response event tree may be linked to a basic event or to the top event of a fault tree that represents equipment failure modes and human failure events that can initiate the specific event. The fault tree models are, in turn, linked to basic events that provide the failure frequencies. Some of the pivotal events represent failure of equipment whose failure probabilities are linked to a separately developed basic event and not to a fault tree.

A4 EVENT TREE DESCRIPTIONS

A4.1 EVENT TREES FOR RF-ESD-01

RF-ESD-01 covers event sequences associated with receipt of a railcar carrying a transportation cask (Ref. 2.2.34, Figure F-1). This ESD covers two types of transportation casks. Corresponding to each type of cask is an initiator event tree (Table A4.1-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.1-1. Summary of Event Trees for RF-ESD-01

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Transportation cask containing a DPC	Initiator: RF-ESD01-DPC Response: RESPONSE-TCASK1	346
Transportation cask containing a TAD canister	Initiator: RF-ESD01-TAD Response: RESPONSE-TCASK1	6,976

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.1.1 Initiating Events for RF-ESD-01

The following initiating events are associated with RF-ESD-01. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.1-2.

Railcar Derailment. This initiating event accounts for the potential impact to the transportation cask on the railcar due to a derailment. The probability of derailment per railcar received is derived from empirical data in Section 6.3 and is modeled as a single event fault tree as described in Section 6.2.2. The fault tree reflects the expectation that only rail casks will be received at the RF. The initiating event is specified as a probability of derailment per cask.

Railcar Collision. This initiating event covers the potential impact to the transportation cask on the conveyance due to a collision with another vehicle. The vehicular collision event is modeled as a fault tree and is listed in Section 6.2.2. The initiating event is specified as a probability of collision per cask.

Table A4.1-2. Initiating Event Assignments for RF-ESD-01

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Railcar derailment	RF-ESD01-DPC	ESD1-DPC-DERAIL	200-SPMRC-DERIL-PER-MILE **AND** 200-SPMRC-MILES-IN-RF
	RF-ESD01-TAD	ESD1-TAD-DERAIL	
Railcar collision	RF-ESD01-DPC	ESD1-DPC-COLLIDE	No further transfers
	RF-ESD01-TAD	ESD1-TAD-COLLIDE	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.1.2 System Response Event Tree RESPONSE-TCASK1

The pivotal events that appear in RESPONSE-TCASK1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

TRANSCASK. Table A4.1-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.1-3. Basic Event Associated with the TRANSCASK Pivotal Events of RF-ESD-01

Initiator Event Tree	Initiating Event Name	Name Assigned to TRANSCASK	Associated Fault Tree or Basic Event
RF-ESD01-DPC	ESD1-DPC-DERAIL	ESD1-DPC-DERAIL-TCASK	TCASK
	ESD1-DPC-COLLIDE	ESD1-DPC-COLLIDE-TCASK	
RF-ESD01-TAD	ESD1-TAD-DERAIL	ESD1-TAD-DERAIL-TCASK	
	ESD1-TAD-COLLIDE	ESD1-TAD-COLLIDE-TCASK	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CANISTER. Table A4.1-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.1-4. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-01

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD01-DPC	ESD1-DPC-DERAIL	ESD1-DPC-DERAIL-CAN	DPC-FAIL-IN-TC
	ESD1-DPC-COLLIDE	ESD1-DPC-COLLIDE-CAN	
RF-ESD01-TAD	ESD1-TAD-DERAIL	ESD1-TAD-DERAIL-CAN	TAD-FAIL-IN-TC
	ESD1-TAD-COLLIDE	ESD1-TAD-COLLIDE-CAN	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

SHIELDING. Table A4.1-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.1-5. Basic Event Associated with the SHIELDING Pivotal Events of RF-ESD-01

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD01-DPC	ESD1-DPC-DERAIL	ESD1-DPC-DERAIL-SHIELD	TCASK-SHIELDING
	ESD1-DPC-COLLIDE	ESD1-DPC-COLLIDE-SHIELD	
RF-ESD01-TAD	ESD1-TAD-DERAIL	ESD1-TAD-DERAIL-SHIELD	
	ESD1-TAD-COLLIDE	ESD1-TAD-COLLIDE-SHIELD	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.1-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.1-6. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-01

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD01-DPC	ESD1-DPC-DERAIL	200-CONFINEMENT	200-CONFINEMENT
	ESD1-DPC-COLLIDE		
RF-ESD01-TAD	ESD1-TAD-DERAIL		
	ESD1-TAD-COLLIDE		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.1-7 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.1-7. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-01

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD01-DPC	ESD1-DPC-DERAIL	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD1-DPC-COLLIDE		
RF-ESD01-TAD	ESD1-TAD-DERAIL		
	ESD1-TAD-COLLIDE		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.2 EVENT TREES FOR RF-ESD-02

RF-ESD-02 covers event sequences associated with removal of impact limiters from the transportation cask, upending the transportation cask, and transferring it to the CTT (Ref. 2.2.34, Figure F-3). This ESD covers two types of transportation casks. Corresponding to each type of cask is an initiator event tree (Table A4.2-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.2-1. Summary of Event Trees for RF-ESD-02

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Transportation cask containing a DPC	Initiator: RF-ESD02-DPC Response: RESPONSE-TCASK1	346
Transportation cask containing a TAD canister	Initiator: RF-ESD02-TAD Response: RESPONSE-TCASK1	6,976

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.2.2 Initiating Events for RF-ESD-02

The following initiating events are associated with RF-ESD-02. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.2-2.

Cask Drop from Operational Height. This initiating event accounts for the potential impact to the transportation cask due to having been dropped from the normal operational height during transfer by the cask handling crane. The probability of drop per transfer is derived from empirical data in Section 6.3 and is modeled as a single event fault tree as described in Attachment B. The initiating event is specified as a probability of a drop per cask.

Cask Tipover. This initiating event covers the potential impact to the transportation cask due to a tipover. The tipover event is modeled as a single event fault tree and is listed in Attachment B. The initiating event is specified as a probability of a tipover per cask.

Side Impact to Cask. This initiating event covers the potential impact to the transportation cask due to a vehicular collision or (for transportation casks that are upended on a railcar (TTCs)) a failure of the tilt frame. This event is modeled as a fault tree and is listed in Attachment B. The initiating event is specified as a probability of an impact per cask.

Unplanned Conveyance Movement. This initiating event covers the potential impact to the transportation cask due to an unplanned movement of the cask handling crane or cask transfer trolley. This event is modeled as a fault tree and is listed in Attachment B. The initiating event is specified as a probability of movement per cask.

Object Dropped on Cask. This initiating event covers the potential impact to the transportation cask due to the drop of a heavy object, such as an impact limiter, on the cask. This event is modeled as a fault tree and is listed in Attachment B. The initiating event is specified as a probability of an object drop per cask.

Cask Drop from Above Operational Height. This initiating event accounts for the potential impact to the transportation cask due to having been dropped from above the normal operational height (for example, due to two-blocking) during transfer by the cask handling crane. The probability of drop per transfer is modeled as a fault tree as described in Attachment B. The initiating event is specified as a probability of a drop per cask.

Table A4.2-2. Initiating Event Assignments for RF-ESD-02

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Cask drop from operational height	RF-ESD02-DPC	ESD2-DPC-DROP	200-TILTFRAME-CSC-FOH **OR** 200-DPC-CRANE-DROP
	RF-ESD02-TAD	ESD2-TAD-DROP	200-CRN2-DROPTAD-CRN-DRP **AND** 200-TRANSNSCTTLIFTNUMBER
Transportation cask tipover	RF-ESD02-DPC	ESD2-DPC-TIP	200-OPTIPOVER001-HFI-NOD
	RF-ESD02-TAD	ESD2-TAD-TIP	200-OP-TIPOVER
Side impact	RF-ESD02-DPC	ESD2-DPC-IMPACT	No further transfers
	RF-ESD02-TAD	ESD2-TAD-IMPACT	
Unplanned conveyance movement	RF-ESD02-DPC	ESD2-DPC-MOVE	200-CRANE-SPURMOVE **OR** 200-CTT-SPURMOVE
	RF-ESD02-TAD	ESD2-TAD-MOVE	
Object dropped on a cask	RF-ESD02-DPC	ESD2-DPC-DROPON	200-200T-CRANE-DROPON
	RF-ESD02-TAD	ESD2-TAD-DROPON	
Cask drop from above operational height	RF-ESD02-DPC	ESD2-DPC-2BLK	200-CRN2-2-BLOCK-CRN-TBK **AND** 200-TRANSCTTLIFTNUMBER
	RF-ESD02-TAD	ESD2-TAD-2BLK	200-CRN2-2-BLOCK-CRN-TBK **AND** 200-TRANSNSCTTLIFTNUMBER

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.2.3 System Response Event Tree RESPONSE-TCASK1

The pivotal events that appear in RESPONSE-TCASK1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

TRANSCASK. Table A4.2-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.2-3. Basic Event Associated with the TRANSCASK Pivotal Events of RF-ESD-02

Initiator Event Tree	Initiating Event	Name Assigned to TRANSCASK	Associated Fault Tree or Basic Event
RF-ESD02-DPC	ESD2-DPC-DROP	ESD2-DPC-DROP-TCASK	TCASK-MISC-DROP
	ESD2-DPC-TIP	ESD2-DPC-TIP-TCASK	TCASK-TIPOVER
	ESD2-DPC-IMPACT	ESD2-DPC-IMPACT-TCASK	TCASK-MISC-IMP
	ESD2-DPC-MOVE	ESD2-DPC-MOVE-TCASK	TCASK-SPURMOVE
	ESD2-DPC-DROPON	ESD2-DPC-DROPON-TCASK	TCASK-MISC-DROP
	ESD2-DPC-2BLK	ESD2-DPC-2BLK-TCASK2	TCASK-2BLOCK
RF-ESD02-TAD	ESD2-TAD-DROP	ESD2-TAD-DROP-TCASK	TCASK-MISC-DROP
	ESD2-TAD-TIP	ESD2-TAD-TIP-TCASK	TCASK-TIPOVER
	ESD2-TAD-IMPACT	ESD2-TAD-IMPACT-TCASK	TCASK-MISC-IMP
	ESD2-TAD-MOVE	ESD2-TAD-MOVE-TCASK	TCASK-SPURMOVE
	ESD2-TAD-DROPON	ESD2-TAD-DROPON-TCASK	TCASK-MISC-DROP
	ESD2-TAD-2BLK	ESD2-TAD-2BLK-TCASK2	TCASK-2BLOCK

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CANISTER. Table A4.2-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.2-4. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-02

Initiator Event Tree	Initiating Event	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD02-DPC	ESD2-DPC-DROP	ESD2-DPC-DROP-CAN	DPC_FAIL_IN_TC
	ESD2-DPC-TIP	ESD2-DPC-TIP-CAN	
	ESD2-DPC-IMPACT	ESD2-DPC-IMPACT-CAN	
	ESD2-DPC-MOVE	ESD2-DPC-MOVE-CAN	
	ESD2-DPC-DROPON	ESD2-DPC-DROPON-CAN	
	ESD2-DPC-2BLK	ESD2-DPC-2BLK-CAN2	
RF-ESD02-TAD	ESD2-TAD-DROP	ESD2-TAD-DROP-CAN	TAD_FAIL_IN_TC
	ESD2-TAD-TIP	ESD2-TAD-TIP-CAN	
	ESD2-TAD-IMPACT	ESD2-TAD-IMPACT-CAN	
	ESD2-TAD-MOVE	ESD2-TAD-MOVE-CAN	
	ESD2-TAD-DROPON	ESD2-TAD-DROPON-CAN	
	ESD2-TAD-2BLK	ESD2-TAD-2BLK-CAN2	

NOTE DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

SHIELDING. Table A4.2-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.2-5. Basic Events Associated with the SHIELDING Pivotal Events of RF-ESD-02

Initiator Event Tree	Initiating Event	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD02-DPC	ESD2-DPC-DROP	ESD2-DPC-DROP-SHIELD	TCASK-SHIELDING-DROP
	ESD2-DPC-TIP	ESD2-DPC-TIP-SHIELD	TCASK-SHIELDING-DROP
	ESD2-DPC-MOVE	ESD2-DPC-MOVE-SHIELD	TCASK-SHIELDING-IMP
	ESD2-DPC-DROPON	ESD2-DPC-DROPON-SHIELD	TCASK-SHIELDING-DROP
	ESD2-DPC-IMPACT	ESD2-DPC-IMPACT-SHIELD	TCASK-SHIELDING-IMP
	ESD2-DPC-2BLK	ESD2-DPC-2BLK-SHIELD2	TCASK-SHIELDING-2BLK
RF-ESD02-TAD	ESD2-TAD-DROP	ESD2-TAD-DROP-SHIELD	TCASK-SHIELDING-DROP
	ESD2-TAD-TIP	ESD2-TAD-TIP-SHIELD	TCASK-SHIELDING-DROP
	ESD2-TAD-MOVE	ESD2-TAD-MOVE-SHIELD	TCASK-SHIELDING-IMP
	ESD2-TAD-DROPON	ESD2-TAD-DROPON-SHIELD	TCASK-SHIELDING-DROP
	ESD2-TAD-IMPACT	ESD2-TAD-IMPACT-SHIELD	TCASK-SHIELDING-IMP
	ESD2-TAD-2BLK	ESD2-TAD-2BLK-SHIELD2	TCASK-SHIELDING-2BLK

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.2-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.2-6. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-02

Initiator Event Tree	Initiating Event	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD02-DPC	ESD2-DPC-DROP	200-CONFINEMENT	200-CONFINEMENT
	ESD2-DPC-TIP		
	ESD2-DPC-IMPACT		
	ESD2-DPC-MOVE		
	ESD2-DPC-DROPON		
	ESD2-DPC-2BLK		
RF-ESD02-TAD	ESD2-TAD-DROP		
	ESD2-TAD-TIP		
	ESD2-TAD-IMPACT		
	ESD2-TAD-MOVE		
	ESD2-TAD-DROPON		
	ESD2-TAD-2BLK		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.2-7 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.2-7. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-02

Initiator Event Tree	Initiating Event	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD02-DPC	ESD2-DPC-DROP	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD2-DPC-TIP		
	ESD2-DPC-IMPACT		
	ESD2-DPC-MOVE		
	ESD2-DPC-DROPON		
	ESD2-DPC-2BLK		
RF-ESD02-TAD	ESD2-TAD-DROP		
	ESD2-TAD-TIP		
	ESD2-TAD-IMPACT		
	ESD2-TAD-MOVE		
	ESD2-TAD-DROPON		
	ESD2-TAD-2BLK		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.3 EVENT TREES FOR RF-ESD-03

RF-ESD-03 covers event sequences for cask preparation activities associated with unbolting and installation of the cask lid adaptor (Ref. 2.2.34, Figure F-4). This ESD covers two types of transportation casks. Corresponding to each type of cask is an initiator event tree (Table A4.3-1). Although the initiator event trees transfer to the same system response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.3-1. Summary of Event Trees for RF-ESD-03

Waste Form Units	Associated Event Trees	Number of Waste Form Units
Transportation cask containing a DPC	Initiator: RF-ESD03-DPC Response: RESPONSE-TCASK1	346
Transportation cask containing a TAD	Initiator: RF-ESD03-TAD Response: RESPONSE-TCASK1	6,976

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.3.1 Initiating Events for RF-ESD-03

The following initiating events are associated with RF-ESD-03. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.3-2.

Cask Drop. This initiating event represents a potential impact to the transportation cask due to having been dropped by the cask handling crane due to a failure to remove the lid bolts before attempting to lift off the lid. The probability of this initiating event per cask received is modeled as a fault tree and is discussed in Attachment B. The initiating event is specified as a probability of a drop per cask.

Cask Tipover. This initiating event covers a tipover of the unsealed transportation cask due to an improper interaction of the cask or cask transfer trolley with the cask handling crane or cask preparation crane. The probability of this initiating event per cask received is modeled as a fault tree and is discussed in Attachment B. The initiating event is specified as a probability of a tipover per cask.

Side Impact to Cask. This initiating event covers an impact to the side of the cask due to improper movement by the cask preparation crane. The probability of this initiating event per cask received is modeled as a fault tree and is discussed in Attachment B. The initiating event is specified as a probability of a tipover per cask handled.

Drop of Heavy Load onto Cask. This initiating event covers the drop of a heavy object onto the cask by the cask preparation crane. The probability of this initiating event per cask received is modeled as a fault tree and is discussed in Attachment B. The initiating event is specified as a probability of a drop per cask.

Table A4.3-2. Initiating Event Assignments for RF-ESD-03

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Cask drop	RF-ESD03-DPC	ESD3-DPC-DROP	200-OPCASKDROP01-HFI-NOD
	RF-ESD03-TAD	ESD3-TAD-DROP	
Transportation cask tipover	RF-ESD03-DPC	ESD3-DPC-TIP	200-CRANE-SPURMOVE **OR** 200-OPTIPOVER002-HFI-NOD
	RF-ESD03-TAD	ESD3-TAD-TIP	
Side impact	RF-ESD03-DPC	ESD3-DPC-IMPACT	No further transfers
	RF-ESD03-TAD	ESD3-TAD-IMPACT	
Drop of heavy load onto cask	RF-ESD03-DPC	ESD3-DPC-DROPON	No further transfers
	RF-ESD03-TAD	ESD3-TAD-DROPON	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.3.2 System Response Event Tree RESPONSE-TCASK1

The pivotal events that appear in RESPONSE-TCASK1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

TRANSCASK. Table A4.3-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.3-3. Basic Event Associated with the TRANSCASK Pivotal Events of RF-ESD-03

Initiator Event Tree	Initiating Event Name	Name Assigned to TRANSCASK	Associated Fault Tree or Basic Event
RF-ESD03-DPC	ESD3-DPC-DROP	ESD3-DPC-DROP-TCASK	TCASK-MISC-DROP
	ESD3-DPC-TIP	ESD3-DPC-TIP-TCASK	TCASK-TIPOVER
	ESD3-DPC-IMPACT	ESD3-DPC-IMPACT-TCASK	TCASK-MISC-IMP
	ESD3-DPC-DROPON	ESD3-DPC-DROPON-TCASK	TCASK-MISC-DROPON
RF-ESD-03-TAD	ESD3-TAD-DROP	ESD3-TAD-DROP-TCASK	TCASK-MISC-DROP
	ESD3-TAD-TIP	ESD3-TAD-TIP-TCASK	TCASK-TIPOVER
	ESD3-TAD-IMPACT	ESD3-TAD-IMPACT-TCASK	TCASK-MISC-IMP
	ESD3-TAD-DROPON	ESD3-TAD-DROPON-TCASK	TCASK-MISC-DROPON

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CANISTER. Table A4.3-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.3-4. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-03

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD03-DPC	ESD3-DPC-DROP	ESD3-DPC-DROP-CAN	DPC_FAIL_IN_TC
	ESD3-DPC-TIP	ESD3-DPC-TIP-CAN	
	ESD3-DPC-IMPACT	ESD3-DPC-IMPACT-CAN	
	ESD3-DPC-DROPON	ESD3-DPC-DROPON-CAN	
RF-ESD-03-TAD	ESD3-TAD-DROP	ESD3-TAD-DROP-CAN	TAD_FAIL_IN_TC
	ESD3-TAD-TIP	ESD3-TAD-TIP-CAN	
	ESD3-TAD-IMPACT	ESD3-TAD-IMPACT-CAN	
	ESD3-TAD-DROPON	ESD3-TAD-DROPON-CAN	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

SHIELDING. Table A4.3-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.3-5. Basic Event Associated with the SHIELDING Pivotal Events of RF-ESD-03

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD03-DPC	ESD3-DPC-DROP	ESD3-DPC-DROP-SHIELD	TCASK-SHIELDING-DROP
	ESD3-DPC-TIP	ESD3-DPC-TIP-SHIELD	TCASK-SHIELDING-DROP
	ESD3-DPC-DROPON	ESD3-DPC-DROPON-SHIELD	TCASK-SHIELDING-DROP
	ESD3-DPC-IMPACT	ESD3-DPC-IMPACT-SHIELD	TCASK-SHIELDING-IMP
RF-ESD-03-TAD	ESD3-TAD-DROP	ESD3-TAD-DROP-SHIELD	TCASK-SHIELDING-DROP
	ESD3-TAD-TIP	ESD3-TAD-TIP-SHIELD	TCASK-SHIELDING-DROP
	ESD3-TAD-DROPON	ESD3-TAD-DROPON-SHIELD	TCASK-SHIELDING-DROP
	ESD3-TAD-IMPACT	ESD3-TAD-IMPACT-SHIELD	TCASK-SHIELDING-IMP

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. This pivotal event represents the success or failure of the HVAC system in continuing to provide radiological confinement after the initiating event. Success of the pivotal event requires the facility structural integrity as well as the functioning of equipment associated with the HVAC system. Table A4.3-6 specifies the fault tree that is associated with this pivotal event for each initiating event.

Table A4.3-6. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-03

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD03-DPC	ESD3-DPC-DROP	200-CONFINEMENT	200-CONFINEMENT
	ESD3-DPC-TIP		
	ESD3-DPC-IMPACT		
	ESD3-DPC-DROPON		
RF-ESD-03-TAD	ESD3-TAD-DROP		
	ESD3-TAD-TIP		
	ESD3-TAD-IMPACT		
	ESD3-TAD-DROPON		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.3-7 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.3-7. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-03

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD03-DPC	ESD3-DPC-DROP	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD3-DPC-TIP		
	ESD3-DPC-IMPACT		
	ESD3-DPC-DROPON		
RF-ESD-03-TAD	ESD3-TAD-DROP		
	ESD3-TAD-TIP		
	ESD3-TAD-IMPACT		
	ESD3-TAD-DROPON		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.4 EVENT TREES FOR RF-ESD-04

RF-ESD-04 covers event sequences for transferring either a cask or aging overpack from the Cask Preparation Area to the Cask Unloading Room (Ref. 2.2.34, Figure F-6). This ESD covers aging overpacks and four types of transportation casks. Corresponding to each type of cask or aging overpack is an initiator event tree (Table A4.4-1). Although the initiator event tree transfers to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.4-1. Summary of Event Trees for RF-ESD-04

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Transportation cask containing a DPC	Initiator: RF-ESD04-DPC Response: RESPONSE-TCASK2	346
Transportation cask containing a TAD canister	Initiator: RF-ESD04-TAD Response: RESPONSE-TCASK2	6,976

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.4.1 Initiating Events for RF-ESD-04

The following initiating events are associated with RF-ESD-04. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.4-2.

Impact Affecting a Transportation Cask or Aging Overpack. This initiating event represents a potential impact to the cask or aging overpack. The probability of impact per transfer is described in Section 6.2. The initiating event is specified as a probability of a drop per cask.

Collision Involving the CTT or Site Transporter. This initiating event represents a potential collision involving the CTT or site transporter. The probability of a collision is modeled as a fault tree as described in Attachment B. The initiating event is specified as a probability of a drop per cask.

Table A4.4-2. Initiating Event Assignments for RF-ESD-04

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Impact affecting transportation cask or aging overpack	RF-ESD04-DPC	ESD4-DPC-IMPACT	200-OPIMPACT0000-HFI-NOD
	RF-ESD04-TAD	ESD4-TAD-IMPACT	
Collision of CTT or site transporter	RF-ESD04-DPC	ESD4-DPC-COLLIDE	200-CTT-FAIL-STOP **OR** 200-OPCTCOLLIDE2-HFI-NOD
	RF-ESD04-TAD	ESD4-TAD-COLLIDE	

NOTE: CTT = cask transfer trolley; DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.4.2 System Response Event Tree RESPONSE-TCASK2

The pivotal events that appear in RESPONSE-TCASK2 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

CANISTER. Table A4.4-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.4-3. Fault Trees Associated with the CANISTER Pivotal Events of RF-ESD-04

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD04-DPC	ESD4-DPC-IMPACT	ESD4-DPC-IMPACT-CAN	DPC-FAIL-NO-CASK-IMP
	ESD4-DPC-COLLIDE	ESD4-DPC-COLLIDE-CAN	
RF-ESD04-TAD	ESD4-TAD-IMPACT	ESD4-TAD-IMPACT-CAN	TAD-FAIL- NO-CASK-IMP
	ESD4-TAD-COLLIDE	ESD4-TAD-COLLIDE-CAN	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

SHIELDING. Table A4.4-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.4-4. Fault Trees Associated with the SHIELDING Pivotal Events of RF-ESD-04

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD04-DPC	ESD4-DPC-IMPACT	ESD4-DPC-IMPACT-SHIELD	TCASK-SHIELDING-IMP
	ESD4-DPC-COLLIDE	ESD4-DPC-COLLIDE-SHIELD	
RF-ESD04-TAD	ESD4-TAD-IMPACT	ESD4-TAD-IMPACT-SHIELD	
	ESD4-TAD-COLLIDE	ESD4-TAD-COLLIDE-SHIELD	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.4-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.4-5. Fault Trees Associated with the CONFINEMENT Pivotal Events of RF-ESD-04

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD04-DPC	ESD4-DPC-IMPACT	200-CONFINEMENT	200-CONFINEMENT
	ESD4-DPC-COLLIDE		
RF-ESD04-TAD	ESD4-TAD-IMPACT		
	ESD4-TAD-COLLIDE		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.4-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.4-6. Fault Trees Associated with the MODERATOR Pivotal Events of RF-ESD-04

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD04-DPC	ESD4-DPC-IMPACT	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD4-DPC-COLLIDE		
RF-ESD04-TAD	ESD4-TAD-IMPACT		
	ESD4-TAD-COLLIDE		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.5 EVENT TREES FOR RF-ESD-05

RF-ESD-05 covers event sequences associated with collision of the shield door into the CTT or site transporter (Ref. 2.2.34, Figure F-1). For the CTT, the shield door involved is the door from the Cask Preparation Area to the Cask Unloading Room. For the site transporter, this door and the shield door between the site transporter entrance vestibule and the Cask Preparation Area apply to this event. This ESD covers aging overpacks and transportation casks.

The conveyance could collide into a stationary shield door or a moving shield door could collide into the conveyance. Since the shield doors are designed in accordance with the applicable provisions of *American National Standard Specification for the Design, Fabrication, and Erection of Steel Safety-Related Structures for Nuclear Facilities* (Ref.2.2.4) to withstand the load and acceleration produced by a DBGM-2 seismic event, it is reasonable to conclude that the shield doors would remain attached to their moorings in the event of a slow speed (maximum of 2.5 mph) collision of a conveyance with the shield door. Therefore the analysis only evaluates the impact of a moving shield door with the conveyance.

A4.5.1 Initiating Events for RF-ESD-05

Collision of Shield Door into CTT or site transporter. This initiating event accounts for a collision of a moving shield door with the CTT or site transporter. Since normal operations would not include the movement of the conveyance through the doorway while the shield door is closing, it is postulated that the door closes due to inadvertent actuation of the door. The probability of impact per transfer is derived from empirical data in Section 6.3 and is modeled as either a hardware failure or a human failure. The assignments made within SAPHIRE for quantification of this initiating event are indicated in Table A4.5-1.

Table A4.5-1. Initiating Event Assignments for RF-ESD-05

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Collision of shield door with CTT or site transporter	RF-ESD5-DPC	ESD5-DPC-IMPACT	200-CTT-COLLIDE-SDR **OR** 200-ST-COLLIDE-SDR[a,b] **AND** 200-ST-#-OF-SHIELD-DOORS
	RF-ESD5-TAD	ESD5-TAD-IMPACT	200-CTT-COLLIDE-SDR **OR** 200-ST-COLLIDE-SDR[a,b] **AND** 200-ST-#-OF-SHIELD-DOORS

NOTE: [a]Result of this fault tree is multiplied by factor of two to account for two shield doors.
[b]Split-fractions are used to account for percentage of operations involving the CTT and the site transporter.
CTT = cask transfer trolley; ESD = event sequence diagram; RF = Receipt Facility;
TAD = transportation, aging, and disposal canister.

Source: Original

A4.5.2 Pivotal Events

The pivotal events that appear in the event tree are listed below and summarized in Section A.3. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

CELL-DOOR. Table A4.5-2 indicates the fault trees or basic events that are associated with this pivotal event for each initiating event.

Table A4.5-2. Basic Events Associated with the CELL-DOOR Pivotal Events of RF-ESD-05

Initiator Event Tree	Initiating Event Name	Name Assigned to CELL-DOOR	Associated Fault Tree or Basic Event[a]
RF-ESD5-DPC	ESD5-DPC-IMPACT	ESD5-DPC-IMPACT-DOOR	SHIELD_DOOR_FAILURE
RF-ESD5-TAD	ESD5-TAD-IMPACT	ESD5-TAD-IMPACT-DOOR	

NOTE: [a]This column may contain fault trees and basic events. See Attachment B for fault trees and Attachment C for basic events.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility;
TAD = transportation, aging, and disposal canister.

Source: Original

CONTAINMENT. Table A4.5-3 indicates the fault trees or basic events that are associated with this pivotal event for each initiating event.

Table A4.5-3. Basic Events Associated with the CONTAINMENT Pivotal Events of RF-ESD-05

Initiator Event Tree	Initiating Event Name	Name Assigned to CONTAINMENT	Associated Fault Tree or Basic Event[a]
RF-ESD05-DPC	ESD5-DPC-IMPACT	ESD5-DPC-IMPACT-CONT	CAN-FAIL-SD-IMPACT
RF-ESD05-TAD	ESD5-TAD-IMPACT	ESD5-TAD-IMPACT-CONT	CAN-FAIL-SD-IMPACT

NOTE: [a]This column may contain fault trees and basic events. See Attachment B for fault trees and Attachment C for basic events.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

SHIELDING. Table A4.5-4 indicates the fault trees or basic events that are associated with this pivotal event for each initiating event.

Table A4.5-4. Basic Events Associated with the SHIELDING Pivotal Events of RF-ESD-05

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event[a]
RF-ESD05-DPC	ESD5-DPC-IMPACT	ESD5-DPC-IMPACT-SHIELD	TCASK-SHIELDING-IMP
RF-ESD05-TAD	ESD5-TAD-IMPACT	ESD5-TAD-IMPACT-SHI	TCASK-SHIELDING-IMP

NOTE: [a]This column may contain fault trees and basic events. See Attachment B for fault trees and Attachment C for basic events.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.5-5 indicates the fault trees or basic events that are associated with this pivotal event for each initiating event.

Table A4.5-5. Basic Events Associated with the CONFINEMENT Pivotal Events of RF-ESD-05

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event[a]
RF-ESD5-DPC	ESD5-DPC-IMPACT	200-CONFINEMENT	200-CONFINEMENT
RF-ESD5-TAD	ESD5-TAD-IMPACT		

NOTE: [a]This column may contain fault trees and basic events. See Attachment B for fault trees and Attachment C for basic events.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.5-6 indicates the fault trees or basic events that are associated with this pivotal event for each initiating event.

Table A4.5-6. Basic Events Associated with the MODERATOR Pivotal Events of RF-ESD-05

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event[a]
RF-ESD05-DPC	ESD05-DPC-IMPACT	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
RF-ESD05-TAD	ESD05-TAD-IMPACT		

NOTE: [a]This column may contain fault trees and basic events. See Attachment B for fault trees and Attachment C for basic events.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.6 EVENT TREES FOR RF-ESD-06

RF-ESD-06 covers event sequences associated with CTM transfers (Ref. 2.2.34, Figure F-9). This ESD covers all canister types. Corresponding to each canister type is an initiator event tree (Table A4.6-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.6-1. Summary of Event Trees for RF-ESD-06

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
DPC	Initiator: RF-ESD06-DPC Response: RESPONSE-CANISTER1	346
TAD canister	Initiator: RF-ESD06-TAD Response: RESPONSE-CANISTER1	6,976

NOTE: Numbers of units given are the total numbers available because, from the perspective of a CTM collision involving a given type of waste form, it is not known what the waste form inside the other CTM might be.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4).

A4.6.1 Initiating Events for RF-ESD-06

The following initiating events are associated with RF-ESD-06. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.6-2. The initiating events are specified as frequency of occurrence per canister.

Impact Associated with Lid Removal. This initiating event covers the potential impact during cask or aging overpack lid removal due to a human failure to remove all of the lid bolts.

Canister Drop from Operational Height. This initiating event accounts for the potential impact to the canister due to having been dropped from the normal operational height during transfer by the CTM.

Impact to Canister due to Conveyance Movement. This initiating event covers the potential impact to or shear of the canister due to untimely movement of the CTM, CTT, or site transporter during loading or unloading of the canister.

Side Impact to Canister. This initiating event covers the potential impact to the canister due to a CTM collision.

Object Dropped on Canister. This initiating event covers the potential impact to the canister due to the drop of a heavy object (e.g., cask lid) by the CTM.

Canister Drop inside Bell. This initiating event accounts for the potential impact to the canister due to having been dropped on the second floor during horizontal transfer by the CTM. This event has been subsumed within the canister drop from operational height event.

Canister Drop above Operational Height. This initiating event accounts for the potential impact to the canister due to having been dropped from above the normal operational height due to a two-blocking event during transfer by the CTM.

Table A4.6-2. Initiating Event Assignments for RF-ESD-06

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Impact with lid removal	RF-ESD06-DPC	ESD6-DPC-LIDIMP	Screened out, no lid removal for DPCs
	RF-ESD06-TAD	ESD6-TAD-LIDIMP	No further transfers
Canister drop (from operational height)	RF-ESD06-DPC	ESD6-DPC-DROP	200-LIFTS-PER-DPC-CAN **AND** CTM-DROP---ALL-HEIGHTS[a]
	RF-ESD06-TAD	ESD6-TAD-DROP	200-LIFTS-PER-TAD-CAN **AND** CTM-DROP---ALL-HEIGHTS[a]
Spurious movement	RF-ESD06-DPC	ESD6-DPC-SPUR	200-CTT-SPUR-MOVE **OR** 200-ST-SPURMOVE **OR** CTM-SHEAR
	RF-ESD06-TAD	ESD6-TAD-SPUR	
Side impact	RF-ESD06-DPC	ESD6-DPC-SIMPACT	200-LIFTS-PER-DPC-CAN **AND** CTM-COLLISION[a]
	RF-ESD06-TAD	ESD6-TAD-SIMPACT	200-LIFTS-PER-TAD-CAN **AND** CTM-COLLISION [a]
Object drop on canister	RF-ESD06-DPC	ESD6-DPC-DROPON	200-CTMOBJLIFTNUMBERD **AND** CTM-DROP-ONTO-CASK[a]
	RF-ESD06-TAD	ESD6-TAD-DROPON	200-CTMOBJLIFTNUMBER **AND** CTM-DROP-ONTO-CASK[a]
Canister drop inside bell	RF-ESD06-DPC	ESD6-DPC-CTMBELL	200-LIFTS-PER-DPC-CAN **AND** SHIELD-BELL-DROPS-SUBSUM
	RF-ESD06-TAD	ESD6-TAD-CTMBELL	200-LIFTS-PER-TAD-CAN **AND** SHIELD-BELL-DROPS-SUBSUM
Canister drop (above operational height)	RF-ESD06-DPC	ESD6-DPC-2BLK	200-LIFTS-PER-DPC-CAN **AND** CTM-2-BLOCK[a]
	RF-ESD06-TAD	ESD6-TAD-2BLK	200-LIFTS-PER-TAD-CAN **AND** CTM-2-BLOCK[a]

NOTE: [a]Basic event and fault tree connected by an AND gate.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility;
TAD = transportation, aging, and disposal canister.

Source: Original

A4.6.3 System Response Event Tree RESPONSE-CANISTER1

The pivotal events that appear in RESPONSE-CANISTER1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

CANISTER. Table A4.6-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.6-3. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-06

Initiator Event Tree	Initiating Event	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD06-DPC	ESD6-DPC-LIDIMP	ESD6-DPC-LIDIMP-CAN	DPC-FAIL-NO-CASK
	ESD6-DPC-DROP	ESD6-DPC-DROP-CAN	
	ESD6-DPC-SPUR	ESD6-DPC-SPUR-CAN	DPC-FAIL-SPURMOVE
	ESD6-DPC-SIMPACT	ESD6-DPC-SIMPACT-CAN	DPC-FAIL-CTM-IMPACT
	ESD6-DPC-DROPON	ESD6-DPC-DROPON-CAN	DPC-FAIL-NO-CASK
	ESD6-DPC-CTMBELL	ESD6-DPC-CTMBELL-CAN	
	ESD6-DPC-2BLK	ESD6-DPC-2BLK-CAN2	CANISTER-FAIL-CTM-2BLOCK
RF-ESD06-TAD	ESD6-TAD-LIDIMP	ESD6-TAD-LIDIMP-CAN	TAD-FAIL-NO-CASK
	ESD6-TAD-DROP	ESD6-TAD-DROP-CAN	
	ESD6-TAD-SPUR	ESD6-TAD-SPUR-CAN	TAD-FAIL-SPURMOVE
	ESD6-TAD-SIMPACT	ESD6-TAD-SIMPACT-CAN	TAD-FAIL-CTM-IMPACT
	ESD6-TAD-DROPON	ESD6-TAD-DROPON-CAN	TAD-FAIL-NO-CASK
	ESD6-TAD-CTMBELL	ESD6-TAD-CTMBELL-CAN	
	ESD6-TAD-2BLK	ESD6-TAD-2BLK-CAN2	CANISTER-FAIL-CTM-2BLOCK

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

SHIELDING. Table A4.6-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.6-4. Basic Events Associated with the SHIELDING Pivotal Events of RF-ESD-06

Initiator Event Tree	Initiating Event	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD06-DPC	ESD6-DPC-LIDIMP	ESD6-DPC-LIDIMP-SHIELD	CTM-SHIELDING
	ESD6-DPC-DROP	ESD6-DPC-DROP-SHIELD	
	ESD6-DPC-SPUR	ESD6-DPC-SPUR-SHIELD	
	ESD6-DPC-SIMPACT	ESD6-DPC-SIMPACT-SHIELD	
	ESD6-DPC-DROPON	ESD6-DPC-DROPON-SHIELD	
	ESD6-DPC-CTMBELL	ESD6-DPC-CTMBELL-SHIELD	
	ESD6-DPC-2BLK	ESD6-DPC-2BLK-SHIELD	
RF-ESD06-TAD	ESD6-TAD-LIDIMP	ESD6-TAD-LIDIMP-SHIELD	
	ESD6-TAD-DROP	ESD6-TAD-DROP-SHIELD	
	ESD6-TAD-SPUR	ESD6-TAD-SPUR-SHIELD	
	ESD6-TAD-SIMPACT	ESD6-TAD-SIMPACT-SHIELD	
	ESD6-TAD-DROPON	ESD6-TAD-DROPON-SHIELD	
	ESD6-TAD-CTMBELL	ESD6-TAD-CTMBELL-SHIELD	
	ESD6-TAD-2BLK	ESD6-TAD-2BLK-SHIELD	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.6-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.6-5. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-06

Initiator Event Tree	Initiating Event	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD06-DPC	ESD6-DPC-LIDIMP	200-CONFINEMENT	200-CONFINEMENT
	ESD6-DPC-DROP		
	ESD6-DPC-SPUR		
	ESD6-DPC-SIMPACT		
	ESD6-DPC-DROPON		
	ESD6-DPC-CTMBELL		
	ESD6-DPC-2BLK		
RF-ESD06-TAD	ESD6-TAD-LIDIMP		
	ESD6-TAD-DROP		
	ESD6-TAD-SPUR		
	ESD6-TAD-SIMPACT		
	ESD6-TAD-DROPON		
	ESD6-TAD-CTMBELL		
	ESD6-TAD-2BLK		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.6-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.6-6. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-06

Initiator Event Tree	Initiating Event	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD06-DPC	ESD6-DPC-LIDIMP	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD6-DPC-DROP		
	ESD6-DPC-SPUR		
	ESD6-DPC-SIMPACT		
	ESD6-DPC-DROPON		
	ESD6-DPC-CTMBELL		
	ESD6-DPC-2BLK		
RF-ESD06-TAD	ESD6-TAD-LIDIMP		
	ESD6-TAD-DROP		
	ESD6-TAD-SPUR		
	ESD6-TAD-SIMPACT		
	ESD6-TAD-DROPON		
	ESD6-TAD-CTMBELL		
	ESD6-TAD-2BLK		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.7 EVENT TREES FOR RF-ESD-07

RF-ESD-07 covers event sequences associated with assembly and closure of an aging overpack (Ref. 2.2.34, Figure F-12). This ESD covers the two waste forms that are placed in aging overpacks in the RF: TAD canisters and DPCs. Corresponding to each waste form unit is an initiator event tree (Table A4.7-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.7-1. Summary of Event Trees for RF-ESD-07

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Aging overpack containing DPC	Initiator: RF-ESD07-DPC Response: RESPONSE-AO1	346
Aging overpack containing TAD canister	Initiator: RF-ESD07-TAD Response: RESPONSE-AO1	6,976

NOTE: AO = aging overpack; DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.7.1 Initiating Events for RF-ESD-07

The following initiating events are associated with RF-ESD-07. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.7-2.

Impact to an Aging Overpack. This initiating event accounts for the potential impact to the aging overpack during assembly and closure of the aging overpack.

Tipover of an Aging Overpack. This initiating event accounts for the potential tipover of the aging overpack.

Object Dropped onto Aging Overpack. This initiating event accounts for the potential for the CTM to drop an object on the aging overpack

Collision between Site Transporter and Facility Structures or Equipment. This initiating event accounts for the potential for a site transporter collision

Table A4.7-2. Initiating Event Assignments for RF-ESD-07

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Impact to an aging overpack.	RF-ESD07-DPC	ESD07-DPC-IMPACT	200-ST-IMPACT
	RF-ESD07-TAD	ESD07-TAD-IMPACT	
Object dropped onto aging overpack	RF-ESD07-DPC	ESD07-DPC-DROPON	200-CTMOBJLIFTNUMBER **OR** CTM-DROP-ONTO-CASK[a]
	RF-ESD07-TAD	ESD07-TAD-DROPON	
Site transporter collision	RF-ESD07-DPC	ESD07-DPC-COLLIDE	200-ST-COLLISION
	RF-ESD07-TAD	ESD07-TAD-COLLIDE	
Tipover of an aging overpack	RF-ESD07-DPC	ESD07-DPC-TIP	200-OPTIPOVER003-HFI-NOD
	RF-ESD07-TAD	ESD07-TAD-TIP	

NOTE: [a]Basic event and fault tree connected by an AND gate.
DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.7.2 System Response Event Tree RESPONSE-AO1

The pivotal events that appear in RESPONSE-AO1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

CANISTER. Table A4.7-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.7-3. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-07

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD07-DPC	ESD07-DPC-IMPACT	ESD07-DPC-IMPACT-CAN	CAN-IN-AO-IMPACT
	ESD07-DPC-TIP	ESD07-DPC-TIP-CAN	CAN IN AO TIP
	ESD07-DPC-COLLIDE	ESD07-DPC-COLLIDE-CAN	DPC-CAN-IN-AO-COLL
	ESD07-DPC-DROPON	ESD07-DPC-DROPON-CAN	CAN-IN-AO-DROPON
RF-ESD07-TAD	ESD07-TAD-IMPACT	ESD07-TAD-IMPACT-CAN	CAN-IN-AO-IMPACT
	ESD07-TAD-TIP	ESD07-TAD-TIP-CAN	CAN IN AO TIP
	ESD07-TAD-COLLIDE	ESD07-TAD-COLLIDE-CAN	TAD-CAN-IN-AO-COLL
	ESD07-TAD-DROPON	ESD07-TAD-DROPON-CAN	CAN-IN-AO-DROPON

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

SHIELDING. Table A4.7-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.7-4. Basic Event Associated with the SHIELDING Pivotal Events of RF-ESD-07

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD-7-DPC	ESD07-DPC-IMPACT	ESD07-DPC-IMPACT-SHIELD	AO-SHIELDING
	ESD07-DPC-TIP	ESD07-DPC-TIP-SHIELD	
	ESD07-DPC-COLLIDE	ESD07-DPC-COLLIDE-SHIELD	
	ESD07-DPC-DROPON	ESD07-DPC-DROPON-SHIELD	
RF-ESD07-TAD	ESD07-TAD-IMPACT	ESD07-TAD-IMPACT-SHIELD	
	ESD07-TAD-TIP	ESD07-TAD-TIP-SHIELD	
	ESD07-TAD-COLLIDE	ESD07-TAD-COLLIDE-SHIELD	
	ESD07-TAD-DROPON	ESD07-TAD-DROPON-SHIELD	

NOTE: AO = aging overpack; DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.7-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.7-5. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-07

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD-7-DPC	ESD07-DPC-IMPACT	200-CONFINEMENT	200-CONFINEMENT
	ESD07-DPC-TIP		
	ESD07-DPC-COLLIDE		
	ESD07-DPC-DROPON		
RF-ESD07-TAD	ESD07-TAD-IMPACT		
	ESD07-TAD-TIP		
	ESD07-TAD-COLLIDE		
	ESD07-TAD-DROPON		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.7-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.7-6. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-07

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD-7-DPC	ESD07-DPC-IMPACT	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD07-DPC-TIP		
	ESD07-DPC-COLLIDE		
	ESD07-DPC-DROPON		
RF-ESD07-TAD	ESD07-TAD-IMPACT		
	ESD07-TAD-TIP		
	ESD07-TAD-COLLIDE		
	ESD07-TAD-DROPON		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.8 EVENT TREES FOR RF-ESD-08

RF-ESD-08 covers event sequences associated with the export of an aging overpack from the RF (Ref. 2.2.34, Figure F-16). This ESD covers aging overpacks. Corresponding to each waste form unit is an initiator event tree (Table A4.8-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.8-1. Summary of Event Trees for RF-ESD-08

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Aging overpack containing DPC	RF-ESD8-DPC Response: RESPONSE-AO1	346
Aging overpack containing TAD canister	RF-ESD8-TAD Response: RESPONSE-AO1	6,976

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.8.1 Initiating Events for RF-ESD-08

The following initiating events are associated with RF-ESD-08. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.8-2.

Aging Overpack Dropped. This initiating event accounts for the potential impact to an aging overpack due to a malfunction of the site transporter.

Site Transporter Rollover. For a site transporter to roll over, the center of mass would have to shift laterally. This could result from traversing a significantly uneven surface or running over a very large object. There are no significantly uneven surfaces in the RF Entrance Vestibule or Cask Preparation Area. Therefore, this failure mode was omitted from analysis by assignment of guaranteed success in the event tree.

Site Transporter Collision. This initiating event accounts for the potential impact to the TAD canister due to a collision involving the site transporter. The probability of collision per TAD canister received is modeled as a fault tree as described in Attachment B. The initiating event is specified as a probability of collision per TAD canister.

Table A4.8-2. Initiating Event Assignments for RF-ESD-08

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Aging overpack dropped	RF-ESD8-DPC	ESD8-DPC-DROP	200-ST-DROP
	RF-ESD8-TAD	ESD8-TAD-DROP	
Site transporter rollover	RF-ESD8-DPC	ESD8-DPC-ROLL	200-ST-ROLLOVER
	RF-ESD8-TAD	ESD8-TAD-ROLL	
Site transporter collision	RF-ESD8-DPC	ESD8-DPC-COLLIDE	200-ST-COLLISION
	RF-ESD8-TAD	ESD8-TAD-COLLIDE	

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; ST = site transporter; TAD = transportation, aging, and disposal canister.

Source: Original

A4.8.2 System Response Event Tree RESPONSE-AO1

The pivotal events that appear in RESPONSE-AO1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

CANISTER. Table A4.8-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.8-3. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-08

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD8-DPC	ESD8-DPC-DROP	ESD8-DPC-DROP-CAN	CAN IN AO DROP
	ESD8-DPC-COLLIDE	ESD8-DPC-COLLIDE-CAN	DPC-CAN-IN-AO-COLL
	ESD8-DPC-ROLL	ESD8-DPC-ROLL-CAN	CAN IN AO ROLLOVER
RF-ESD8-TAD	ESD8-TAD-DROP	ESD8-TAD-DROP-CAN	CAN IN AO DROP
	ESD8-TAD-COLLIDE	ESD8-TAD-COLLIDE-CAN	TAD-CAN-IN-AO-COLL
	ESD8-TAD-ROLL	ESD8-TAD-ROLL-CAN	CAN IN AO ROLLOVER

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

SHIELDING. Table A4.8-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.8-4. Basic Event Associated with the SHIELDING Pivotal Events of RF-ESD-08

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD8-DPC	ESD8-DPC-DROP	ESD8-DPC-DROP-SHIELD	AO_SHIELDING
	ESD8-DPC-COLLIDE	ESD8-DPC-COLLIDE-SHIELD	
	ESD8-DPC-ROLL	ESD8-DPC-ROLL-SHIELD	
RF-ESD8-TAD	ESD8-TAD-DROP	ESD8-TAD-DROP-SHIELD	
	ESD8-TAD-COLLIDE	ESD8-TAD-COLLIDE-SHIELD	
	ESD8-TAD-ROLL	ESD8-TAD-ROLL-SHIELD	

NOTE: AO = aging overpack; DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

CONFINEMENT. Table A4.8-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.8-5. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-08

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD8-DPC	ESD8-DPC-DROP	200-CONFINEMENT	200-CONFINEMENT
	ESD8-DPC-COLLIDE		
	ESD8-DPC-ROLL		
RF-ESD8-TAD	ESD8-TAD-DROP		
	ESD8-TAD-COLLIDE		
	ESD8-TAD-ROLL		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.8-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.8-6. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-08

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD8-DPC	ESD8-DPC-DROP	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD8-DPC-COLLIDE		
	ESD8-DPC-ROLL		
RF-ESD8-TAD	ESD8-TAD-DROP		
	ESD8-TAD-COLLIDE		
	ESD8-TAD-ROLL		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A4.9 EVENT TREES FOR RF-ESD-09

RF-ESD-09 covers event sequences associated with export of the horizontal cask on a cask transfer trailer (Ref. 2.2.34). This ESD only covers DPCs since TAD canisters are not transported using this vehicle (Table A4.9-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.9-1. Summary of Event Trees for RF-ESD-09

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Transportation cask containing a DPC	Initiator: RF-ESD09-DPC Response: RESPONSE-TCASK1	346

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.9.1 Initiating Events for RF-ESD-09

The following initiating events are associated with RF-ESD-09. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.9-2.

Cask Transfer Trailer Rollover. This initiating even accounts for the potential of the cask transfer trailer rolling over in the Receipt Facility. However, per HFE Section 6.4, this initiating event has been screened out as a non-credible event.

Cask Transfer Trailer Collision. This initiating event covers the potential impact to the transportation cask on the cask transfer trailer due to a collision with another vehicle, facility structures or equipment.

Table A4.9-2. Initiating Event Assignments for RF-ESD-09

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Cask transfer trailer collision	RF-ESD9	ESD9-COLLIDE	200-HCTT-COLLISION
Cask transfer trailer rollover	RF-ESD9	ESD9-ROLL	200-HCTT-ROLL

NOTE: ESD = event sequence diagram; HCTT = horizontal cask transfer trailer; RF = Receipt Facility.

Source: Original

A4.9.2 System Response Event Tree RESPONSE-TCASK1

The pivotal events that appear in RESPONSE-TCASK1 are summarized below. The accompanying tables show the association of pivotal event names with basic event or fault tree names.

TRANSCASK. Table A4.9-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.9-3. Basic Event Associated with the TRANSCASK Pivotal Events of RF-ESD-09

Initiator Event Tree	Initiating Event Name	Name Assigned to TRANSCASK	Associated Fault Tree or Basic Event
RF-ESD9	ESD9-COLLIDE	ESD9-COLLIDE-TCASK	TCASK-FAIL-COLL
	ESD9-ROLL	ESD9-ROLL-TCASK	TCASK-FAIL ROLLOVER

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

CANISTER. Table A4.9-4 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.9-4. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-09

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD9	ESD9-COLLIDE	ESD9-COLLIDE-CAN	DPC_FAIL_IN_TC
	ESD9-ROLL	ESD9-ROLL-CAN	

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

SHIELDING. Table A4.9-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.9-5. Basic Event Associated with the SHIELDING Pivotal Events of RF-ESD-09

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD9	ESD9-COLLIDE	ESD9-COLLIDE-SHIELD	TCASK-SHIELDING
	ESD9-ROLL	ESD9-ROLL-SHIELD	

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

CONFINEMENT. Table A4.9-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.9-6. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-09

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD9	ESD9-COLLIDE	200-CONFINEMENT	200-CONFINEMENT
	ESD9-ROLL		

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

MODERATOR. Table A4.9-7 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.9-7. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-09

Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD9	ESD9-COLLIDE	200-MODERATOR-SOURCE	200-MODERATOR-SOURCE
	ESD9-ROLL		

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

A4.10 EVENT TREES FOR RF-ESD-10

RF-ESD-10 covers event sequences associated with direct exposure during cask preparation activities (Ref. 2.2.34, Figure F-17). This ESD is only applicable to DPCs because the lid is not removed from the TAD container in this operation. Corresponding to each waste form unit is an initiator event tree (Table A4.10-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.10-1. Summary of Event Trees for RF-ESD-10

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Transportation cask containing a DPC	RF-ESD10	346

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.10.1 Initiating Events for RF-ESD-10

The following initiating events are associated with RF-ESD-10. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.10-2.

Temporary Shielding Loss during Cask Preparation Activities. This initiating event accounts for the loss of shielding during cask preparation activities. Loss of shielding could occur due to the failure to close the cask preparation platform shield plate or the inadvertent opening of the cask preparation platform shield plate. The probability of drop per transfer is derived from empirical data in Section 6.3 and is modeled as a single event fault tree as described in Section 6.2. The initiating event is specified as a probability of a drop per cask.

Table A4.10-2. Initiating Event Assignments for RF-ESD-10

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Temporary loss of shielding during preparation activities	RF-ESD10	PREPSHIELD	200-LIDDISPLACE1-HFI-NOD **OR** 200-OPDPCSHIELD1-HFI-NOW

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

A4.10.2System Response Event Tree for RF-ESD-10

There are no pivotal events associated with RF-ESD-10.

A4.11 EVENT TREES FOR RF-ESD-11

RF-ESD-11 covers event sequences associated with direct exposure during canister transfer activities (Ref. 2.2.34, Figure F-18). This ESD covers all waste forms. Corresponding to each waste form unit is an initiator event tree (Table A4.11-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.11-1. Summary of Event Trees for RF-ESD-11

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
All waste forms	RF-ESD11	7,324

NOTE: ESD = event sequence diagram; RF = Receipt Facility.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.11.1 Initiating Events for RF-ESD-11

The following initiating events are associated with RF-ESD-11. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.11-2.

Temporary Shielding Loss during CTM Activities. This initiating event accounts for the loss of shielding during cask preparation activities. Loss of shielding could occur due to the failure of the shield bell or the inadvertent opening of a slide gate or shield skirt. The probability of drop per transfer is derived from empirical data in Section 6.3 and is modeled as a single event fault tree as described in Section 6.2. The initiating event is specified as a probability of a drop per cask.

Table A4.11-2. Initiating Event Assignments for RF-ESD-11

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level[a]
Temporary loss of shielding during CTM activities	RF-ESD11	CTMSHIELD	No further transfers

NOTE: CTM = canister transfer machine; ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

A4.11.2 System Response Event Tree for RF-ESD-11

There are no pivotal events associated with RF-ESD-11.

A4.12 EVENT TREES FOR RF-ESD-12

RF-ESD-12 covers event sequences associated with fires in the RF (Ref. 2.2.34, Figure F-20). This ESD covers all waste forms (Table A4.12-1). Although the initiator event trees transfer to the same response tree, the response tree is customized within SAPHIRE for each initiator event tree by the use of basic rules. The rules instruct SAPHIRE where to look for the fault tree that models each pivotal event. The assignments made in the rules files are indicated in this section.

Table A4.12-1. Summary of Event Trees for RF-ESD-12

Waste Form Unit	Associated Event Trees	Number of Waste Form Units
Transportation cask or aging overpack containing a DPC	Initiator: RF-ESD12-DPC Response: RESPONSE-FIRE	346
Transportation cask or aging overpack containing a TAD canister	Initiator: RF-ESD12-TAD Response: RESPONSE-FIRE	6,976

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: *Waste Form Throughputs for Preclosure Safety Analysis*, (Ref. 2.2.27, Table 4) for numbers of waste form units.

A4.12.1 Initiating Events for RF-ESD-12

The following initiating events are associated with RF-ESD-12. The assignments made within SAPHIRE for quantification of these initiating events are indicated in Table A4.12-2.

Localized Fire Threatens TAD Canister or DPC in Aging Overpack in Vestibule/Lid Bolting Room (diesel present) on Site Transporter. This initiating event accounts for the potential impact from a fire threatening a TAD canister in an aging overpack in the Vestibule/Lid Bolting Room with diesel present.

Localized Fire Threatens TAD Canister or DPC in Aging Overpack in Loading Room (diesel present) on Site Transporter. This initiating event accounts for the potential impact from a fire threatening a TAD canister in an aging overpack in the Loading Room with diesel present.

Localized Fire Threatens TAD Canister or DPC in Transportation Cask in Vestibule/Preparation Area (diesel present) on Site Prime Mover. This initiating event accounts for the potential impact from a fire threatening a cask in the Preparation Area with diesel present.

Localized Fire Threatens TAD Canister or DPC in Transportation Cask in Preparation Area on Railcar. This initiating event accounts for the potential impact from a fire threatening a cask in the Preparation Area

Localized Fire Threatens TAD Canister or DPC in Transportation Cask in Preparation Area on CTT. This initiating event accounts for the potential impact from a fire threatening a waste form in the Preparation Area

Localized Fire Threatens TAD Canister or DPC in Transportation Cask in Cask Unloading Room on CTT. This initiating event accounts for the potential impact from a fire in the Cask Unloading Room.

Localized Fire Threatens TAD Canister or DPC (including TTCs) in Transfer Room in CTM. This initiating event accounts for the potential impact from a fire in the Transfer Room.

Large Fire Threatens Waste Forms in RF. This initiating event accounts for the potential impact from a large fire in the RF.

Table A4.12-2. Initiating Event Assignments for RF-ESD-12

Initiating Event Description	Initiator Event Tree	SAPHIRE Assignment by Basic Rules	SAPHIRE Assignment at Fault Tree Level
Localized Fire Threatens Waste Form in AO in Vestibule/Lid Bolting Room (Diesel Present)	RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-DPC	ESD12-FIRE-IN-BOLT-DPC
	RF-ESD12-TAD	ESD12-BOLT-FIRE-CSK-TAD	ESD12-FIRE-IN-BOLT-TAD
Localized Fire Threatens Waste Form in AO in Loading Room (Diesel Present)	RF-ESD12-DPC	ESD12-LOAD-FIRE-CSK-DPC	ESD12-FIRE-IN-LOAD-DPC
	RF-ESD12-TAD	ESD12-LOAD-FIRE-CSK-TAD	ESD12-FIRE-IN-LOAD-TAD
Localized Fire Threatens Waste Form in Vestibule/Preparation Area (Diesel Present)	RF-ESD12-DPC	ESD12-PREP-FIRE-CSK-DC	ESD12-DFIRE-IN-PREP-DPC
	RF-ESD12-TAD	ESD12-PREP-FIRE-CSK-TD	ESD12-DFIRE-IN-PREP-TAD
Localized Fire Threatens Waste Form in Preparation Area	RF-ESD12-DPC	ESD12-PREP-FIRE-CSK-DPC	ESD12-FIRE-IN-PREP-DPC
	RF-ESD12-TAD	ESD12-PREP-FIRE-CSK-TAD	ESD12-FIRE-IN-PREP-TAD
Localized Fire Threatens Waste Form in Preparation Area	RF-ESD12-DPC	ESD12-PREP-FIRE-CAN-DPC	ESD12-FIRE-IN-PREPCT-DPC
	RF-ESD12-TAD	ESD12-PREP-FIRE-CAN-TAD	ESD12-FIRE-IN-PREPCT-TAD
Localized Fire Threatens Waste Form in Cask Unloading Room	RF-ESD12-DPC	ESD12-UNLD-FIRE-CAN-DPC	ESD12-FIRE-IN-UNLD-DPC
	RF-ESD12-TAD	ESD12-UNLD-FIRE-CAN-TAD	ESD12-FIRE-IN-UNLD-TAD
Localized Fire Threatens Waste Form in Transfer Room	RF-ESD12-DPC	ESD12-XFER-FIRE-CSK-DPC	ESD12-FIRE-CTM-DPC
	RF-ESD12-TAD	ESD12-XFER-FIRE-CSK-TAD	ESD12-FIRE-CTM-TAD
Large fire in RF	RF-ESD12-DPC	ESD12-LARGE-FIRE-DPC	ESD12-DPC-IN-LG-FIRE
	RF-ESD12-TAD	ESD12-LARGE-FIRE-TAD	ESD12-TAD-IN-LG-FIRE

NOTE: AO = aging overpack; RF = Receipt Facility.

Source: Original

A4.12.2 System Response Event Tree RESPONSE-FIRE

CANISTER. Table A4.12-3 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.12-3. Basic Events Associated with the CANISTER Pivotal Events of RF-ESD-12

Initiator Event Tree	Initiating Event Name	Name Assigned to CANISTER	Associated Fault Tree or Basic Event
RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-DPC	ESD12-CAN-AO	CANISTER-FIRE-AO
	ESD12-LOAD-FIRE-CSK-DPC	ESD12-CAN-AO	CANISTER-FIRE-AO
	ESD12-PREP-FIRE-CSK-DC	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-PREP-FIRE-CSK-DPC	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-PREP-FIRE-CAN-DPC	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-UNLD-FIRE-CAN-DPC	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-XFER-FIRE-CSK-DPC	ESD12-CAN	ESD12-BARE-CAN
	ESD12-LARGE-FIRE-DPC	ESD12-CAN-SPLIT-DPC	See fault tree in Attachment B
RF-ESD12-TAD	ESD12-BOLT-FIRE-CSK-TAD	ESD12-CAN-AO	CANISTER-FIRE-AO
	ESD12-LOAD-FIRE-CSK-TAD	ESD12-CAN-AO	CANISTER-FIRE-AO
	ESD12-PREP-FIRE-CSK-TD	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-PREP-FIRE-CSK-TAD	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-PREP-FIRE-CAN-TAD	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-UNLD-FIRE-CAN-TAD	ESD12-CAN-TC	CANISTER-FIRE-TC
	ESD12-XFER-FIRE-CSK-TAD	ESD12-CAN	ESD12-BARE-CAN
	ESD12-LARGE-FIRE-TAD	ESD12-CAN-SPLIT-TAD	See fault tree in Attachment B

NOTE: AO = aging overpack; DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

SHIELDING. This pivotal event represents the success or failure of the shielding provided by the transportation cask, aging overpack, or CTM shield bell as a result of the initiating event. Table A4.12-4 indicates the fault trees or basic events that are associated with this pivotal event for each initiating event.

Table A4.12-4. Fault Tree Associated with the SHIELDING Pivotal Events of RF-ESD-12

Initiator Event Tree	Initiating Event Name	Name Assigned to SHIELDING	Associated Fault Tree or Basic Event
RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-DPC	ESD12-DPC-SHIELD-AO	200-DPC-AO-SHIELD-FIRE
	ESD12-LOAD-FIRE-CSK-DPC	ESD12-DPC-SHIELD-AO	200-DPC-AO-SHIELD-FIRE
	ESD12-PREP-FIRE-CSK-DC	ESD12-DPC-SHIELD-TC	200-DPC-TC-SHIELD-FIRE
	ESD12-PREP-FIRE-CSK-DPC	ESD12-DPC-SHIELD-TC	200-DPC-TC-SHIELD-FIRE
	ESD12-PREP-FIRE-CAN-DPC	ESD12-DPC-SHIELD-TC	200-DPC-TC-SHIELD-FIRE
	ESD12-UNLD-FIRE-CAN-DPC	ESD12-DPC-SHIELD-TC	200-DPC-TC-SHIELD-FIRE
	ESD12-XFER-FIRE-CSK-DPC	ESD12-DPC-SHIELD-CAN	200-DPC-CAN-SHIELD-FIRE
	ESD12-LARGE-FIRE-DPC	ESD12-DPC-SHIELD-LF	PROB-DPC-IN-TC-IN-LF **AND** ESD12-DPC-SHIELD-TC
RF-ESD12-TAD	ESD12-BOLT-FIRE-CSK-TAD	ESD12-TAD-SHIELD-AO	200-TAD-AO-SHIELD-FIRE
	ESD12-LOAD-FIRE-CSK-TAD	ESD12-TAD-SHIELD-AO	200-TAD-AO-SHIELD-FIRE
	ESD12-PREP-FIRE-CSK-TD	ESD12-TAD-SHIELD-TC	200-TAD-TC-SHIELD-FIRE
	ESD12-PREP-FIRE-CSK-TAD	ESD12-TAD-SHIELD-TC	200-TAD-TC-SHIELD-FIRE
	ESD12-PREP-FIRE-CAN-TAD	ESD12-TAD-SHIELD-TC	200-TAD-TC-SHIELD-FIRE
	ESD12-UNLD-FIRE-CAN-TAD	ESD12-TAD-SHIELD-TC	200-TAD-TC-SHIELD-FIRE
	ESD12-XFER-FIRE-CSK-TAD	ESD12-TAD-SHIELD-CAN	200-TAD-CAN-SHIELD-FIRE
	ESD12-LARGE-FIRE-TAD	ESD12-TAD-SHIELD-LF	PROB-TAD-IN-TC-IN-LF **AND** ESD12-TAD-SHIELD-TC

NOTE: AO = aging overpack; DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

CONFINEMENT. Table A4.12-5 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.12-5. Basic Event Associated with the CONFINEMENT Pivotal Events of RF-ESD-12

Initiator Event Tree	Initiating Event Name	Name Assigned to CONFINEMENT	Associated Fault Tree or Basic Event
RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-DPC	200-CONFINEMENT	No further transfers
	ESD12-LOAD-FIRE-CSK-DPC		
	ESD12-PREP-FIRE-CSK-DC		
	ESD12-PREP-FIRE-CSK-DPC		
	ESD12-PREP-FIRE-CAN-DPC		
	ESD12-UNLD-FIRE-CAN-DPC		
	ESD12-XFER-FIRE-CSK-DPC		
	ESD12-LARGE-FIRE-DPC		
RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-TAD		
	ESD12-LOAD-FIRE-CSK-TAD		
	ESD12-PREP-FIRE-CSK-TD		
	ESD12-PREP-FIRE-CSK-TAD		
	ESD12-PREP-FIRE-CAN-TAD		
	ESD12-UNLD-FIRE-CAN-TAD		
	ESD12-XFER-FIRE-CSK-TAD		
	ESD12-LARGE-FIRE-TAD		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

MODERATOR. Table A4.12-6 indicates the basic event that is associated with this pivotal event for each initiating event.

Table A4.12-6. Basic Event Associated with the MODERATOR Pivotal Events of RF-ESD-12

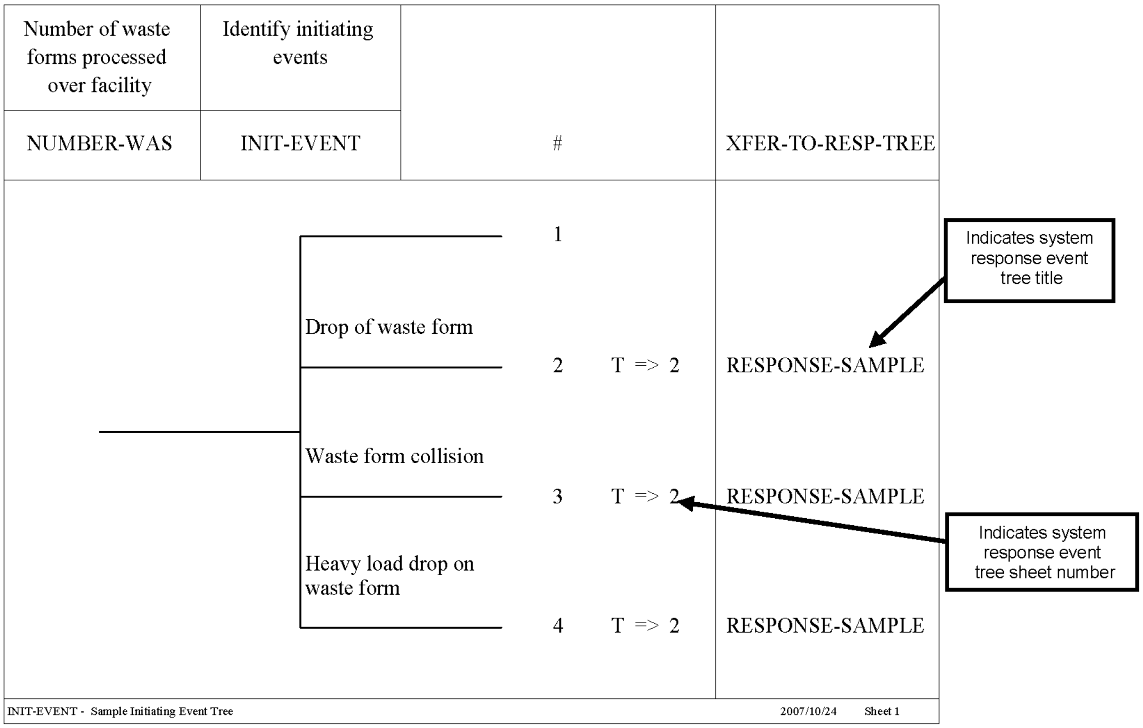
Initiator Event Tree	Initiating Event Name	Name Assigned to MODERATOR	Associated Fault Tree or Basic Event
RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-DPC	200-MODERATOR-SOURCE	No further transfers
	ESD12-LOAD-FIRE-CSK-DPC		
	ESD12-PREP-FIRE-CSK-DC		
	ESD12-PREP-FIRE-CSK-DPC		
	ESD12-PREP-FIRE-CAN-DPC		
	ESD12-UNLD-FIRE-CAN-DPC		
	ESD12-XFER-FIRE-CSK-DPC		
	ESD12-LARGE-FIRE-DPC		
RF-ESD12-DPC	ESD12-BOLT-FIRE-CSK-TAD		
	ESD12-LOAD-FIRE-CSK-TAD		
	ESD12-PREP-FIRE-CSK-TD		
	ESD12-PREP-FIRE-CSK-TAD		
	ESD12-PREP-FIRE-CAN-TAD		
	ESD12-UNLD-FIRE-CAN-TAD		
	ESD12-XFER-FIRE-CSK-TAD		
	ESD12-LARGE-FIRE-TAD		

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original

A5 EVENT TREES

Navigation from an initiator event tree to the corresponding response event tree is assisted by the rightmost two columns on the initiator event trees as shown in Figure A5-1. The numbers under the "#" symbol may be used by the reader to refer to a particular branch of an event tree, but it is not used elsewhere in this analysis.



Source: Original

Figure A5-1. Example Initiator Event Tree Showing Navigation Aids

Table A5-1. ESDs to Event Trees

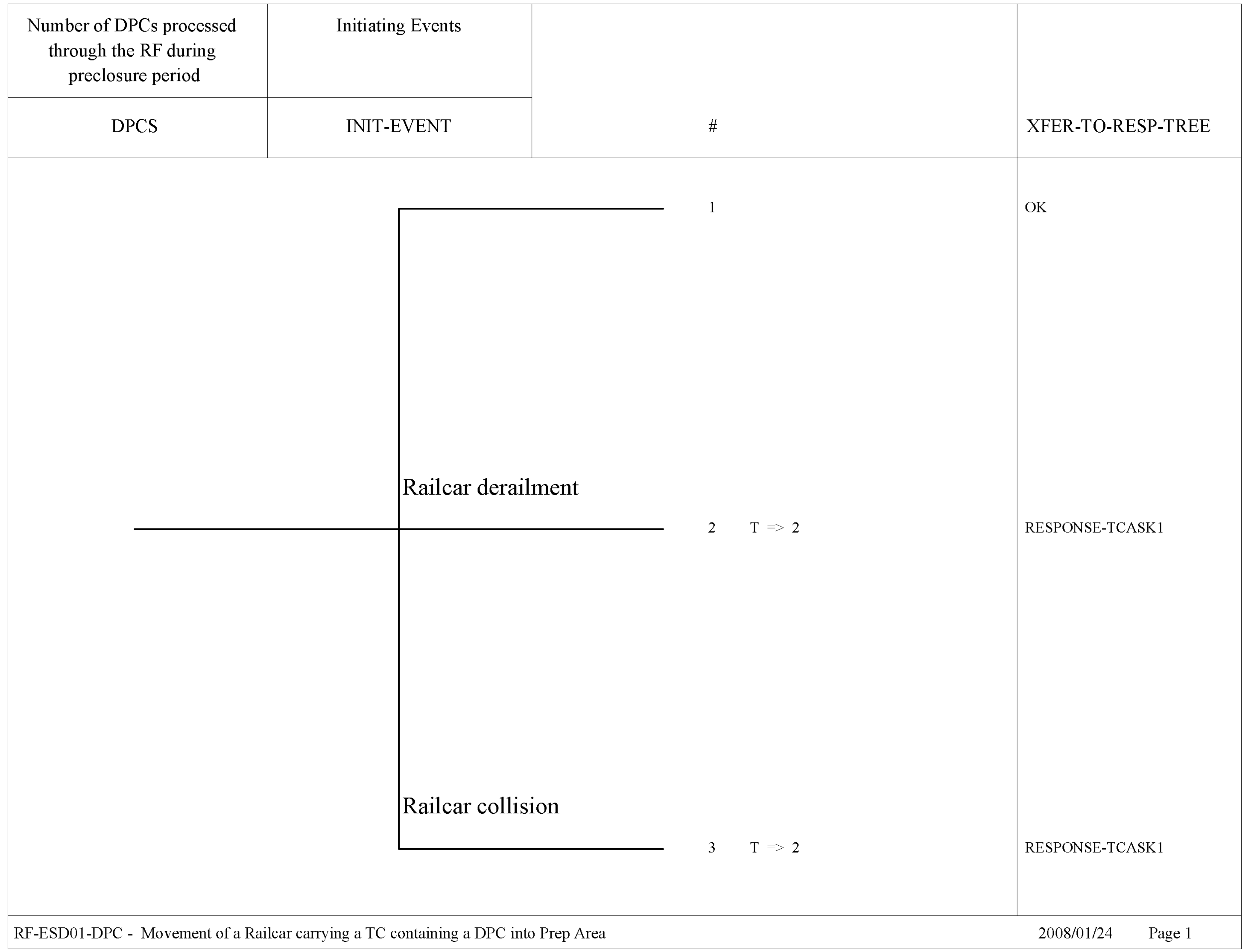
ESD#	ESD Title	IE Event Tree Name	IE Event Tree Figure	Response Tree Name	Response Tree Figure
RF-ESD-01	Event Sequences for Activities Associated with Receipt of Transportation Cask into Cask Preparation Room	RF-ESD01-DPC RF-ESD01-TAD	Figure A5-2 Figure A5-4	RESPONSE-TCASK1	Figure A5-3
RF-ESD-02	Event Sequences for Activities Associated with Removal of Impact Limiters, Cask Upending, and transfer to CTT or Cast Transfer Trailer	RF-ESD02-DPC RF-ESD02-TAD	Figure A5-5 Figure A5-6	RESPONSE-TCASK1	Figure A5-3
RF-ESD-03	Event Sequences for Activities Associated with Unbolting and Lid Adapter Installation	RF-ESD03-DPC RF-ESD03-TAD	Figure A5-7 Figure A5-8	RESPONSE-TCASK1	Figure A5-3
RF-ESD-04	Event Sequences for Activities Associated with Transfer of a Cask on CTT from Cask Preparation Room to Cask Unloading Room	RF-ESD04-DPC RF-ESD04-TAD	Figure A5-9 Figure A5-11	RESPONSE-TCASK2	Figure A5-10
RF-ESD-05	Event Sequences for Activities Associated with a Transportation Cask on a CTT or Site Transporter Colliding with Lid Bolting Room or Cask Unloading Room Shield Doors	RF-ESD05-DPC RF-ESD05-TAD	Figure A5-12 Figure A5-13	N/A	N/A
RF-ESD-06	Event Sequences for Activities Associated with the Transfer of a Canister from Transportation Cask to Aging Overpack with CTM	RF-ESD06-DPC RF-ESD06-TAD	Figure A5-14 Figure A5-16	RESPONSE-CANISTER1	Figure A5-15
RF-ESD-07	Event Sequences for Activities Associated with Assembly and Closure of an Aging Overpack	RF-ESD07-DPC RF-ESD07-TAD	Figure A5-17 Figure A5-19	RESPONSE-AO1	Figure A5-18

Table A5-1. ESDs to Event Trees (Continued)

ESD#	ESD Title	IE Event Tree Name	IE Event Tree Figure	Response Tree Name	Response Tree Figure
RF-ESD-08	Event Sequences for Activities Associated with the Exporting of an Aging Overpack from the Receipt Facility	RF-ESD08-DPC RF-ESD08-TAD	Figure A5-20 Figure A5-21	RESPONSE-AO1	Figure A5-18
RF-ESD-09	Event Sequences for Activities Associated with Export of Horizontal Cask on Cask Transfer Trailer	RF-ESD09	Figure A5-22	RESPONSE-TCASK1	Figure A5-3
RF-ESD-10	Event Sequences for Activities Associated with Direct Exposure During DPC handling Activities	RF-ESD10	Figure A5-23	N/A	N/A
RF-ESD-11	Event Sequences for Activities Associated with Direct Exposure During CTM Activities	RF-ESD11	Figure A5-24	N/A	N/A
RF-ESD-12	Event Sequences for a Fire Occurring in Receipt Facility	RF-ESD12-DPC RF-ESD12-TAD	Figure A5-25 Figure A5-27	RESPONSE-FIRE	Figure A5-26

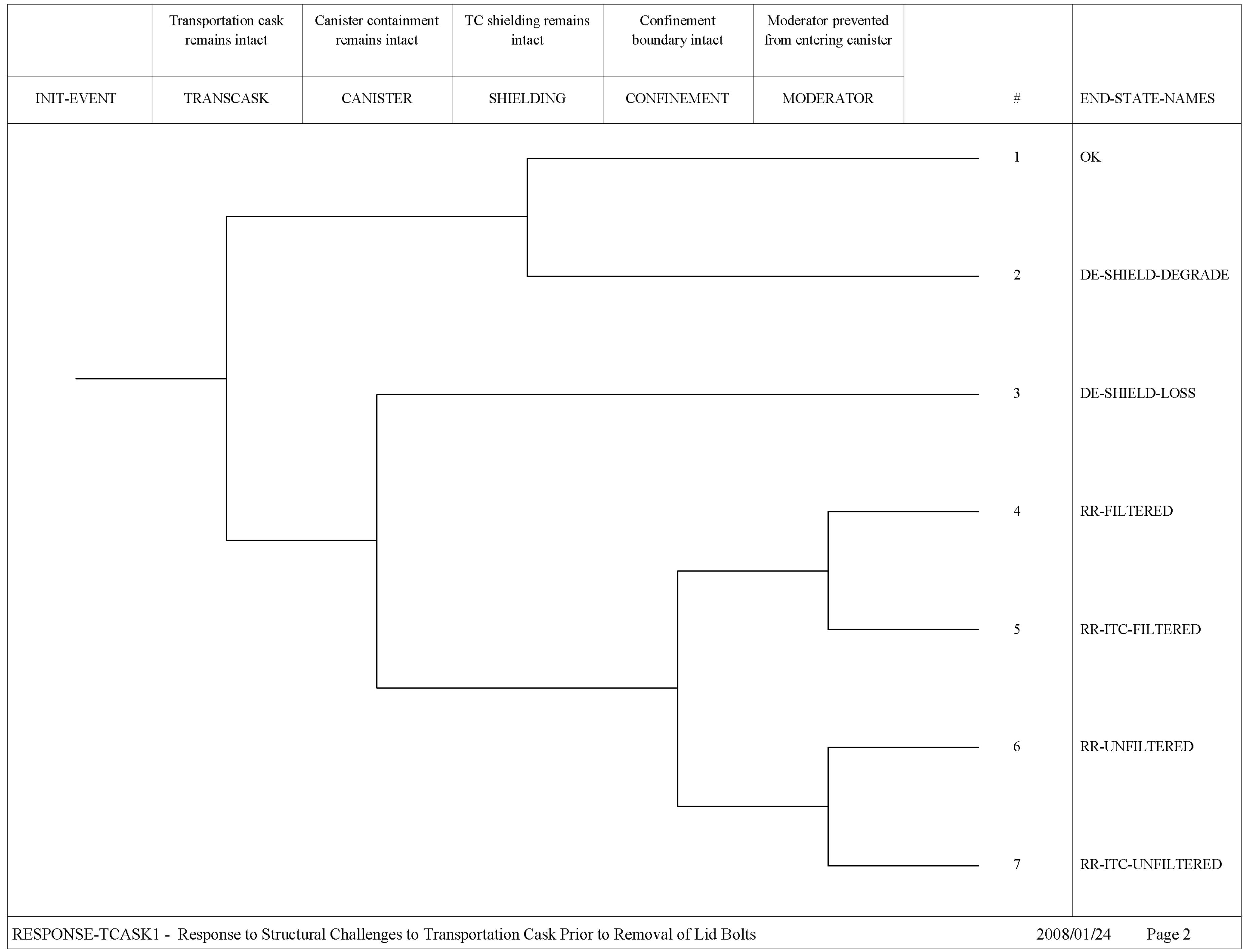
NOTE: AO = aging overpack; CTM = canister transfer machine; CTT = cask transfer trolley; DPC = dual-purpose canister; ESD = event sequence diagram; IE = initiating event; N/A = not applicable; RF = Receipt Facility; TAD = transportation, aging, and disposal canister.

Source: Original



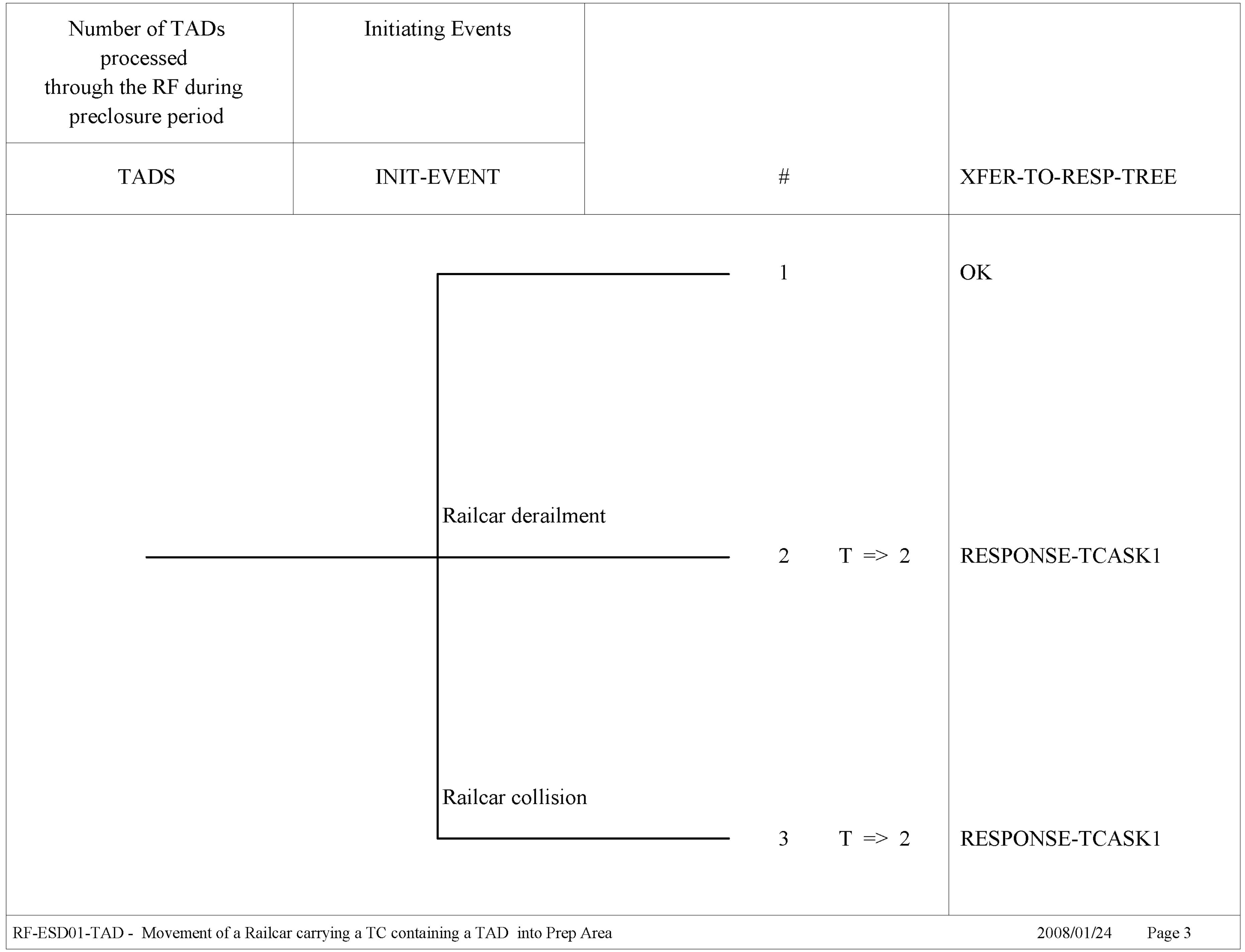
Source: Original

Figure A5-2. Event Tree RF-ESD01-DPC – Movement of a Railcar Carrying a Transportation Cask Containing a DPC into the Preparation Area



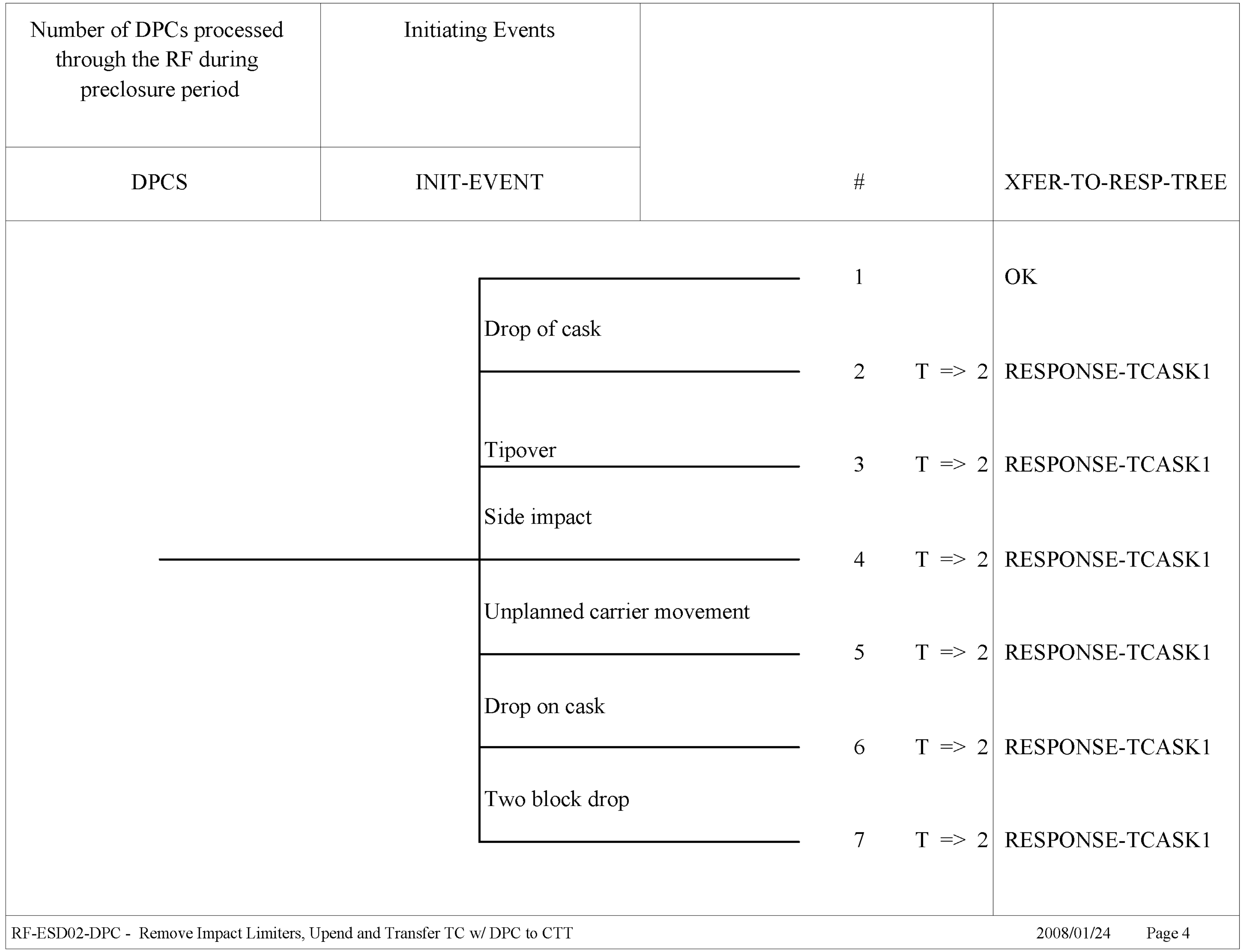
Source: Original

Figure A5-3. Event Tree RESPONSE-TCASK1 – Response to Structural Challenges to Transportation Cask Prior to Removal of Lid Bolts



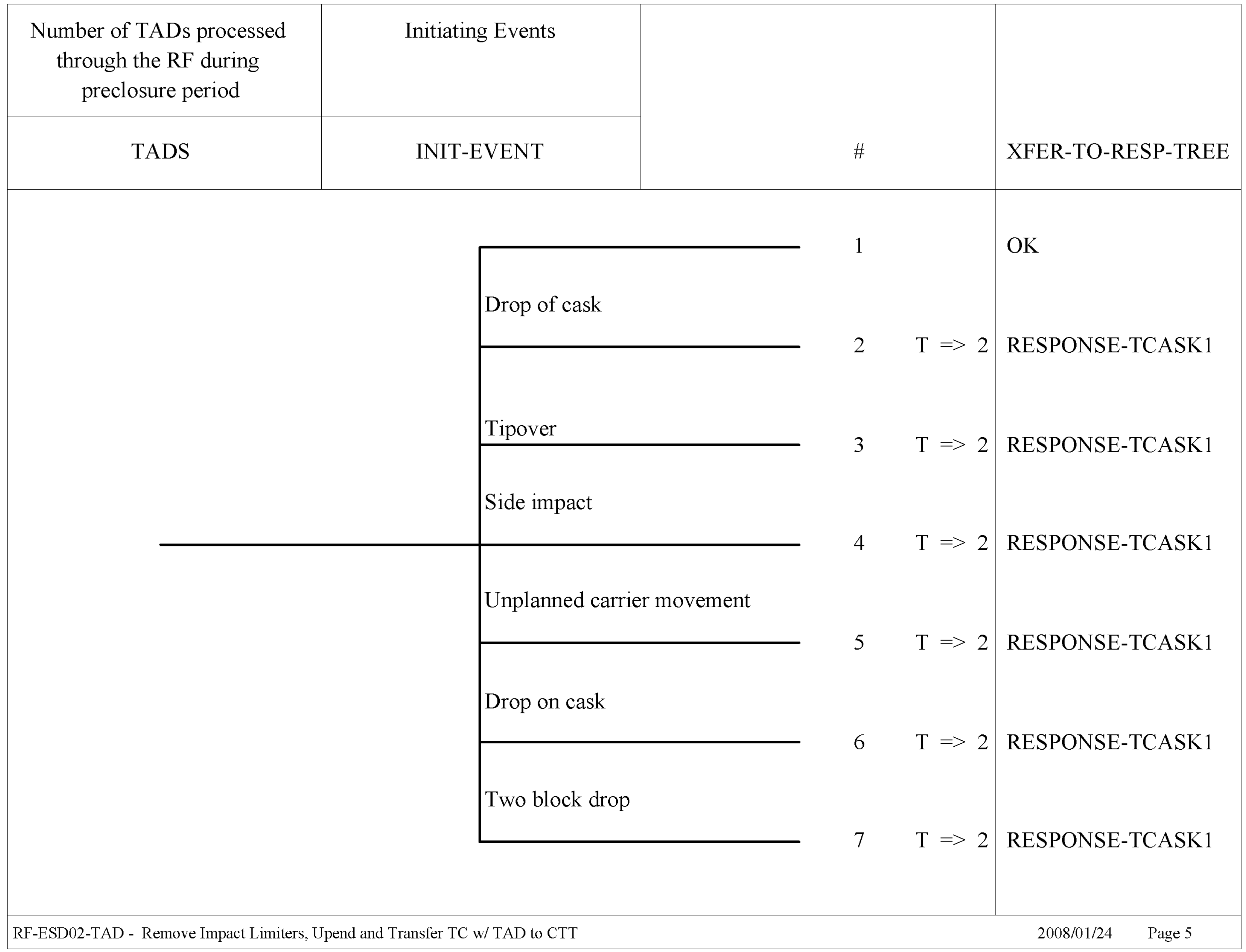
Source: Original

Figure A5-4. Event Tree RF-ESD01-TAD – Movement of Railcar Carrying a Transportation Cask Containing a TAD Canister into the Preparation Area



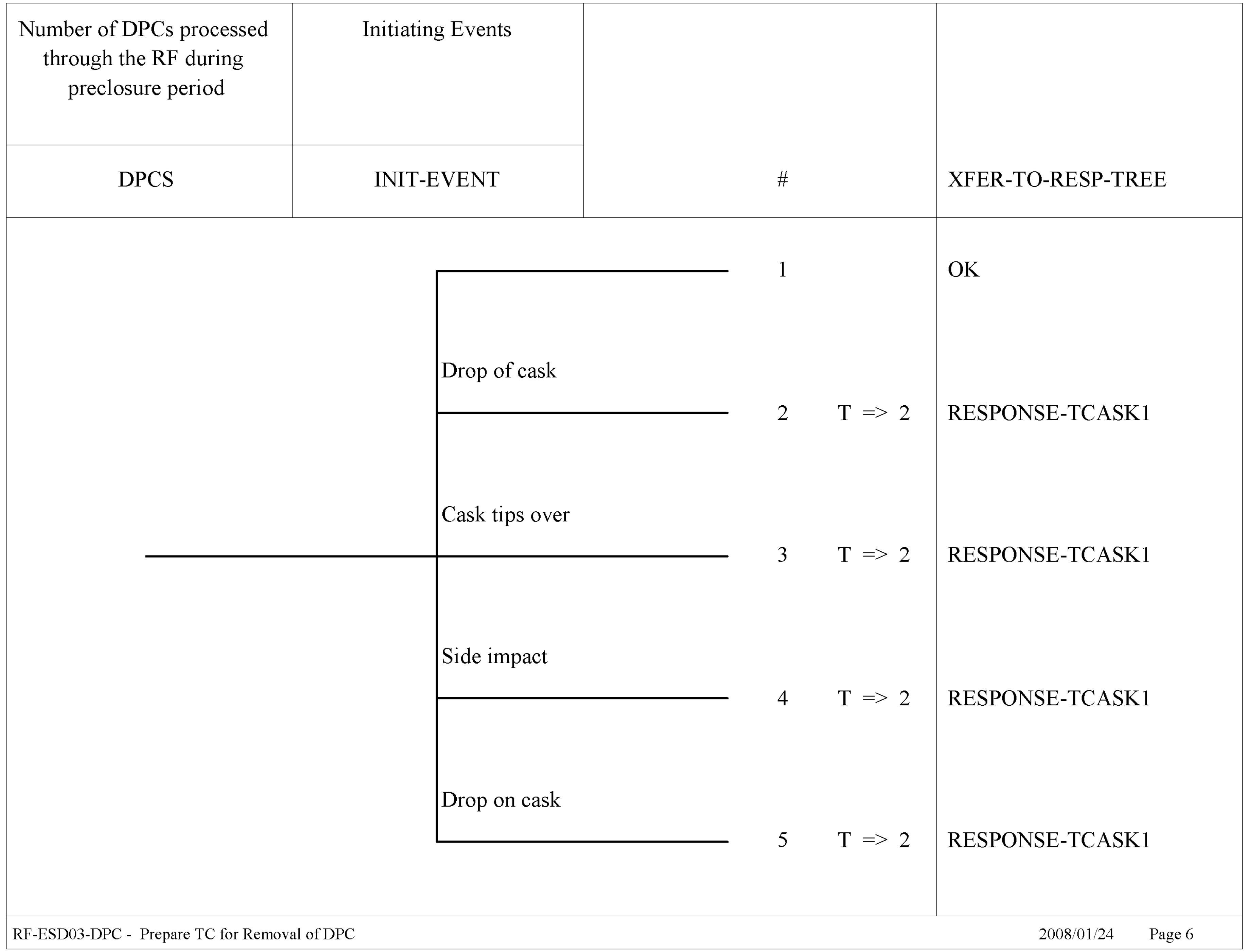
Source: Original

Figure A5-5. Event Tree RF-ESD02-DPC – Remove Impact Limiters, Upend, and Transfer a Transportation Cask with a DPC to a CTT



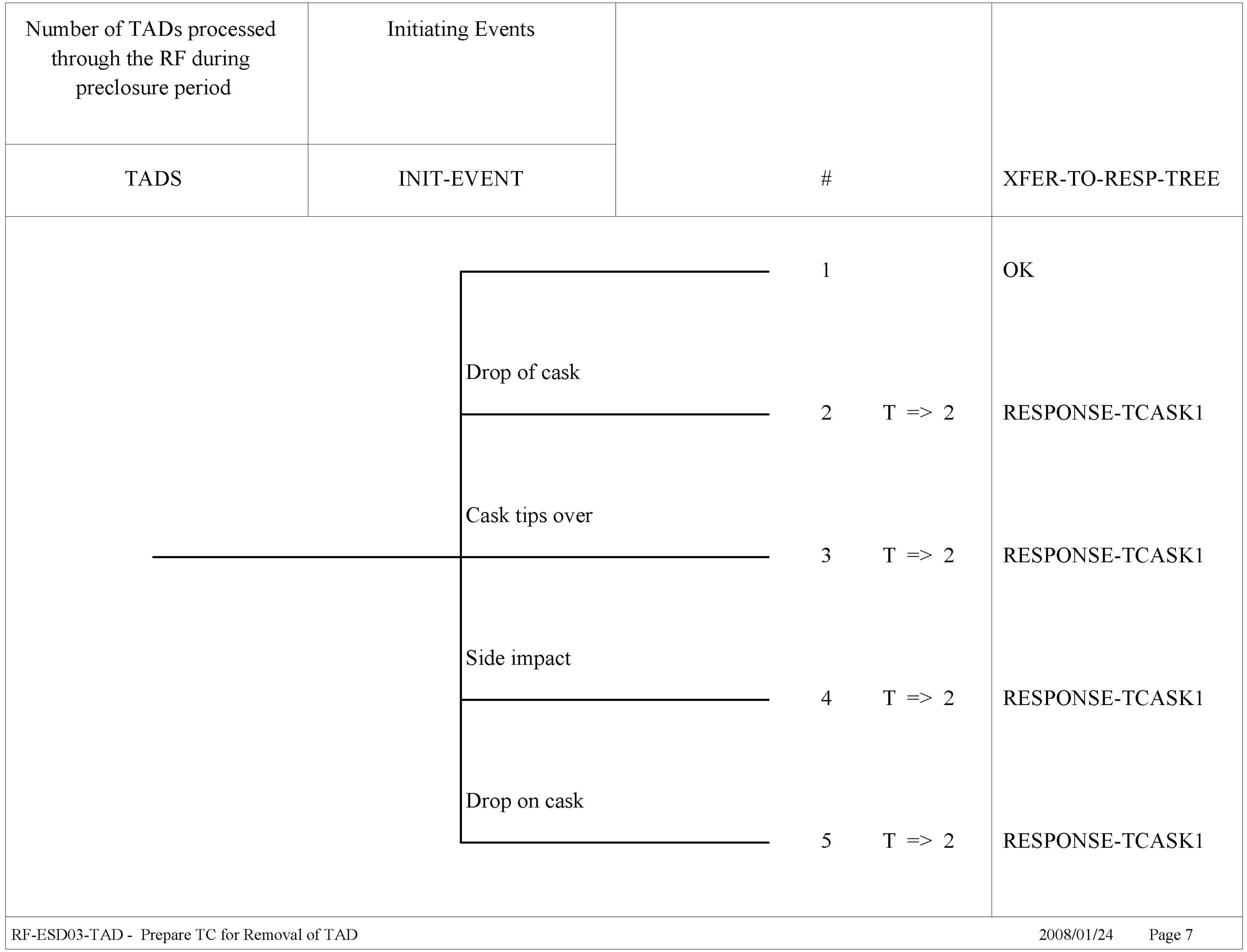
Source: Original

Figure A5-6. Event Tree RF-ESD02-TAD – Remove Impact Limiters, Upend, and Transfer a Transportation with a TAD Canister to a CTT



Source: Original

Figure A5-7. Event Tree RF-ESD03-DPC – Prepare a Transportation Cask for Removal of a DPC



Source: Original

Figure A5-8. Event Tree RF-ESD03-TAD – Prepare a Transportation Cask for Removal of a TAD Canister

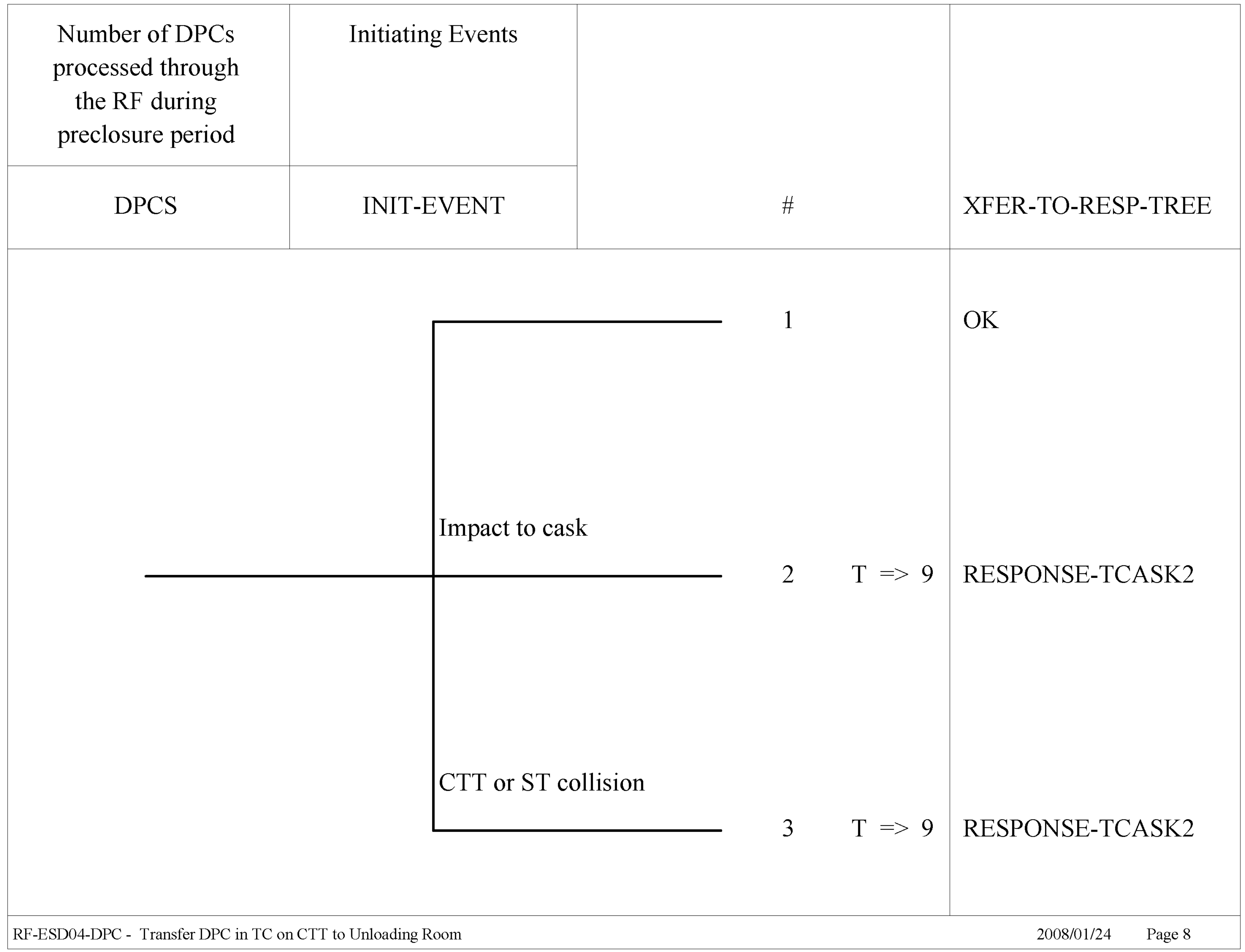


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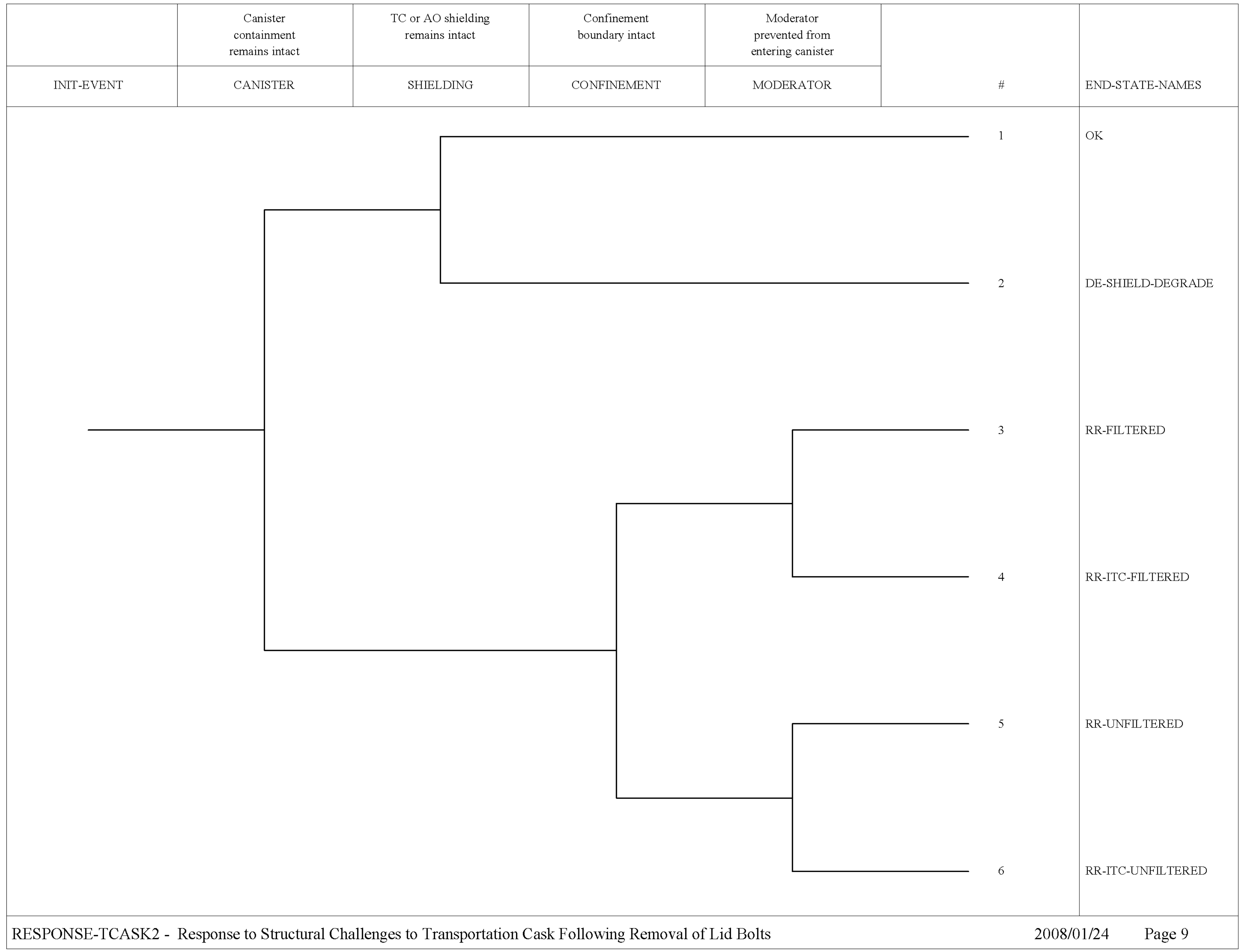
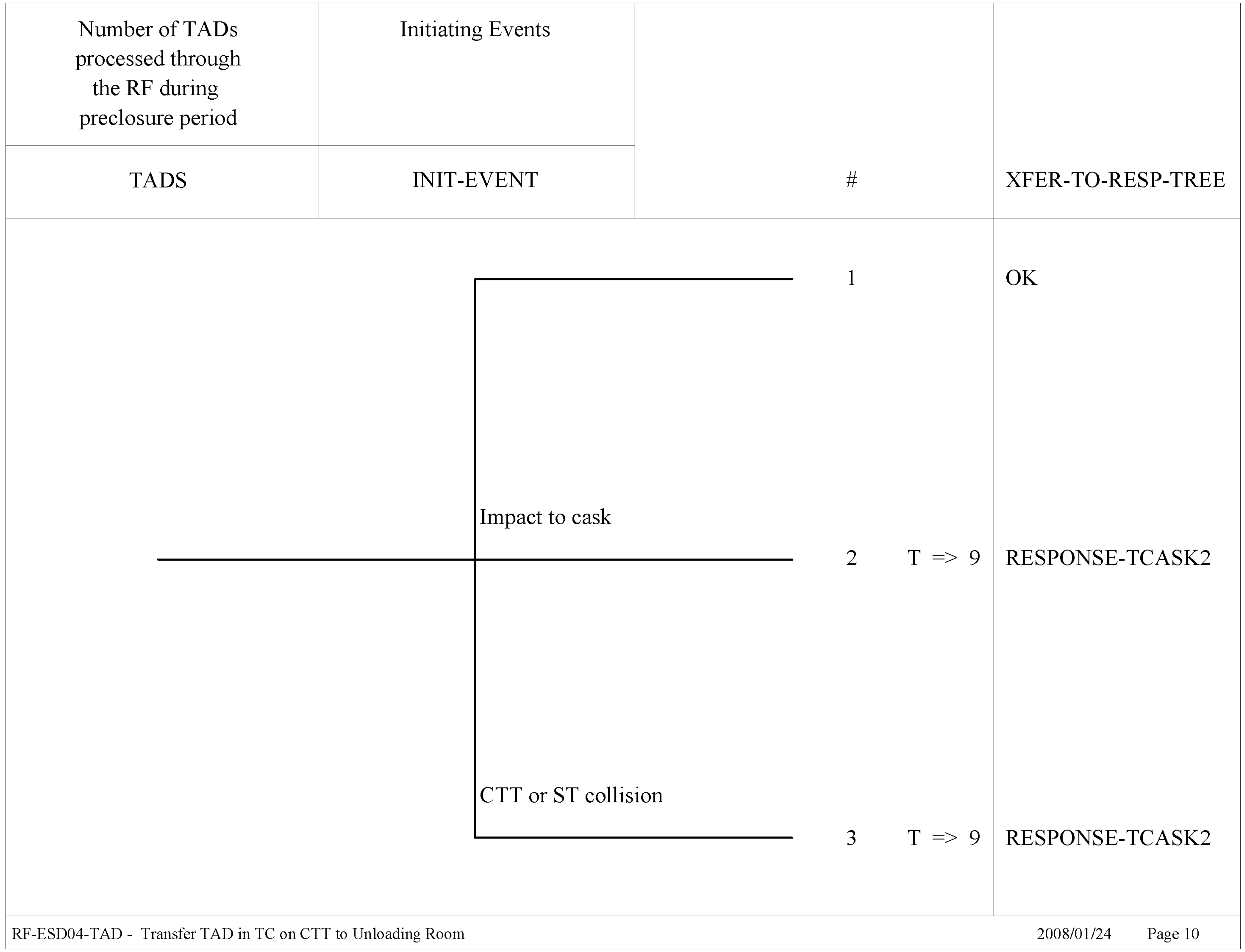
Figure A5-9. Event Tree RF-ESD04-DPC – Transfer a DPC in a Transportation Cask on a CTT to the Unloading Room



Source: Original

Figure A5-10. Event Tree RESPONSE-TCASK2 – Response to Structural Challenges to Transportation Cask Following Removal of Lid Bolts



Source: Original

Figure A5-11. Event Tree RF-ESD04-TAD – Transfer a TAD Canister in a Transportation Cask on the CTT to the Unloading Room

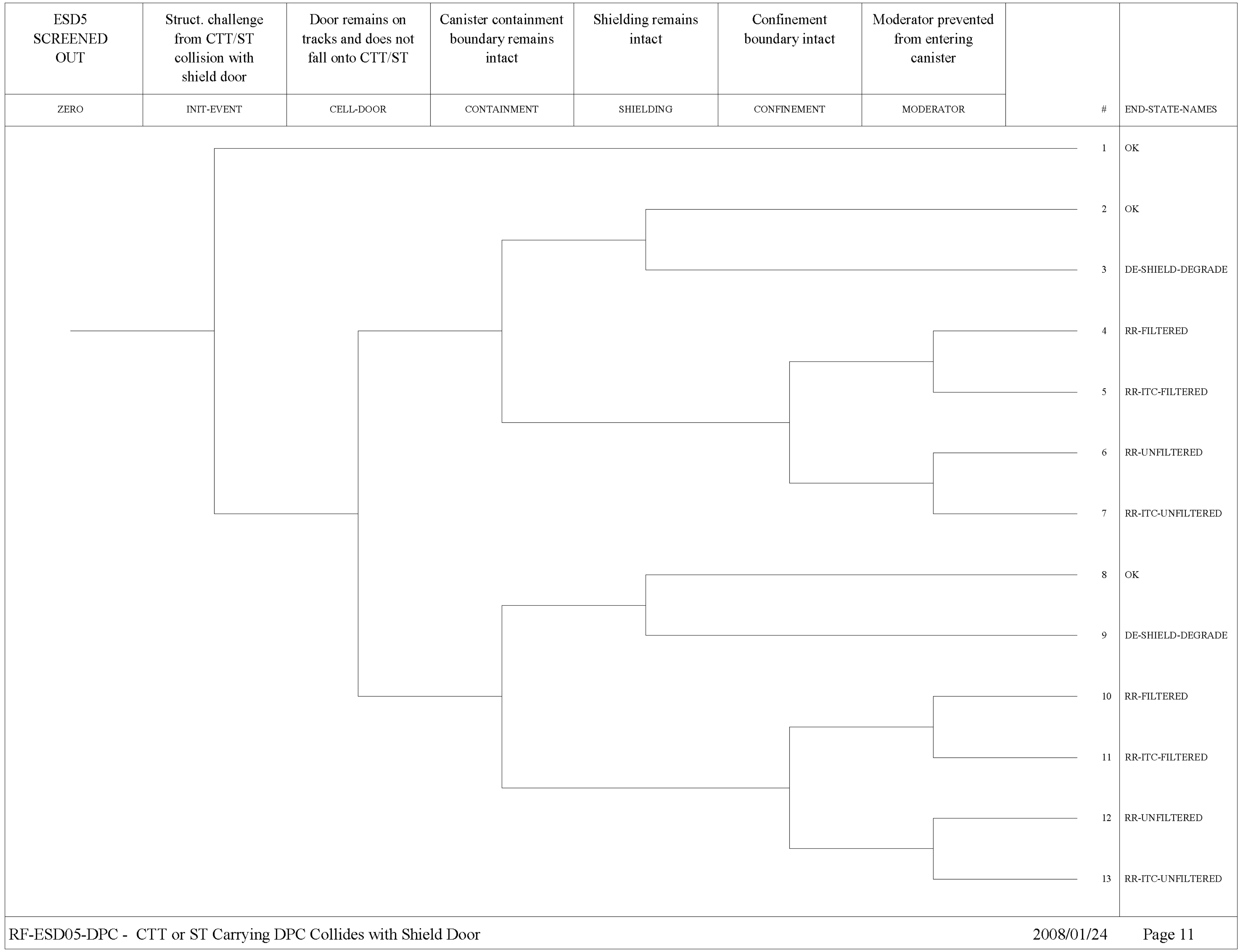


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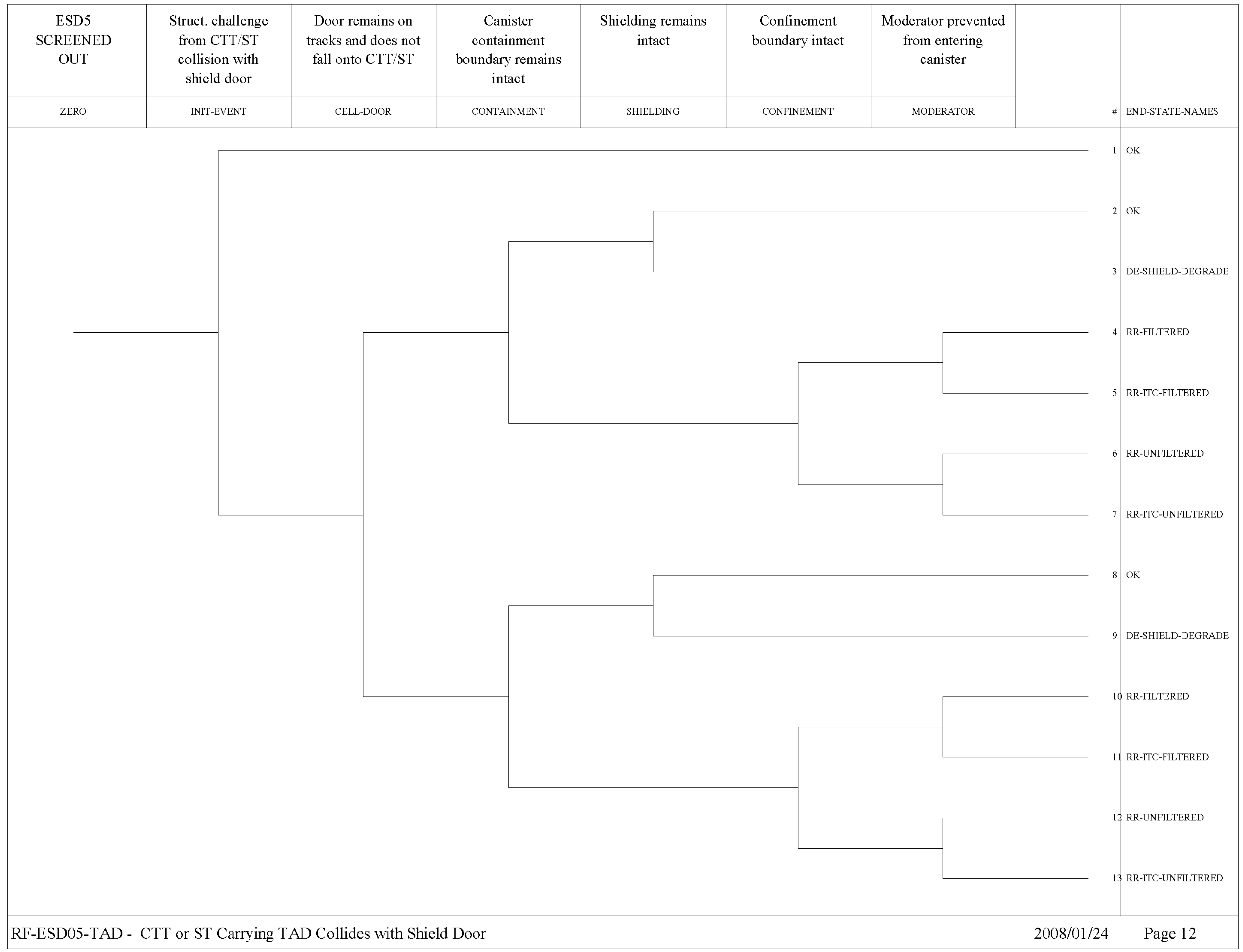
Figure A5-12. Event Tree RF-ESD05-DPC – CTT or Site Transporter Carrying a DPC Collides with a Shield Door



Source: Original

Figure A5-13. Event Tree RF-ESD05-TAD – CTT or Site Transporter Carrying a TAD Canister Collides with a Shield Door

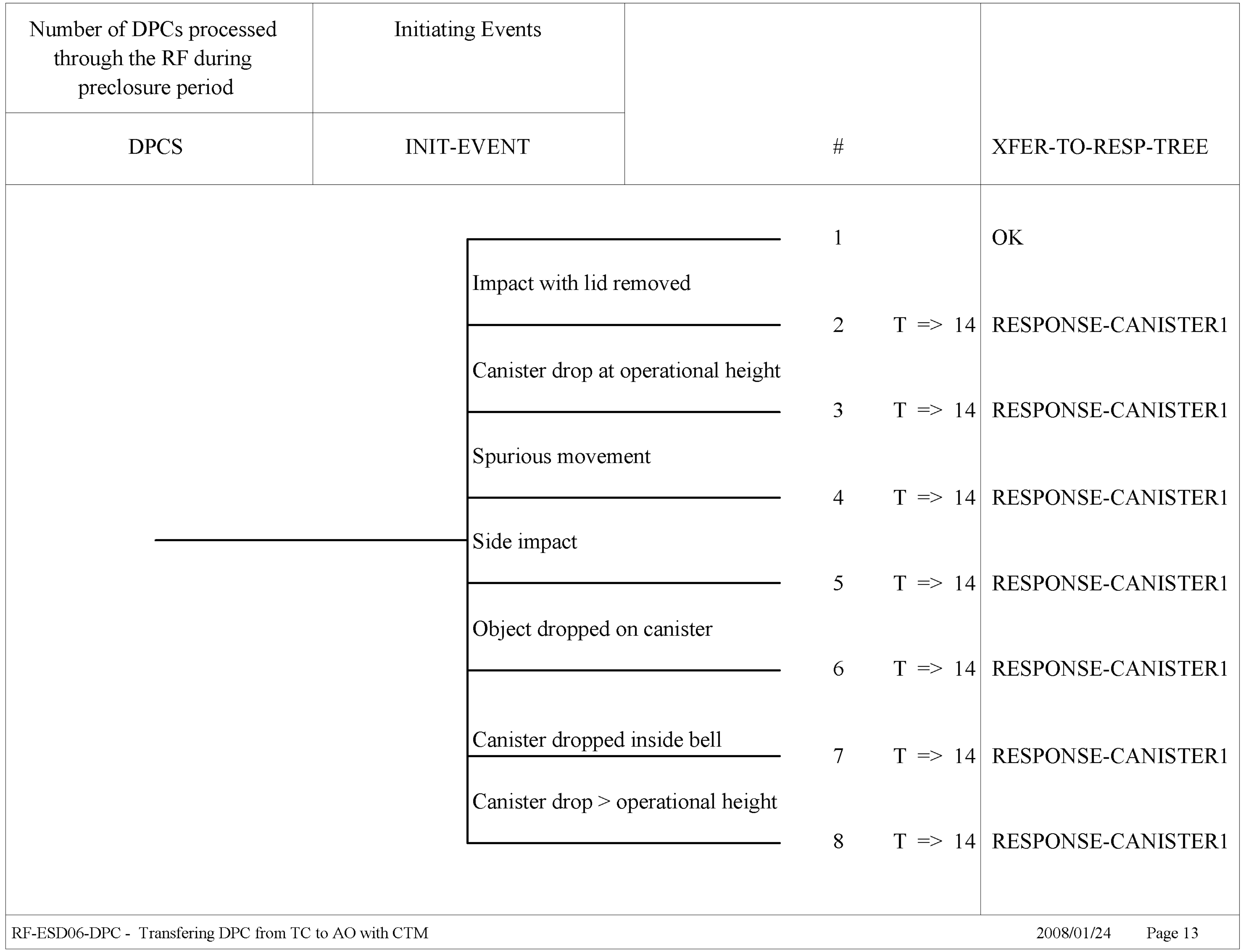


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Figure A5-14. Event Tree RF-ESD06-DPC – Transferring a DPC from a Transportation Cask to an Aging Overpack with the CTM

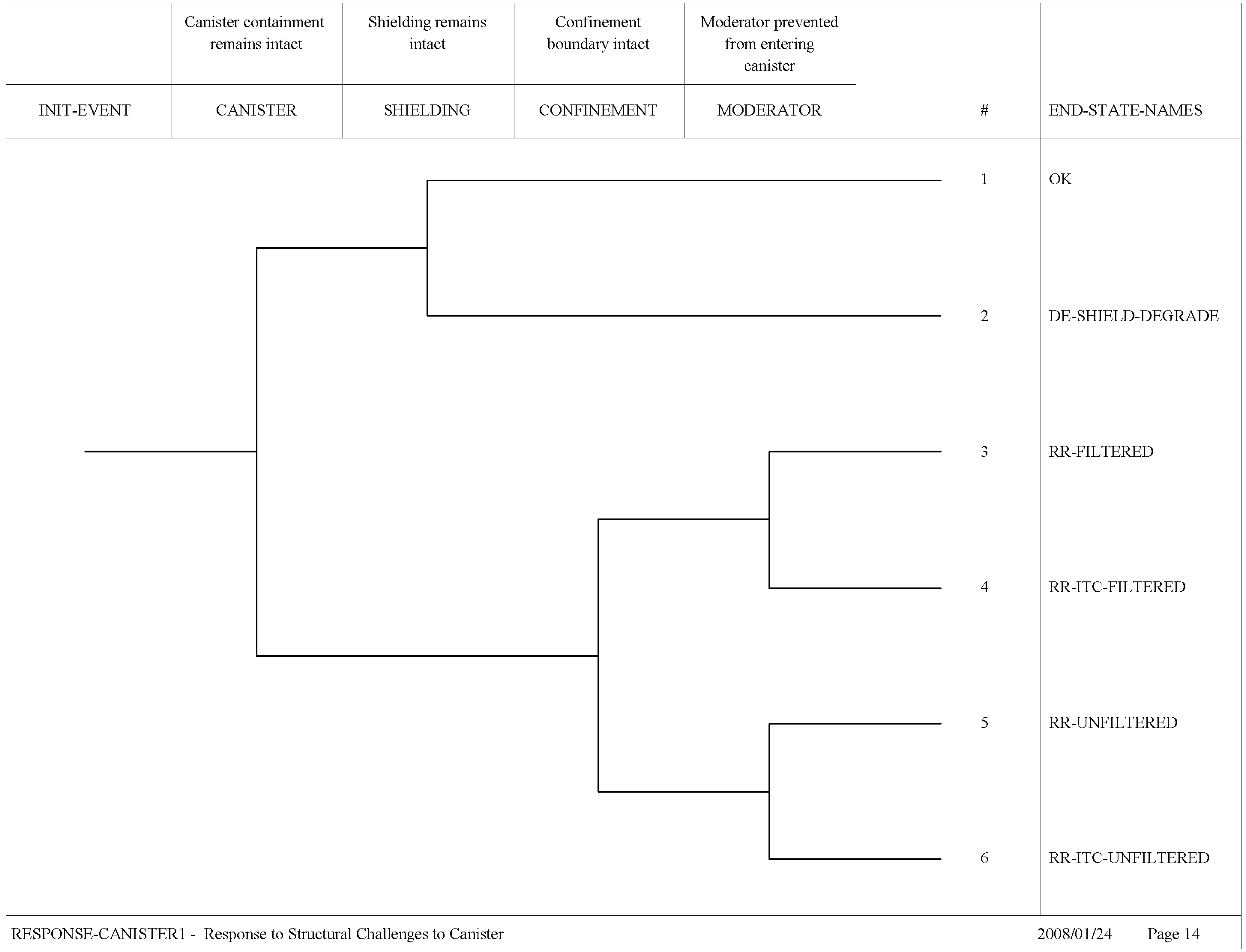

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Figure A5-15. Event Tree RESPONSE-CANISTER1 – Response to Structural Challenges to Canister

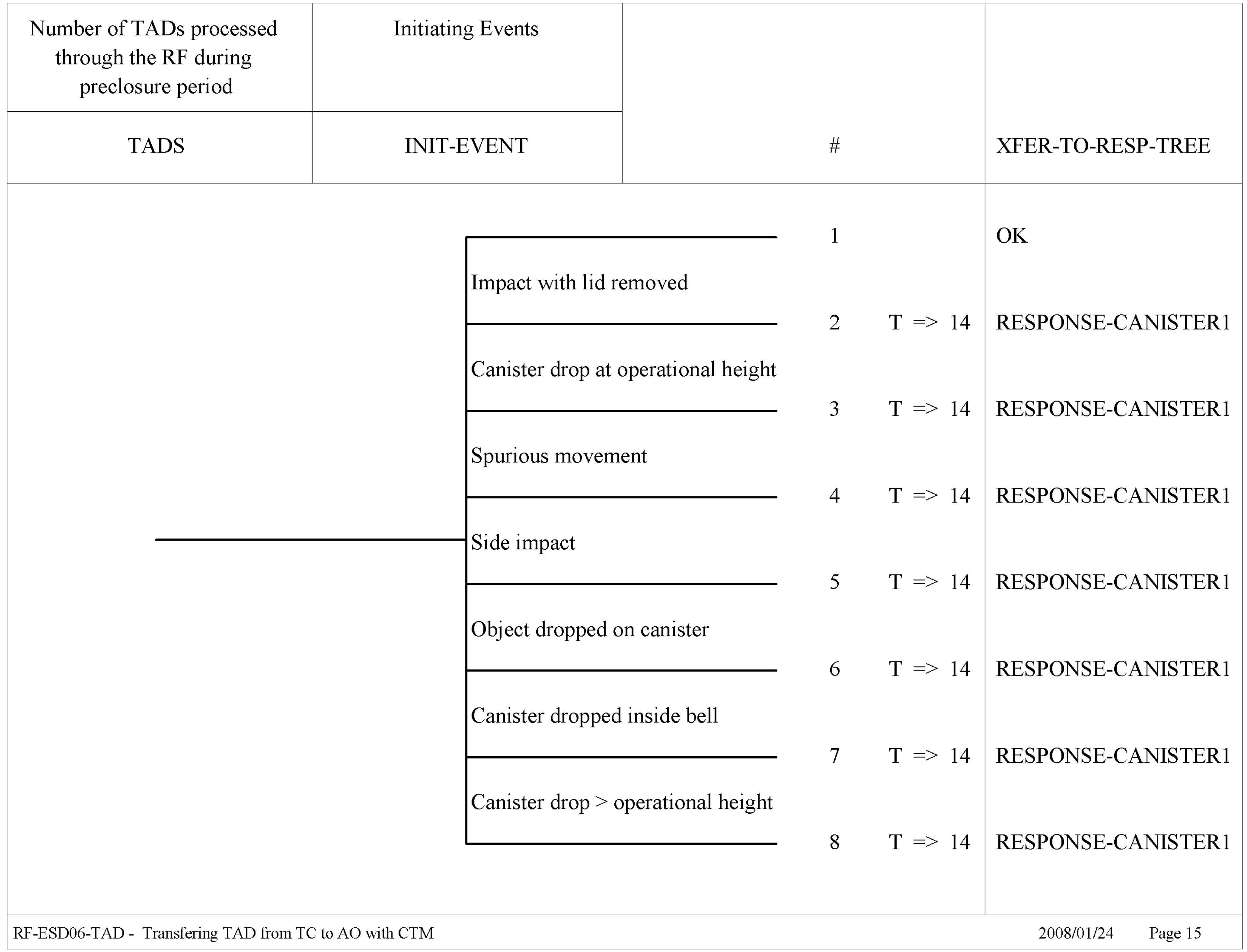

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Figure A5-16. Event Tree RF-ESD06-TAD – Transferring a TAD Canister from a Transportation Cask to an Aging Overpack with the CTM

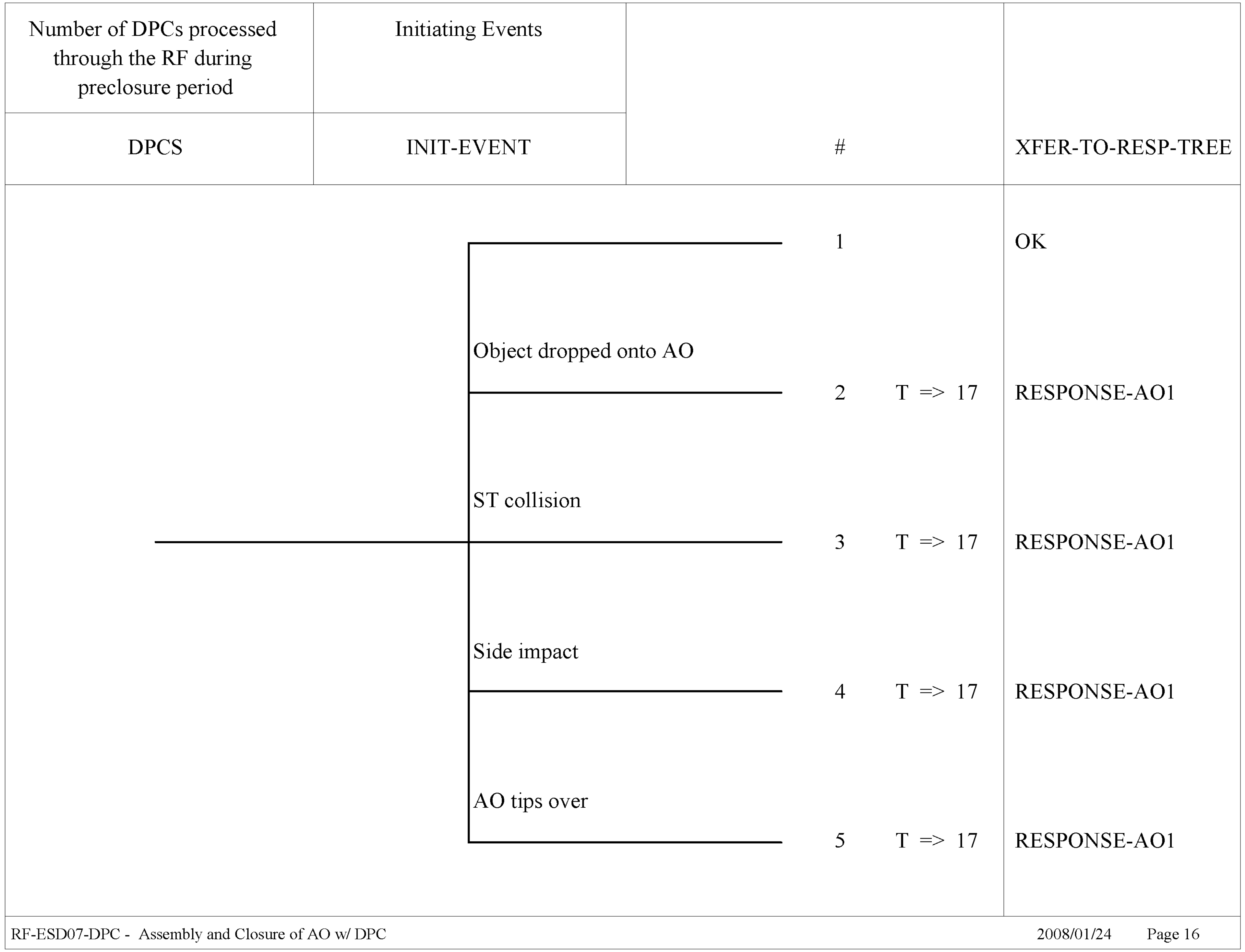

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Figure A5-17. Event Tree RF-ESD07-DPC – Assembly and Closure of an Aging Overpack with a DPC

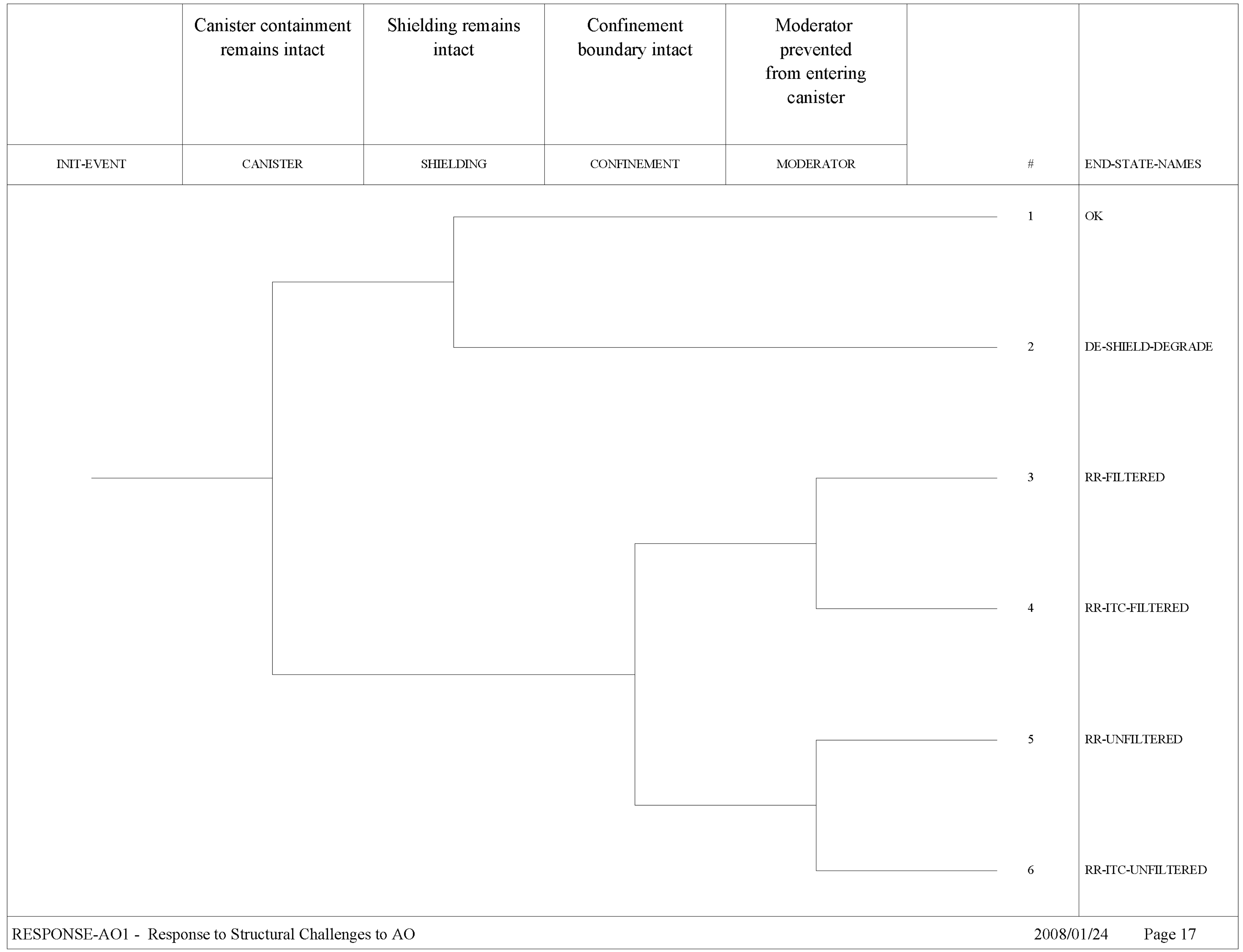

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Figure A5-18. Event Tree RESPONSE-AO1 – Response to Structural Challenges to an Aging Overpack

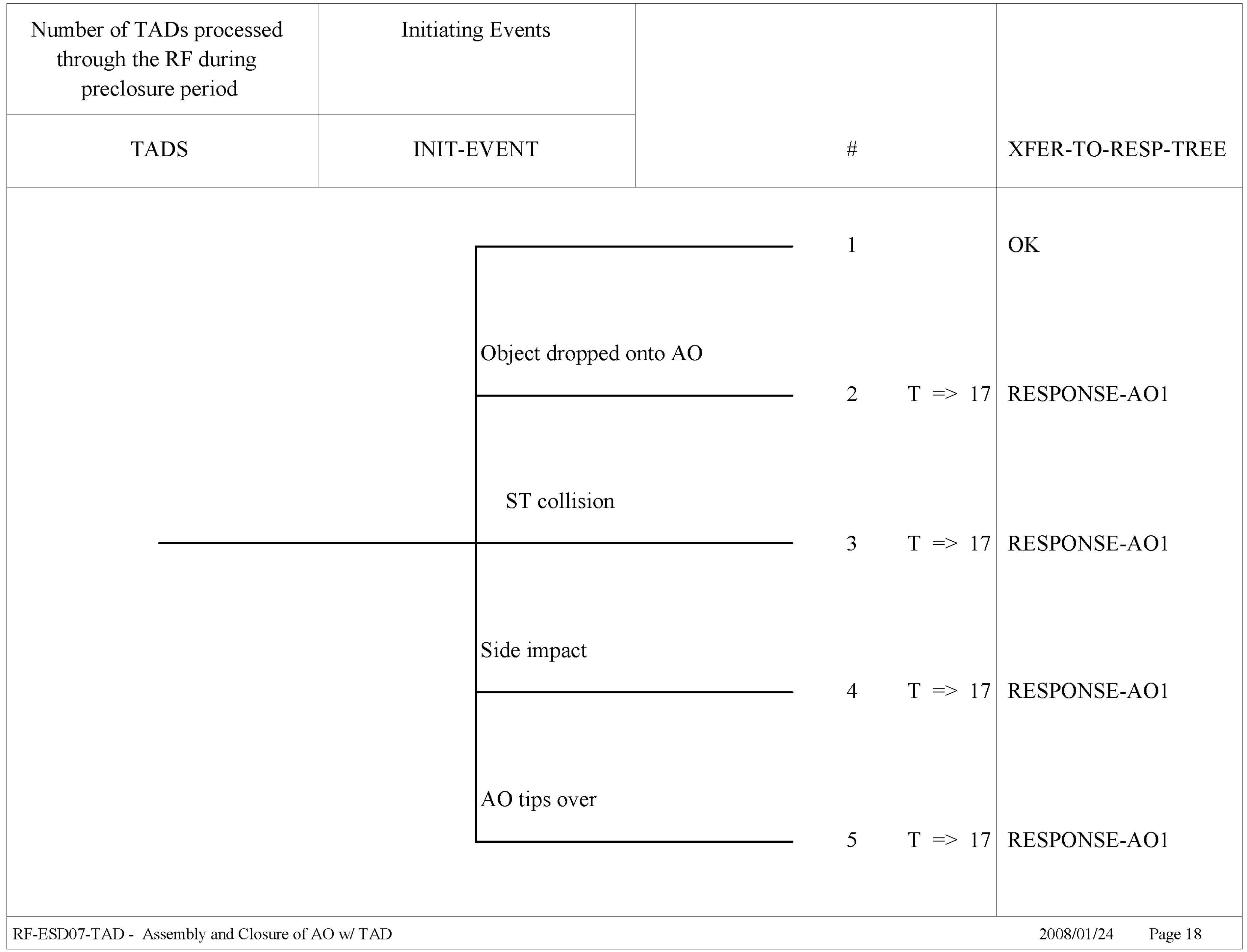

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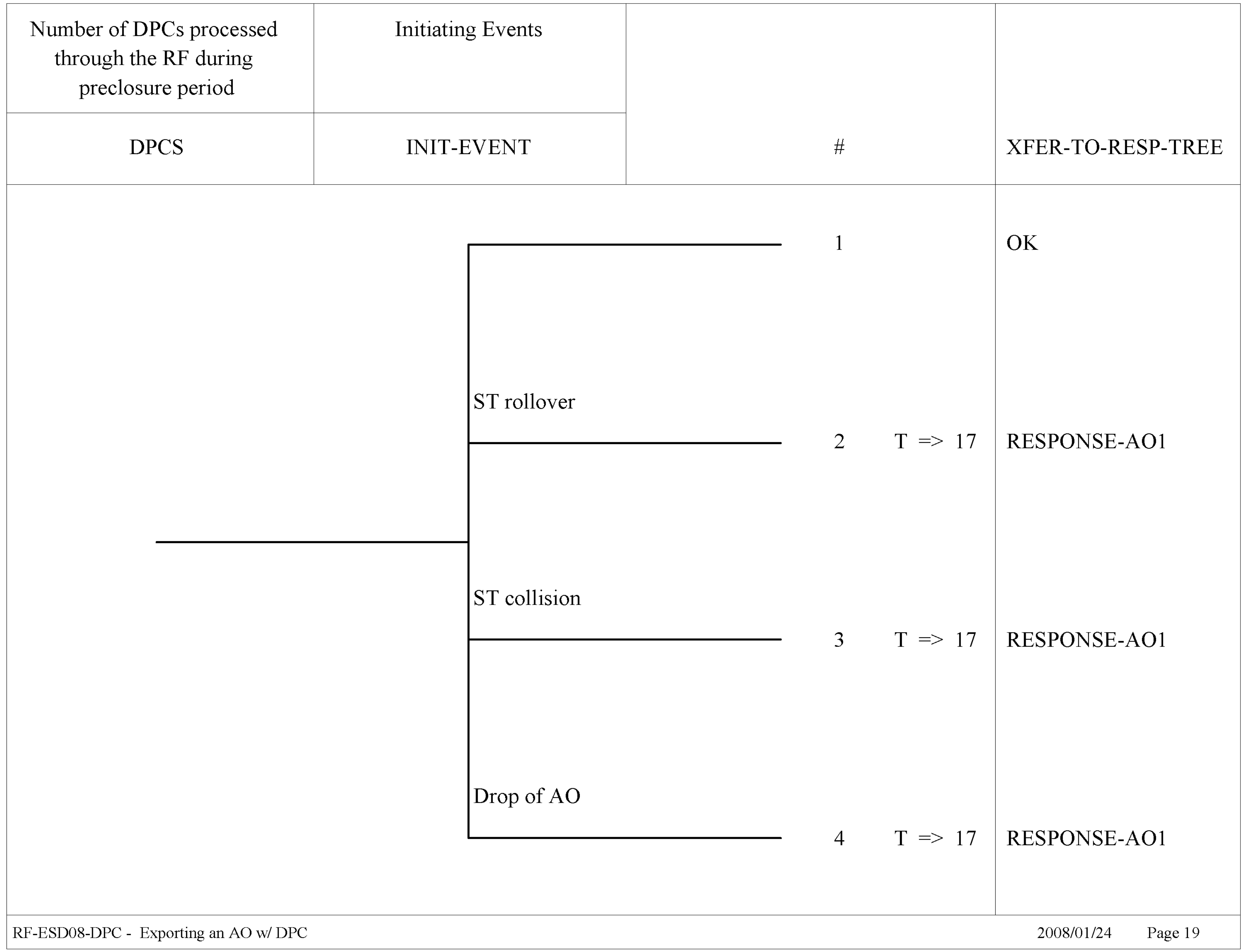
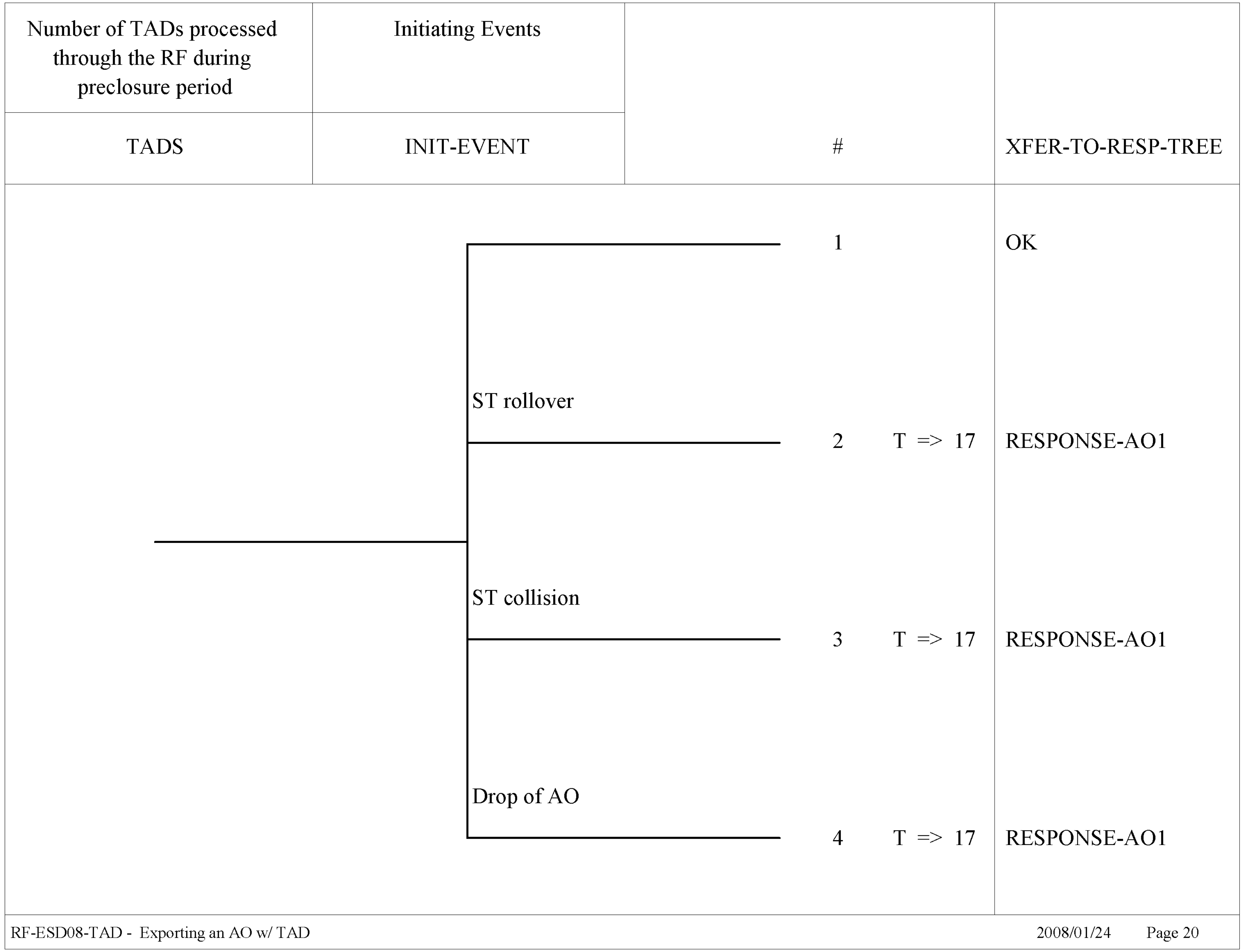
Figure A5-19. Event Tree RF-ESD07-TAD – Assembly and Closure of an Aging Overpack with a TAD Canister



Source: Original

Figure A5-20. Event Tree RF-ESD08-DPC – Export of an Aging Overpack with a DPC



Source: Original

Figure A5-21. Event Tree RF-ESD08-TAD – Export of an Aging Overpack with a TAD Canister

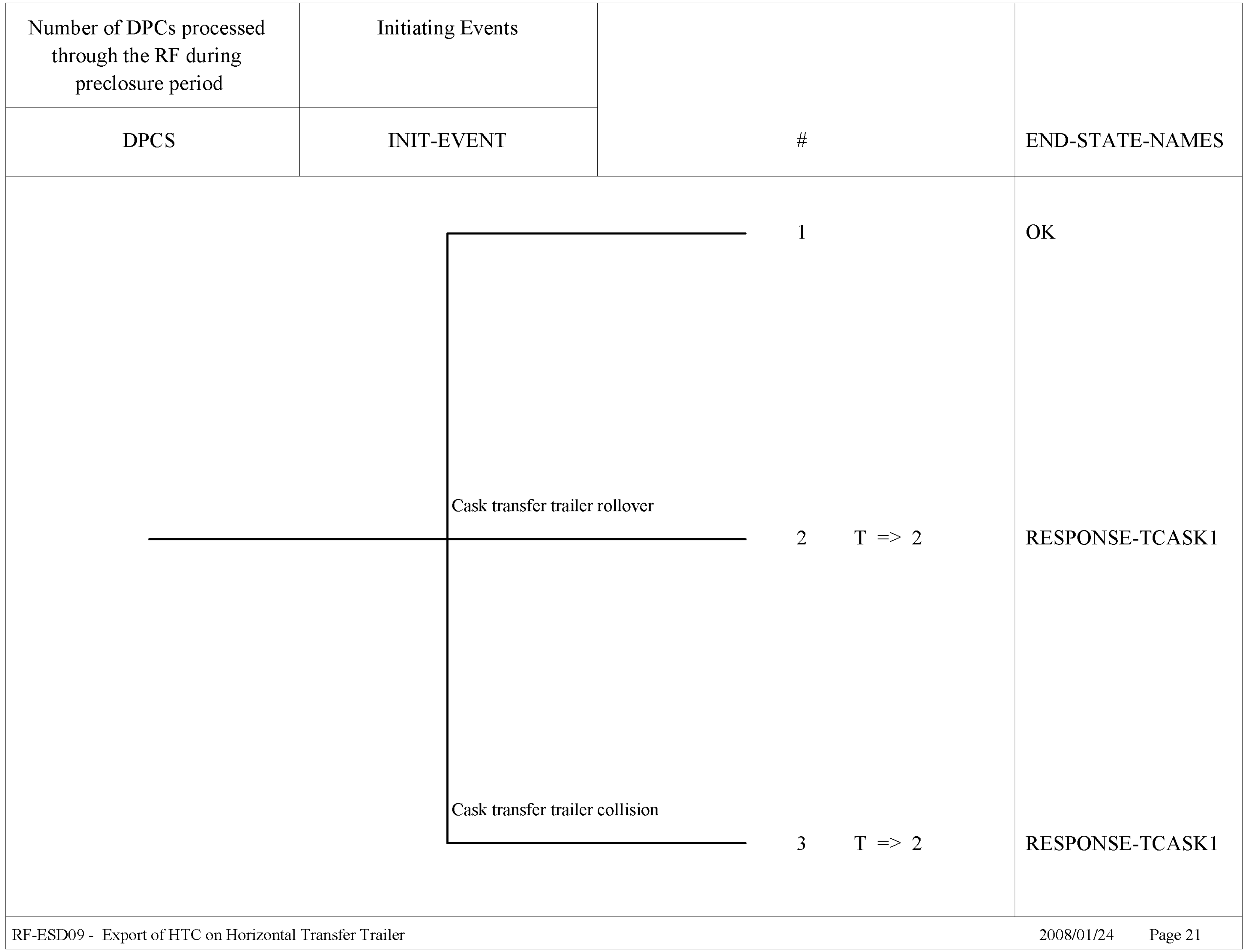


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Figure A5-22. Event Tree RF-ESD09 – Export of an HTC on a Horizontal Transfer Trailer

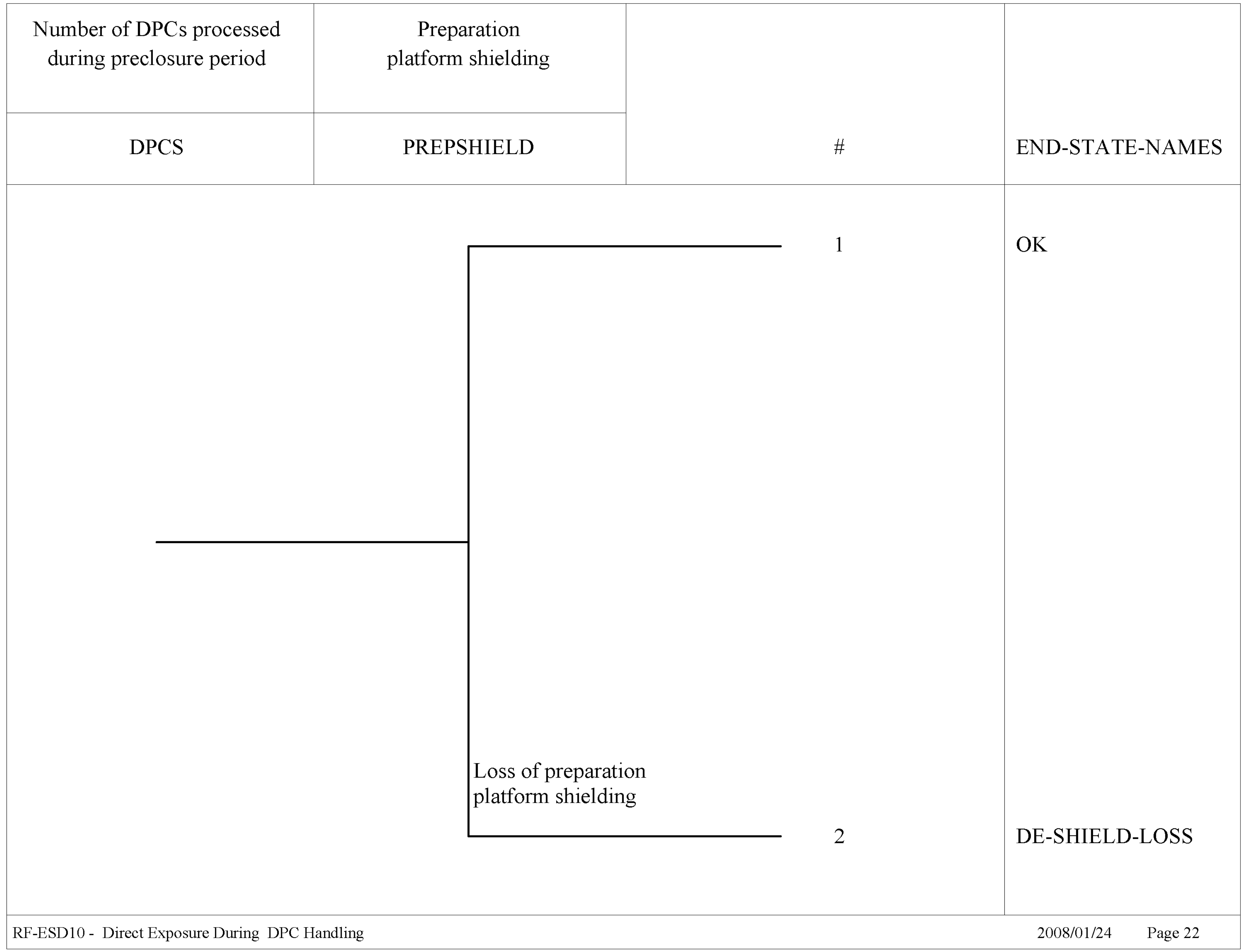

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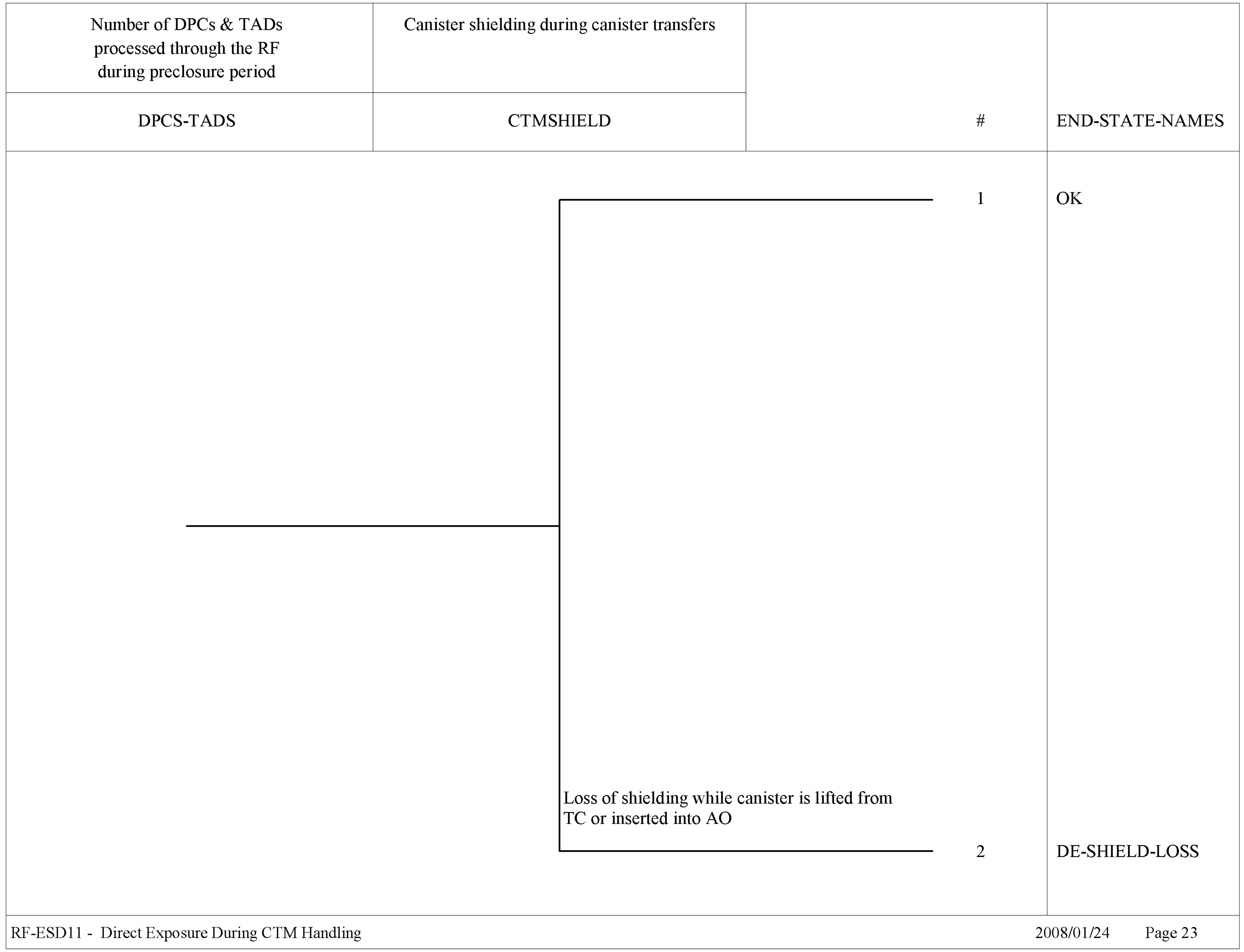
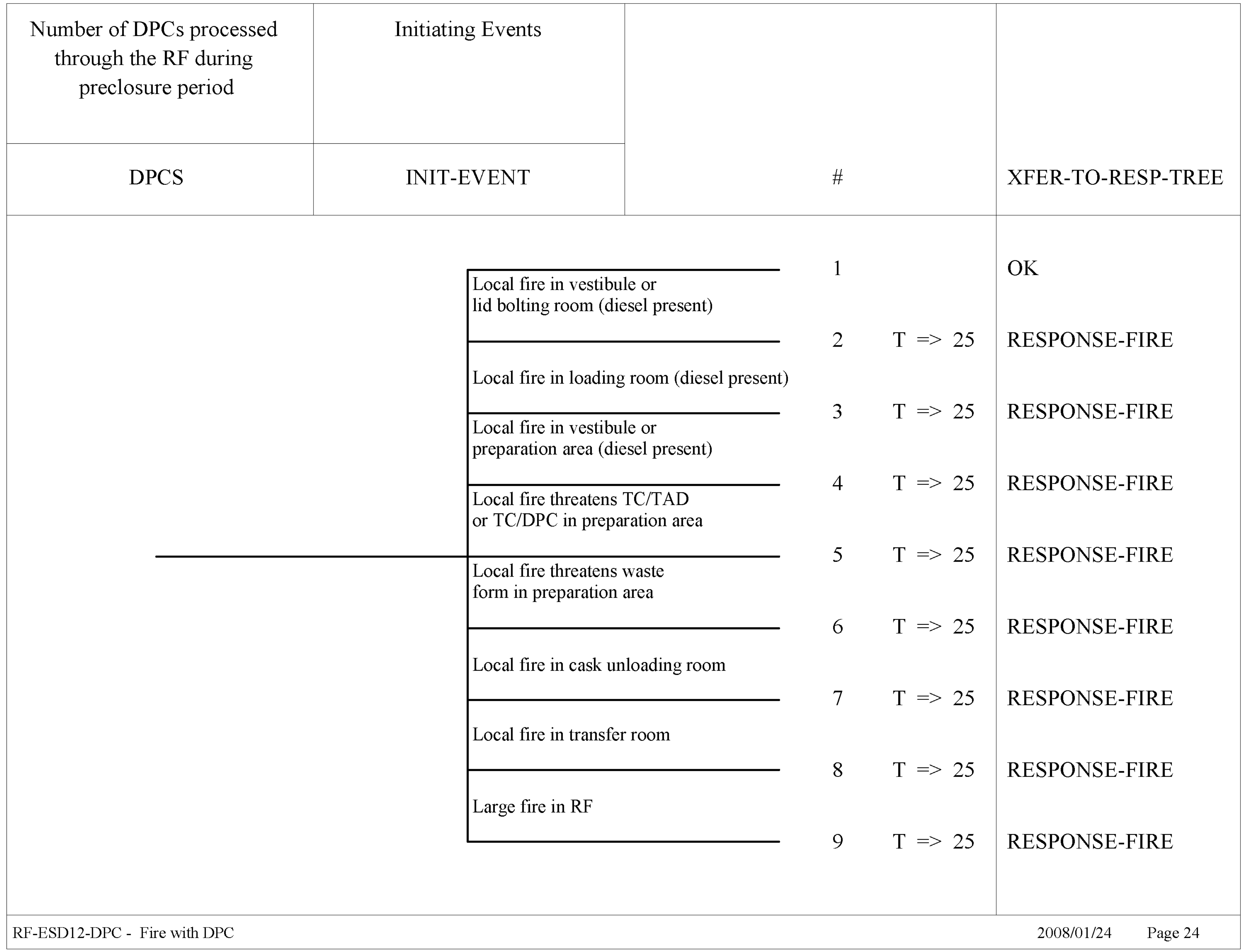
Figure A5-23. Event Tree RF-ESD10 – Direct Exposure during DPC Handling



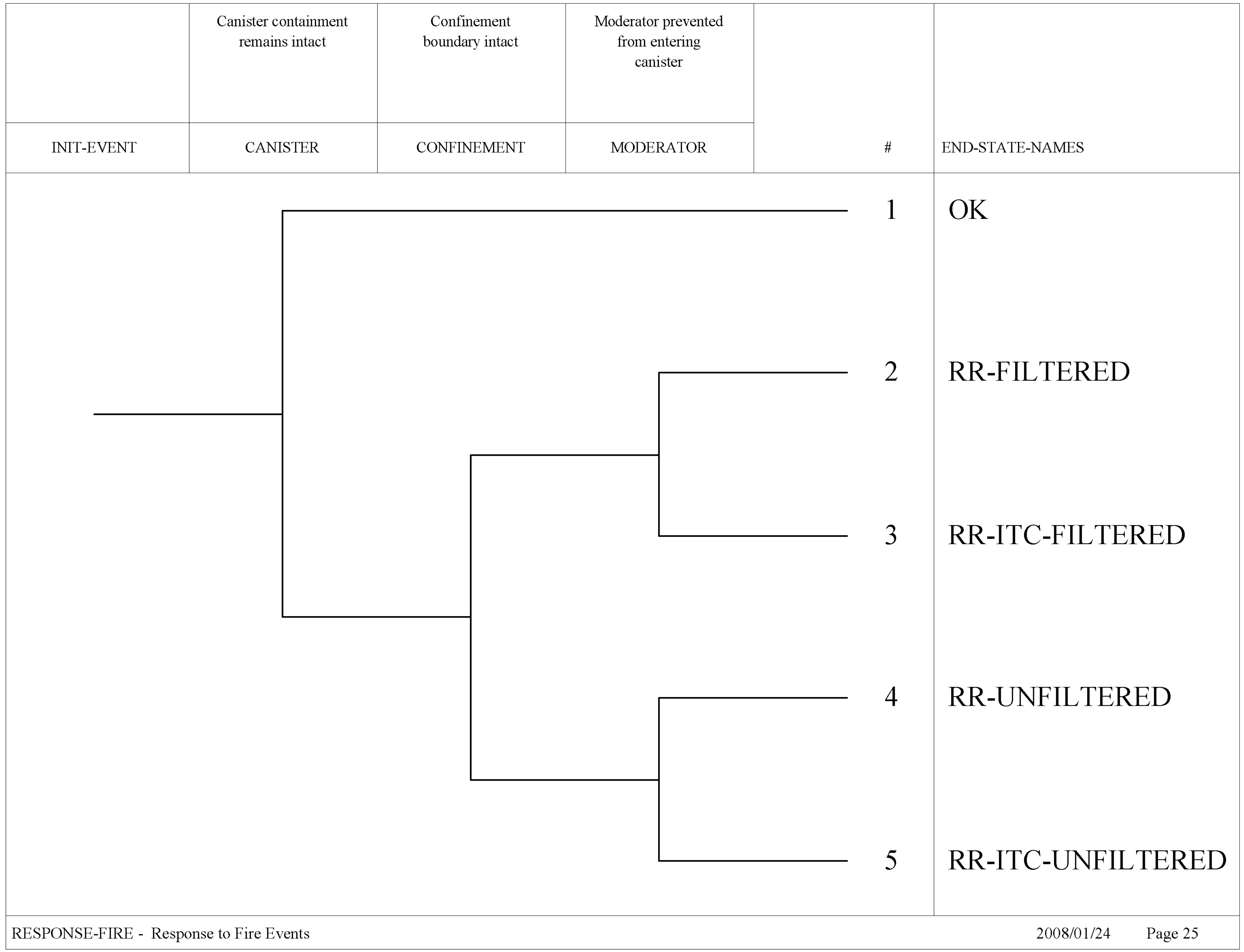
Source: Original

Figure A5-24. Event Tree RF-ESD11 – Direct Exposure during CTM Handling



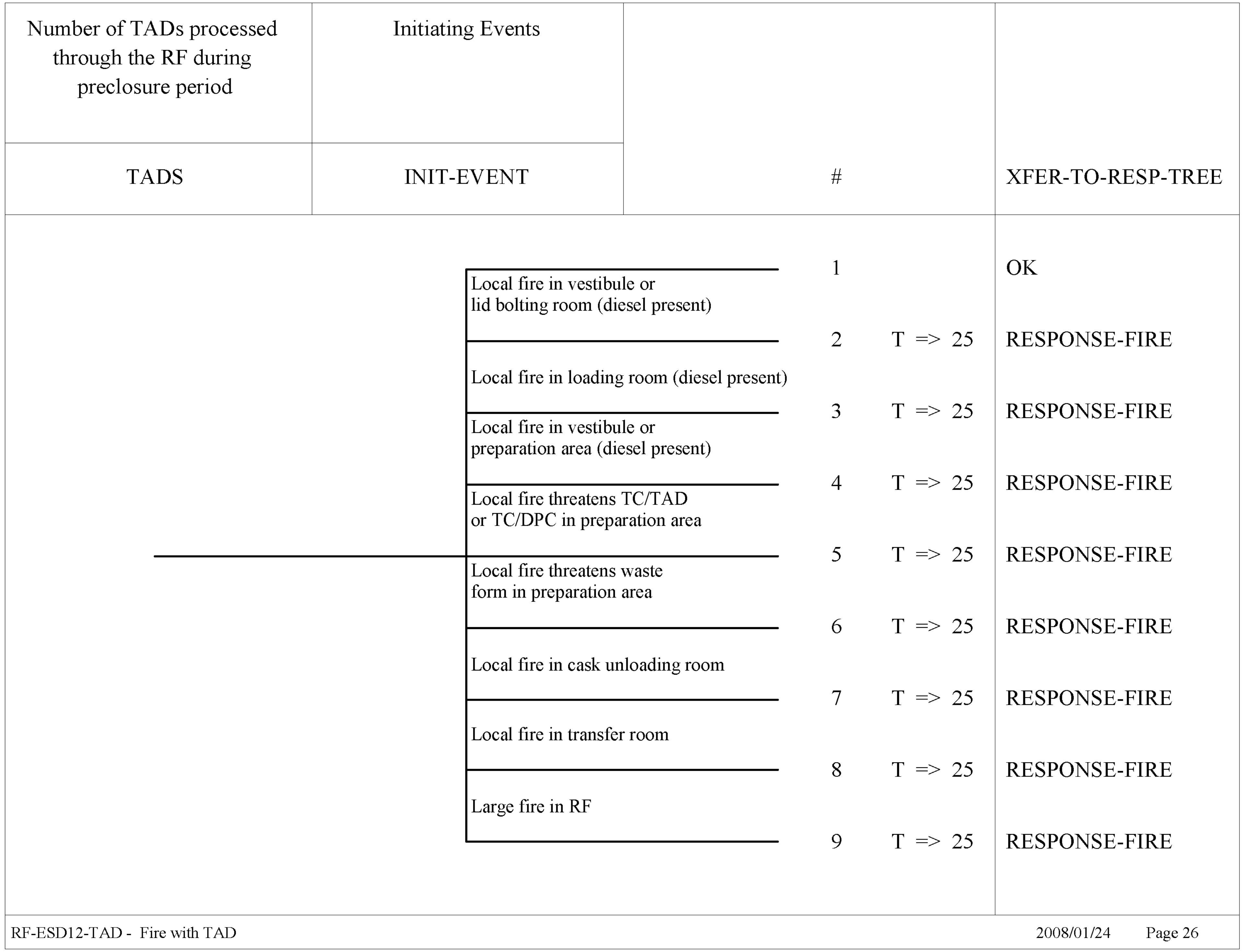
Source: Original

Figure A5-25. Event Tree RF-ESD12-DPC – Fire with a DPC



Source: Original

Figure A5-26. Event Tree RESPONSE-FIRE – Response to Fire Events



Source: Original

Figure A5-27. Event Tree RF-ESD12-TAD – Fire with a TAD Canister

ATTACHMENT B
SYSTEM/PIVOTAL EVENT ANALYSIS – FAULT TREES

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ACRONYMS AND ABBREVIATIONS

Acronyms

AAR	Association of American Railroads
ASD	adjustable speed drive
AHU	air handling unit
CCF	common-cause failure
CRCF	Canister Receipt and Closure Facility
CTT	cask transfer trolley
CTM	canister transfer machine
DOE	U.S. Department of Energy
DPC	dual-purpose canister
EDGF	Emergency Diesel Generator Facility
EPROM	erasable programmable read-only memory
ESD	event sequence diagram
FRA	Federal Railroad Administration
HAM	horizontal aging module
HCTT	cask tractor and the cask transfer trailer
HEP	human error probability
HEPA	high-efficiency particulate air (filter)
HFE	human failure event
HVAC	heating, ventilation and air-conditioning
IHF	Initial Handling Facility
ITS	important to safety
LOSP	loss of offsite power
MCC	motor control center
MCO	multicanister overpack
OOS	out of service
PCSA	preclosure safety analysis
PLC	programmable logic controller
RF	Receipt Facility
SPM	site prime mover
SPMRC	site prime mover railcar
TAD	transportation, aging, and disposal

ACRONYMS AND ABBREVIATIONS (Continued)

UPS	uninterruptible power system
WHF	Wet Handling Facility

Abbreviations

AC	alternating current
cfm	cubic foot per minute
DC	direct current
fpm	foot per minute
hp	horsepower
Hz	Hertz
in.	inch
kV	kilovolt
kW	kilowatt
mph	mile per hour
psi	pound per square inch
rpm	revolution per minute
scfm	standard cubic foot per minute
V	volt

ATTACHMENT B
SYSTEM/PIVOTAL EVENT ANALYSIS – FAULT TREES

This attachment presents system and pivotal event fault trees that are used in the event trees described in Attachment A. The system fault trees are presented and described in Sections B1 through B8, on a system basis. The pivotal event fault trees are presented in Section B9. For the most part, the pivotal events link to a basic event and these are presented in tables. In a few cases, the assignment is not straightforward and a supplemental fault tree provides a link to the generic fault tree or basic event level. These supplemental fault trees are presented and described.

B1 SITE PRIME MOVER ANALYSIS – FAULT TREES

B1.1 REFERENCES

Design Input

The preclosure safety analysis (PCSA) is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

The inputs in this Section noted with an asterisk (*) indicate that they fall into one of the designated categories described in Section 4.1, relative to suitability for intended use.

B1.1.1 *AAR S-2043. 2003. *Performance Specification for Trains Used to Carry High-Level Radioactive Material.* Washington, D.C.: Association of American Railroads. TIC: 257585.

B1.2 SITE PRIME MOVER DESCRIPTION

B1.2.1 Overview

The site prime mover (SPM) is a diesel/electric self-propelled vehicle that is designed to move railcars or truck trailers loaded with transportation casks. The transport occurs both in the Intra-Site Operations and within the Canister Receipt and Closure Facility (CRCF), the Wet Handling Facility (WHF), the Initial Handling Facility (IHF), and the Receipt Facility (RF).

Only the site prime mover railcar (SPMRC) enters the RF. Movement of SPMRC within the RF is limited to the Transportation Cask Vestibule (1021A), Transportation Cask Vestibule Annex (1021), the Cask Preparation Room Annex (1017A), and the Cask Preparation Room (1017).

Transportation casks arriving at the RF can contain:

- Dual-purpose canisters (DPCs)
- Transportation, aging, and disposal (TAD) canisters.

B1.2.2 System Description

B1.2.2.1 Site Prime Mover

The SPM is a commercially available vehicle that has the capability of moving both railcars and truck trailers loaded with transportation casks. Retractable railroad wheels attached to the front and rear axles of the SPM are used for rail operations.

The driving and braking power comes directly from the road tires as they are in contact with the rails. Weight sharing between the flanged rail and regular road wheels is automatically varied to achieve the required power transmission needs. More weight can be distributed on the rail wheels when moving, or more on the road wheels when braking, accelerating, and negotiating inclines. The SPM has speed limiters that set the maximum speed of the vehicle to less than 9.0 mph.

During Intra-Site Operation activities, the diesel engine drives the generator, which provides the required 480V, 3-phase, 60 Hz power to the vehicle. During facility operations, the diesel engine is disabled and facility 480V, 3-phase, 60 Hz power is supplied to the generator. The diesel engine is not used to move the railcar inside the facility.

The SPM is equipped with an automatic wagon coupling system for railcars. In addition, the SPM is equipped with high-performance compressors, a priority filling system, an electronic regulating valve with filling speed adjustments, and a 99 gallon diesel fuel tank.

B1.2.2.2 Railcars

Railcars used for movement of transportation casks are designed in accordance with Federal Railroad Administration (FRA) requirements under authority delegated by the Secretary of Transportation. The FRA administers a safety program that oversees the movement of nuclear shipments throughout the national rail transportation system. Performance standards are addressed in the Association of American Railroads (AAR) Standard S-2043 (Ref. B1.1.1).

B1.2.2.3 Subsystems

The SPMRC system is composed of four subsystems:

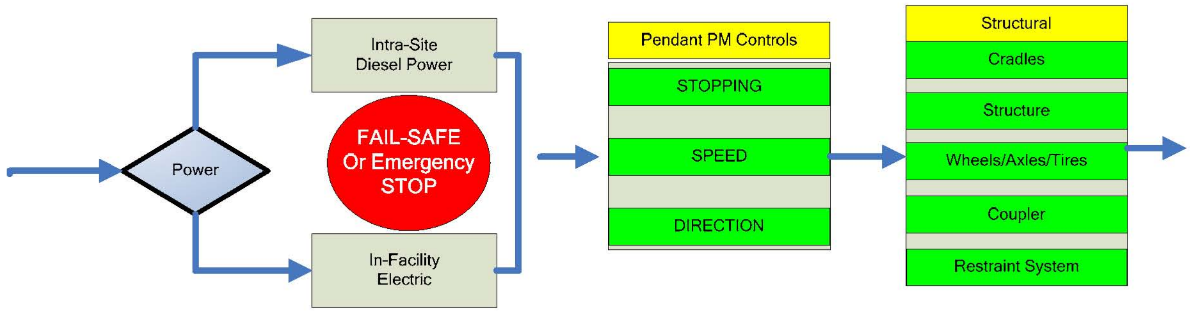
- Power plant–a diesel engine, generator, and diesel fuel tank are enclosed in the SPM. The SPM utilizes a diesel engine for all Intra-Site Operations. For operations conducted inside facilities, the SPM is connected to facility 480V, 3-phase, 60 Hz power.

- Vehicle controls–during Intra-Site Operations, the operator controls the SPM at the operator's console inside the SPM. For all operations inside of facilities, the operator controls the SPM with either a remote (wireless) controller or through a pendant connected to the vehicle.

- Structural controls–these subsystems include restraints for securing the transportation casks to the railcar/truck trailer; automatic coupler hardware; cradles for supporting the transportation cask; and wheels/tires and axles.

- Brakes–for the railcar, brakes comply with FRA requirements.

A simplified block diagram of the functional components on the SPMRC is shown in Figure B1.2-1.



Source: Original

Figure B1.2-1. Site Prime Mover Simplified Block Diagram Intra-Site and In-Facility

B1.2.3 Operations

B1.2.3.1 Normal Operations

In-facility SPM operations begin when the SPM has positioned the railcar outside the Transportation Cask Vestibule at the facility such that the railcar is pushed into the facility. The SPM diesel engine is shut down and the outer and inner vestibule doors are opened. Facility 480V, 3-phase, 60 Hz power is connected to the SPM for all operations inside the facility. The SPM is never operated inside a facility using the diesel engine.

The operator connects the pendant controller or uses a remote (wireless) controller to move the railcar into the Transportation Cask Vestibule and Transportation Cask Vestibule Annex. Once inside, the outer vestibule door is closed. The Cask Preparation Room Annex door is then opened and the SPM moves the railcar into position in the Cask Preparation Room. Once in position, the SPM is disconnected from the railcar and returns to the Transportation Cask Vestibule. The Cask Preparation Room Annex door is then closed. The outer vestibule door can then be opened and the SPM exits the facility. Once outside, the SPM is shut down and the facility power is removed and the inner and outer vestibule doors are closed.

B1.2.3.2 Site Prime Mover Off-Normal Operations

In the event of loss of power, the SPM is designed to stop, retain control of the railcar, and enter a locked mode. Upon the restoration of power the SPM remains in the locked mode until operator action is taken to return to normal operations.

B1.2.3.3 Site Prime Mover Testing and Maintenance

Testing and maintenance of the SPM is done on a periodic basis and does not affect the normal operations of the SPM. Testing and/or maintenance are not performed on a SPM when it is coupled with a railcar. A SPM that has malfunctioned or has a warning light lit on the SPM is deemed unserviceable and turned in for maintenance. Unserviceable vehicles are not used.

If an unserviceable state is identified during movement, the operator puts the SPM into a safe state (as quickly as possible) and recovery actions for the SPM are invoked.

B1.3 DEPENDENCIES AND INTERACTIONS ANALYSIS

Dependencies are broken down into five categories with respect to their interactions with system, structures, and components. The five areas considered are addressed in Table B1.3-1 with the following dependencies:

1. Functional dependence.
2. Environmental dependence.
3. Spatial dependence.
4. Human dependence.
5. Failures based on external events.

Table B1.3-1. Dependencies and Interactions Analysis

Systems, Structures, Components	Dependencies and Interactions				
	Functional	Environ-mental	Spatial	Human	External Events
Structural	—Material failure —Coupler —Wheels/tires/axle	—	—	—	—
Brakes	—Material failure	—	—	—Failure to engage (set)	—
Power plant	—Governor fails —Safe state on	—	—	—Failure to stop	—
Remote control	—Spurious commands	—	—	—Improper command	- Collide end stops

Source: Original

B1.4 SITE PRIME MOVER RELATED FAILURE SCENARIOS

There are two top events for the SPM operating inside the RF:

1. SPMRC collides with RF structures.
2. SPMRC Derailment.

Table B1.4-1 provides a cross reference between the event sequence diagram (ESD) and the SPM fault trees that support them. Potential fire scenarios associated with the SPM are discussed in Section 6.5 and Attachment F.

Table B1.4-1. ESD Cross Reference with SPMRC Fault Trees

RF ESD Number	SPMRC Collision	SPMRC Derailment
ESD01-DPC	X	X
ESD01-TAD	X	X

NOTE: ESD = event sequence diagram, RF = Receipt Facility;
SPMRC = site prime mover railcar.

Source: Original

B1.4.1 SPMRC Collides with RF Structures

B1.4.1.1 Description

The two fault trees for SPMRC collision within the RF are identical for each type of transportation cask. Collision can occur as a result of human error or mechanical failures. Mechanical failures leading to a collision consist of the SPM failure to stop when commanded, the SPM exceeding a safe speed, or the SPM moving in a wrong direction.

B1.4.1.2 Success Criteria

The success criteria for preventing a collision includes safety design features incorporated in the SPM for mechanical failures and the SPM operator maintaining situational awareness and proper control of the movement of the SPM. To avoid collisions, the SPM must stop when commanded, be prevented from entering a runaway situation, or respond correctly to a SPM movement command.

The SPM is designed to stop whenever commanded to stop or when there is a loss of power. The operator can stop the SPM by either commanding a "stop" from the start/stop button or by releasing the palm switch which initiates an emergency stop. At anytime there is a loss of power detected, the SPM immediately stops all movement and enters into a "lock mode" safe state. The SPM remains in this locked mode until power is returned and the operator restarts the SPM.

Runaway situations on the SPM are prevented by hardware constraints. The maximum speed of the SPM is controlled by a speed limiter on the diesel engine for outside facility movement. The speed control on the SPM for in-facility operations is controlled by the physical limitations of the drive system. The SPM gearing prevents the SPM from exceeding 9.0 mph. Simultaneous operation of the railroad wheels and the road tires is prevented by design of the SPM.

B1.4.1.3 Design requirements and Features

Requirements

Since the dominant contributor to a SPMRC collision in the facility is human error, no priority is given to either the remote or the pendant controllers. The SPM is operated on electrical power when inside the building. The SPM is disconnected from the railcar in the Cask Preparation Room and moved out of the building before cask preparation activities begin.

Design Features

The SPM has two off-equipment control devices that have complete control over the SPMRC. The drive system limits the maximum speed of the SPM to 9.0 mph.

System Configuration and Operating Conditions

Requirements

Two means of stopping the SPM are incorporated in the controllers. One is the normal stop button and the other consists of an emergency stop that has the equivalent of a “deadman switch.” On the loss of AC power derived from the facility, the SPM immediately enters the lock mode state. The lock mode state is not reversible without specific operator action.

Design Features and Inputs

Stopping the SPM is accomplished by pushing the “stop” button on the remote or pendant controller. The SPM, upon receiving a stop command from either control source, immediately responds by removing power from the propulsion system on the SPM.

Testing and Maintenance

Requirements

No maintenance or testing is permitted on a SPM loaded with a transportation cask.

Design Feature

None

B1.4.1.4 Fault Tree Model

The fault tree model for “SPMRC Collision in the RF” accounts for both human error and/or SPMRC mechanical problems that could result in a collision. There is only one movement within the RF. Once the SPMRC has been properly positioned within the Cask Preparation Room, the SPM is decoupled from the railcar and is moved out of the facility.

The top event is a collision of the SPMRC in the RF and is shown in Figure B1.4-3. This may occur due to human error coupled with failure of the speed control or interlocks, or failure of the mechanical and/or control system, including failure to stop (Figure B1.4-4) or exceeding a safe speed (Figure B1.4-5). Failure to stop may occur due to mechanical failure of brakes or failure of the control system. Exceeding a safe speed may also occur due to failure of the control system.

This fault tree model for “SPMRC Collision in the RF” is identical for both DPC and TAD canister movements.

B1.4.1.5 Basic Event Data

Table B1.4-2 contains a list of basic events used in the "SPMRC Collides with RF Structures" fault trees. The mission time has been set at one hour. This is a conservative estimate since it does not require one hour to move the railcar into the facility, disconnect the SPM from the railcar, and move the SPM back outside the facility.

Table B1.4-2. Basic Event Probability for SPMRC Collides with RF Structures

Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-OPRCCOLLIDE1-HFI-NOD	1	3.000E-003	3.000E-003	0.000E+000	0.000E+000
200-OPRCINTCOL01-HFI-NOD	1	1.000E+000	1.000E+000	0.000E+000	0.000E+000
200-OPRCINTCOL02-HFI-NOD	1	1.000E+000	1.000E+000	0.000E+000	0.000E+000
200-PWR-LOSS	1	4.100E-006	4.100E-006	0.000E+000	0.000E+000
200-SPMRC-BRP000-BRP-FOD	1	5.020E-005	5.020E-005	0.000E+000	0.000E+000
200-SPMRC-BRP001-BRP-FOD	1	5.020E-005	5.020E-005	0.000E+000	0.000E+000
200-SPMRC-CBP001-CBP-OPC	3	9.130E-008	0.000E+000	9.130E-008	1.000E+000
200-SPMRC-CBP001-CBP-SHC	3	1.880E-008	0.000E+000	1.880E-008	1.000E+000
200-SPMRC-CPL00-CPL-FOH	3	1.910E-006	0.000E+000	1.910E-006	1.000E+000
200-SPMRC-CT000--CT--FOD	1	4.000E-006	4.000E-006	0.000E+000	0.000E+000
200-SPMRC-CT0001-CT-FOD	1	4.000E-006	4.000E-006	0.000E+000	0.000E+000
200-SPMRC-CT002--CT--FOH	3	6.880E-005	0.000E+000	6.880E-005	1.000E+000
200-SPMRC-CT003-CT-SPO	3	2.270E-005	0.000E+000	2.270E-005	1.000E+000
200-SPMRC-G65000-G65-FOH	3	1.160E-005	0.000E+000	1.160E-005	1.000E+000
200-SPMRC-HC001--HC--SPO	3	5.230E-007	0.000E+000	5.230E-007	1.000E+000
200-SPMRC-HC001-HC--FOD	1	1.740E-003	1.740E-003	0.000E+000	0.000E+000
200-SPMRC-IEL011-IEL-FOD	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-SPMRC-MOE000-MOE-FSO	3	1.350E-008	0.000E+000	1.350E-008	1.000E+000
200-SPMRC-SC021--SC--FOH	3	1.280E-004	0.000E+000	1.280E-004	1.000E+000
200-SPMRC-SEL021-SEL-FOH	3	4.160E-006	0.000E+000	4.160E-006	1.000E+000
200-SPMRC-STU001-STU-FOH	3	2.107E-004	0.000E+000	4.810E-008	4.380E+003

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.

Calc. = calculation; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B1.4.1.5.1 Human Failure Events

Three human errors have been identified for this fault tree. Section 6.4 and Attachment E contain a detailed analysis on the derivation of the failure data.

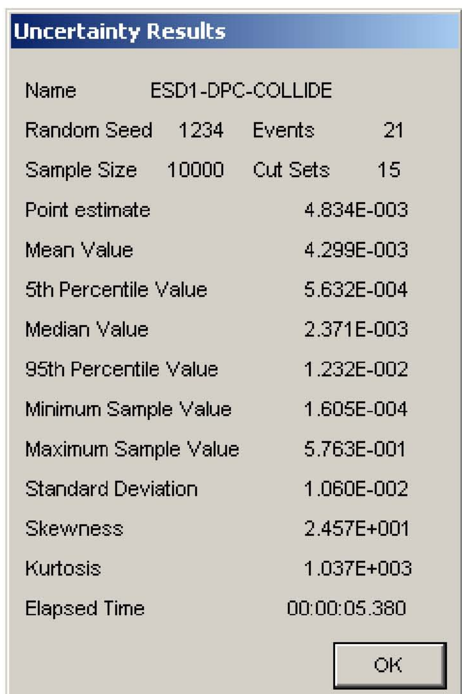
1. Operator causes collision (200-OPRCCOLLIDE1-HFI-NOD)
2. Operator initiates runaway (200-OPRCINTCOL01-HFI-NOD)
3. Operator causes SPMRC collision with mobile platform (200-OPRCINTCOL02-HFI-NOD).

B1.4.1.5.2 Common-Cause Failures

There are no common-cause failures.

B1.4.1.6 Uncertainty and Cut Set Generation Results

Figure B1.4-1 contains the uncertainty results obtained from running the fault tree for the "SPMRC Collides with RF Structures" fault tree. Figure B1.4-2 provides the cut set generation results for the "SPMRC Collides with RF Structures" fault tree.



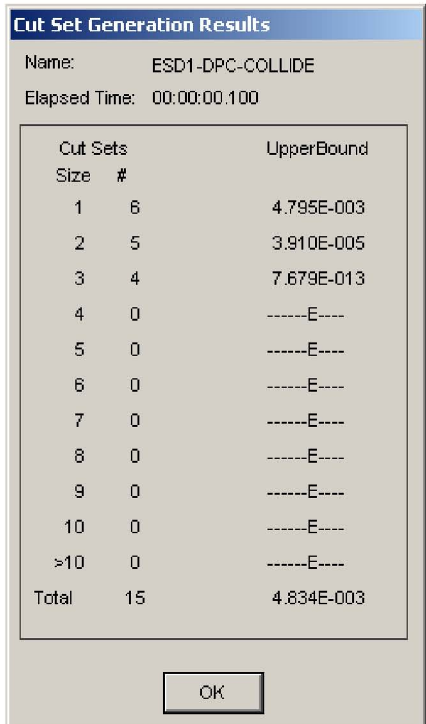
Uncertainty Results

Name	ESD1-DPC-COLLIDE		
Random Seed	1234	Events	21
Sample Size	10000	Cut Sets	15
Point estimate			4.834E-003
Mean Value			4.299E-003
5th Percentile Value			5.632E-004
Median Value			2.371E-003
95th Percentile Value			1.232E-002
Minimum Sample Value			1.605E-004
Maximum Sample Value			5.763E-001
Standard Deviation			1.060E-002
Skewness			2.457E+001
Kurtosis			1.037E+003
Elapsed Time			00:00:05.380

OK

Source: Original

Figure B1.4-1. Uncertainty Results of the SPMRC Collides with RF Structures Fault Tree



Source: Original

Figure B1.4-2. Cut set Generation Results for the SPMRC Collides with RF Structures Fault Tree

B1.4.1.7 Cut sets

Table B1.4-3 contains the cut sets for "SPMRC Collides with RF Structures". The probability of failure is 4.834E-3.

Table B1.4-3. Cut Sets for SPMRC Collides with RF Structures

Fault Tree	Cut set %	Prob./Freq.	Basic Event	Description	Probability
ESD1-DPC-COLLIDE	62.07	3.000E-003	200-OPRCCOLLIDE1-HFI-NOD	Operator Causes Collision	3.0E-003
	36.00	1.740E-003	200-SPMRC-HC001-HC--FOD	Pendant Control Transmits Wrong Signal	1.7E-003
	1.04	5.020E-005	200-SPMRC-BRP000-BRP-FOD	Brake (Pneumatic) Failure on Demand Brake (Pneumatic) Failure on Demand PMRC Fails to Stop on Loss of Power	5.0E-005
	0.57	2.750E-005	200-OPRCINTCOL02-HFI-NOD	Operator Causes Collision with Mobile Platform	1.0E+000

Table B1.4-3. Cut Sets for SPMRC Collides with RF Structures (Continued)

Fault Tree	Cut set %	Prob./Freq.	Basic Event	Description	Probability
			200-SPMRC-IEL011-IEL-FOD	Failure of Mobile Platform Anti-Coll Interlock	2.8E-005
	0.24	1.160E-005	200-OPRCINTCOL01-HFI-NOD	Operator Initiates Runaway	1.0E+000
			200-SPMRC-G65000-G65-FOH	SPMRC Speed Control (Governor) Fails	1.2E-005
	0.08	4.000E-006	200-SPMRC-CT000--CT--FOD	SPMRC Primary Stop Switch Fails	4.0E-006
	0.08	4.000E-006	200-SPMRC-CT0001-CT-FOD	On-Board Controller Fails to Respond	4.0E-006
	0.04	1.910E-006	200-SPMRC-CPL00-CPL-FOH	Railcar Automatic Coupler System Fails	1.9E-006
	0.00	7.275E-013	200-SPMRC-BRP001-BRP-FOD	SPMRC Brake (Pneumatic) Failure on Demand	5.0E-005
			200-SPMRC-CT002--CT--FOH	Pendant Direction Controller Fails	6.9E-005
			200-SPMRC-STU001-STU-FOH	SPMRC End Stops Fail	2.1E-004
	0.00	5.535E-014	200-PWR-LOSS	Loss of Site Power	4.1E-006
			200-SPMRC-MOE000-MOE-FSO	SPMRC Lock Mode State Fails on Loss of Power	1.4E-008
	0.00	3.370E-014	200-SPMRC-CT003-CT-SPO	On-Board Controller Initiates Spurious Signal	2.3E-005
			200-SPMRC-G65000-G65-FOH	SPMRC Speed Control (Governor) Fails	1.2E-005
			200-SPMRC-SC021--SC--FOH	Speed Controller on SPMRC Pendant Fails	1.3E-004
	0.00	5.531E-015	200-SPMRC-BRP001-BRP-FOD	SPMRC Brake (Pneumatic) Failure on Demand	5.0E-005

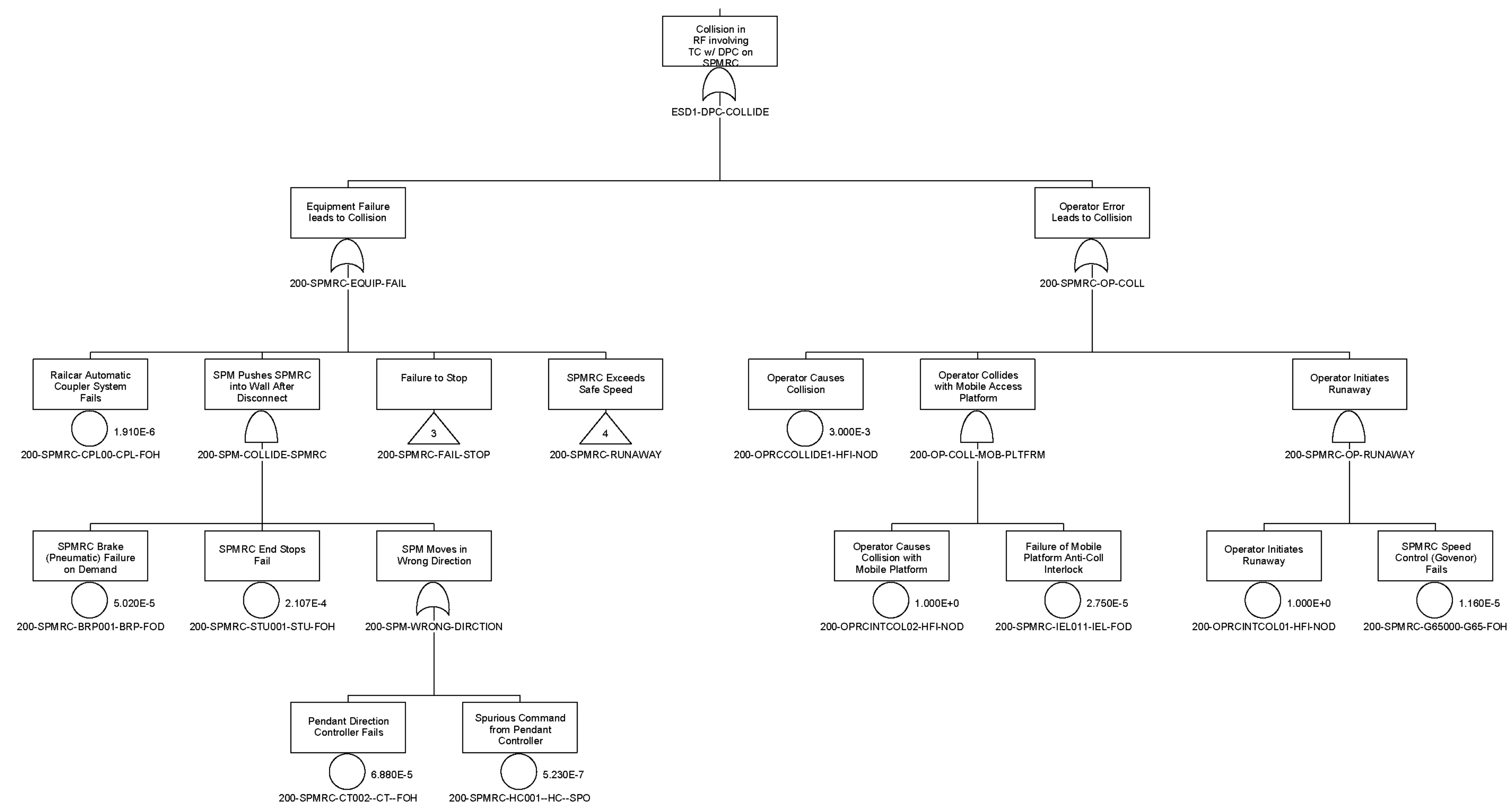
Table B1.4-3. Cut Sets for SPMRC Collides with RF Structures (Continued)

Fault Tree	Cut set %	Prob./Freq.	Basic Event	Description	Probability
			200-SPMRC-HC001--HC--SPO	Spurious Command from Pendant Controller	5.2E-007
			200-SPMRC-STU001-STU-FOH	SPMRC End Stops Fail	2.1E-004
	0.00	1.233E-015	200-SPMRC-CBP001-CBP-OPC	Power Cable to SPMRC - Open Circuit	9.1E-008
			200-SPMRC-MOE000-MOE-FSO	SPMRC Lock Mode State Fails on Loss of Power	1.4E-008
	0.00	1.095E-015	200-SPMRC-CT003-CT-SPO	On-Board Controller Initiates Spurious Signal	2.3E-005
			200-SPMRC-G65000-G65-FOH	SPMRC Speed Control (Governor) Fails	1.2E-005
			200-SPMRC-SEL021-SEL-FOH	Speed Selector on SPMRC Pendant Fails	4.2E-006
	0.00	2.538E-016	200-SPMRC-CBP001-CBP-SHC	SPMRC Power Cable - Short Circuit	1.9E-008
			200-SPMRC-MOE000-MOE-FSO	SPMRC Lock Mode State Fails on Loss of Power	1.4E-008
		4.834E-003	= Total		

NOTE: Freq. = frequency; Prob. = probability; SPMRC = site prime mover railcar.

Source: Original

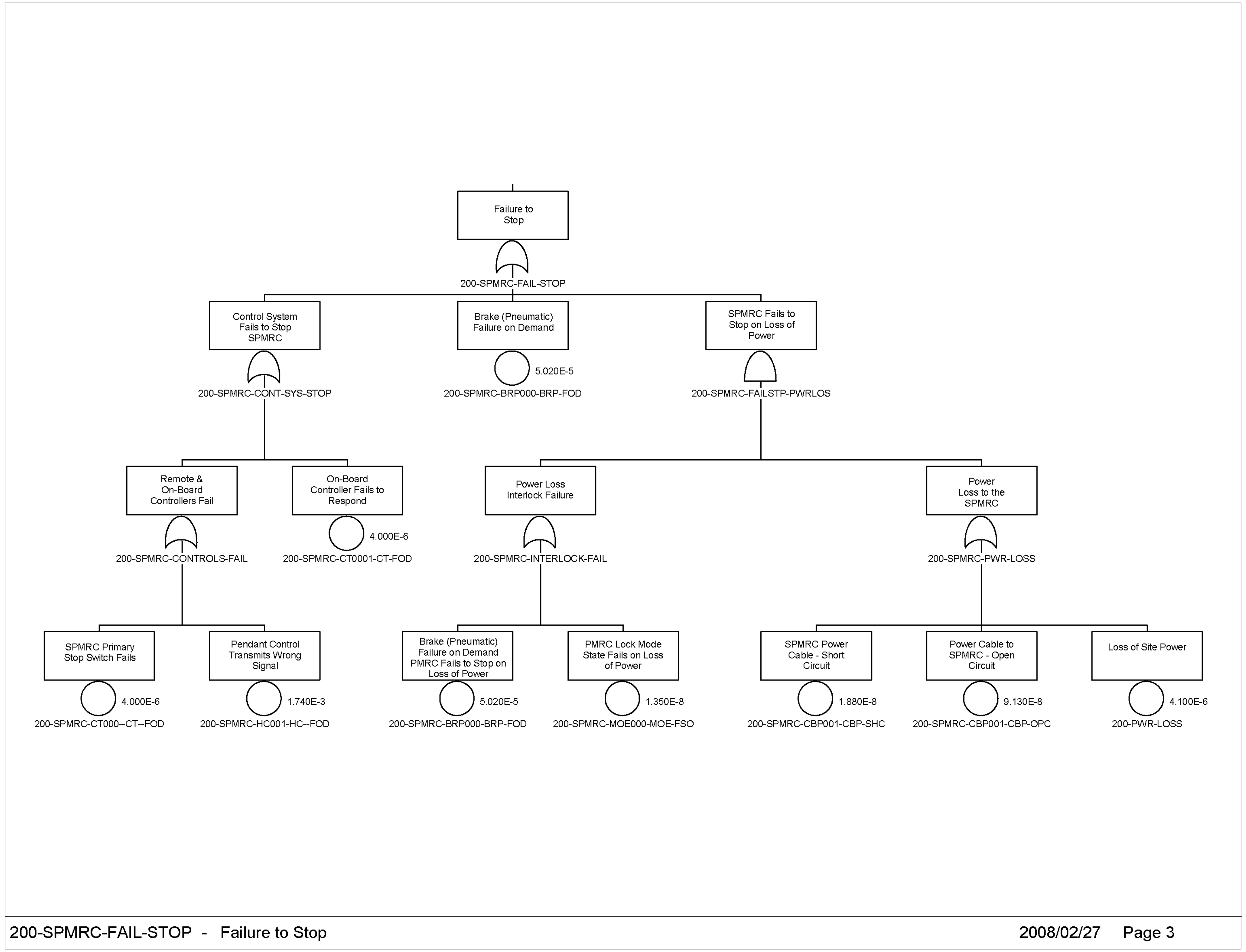
B1.4.1.8 Fault Trees



ESD1-DPC-COLLIDE - PMRC Collision in RF 2008/03/07 Page 1

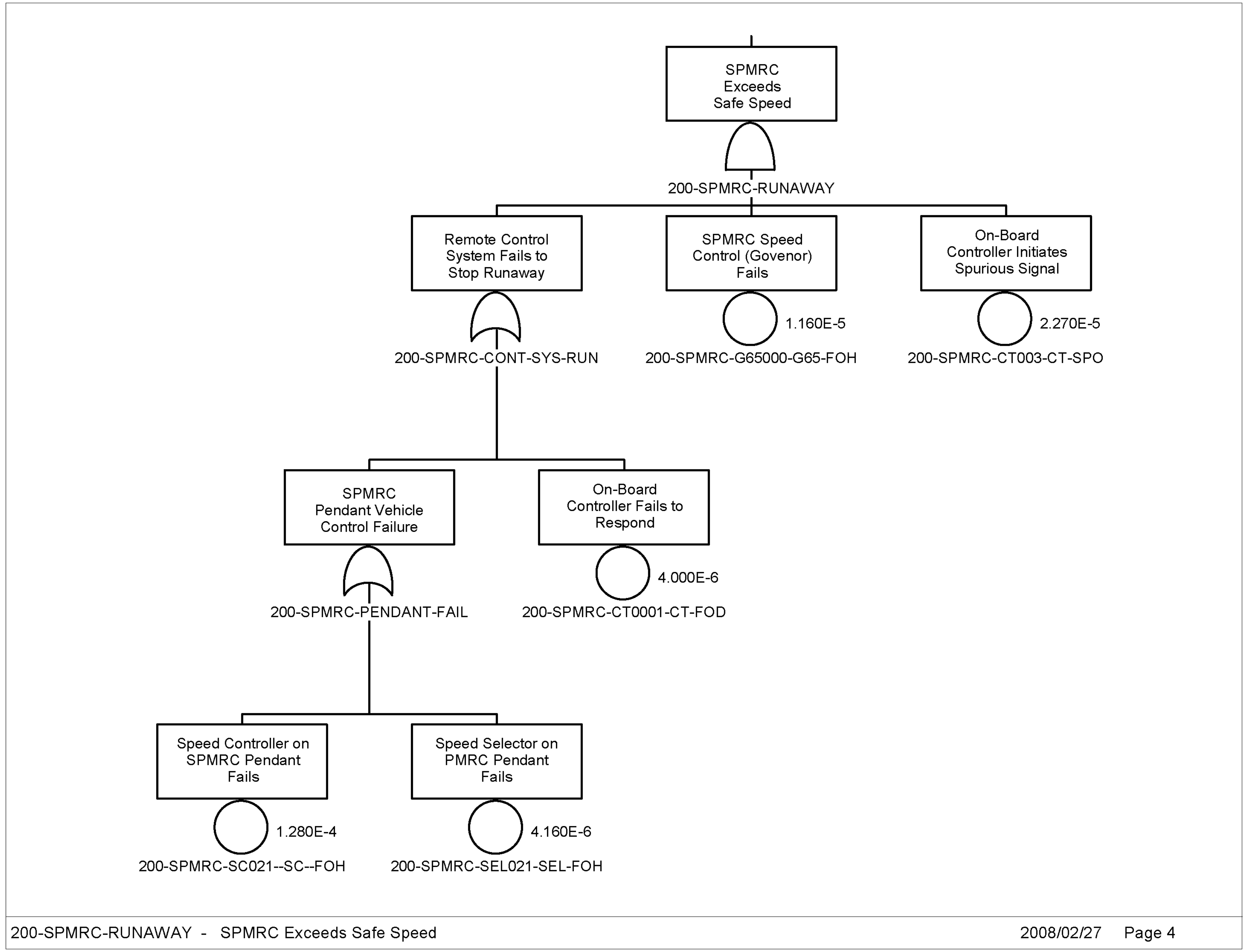
Source: Original

Figure B1.4-3. SPMRC Collision in RF



Source: Original

Figure B1.4-4. SPMRC Fail to Stop



Source: Original

Figure B1.4-5. SPMRC Exceeds Safe Speed

B1.4.2 SPMRC Derailment

B1.4.2.1 Description

The two fault trees for SPMRC derailment within the RF are identical for each type of transportation cask. Derailment is characterized by a basic event that accounts for the probability of a railcar derailment per mile of travel with in the RF.

This fault tree considers the potential for the SPM to derail during movement of the railcar to the preparation area. The top event is "SPMRC Derails Causing Impact to Transportation Cask." This fault tree is shown in Figure B1.4-8.

The probability of derailment is based on historical data for train derailment at low speeds. The probability of derailment per mile is multiplied by the number of miles the SPM travels from the vestibule to the preparation area (approximately 4E-02 miles). Detailed analysis for this basic event is contained in Attachment C.

B1.4.2.2 Success Criteria

The success criterion for this fault tree is that the SPMRC does not derail during the transport process.

B1.4.2.3 Design Requirements and Features

Requirements

- The railcar design requirements comply with AAR Standard S-2043 Performance Specification for Trains Used to Carry High-Level Radioactive Material (Ref. B1.1.1).

Design Feature

- The design features of the railcar are in compliance with AAR Standard S-2043 (Ref. B1.1.1).

Testing and Maintenance

Requirements

- No maintenance or testing is permitted on a railcar loaded with a transportation cask.

Design Feature

- None.

B1.4.2.4 Fault Tree Model

The fault tree model for "SPMRC Derailment Causing a Transportation Cask Impact" consists of the probability for a railcar derailment per mile of travel times the number of occurrences for each type of transportation cask.

B1.4.2.5 Basic Event Data

Table B1.4-4 contains a list of basic events used in the "SPMRC Derailment" fault trees.

Table B1.4-4. Basic Event Probability for SPMRC Derailment

Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-SPMRC- DERIL-PER-MILE	3	1.180E-005	0.000E+000	1.180E-005	1.000E+000
200-SPMRC-MILES-IN- RF	V	4.000E-002	4.000E-002	0.000E+000	0.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. calculation; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B1.4.2.5.1 Human Failure Events

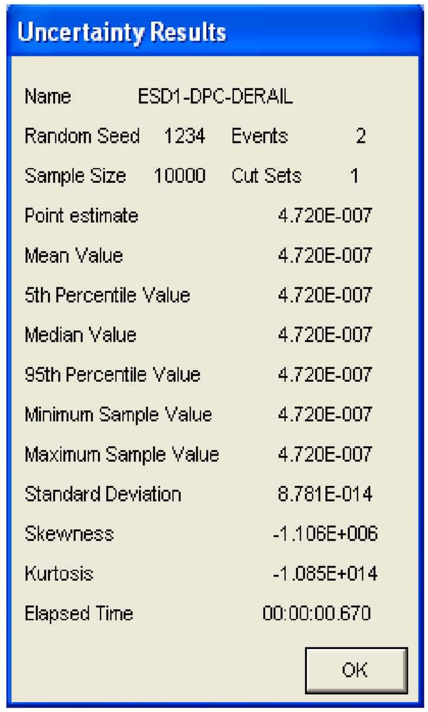
There are no human errors identified for this fault tree.

B1.4.2.5.2 Common-Cause Failures

There are no common-cause failures (CCFs) identified for this fault tree.

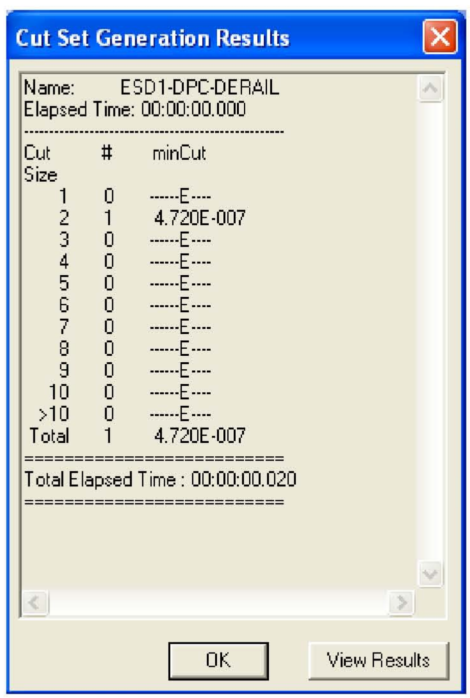
B1.4.2.6 Uncertainty and Cut Set Generation Results

Figure B1.4-6 contains the uncertainty results obtained from running the fault tree for SPMRC derailment. Figure B1.4-7 provides the cut set generation results for the SPMRC derailment fault tree.



Source: Original

Figure B1.4-6. Uncertainty Results of the SPMRC Derailment Fault Tree



Source: Original

Figure B1.4-7. Cut Set Generation Results for the SPMRC Derailment Fault Tree

B1.4.2.7 Cut Sets

Table B1.4-5 contains the cut sets for the "SPMRC Derailment" fault tree. The probability of derailment per cask is 4.720E-007.

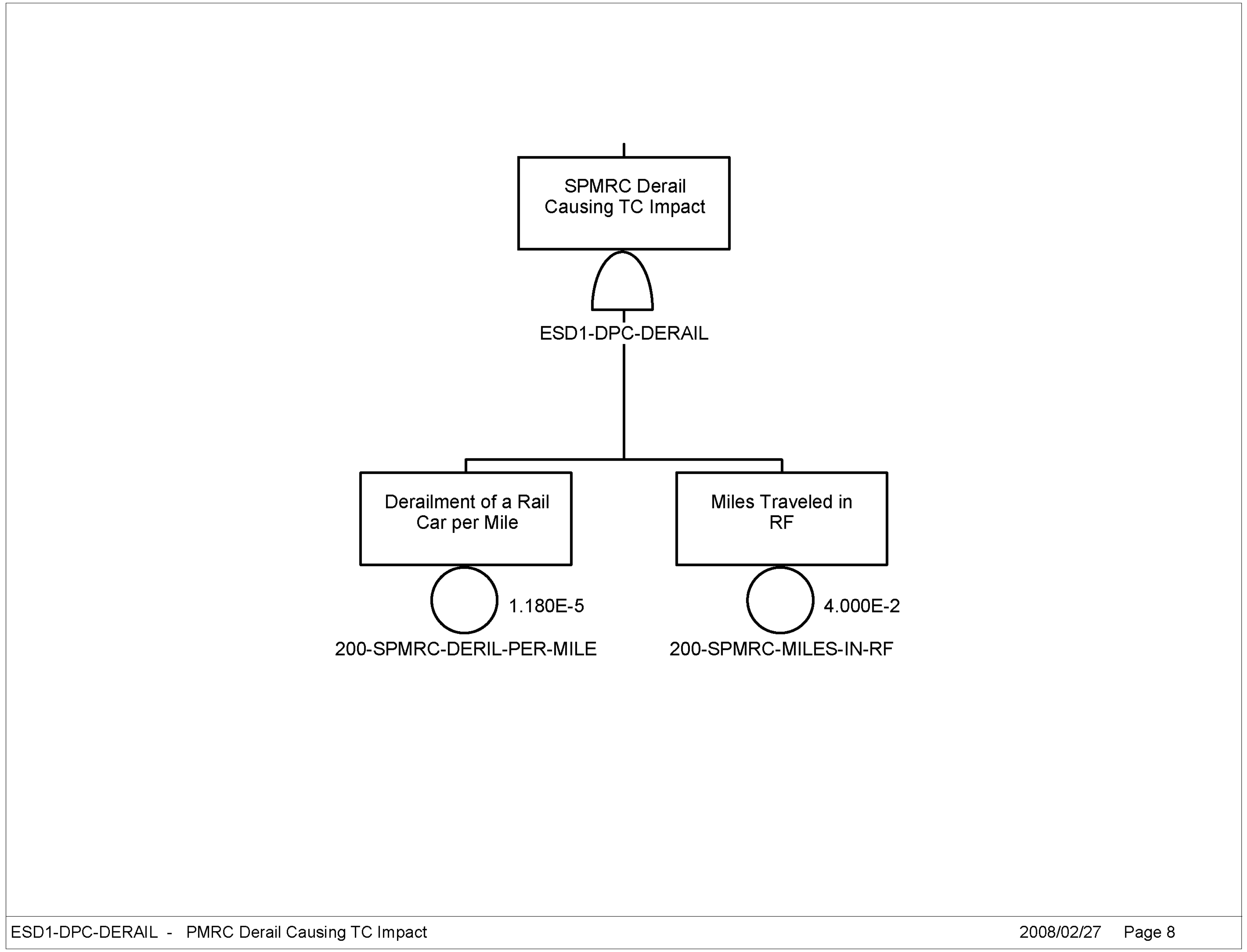
Table B1.4-5. Cut Sets for SPMRC Derailment

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
ESD1-DPC-DERAIL	100.00	4.720E-007	200-SPMRC-DERIL-PER-MILE	Derailment of a railcar per mile	1.2E-005
			200-SPMRC-MILES-IN-RF	Miles traveled in RF	4.0E-002
		4.720E-007	= Total		

NOTE: Freq. = frequency; Prob. = probability.

Source: Original

B1.4.2.8 Fault Trees



Source: Original

Figure B1.4-8. SPMRC Derailment in RF

B2 CASK TRANSFER TROLLEY – FAULT TREES ANALYSIS

B2.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

The inputs in this Section noted with an asterisk (*) indicate that they fall into one of the designated categories described in Section 4.1, relative to suitability for intended use.

B2.1.1 BSC (Bechtel SAIC Company) 2007. *Mechanical Handling Design Report for Cask Transfer Trolley*. 000-30R-HM00-00200-000-001. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071219.0001.

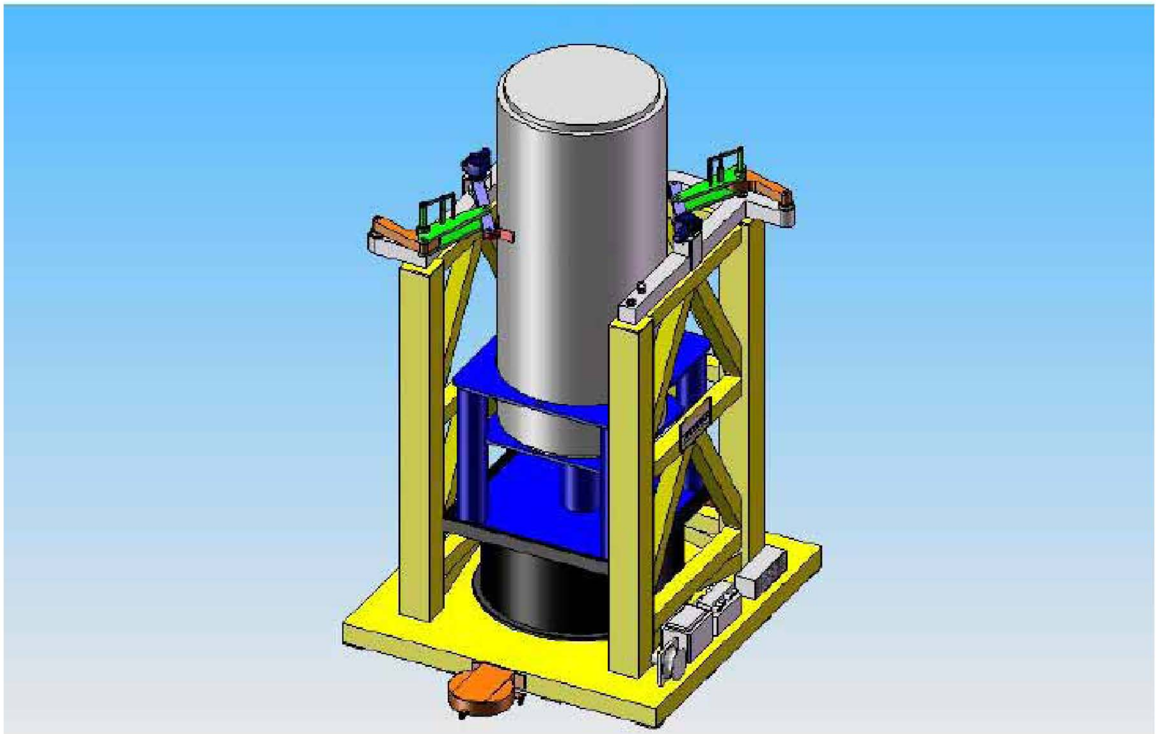
B2.1.2 *BSC 2007. *Preliminary Throughput Study For The Receipt Facility*. 200-30R-RF00-00300-000-002. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071227.0021.

B2.1.3 *Morris Material Handling 2007. *P&ID – Cask Transfer Trolley*. V0-CY05-QHC4-00459-00029-001 Rev. 005. Oak Creek, Wisconsin: Morris Material Handling. ACC: ENG.20071019.0003.

B2.2 CASK TRANSFER TROLLEY DESCRIPTION

B2.2.1 Physical Description

The cask transfer trolley (CTT) is an air powered machine that is used to transport vertically oriented transportation casks from the Cask Preparation Room to the Cask Unloading Room. The trolley consists of a platform, a cask support assembly, a pedestal assembly, a seismic restraint system, and an air system as illustrated in Figure B2.2-1.



Source: Modified from Ref. B2.1.1.

Figure B2.2-1. Cask Transfer Trolley

The platform, or main deck, is the main support structure for the trolley. The structure is designed to hold the air bearings under the deck and simultaneously support the cask support assembly and cask. The cask support assembly is the truss work that is welded to the platform and cradles three sides of the cask. The cask support assembly provides the structural support for the seismic restraint system and pedestal assembly to hold the cask during an earthquake or collision event.

The CTT must handle a number of different types of casks; consequently, different pedestals are used to position the top of the cask at the appropriate height above the floor. Each pedestal sub-component is designed for its respective cask to sit down in a “cavity.” The depth of the cavity is a minimum of 6 in. which is sufficient to prevent the cask from exiting from the pedestal due to uplift during the worst case seismic event. In addition, the cask is restrained in the longitudinal and transverse directions by the cavity walls and restrained in the vertical down direction by the pedestal itself.

This design also ensures the cask is positioned in the correct position in the trolley. The trolley is positioned within a set tolerance under the cask transfer port in the transfer area using bumpers and stops that are bolted to the floor with bolts that shear to allow the CTT to slide during a significant seismic event.

In addition to the cask being restrained at the bottom by the pedestal assembly, the upper section of the cask is restrained to prevent side motions during a seismic event. The system is made up of two linkage systems that are mounted on opposite corners of the cask support assembly. An electric motor extends and retracts the restraint brackets to predetermined positions. Different cask diameters are handled by bolting unique interface clamps onto the seismic restraints.

When the restraint system is properly positioned next to the cask, a locking pin is air-actuated to secure the system. This solid high-strength alloy locking pin can withstand the shear stresses that would be experienced during a seismic event. Both locking pins are monitored by proximity switches (or limit switches) that are hard wired to the control system to verify the pins are in place. If the locking pins are not secured properly, the CTT is not able to power up and move/levitate.

The facility compressed air supply inflates nine 54-in. diameter air casters beneath the trolley platform. Each air caster consists of a urethane torus-shaped bag with a chamber inside the torus. The air film is produced when air is distributed to each air caster causing the air bags to inflate. The inflated bags create a seal against the floor surface and confine the air within the chambers of the bags until the air pressure is sufficient to offset the weight of the loaded trolley. The air bearings allow the CTT to rise above the steel floor approximately 1/2 in. to 7/8 in. The air bearings are supplied with facility air (between 75-100 psi optimal) and consume from 500 to 700 scfm. A hose reel for the 1-1/2-in. diameter air hose is mounted on the platform. The reel is equipped with an air-powered return, a ball valve shut-off, quick-disconnect fittings, and a safety air fuse.

A main "off/on" control valve and separate flow control/monitoring valve for each air bearing allows adjustment and verification of pressure/flow for each individual bearing. There are two interlocks for the air; one pressure monitor verifies the main incoming pressure is not too high, and a second set of monitors verifies that all bearings have sufficient air pressure. This air monitoring system for the air bearings is not important to safety and therefore has not been analyzed.

End mounted turtle-style drive units that are 360-degree steerable, are used to steer the CTT. Traction is produced by down-pressure on the wheels provided by a small air bag on each drive unit. Air is supplied from facility air to a high-speed pneumatic motor in combination with a reducer to limit the wheel speed of the turtle drives. The maximum speed of the system is less than or equal to 10 fpm at the maximum air pressure available from the facility compressed air supply.

The CTT speed is controlled in two ways. First, the electrical control system is designed to provide a control signal to the air valve that produces a speed range of 0-10 fpm. In the event this control system fails, a factory set mechanical throttle valve, in line with each motor drive, restricts the air flow to prevent a "run-away" condition.

B2.2.2 Control System

The control system is relay-based and includes a pendant station for its operator interface.

No programmable logic controller is used–all interlocks are hard wired. The pendant is a standard crane pendant that has all of the controls for the unit including:

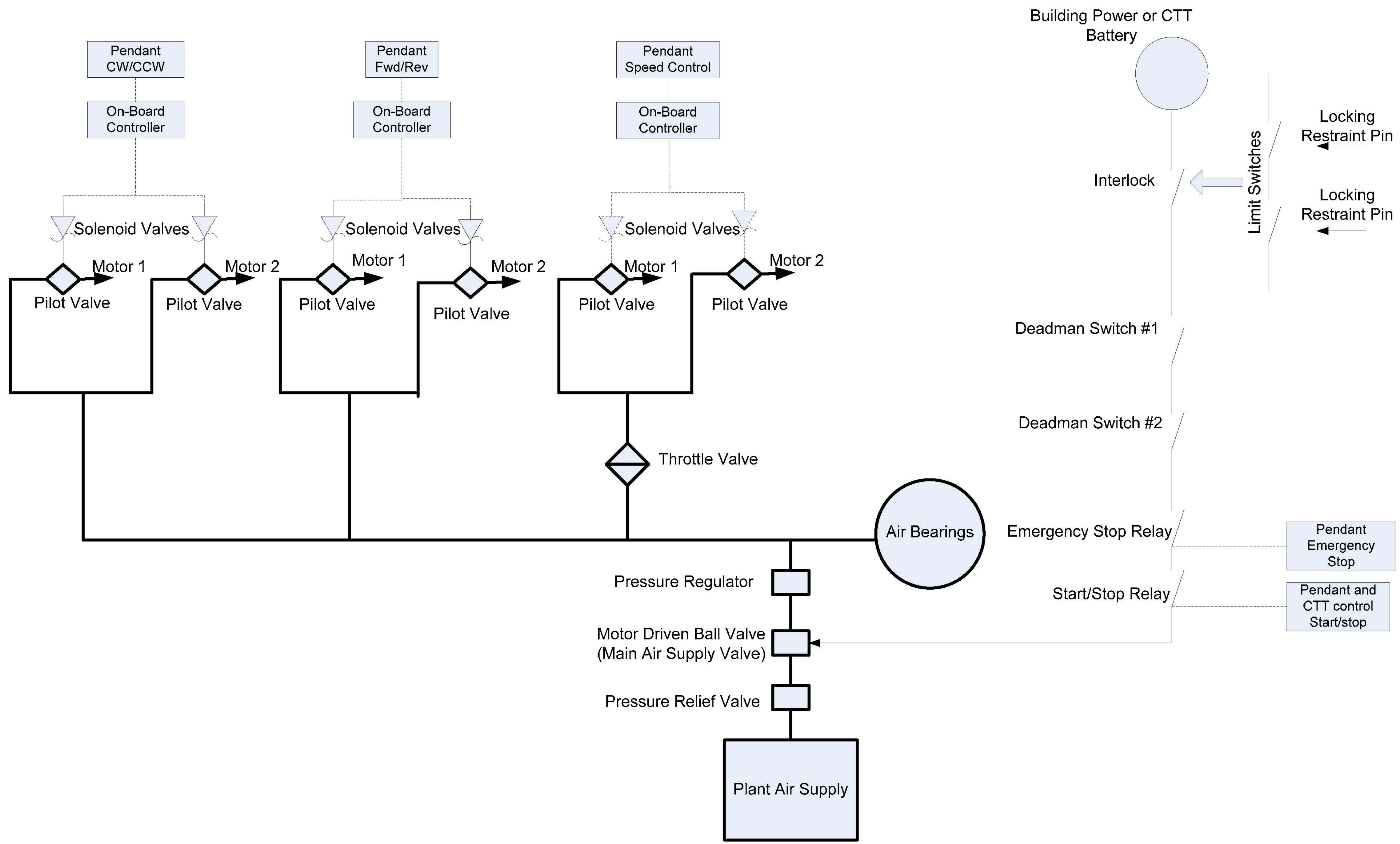
- Deadman handle–The operator presses both handles to allow air to flow to the CTT to levitate and move it horizontally.
- Emergency-stop button–The operator presses the emergency stop button on the pendant control or on the CTT to stop the CTT
- Clockwise/counterclockwise momentary switch– The operator turns this switch to turn the drive units for horizontal movement. This rotational characteristic is used to move the CTT to the storage or maintenance location after it leaves the Cask Preparation Area.
- Forward/reverse switch–The operator uses the forward/reverse switch to determine the direction of the drive units.
- Variable speed control switch–The operator uses the variable speed control switch to adjust the CTT drive speed
- Cask restraint– The operator uses the selector switch to actuate the motor to close the restraints and automatically engage the locking pin.

During normal operations, the controls operate off a battery system contained on the CTT. Only one operator is needed to move the CTT since it only travels in one direction when it is carrying a cask. The CTT moves forward and reverse between the Cask Preparation Room and the Cask Unloading Room and is restrained from side to side by removable barriers that are mounted to the building floor.

A schematic of the control system is shown in Figure B2.2-2.

The main air supply valve is a solenoid operated pilot valve that is fail safe (i.e., it is a spring valve that closes upon loss of electrical power or loss of air pressure). The air supply valve opens when the locking restraint pins actuate the limit switches and the pendant deadman switches are actuated.

The controls on the pendant are clockwise/counterclockwise, forward/reverse, and drive speed to control the valves for the motor drives. These valves are also fail safe solenoid operated pilot valves.



Source: Modified from Ref. B2.1.3.

Figure B2.2-2. Schematic of the CTT Control System

Releasing the deadman switches or pressing the emergency-stop or start/stop buttons on the pendant control or the emergency-stop button on the CTT opens a relay to interrupt power to the main air supply valve, causing it to close. Upon closing the main supply valve the air pressure levitating the CTT and driving the motors is reduced and the CTT lowers to the floor.

B2.2.3 Operation

B2.2.3.1 Initial Conditions

The CTT is initially located in the Cask Preparation Room with the battery fully charged, the seismic restraints retracted, and with no air connected. Based on the next planned cask to be loaded onto the trolley, the corresponding pedestal components are installed into the base and bumpers are bolted onto the seismic restraints and supports. The air hose is then connected to the CTT.

The overhead crane moves a cask onto the pedestal. With the cask still attached to the crane, the operator remotely operates the seismic restraints and secures the cask to the CTT by extending the electric motor driven actuators. When the restraints are in place, the locking pins are pneumatically inserted. With the cask secured to the trolley, the overhead crane is disengaged from the cask.

When the locking pins are inserted properly (thus locking the seismic restraints in place), a pair of proximity switches (limit switches) de-activates the interlock and the main air supply valve can be opened to allow the air bearings and drive motors can be operated. Once all preparations of the cask are complete, the trolley can be moved to the Cask Unloading Room using the pendant controls.

B2.2.3.2 Cask Movement

When all steps are properly completed, air is introduced to the CTT. The operator actuates the air bearings, levitating the CTT with the load. The system continuously and automatically checks the flow and pressure to each air bearing; if a problem is detected, the air supply to all bearings is stopped and the system lowers to the ground.

Once the trolley is raised, the operator drives the CTT into the Cask Unloading Room. By moving forward and reverse, the CTT is driven through the door way. Guides bolted to the floor ensures the CTT can only move forward and back, and in addition, will ensure the CTT is properly positioned directly below the transfer port. Once in position, the air flow to the bearings is stopped and the CTT lowers to the ground and rests in position. The operator disconnects the quick-disconnect air hose and rewinds the hose onto the trolley. The shield doors that separate the Cask Preparation Room from the Cask Unloading Room are then closed.

B2.2.3.3 System/Pivotal Event Success Criteria

Success criteria for loading a cask onto the CTT in the Cask Preparation Room, and unloading the canisters from the cask in the Cask Unloading Room require the CTT remain stationary during these operations with no spurious movement. Success criteria for moving the CTT with a cask from the Cask Preparation Room to the Cask Unloading Room requires the CTT to travel at an allowable speed, and the operator is able to control the CTT movement.

During cask loading at the Cask Preparation Room, compressed air must be available to the CTT to remotely insert the locking pins into the restraint system. Both pin interlocks must function before the main air supply valve can be opened thereby preventing movement of the CTT until the cask has been loaded and restrained. Once the locking pins are in place the crane is removed from the cask. During the time the crane is being removed from the cask, the air supply valve is closed and the valves that control the air to the air bags and motors are closed. Movement is not initiated until both deadman switches on the remote pendant control are pressed to allow air to the air bags to levitate the CTT.

Upon the CTT reaching the Cask Unloading Room, procedures require that the air supply hose to be disconnected and removed from the CTT to prevent any movement while unloading the canisters from the cask. This is accomplished by locating the air supply outside the Cask Unloading Room. An interlock prevents the transfer port slide gate from opening until the shield door to the Cask Unloading Room is closed. Thus, because the air supply is external to the Cask Unloading Room, the air hose must be removed from the CTT before the shield door can be closed, and the shield door must be closed before the port slide gate can be opened, allowing canister transfer from the cask. Therefore, the location of the air supply and the shield door interlock requires removal of the air supply from the CTT before canister transfer can begin.

When moving the cask between the Cask Preparation Room and the Cask Unloading Room, movement in the wrong direction is prevented by the guide rails bolted to the floor along the path of the CTT. This forces the CTT to move only in a straight line forward and back between the two areas. Runaway of the CTT is prevented by the throttle valve which is set at the factory such that the maximum speed is 10 fpm at the maximum facility air pressure.

The CTT is stopped to prevent a collision into a closed shield door or the end stops in the Cask Unloading Room by the operator speed controls on the pendant, by the deadman switches on the pendant, or by the emergency stop buttons on the pendant and on the CTT. The speed controls slow down and stop the CTT by controlling the air flow through the drive speed valve, and the deadman switches and emergency stop buttons remove power to the main air supply valve causing it to close. Because the emergency stop function is a recovery action performed by the operator and requires operator intervention, these functions were not modeled in the analysis.

On loss of electrical power from the battery, the air valves all fail closed, and no air will pass through to the air bearings or drive units and the CTT settles to the floor. If the air pressure and flow is lost, the unit can not levitate or move horizontally and the CTT lowers to the floor and no other action occurs. A separate sustained signal is needed to actuate the air valves to raise the load (positive operator action). Thus, although a spurious signal may cause air to flow momentarily, additional operator controls are needed to cause the unit to levitate or move horizontally.

B2.3 DEPENDENCIES AND INTERACTIONS ANALYSIS

Dependencies are broken down into five categories with respect to their interactions with systems, structures, and components. The five areas considered are addressed in Table B2.3-1 with the following dependencies:

1. Functional dependence
2. Environmental dependence
3. Spatial dependence
4. Human dependence
5. Failures based on external events.

Table B2.3-1. Dependencies and Interactions Analysis

Systems, Structures, Components	Dependencies and Interactions				
	Functional	Environmental	Spatial	Human	External Events
Air supply	Provides levitation and motive force	—	—	Fail to disconnect air hose	—
Locking pin limit switches	Prevents spurious movement	—	—	—	—
Guide rails	Prevents movement in wrong direction	—	—	—	Shear during seismic event allows CTT to slide
Pendant control	Controls direction and speed and initiates movement	—	—	Wrong instructions	—
Deadman switch	Allows operation	—	—	Fail to release	—
Emergency stop	Stops CTT	—	—	Fail to energize	—
Throttle valve	Limits maximum speed	—	—	—	—
Structure	Constrains and supports cask	—	—	—	Seismic causes impact
Shield door	Opens for CTT to pass through	—	—	Close door inadvertently	Closes on CTT

NOTE: CTT = cask transfer trolley

Source: Original

B2.4 CTT-RELATED FAILURE SCENARIOS

There are four fault trees associated with the CTT:

1. Spurious movement of the CTT in the Cask Preparation Room during cask loading.

2 Spurious movement of the CTT in the Cask Preparation Room during cask preparation.

3. Collision of the CTT during cask transfer.

4. Spurious movement of the CTT in the Cask Unloading Room.

An additional fault tree involving the CTT is closing of the shield door on the CTT as the CTT moves a cask from the Cask Preparation Room to the Cask Unloading Room. This fault tree is described in a separate section involving inadvertent shield door closure that satisfies ESD-06, pivotal event "Collision with Cask Unloading Room Shield Door."

In all cases a conservative mission time of one hour per cask transfer was used for each fault tree. The time required to move a cask to the trolley and disconnect the crane is approximately 55 minutes, while the time required moving the trolley from the Cask Preparation Room to the Cask Unloading Room is approximately 15 minutes. The time required to extract the canister from the cask is approximately 20 minutes (Ref. B2.1.2). Therefore, a one-hour mission time is considered a conservative value.

B2.4.1 Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading

B2.4.1.1 Description

This fault tree describes spurious movement of the CTT during cask loading to satisfy ESD-02, pivotal event "Unplanned Conveyance Movement Causes Drop." The top event is "Spurious Movement of the CTT during Cask Loading" which is defined as unplanned movement of the CTT while the cask is being loaded onto the CTT. This fault tree is shown in Figures B2.4-3 and B2.4-4.

Spurious movement can be caused by equipment failures or by a combination of equipment failure and operator error. For equipment failures to cause spurious movement the main air supply valve must open to supply air to the air bags to levitate the CTT. This can occur if the main air supply valve fails open or the locking pin limit switches and control system fail causing the valve to open. For the operator to initiate spurious movement, the locking pin limit switches must fail allowing the operator to open the main air supply valve.

B2.4.1.2 Success Criteria

The success criterion is that the CTT remains motionless during loading of the transportation cask. Movement of the CTT during this operation could cause impact and damage to the transportation cask.

B2.4.1.3 Design Requirements and Features

Requirements

There are no additional design requirements.

Features

The design feature is the two locking restraint pins that prevent power to the main air supply valve until the pins are in place and the limit switches are activated to allow power to the air supply valve.

B2.4.1.4 Fault Tree Model

The top event is "Spurious Movement of the CTT during Cask Loading in the Cask Preparation Room" (Figure B2.4-3). This can occur if the control system initiates a spurious signal and both of the pin limit switches fail, or the operator initiates a command to move the CTT and both of the pin limit switches fail. A third failure mode is the mechanical failure of the main supply valve in conjunction with a spurious signal from the control system to initiate movement or failures of the control valves or the valve to the air bags.

A conservative mission time for this operation has been set at one hour.

B2.4.1.5 Basic Event Data

Table B2.4-1 contains a list of basic events used in the fault trees (Figure B2.4-3 and B2.4-4) for spurious movement of the CTT in the preparation area during cask loading.

Table B2.4-1. Basic Event Probabilities for Spurious Movement of the CTT during Cask Loading

Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200--CTT--SV401--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT--CT001---CT--SPO	3	2.270E-005	0.000E+000	2.270E-005	1.000E+000
200-CTT--HC001---HC--SPO	3	5.230E-007	0.000E+000	5.230E-007	1.000E+000
200-CTT--SV301---SV--SPO	3	4.090E-007	0.000E+000	4.090E-007	1.000E+000
200-CTT--ZS301---ZS--FOD	1	2.930E-004	2.930E-004	0.000E+000	0.000E+000
200-CTT--ZS302---ZS--FOD	1	2.930E-004	2.930E-004	0.000E+000	0.000E+000
200-CTT-FWDREVM1-SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-FWDREVM2-SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-SVROTM1--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-SVROTM2--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-OPSPURMOVE01-HFI-NOD	1	1.000E-004	1.000E-004	0.000E+000	0.000E+000
200-CTT-ZS301-SW-CCF	1	1.380E-005	1.380E-005	0.000E+000	0.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B2.4.1.5.1 Human Failure Events

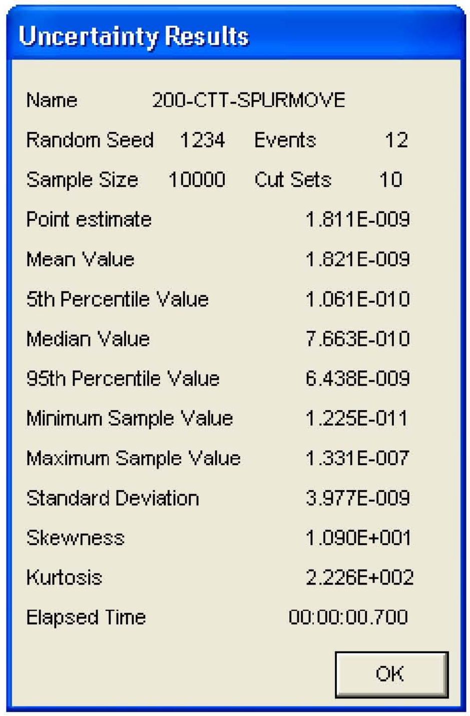
One operator error involves initiation of spurious movement. The operator error is 200-OPSPURMOVE01-HFI-NOD.

B2.4.1.5.2 Common-Cause Failures

One CCF was added to the fault tree to account for the failure of both restraint pin limit switches. An alpha factor of 0.047 was used to determine the common-cause value using two of two as the failure criteria (Table C3-1, CCCG = 2). The CCF is 200-CTT-ZS301-SW-CCF.

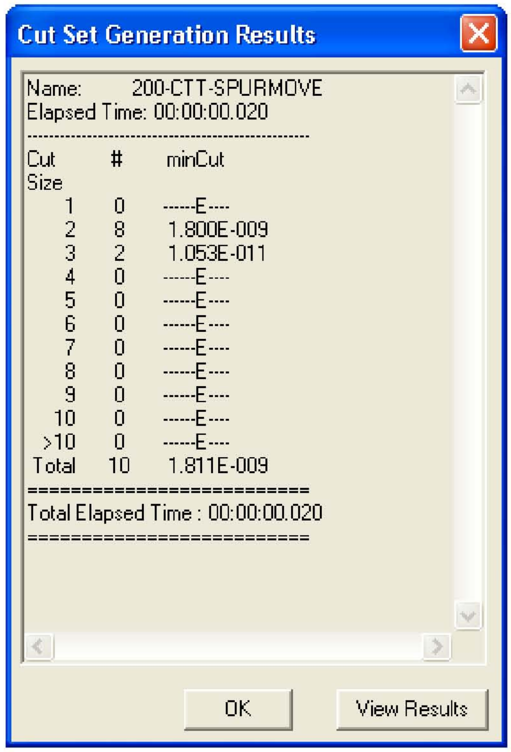
B2.4.1.6 Uncertainty and Cut Set Generation Results

Figure B2.4-1 contains the uncertainty results obtained from running the fault tree for the "Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading" fault tree. Figure B2.4-2 provides the cut set generation results for the "Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading."



Source: Original

Figure B2.4-1. Uncertainty Results of the Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading Fault Tree



Source: Original

Figure B2.4-2. Cut Set Generation Results for the Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading Fault Tree

B2.4.1.7 Cut Sets

Table B2.4-2 contains the cut sets for the "Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading" fault tree. The total probability per cask loading is 1.81E-009.

Table B2.4-2. Cut Sets for Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
200-CTT-SPURMOVE	76.22	1.380E-009	200-CTT-ZS301-SW-CCF	Common Cause Failure of Limit Switches	1.380E-005
			200-OPSPURMOVE01-HFI-NOD	Operator Initiates Spurious Movement	1.000E-004
	17.30	3.133E-010	200-CTT--CT001---CT--SPO	On-Board Controller Initiates Spurious Signal	2.270E-005
			200-CTT-ZS301-SW-CCF	Common Cause Failure of Limit Switches	1.380E-005
	1.10	1.992E-011	200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
			200-CTT-SVROTM1-SV-FOH	Failure of Solenoid Valve Providing Rotation to Motor	4.870E-005

Table B2.4-2. Cut Sets for Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading (Continued)

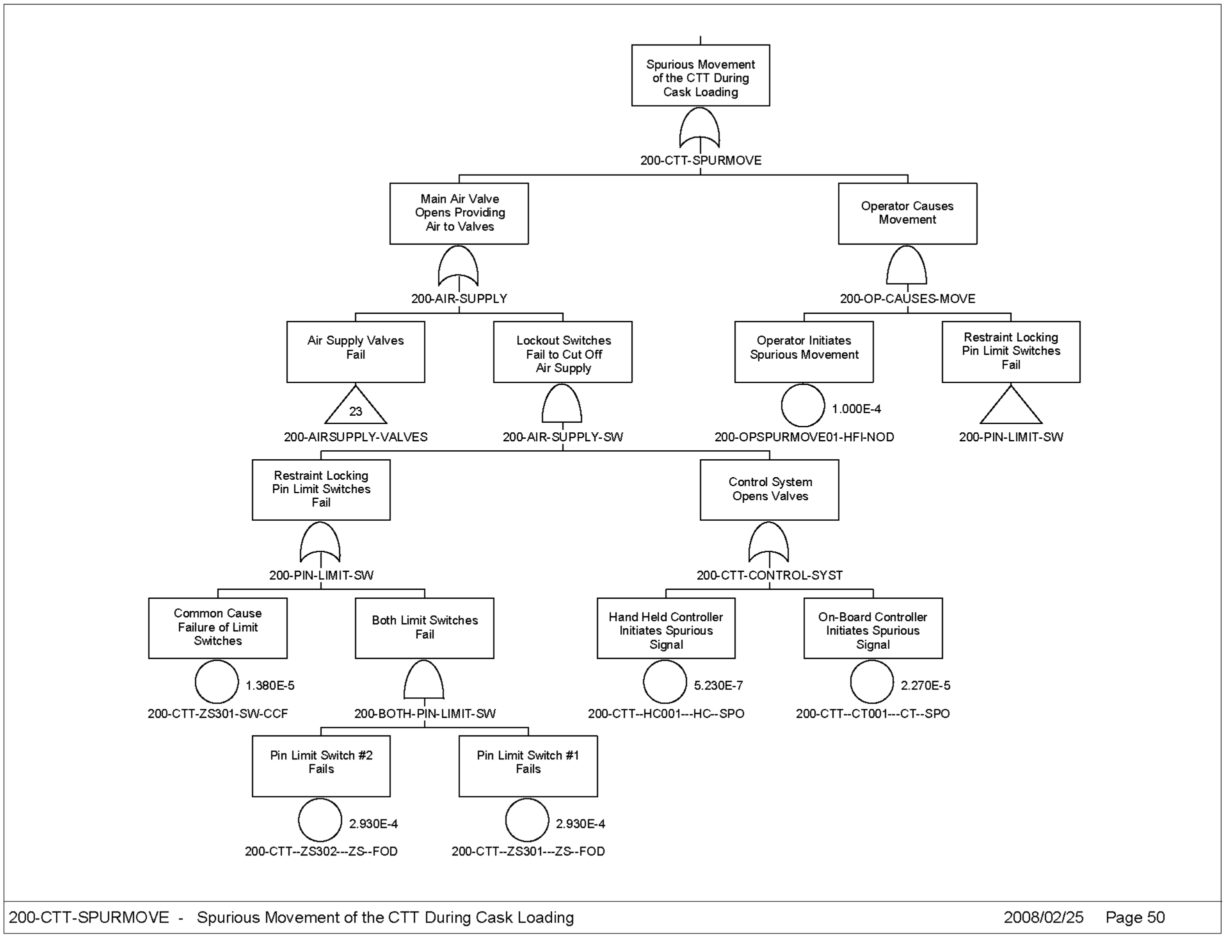
Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
				1	
	1.10	1.992E-011	200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
			200-CTT-SV401-SV-FOH	Failure of Air Supply Solenoid Valve for Air Bags	4.870E-005
	1.10	1.992E-011	200-CTT-FWDREVM2-SV-FOH	Failure of Solenoid Valve Providing Fwd/Rev to Motor 2	4.870E-005
			200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
	1.10	1.992E-011	200-CTT-FWDREVM1-SV-FOH	Failure of Solenoid Valve Providing Fwd/Rev to Motor 1	4.870E-005
			200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
	1.10	1.992E-011	200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
			200-CTT-SVROTM2-SV-FOH	Failure of Solenoid Valve Providing Rotation to Motor 2	4.870E-005
	0.47	8.585E-012	200-CTT--ZS301---ZS--FOD	Pin Limit Switch #1 Fails	2.930E-004
			200-CTT--ZS302---ZS--FOD	Pin Limit Switch #2 Fails	2.930E-004
			200-OPSPURMOVE01-HFI-NOD	Operator Initiates Spurious Movement	1.000E-004
	0.40	7.217E-012	200-CTT--HC001---HC--SPO	Hand Held Controller Initiates Spurious Signal	5.230E-007
			200-CTT-ZS301-SW-CCF	Common Cause Failure of Limit Switches	1.380E-005
	0.11	1.949E-012	200-CTT--CT001---CT--SPO	On-Board Controller Initiates Spurious Signal	2.270E-005
			200-CTT--ZS301---ZS--FOD	Pin Limit Switch #1 Fails	2.930E-004
			200-CTT--ZS302---ZS--FOD	Pin Limit Switch #2 Fails	2.930E-004
	0.00	4.490E-014	200-CTT--HC001---HC--SPO	Hand Held Controller Initiates Spurious Signal	5.230E-007
			200-CTT--ZS301---ZS--FOD	Pin Limit Switch #1 Fails	2.930E-004
			200-CTT--ZS302---ZS--FOD	Pin Limit Switch #2 Fails	2.930E-004
Total	1.811E-009				

NOTE: Prob. = probability

Source: Original

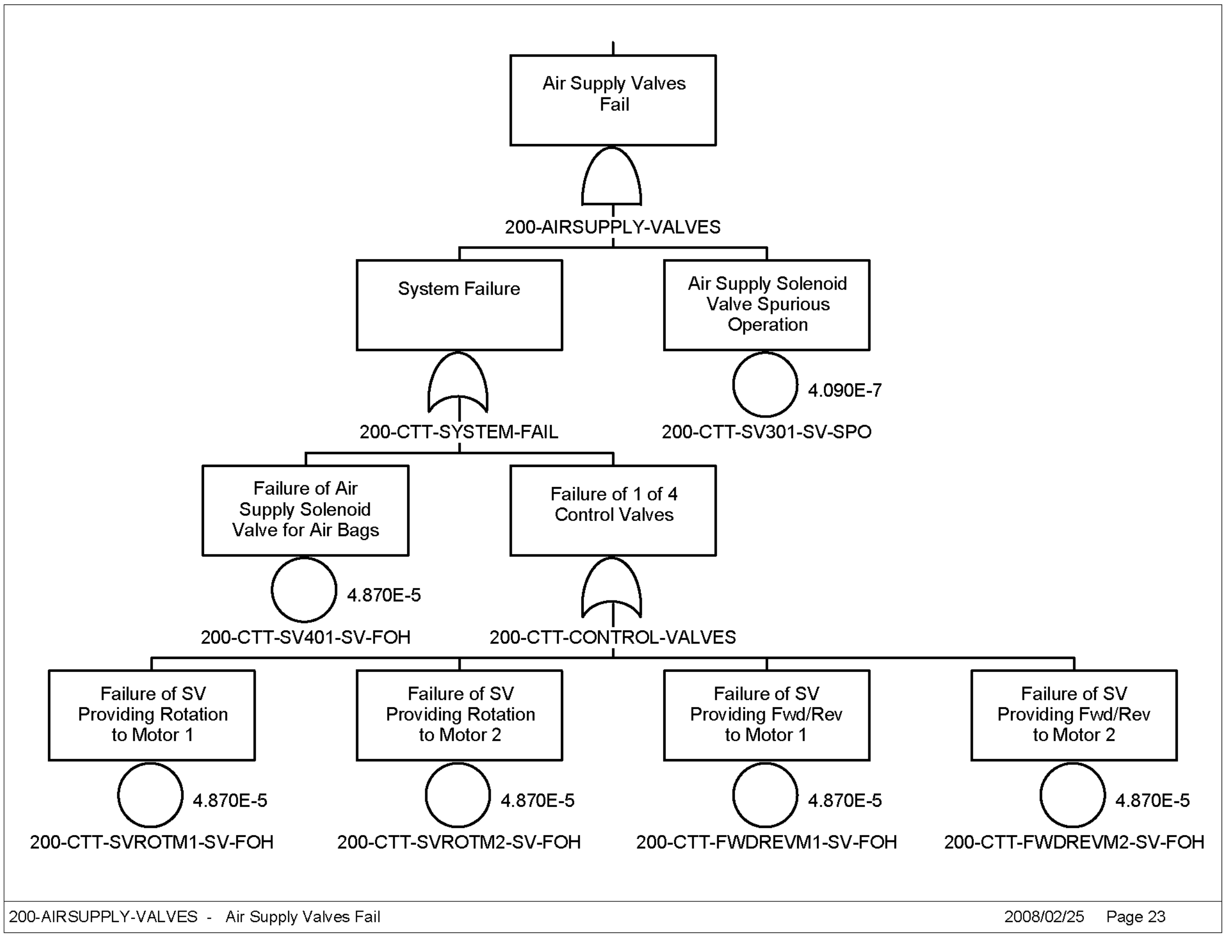
B2.4.1.8 Fault Trees

The fault trees for spurious movement of the CTT during Cask Loading are shown in Figures B2.4-3 and B2.4-4.



Source: Original

Figure B2.4-3. Fault Tree for Spurious Movement of the CTT in the Cask Preparation Room during Cask Loading



Source: Original

Figure B2.4-4. Fault Tree for Air Supply Valves Fail

B2.4.2 Spurious Movement of the CTT in the Preparation Area during Cask Preparation

B2.4.2.1 Description

This fault tree describes spurious movement of the CTT during cask preparation to satisfy ESD-03 pivotal event, "Side Impact to Transportation Cask." The top event is "Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation" which is defined as unplanned movement of the CTT while the cask is being prepared for movement to the Cask Unloading Room by unbolting the lid and installing the lid adapter. This fault tree is shown in Figure B2.4-7.

During this operation, the locking pins have been installed and the limit switches are closed. Spurious movement can be caused by multiple equipment failures or by operator error. For equipment failures to cause spurious movement the main air supply valve must open to supply air to the air bags to levitate the CTT. This can occur through failure of the main air supply valve coupled with spurious commands from the control system or failure of the control valves.

Alternatively, the operator can initiate spurious movement since at this stage of the operation there are no preventive interlocks.

B2.4.2.2 Success Criteria

The success criterion is that the CTT remain motionless during cask preparation. Movement of the CTT during this operation could cause impact to occur resulting in damage to the transportation cask.

B2.4.2.3 Design Requirements and Features

There are no design requirements or features for this operation.

B2.4.2.4 Fault Tree Model

The top event in this fault tree is "Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation" (Figure B2.4-7). This can occur through spurious signals from the control system, spurious operation of the main air supply valve, failure of the control valves, or operator error initiating CTT movement.

B2.4.2.5 Basic Event Data

Table B2.4-3 contains a list of basic events used in the fault tree (Figure B2.4-7) for the "Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation."

Table B2.4-3. Basic Event Probabilities for Spurious movement of the CTT in the Cask Preparation Room during Cask Preparation

Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTT--CT001---CT--SPO	3	2.270E-005	0.000E+000	2.270E-005	1.000E+000
200-CTT--HC001---HC--SPO	3	5.230E-007	0.000E+000	5.230E-007	1.000E+000
200-OPSPURMOVE01-HFI-NOD	1	1.000E-004	1.000E-004	0.000E+000	0.000E+000
200--CTT--SV401--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT--SV301---SV--SPO	3	4.090E-007	0.000E+000	4.090E-007	1.000E+000
200-CTT-FWDREVM1-SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-FWDREVM2-SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-SVROTM1--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-SVROTM2--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B2.4.2.5.1 Human Failure Events

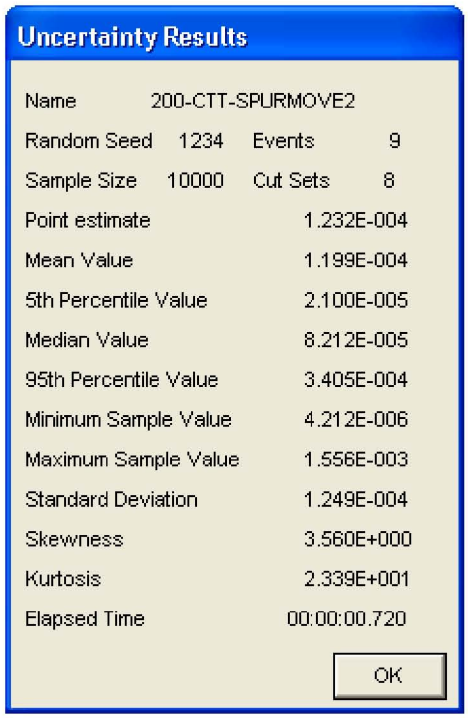
One operator error (200-OPSPURMOVE01-HFI-NOD) involves initiation of spurious movement.

B2.4.2.5.2 Common-Cause Failures

There are no CCFs associated with this fault tree.

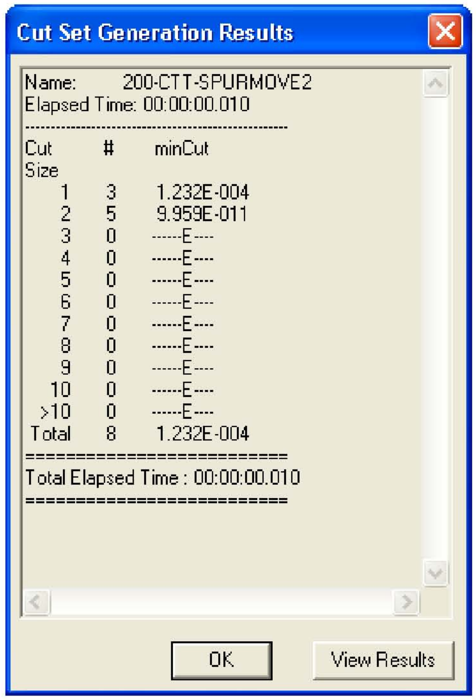
B2.4.2.6 Uncertainty and Cut Set Generation Results

Figure B2.4-5 contains the uncertainty results obtained from running the fault tree for spurious movement of the CTT in “Spurious movement of the CTT in the Cask Preparation Room during Cask Preparation.” Figure B2.4-6 provides the cut set generation results for “Spurious movement of the CTT in the Cask Preparation Room during Cask Preparation.”



Source: Original

Figure B2.4-5. Uncertainty Results of the Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation



Source: Original

Figure B2.4-6. Cut Set Generation Results for Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation

B2.4.2.7 Cut Sets

Table B2.4-4 contains the cut sets for the "Spurious movement of the CTT in the Cask Preparation Room during Cask Preparation" fault tree. The total probability per cask is 1.23E-004 with operator initiation of spurious movement the dominant cause of movement during cask preparation.

Table B2.4-4. Cut Sets for Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
200-CTT-SPURMOVE2	81.16	1.000E-004	200-OPSPURMOVE01-HFI-NOD	Operator Initiates Spurious Movement	1.000E-004
	18.42	2.270E-005	200-CTT--CT001---CT--SPO	On-Board Controller Initiates Spurious Signal	2.270E-005
	0.42	5.230E-007	200-CTT--HC001---HC--SPO	Hand Held Controller Initiates Spurious Signal	5.230E-007
	0.00	1.992E-011	200-CTT-FWDREVM2-SV-FOH	Failure of Solenoid Valve Providing Fwd/Rev to Motor 2	4.870E-005
			200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007

Table B2.4-4. Cut Sets for Spurious Movement of the CTT in the Cask Preparation Room during Cask Preparation (Continued)

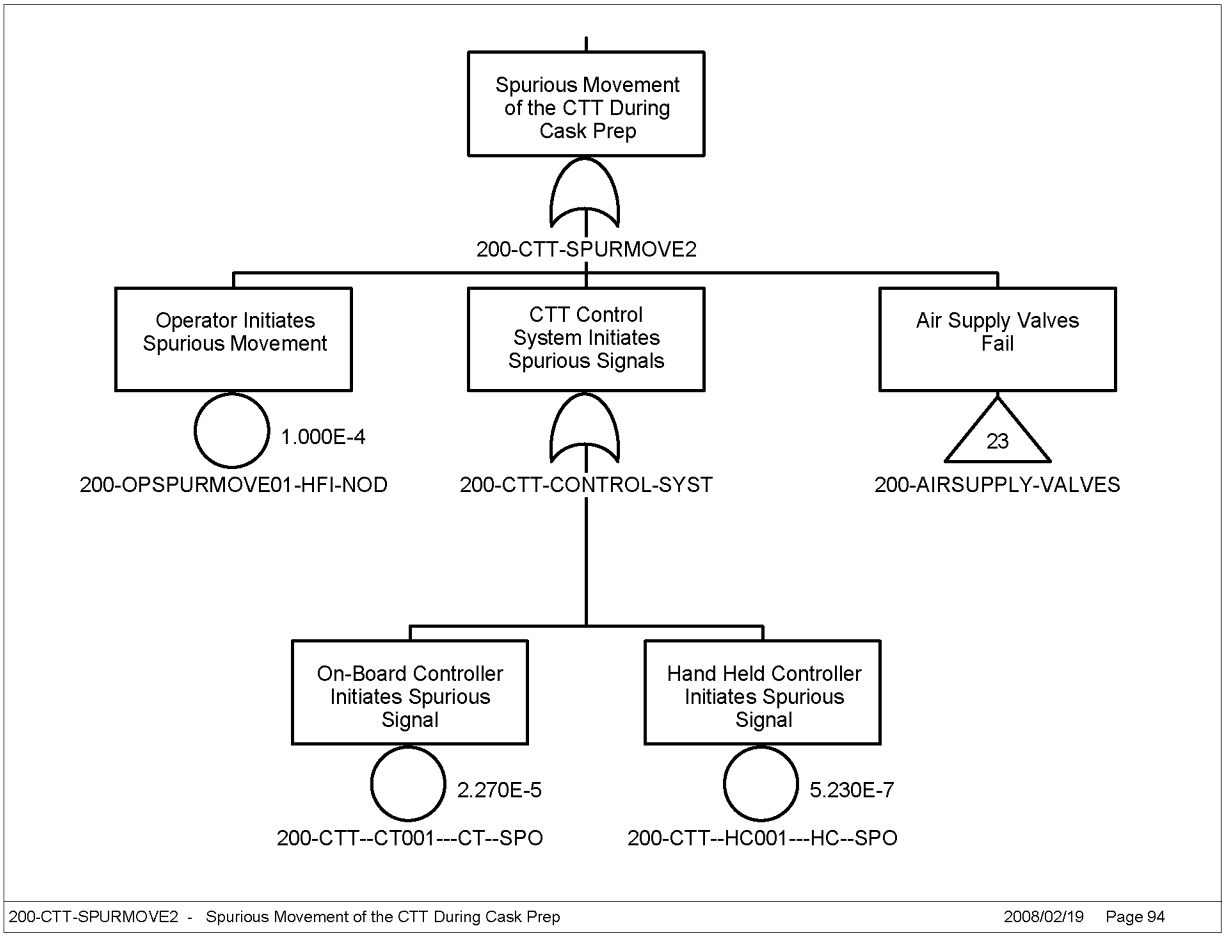
Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
	0.00	1.992E-011	200-CTT-FWDREVM1-SV-FOH	Failure of Solenoid Valve Providing Fwd/Rev to Motor 1	4.870E-005
			200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
	0.00	1.992E-011	200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
			200-CTT-SV401-SV-FOH	Failure of Air Supply Solenoid Valve for Air Bags	4.870E-005
	0.00	1.992E-011	200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
			200-CTT-SVROTM1-SV-FOH	Failure of Solenoid Valve Providing Rotation to Motor 1	4.870E-005
	0.00	1.992E-011	200-CTT-SV301-SV-SPO	Air Supply Solenoid Valve Spurious Operation	4.090E-007
			200-CTT-SVROTM2-SV-FOH	Failure of Solenoid Valve Providing Rotation to Motor 2	4.870E-005
Total	1.232E-004				

NOTE: Prob. = probability

Source: Original

B2.4.2.8 Fault Trees

The fault trees for "Spurious Movement of the CTT in the Cask Preparation Area During Cask Preparation" is shown in Figures B2.4-7. Note that the transfer gate 23 in Figure B2.4-7 refers to the fault tree in Figure B2.4-4.



Source: Original

Figure B2.4-7. Fault Tree for Spurious Movement of the CTT During Cask Preparation

B2.4.3 Collision of CTT during Cask Transfer

B2.4.3.1 Description

This fault tree considers the potential for the CTT to collide into a structure or object while moving a cask from the preparation area to the transfer area to satisfy ESD-04, pivotal event "CTT collision with Another Vehicle, Facility Structure, or Equipment." The top event is "CTT Collision into Structure." This fault tree is shown in Figures B2.4-10 and B2.4-11.

Two primary causes of a collision are operator initiated (possibly through inattention) or failure of the CTT to stop. Movement in the wrong direction as a contributing factor is negated by the use of guide rails forcing the CTT to only move forward and backward. A runaway condition is prevented by the control system, designed to give a proportional signal to the air valve that produces a speed range of only 0-10 fpm, and an in-line factory set mechanical throttle valve that limits the speed to 10 fpm in the event the control system fails. In the event both of these devices fail, the stop functions must also fail. Since all three functions must fail for a runaway condition, the primary events leading to a collision are operator error or failure to stop.

Failure to stop the CTT requires that failure of the normal stop function, deadman switches, and the air supply valve all fail to close on demand. The emergency stop buttons, one on the pendant and one on the CTT, must also fail; however, because these are recovery actions to be taken by the operator, the emergency stop functions are not credited in the fault tree.

B2.4.3.2 Success Criteria

The success criterion for this event is that the CTT does not experience a collision with any object, including the shield door, during transfer of a cask from the Cask Preparation Room to the Cask Unloading Room. A collision of the CTT could cause damage to the transportation cask.

B2.4.3.3 Design Requirements and Features

The design feature is the deadman switches on the pendant control that must be pressed for air to be supplied to the CTT to provide motive power. There are no requirements for this operation.

B2.4.3.4 Fault Tree Model

The top event of the fault tree is a collision of the CTT into an object or structure during transfer of a cask from the Cask Preparation Room to the Cask Unloading Room (Figures B2.4-10 and B2.4-11). This may occur through operator error or equipment failure of the normal or emergency stop functions. A conservative mission time for this operation has been set at one hour.

B2.4.3.5 Basic Event Data

Table B2.4-5 contains a list of basic events used in the "Collision of CTT during Cask Transfer" fault tree (Figures B2.4-10 and B2.4-11).

Table B2.4-5. Basic Event Probability for Collision of CTT during Cask Transfer

Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTT--DSW000--ESC-CCF	1	1.180E-005	1.180E-005	0.000E+000	0.000E+000
200-CTT--DSW001--ESC-FOD	1	2.500E-004	2.500E-004	0.000E+000	0.000E+000
200-CTT--DSW002--ESC-FOD	1	2.500E-004	2.500E-004	0.000E+000	0.000E+000
200-CTT--HC021---HC--FOD	1	1.740E-003	1.740E-003	0.000E+000	0.000E+000
200-CTT--SV601---SV--FOD	1	6.280E-004	6.280E-004	0.000E+000	0.000E+000
200-CTT--SV602---SV--FOD	1	6.280E-004	6.280E-004	0.000E+000	0.000E+000
200-OPCTTCOLLID2-HFI-NOD	1	1.000E-003	1.000E-003	0.000E+000	0.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B2.4.3.5.1 Human Failure Events

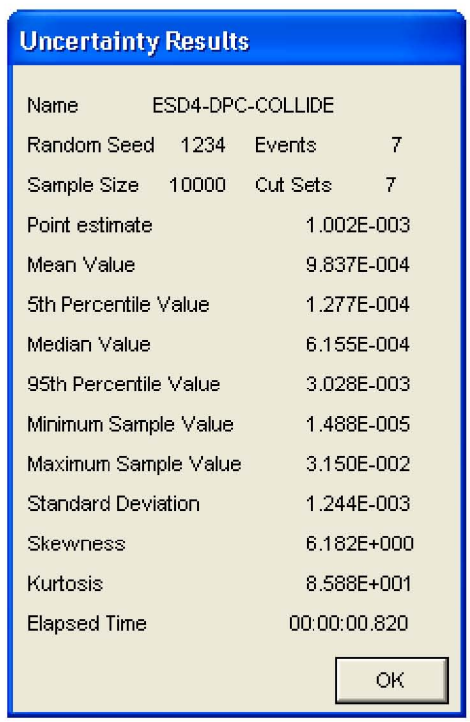
A collision may be caused by an operator error (200-OPCTTCOLLID2-HFI-NOD) failing to stop the CTT.

B2.4.3.5.2 Common-Cause Failures

One CCF (200-CTT--DSW000--ESC-CCF) involves the failure of both deadman switches, both of which must be pressed for the main air supply valve to open. An alpha factor of 0.047 was used to determine the CCF value using two of two as the failure criteria (Table C3-1, CCCG = 2).

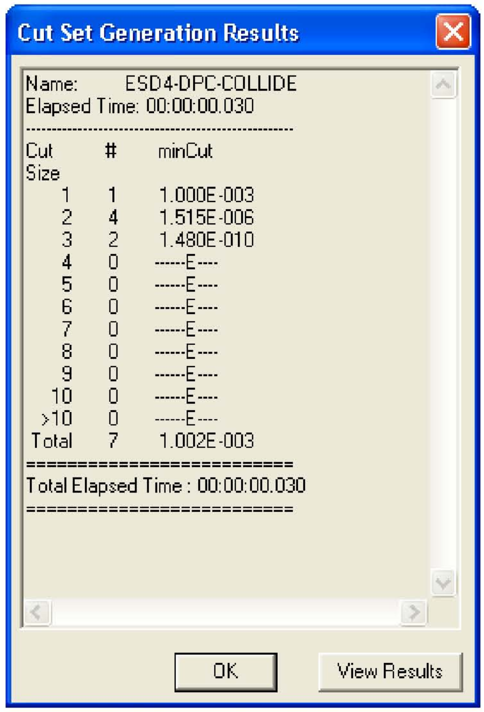
B2.4.3.6 Uncertainty and Cut Set Generation Results

Figure B2.4-8 contains the uncertainty results obtaining from running the fault trees for the "Collision of CTT during Cask Transfer" fault tree. Figure B2.4-9 provides the cut set generation results for "Collision of CTT during Cask Transfer."



Source: Original

Figure B2.4-8. Uncertainty Results for the Collision of CTT during Cask Transfer Fault Tree



Source: Original

Figure B2.4-9. Cut Set Generation Results for the Collision of CTT during Cask Transfer Fault Tree

B2.4.3.7 Cut Sets

Table B2.4-6 contains the cut sets for collision of the "Collision of CTT during Cask Transfer" from the Cask Preparation Room to the Cask Unloading Room fault trees. The total frequency per cask is 1.00E-003 with operator error the dominant cause of collision.

Table B2.4-6. Cut Sets for Collision of the Collision of CTT during Cask Transfer

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
ESD4-DPC-COLLIDE	99.85	1.000E-003	200-OPCTCOLLIDE2-HFI-NOD	Operator Causes CTT Collision	1.000E-003
	0.11	1.093E-006	200-HTC--HC021---HC--FOD	Remote Stop Control Transmits Wrong Instruction	1.740E-003
			200-HTC--SV601---SV--FOD	Main Air Supply Valve Fails on Demand	6.280E-004
	0.04	3.944E-007	200-HTC--SV601---SV--FOD	Main Air Supply Valve Fails on Demand	6.280E-004
			200-HTC--SV602---SV--FOD	Solenoid Valve Fails to Close	6.280E-004
	0.00	2.053E-008	200-CTT--DSW000--ESC-CCF	Common Cause Failure of Deadman Switches	1.180E-005

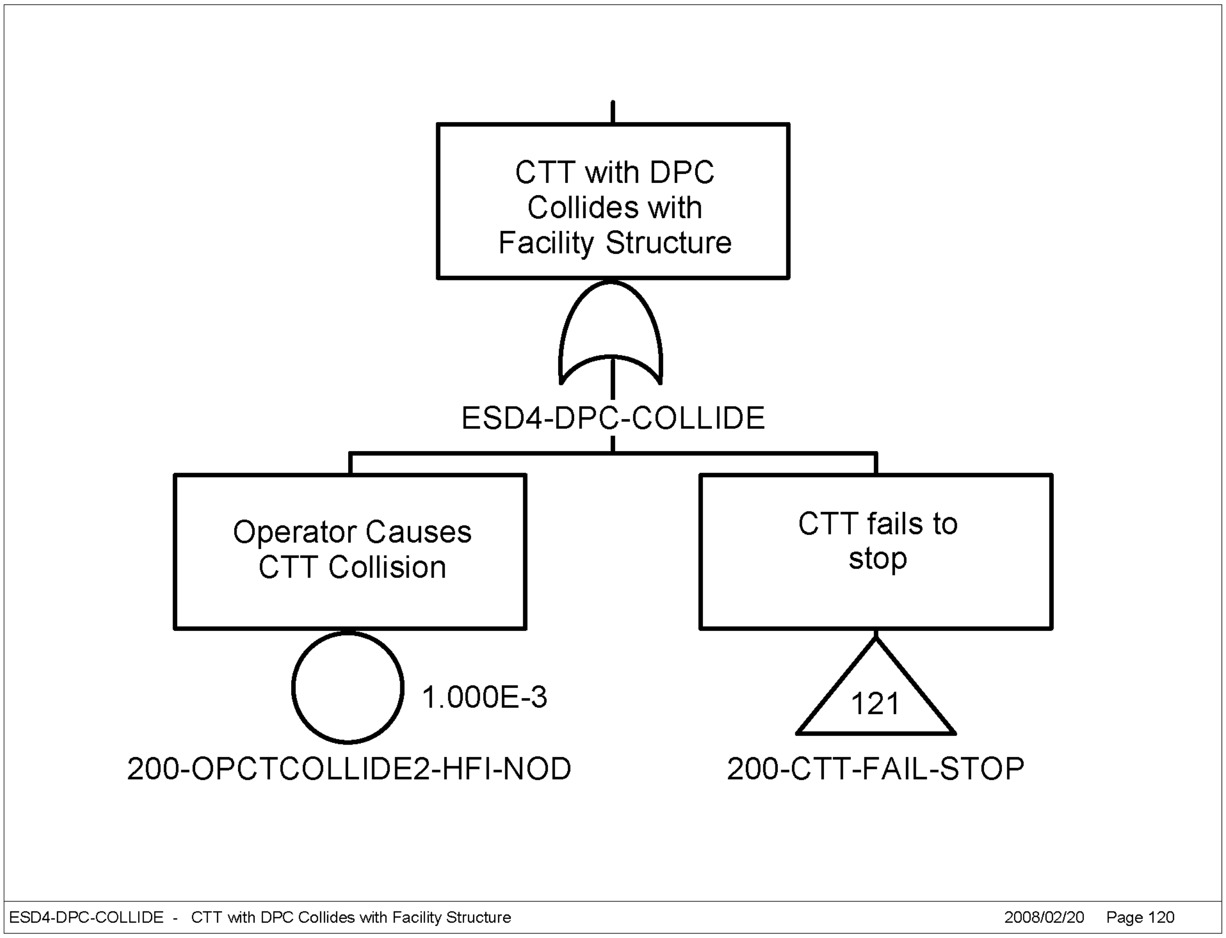
Table B2.4-6. Cut Sets for Collision of the CTT During Cask Transfer (Continued)

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
			200-HTC--HC021---HC--FOD	Remote Stop Control Transmits Wrong Instruction	1.740E-003
	0.00	7.410E-009	200-CTT--DSW000--ESC-CCF	Common Cause Failure of Deadman Switches	1.180E-005
			200-HTC--SV602---SV--FOD	Solenoid Valve Fails to Close	6.280E-004
	0.00	1.088E-010	200-CTT--DSW001--ESC-FOD	Deadman Switch #1 Fails Closed	2.500E-004
			200-CTT--DSW002--ESC-FOD	Deadman Switch #2 Fails Closed	2.500E-004
			200-HTC--HC021---HC--FOD	Remote Stop Control Transmits Wrong Instruction	1.740E-003
	0.00	3.925E-011	200-CTT--DSW001--ESC-FOD	Deadman Switch #1 Fails Closed	2.500E-004
			200-CTT--DSW002--ESC-FOD	Deadman Switch #2 Fails Closed	2.500E-004
			200-HTC--SV602---SV--FOD	Solenoid Valve Fails to Close	6.280E-004
Total	1.002E-003				

NOTE: CTT = cask transfer trolley; Prob. = probability

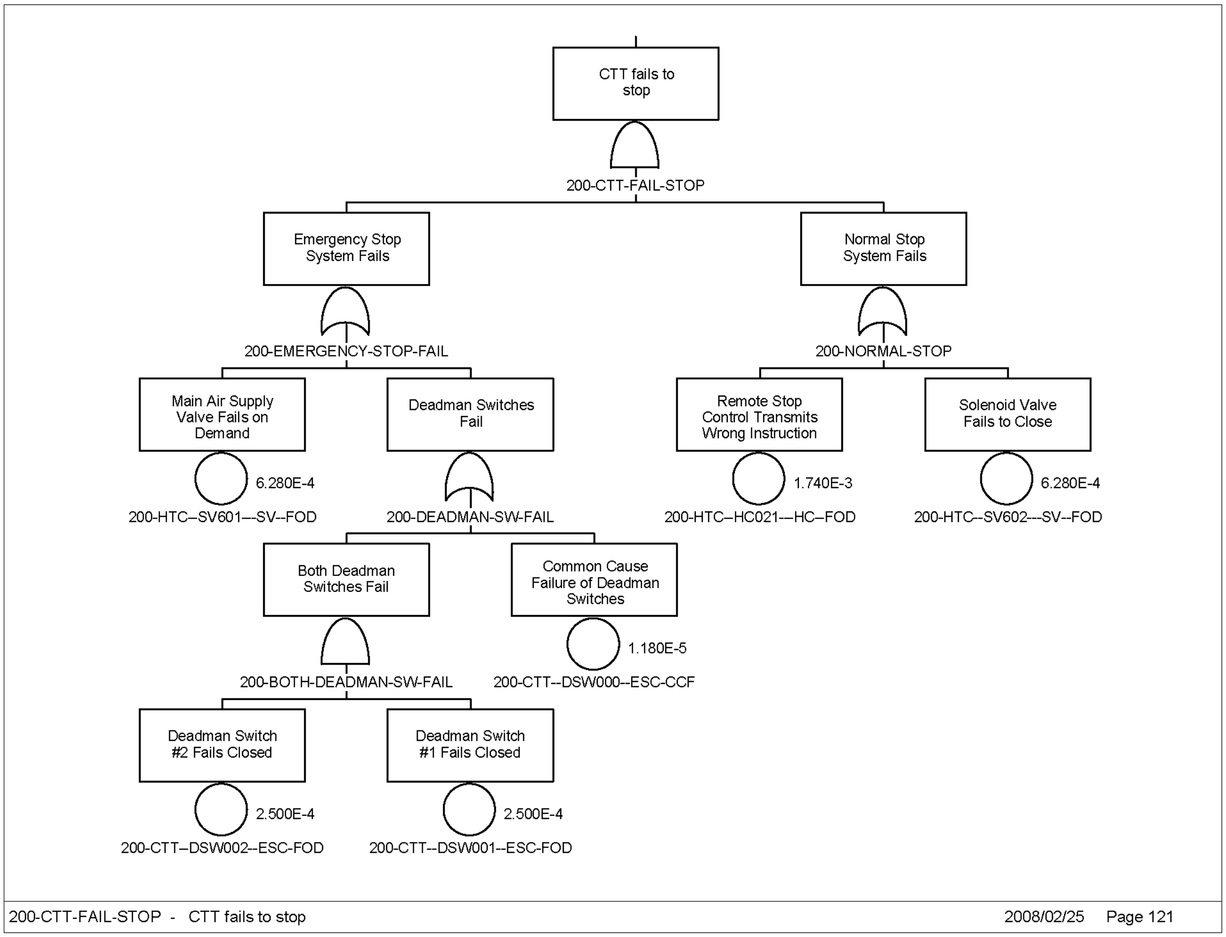
Source: Original

B2.4.3.8 Fault Trees



Source: Original

Figure B2.4-10. Fault Tree for Collision of the Collision of CTT during Cask Transfer (Page 1)



Source: Original

Figure B2.4-11. Fault Tree for Collision of the Collision of CTT during Cask Transfer (Page 2)

B2.4.4 Spurious Movement of the CTT in the Cask Unloading Room

B2.4.4.1 Description

This fault tree describes spurious movement of the CTT during extraction, or unloading, of the canister from the transportation cask on the CTT to satisfy ESD-06, “Canister Impact Due to Movement of CTT during Lift.” The top event is “Spurious Movement during Canister Transfer” which is defined as unplanned movement of the CTT while the canister is being removed from the transportation cask. This fault tree is shown in Figures B2.4-14.

Spurious movement is prevented in the Cask Unloading Room by disconnecting the air supply hose from the CTT. The shield door interlock (external to the CTT) must be closed to allow the port slide gate to open and canister extraction to begin. Thus, if the shield door is not closed the slide gate cannot open and extraction of the canister cannot begin. With the air supply located outside the transfer room, the operator must disconnect the air supply hose to the CTT for the shield door to be closed, or the shield door cuts through the hose upon closing. If the operator fails to disconnect the hose, movement may be initiated by failure of the door interlocks and the control system causing the main air supply valve to open, or the main air supply valve to "fail open" in conjunction with failure of the controls or the control valves. During this transfer process the operator is not in the transfer room and cannot access the controls to initiate spurious movement.

B2.4.4.2 Success Criteria

Success criterion is that the CTT remain motionless during canister extraction from the transportation cask. Movement of the CTT during this operation could cause impact and/or shear and damage to the canister.

B2.4.4.3 Design Requirements and Features

The design feature is the shield door interlocks that prevent the extraction operation until the shield door is closed. Requirements include locating the air supply outside the canister transfer room, and for the operator to disconnect the air supply to the CTT prior to unloading.

B2.4.4.4 Fault Tree Model

The top event is the "Spurious Movement of the CTT in the Cask Unloading Room" during extraction of the canister from the transportation cask on the CTT. This may occur through failure to disconnect the air supply resulting in operation of the main air supply valve. The air supply valve may fail through spurious operation of the valve or spurious signals generated by the control system. Compressed air may be available to the CTT through failure of the operator to disconnect the air hose, or failure of the shield door interlocks. A conservative mission time for this operation has been set at one hour.

B2.4.4.5 Basic Event Data

Table B2.4-7 contains a list of basic events used in the fault tree (Figure B2.4-14) for "Spurious Movement of the CTT in the Cask Unloading Room."

Table B2.4-7. Basic Event Probability for Spurious Movement of the CTT in the Cask Unloading Room

Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CR---IEL001--IEL-FOD	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-CR---IEL002--IEL-FOD	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-CR---IELCCF--IEL-CCF	1	1.290E-006	1.290E-006	0.000E+000	0.000E+000
200-CTT--CT001---CT--SPO	3	2.270E-005	0.000E+000	2.270E-005	1.000E+000
200-CTT--HC001---HC--SPO	3	5.230E-007	0.000E+000	5.230E-007	1.000E+000
200-OPNODISCOAIR-HFI-NOD	1	1.000E-003	1.000E-003	0.000E+000	0.000E+000
200-CTT--SV301---SV--SPO	3	4.090E-007	0.000E+000	4.090E-007	1.000E+000
200--CTT--SV401--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-FWDREVM1-SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-FWDREVM2-SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-SVROTM1--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000
200-CTT-SVROTM2--SV--FOH	3	4.870E-005	0.000E+000	4.870E-005	1.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B2.4.4.5.1 Human Failure Events

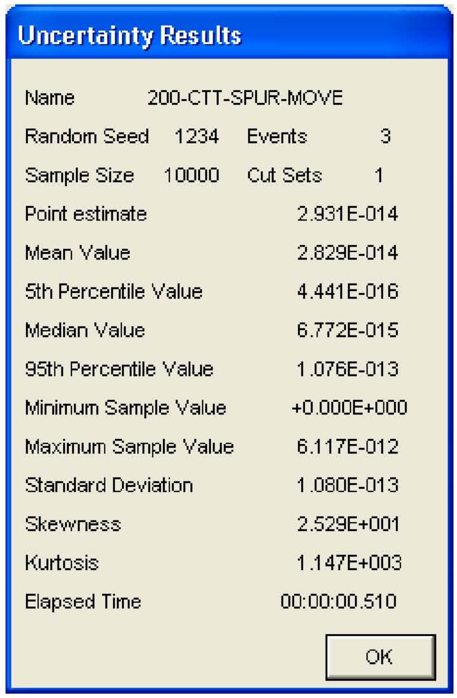
One operator error involves failure to disconnect the air supply. (200-OPNODISCOAIR-HFI-NOD).

B2.4.4.5.2 Common-Cause Failures

One CCF (200-CR---IELCCF--IEL-CCF) involves failure of both shield door interlocks allowing the shield door to close and the slide port gate to open. An alpha factor of 0.047 was used to determine the CCF value using two of two as the failure criteria (Table C3-1, CCCG = 2).

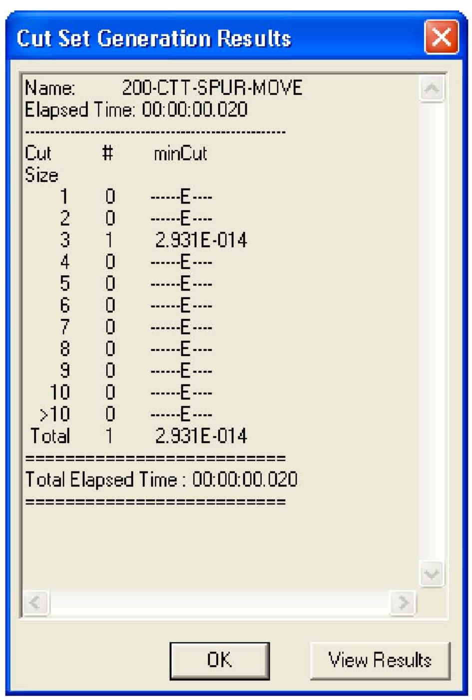
B2.4.4.6 Uncertainty and Cut Set Generation Results

Figure B2.4-12 contains the uncertainty results obtained from running the fault trees for "Spurious Movement of the CTT in the Cask Unloading Room" while extracting the canister from the transportation cask in the unloading area. Figure B2.4-13 provides the cut set generation results for the "Spurious Movement of the CTT in the Cask Unloading Room" fault tree.



Source: Original

Figure B2.4-12. Uncertainty Results for the Spurious Movement of the CTT in the Cask Unloading Room Fault Tree



Source: Original

Figure B2.4-13. Cut Set Generation Results for the Spurious Movement of the CTT in the Cask Unloading Room Fault Tree

B2.4.4.7 Cut Sets

Table B2.4-8 contains the cut sets for the "Spurious Movement of the CTT in the Cask Unloading Room" fault tree. The total frequency per cask is 2.93E-014.

Table B2.4-8. Cut Sets for Spurious Movement of the CTT in the Cask Unloading Room

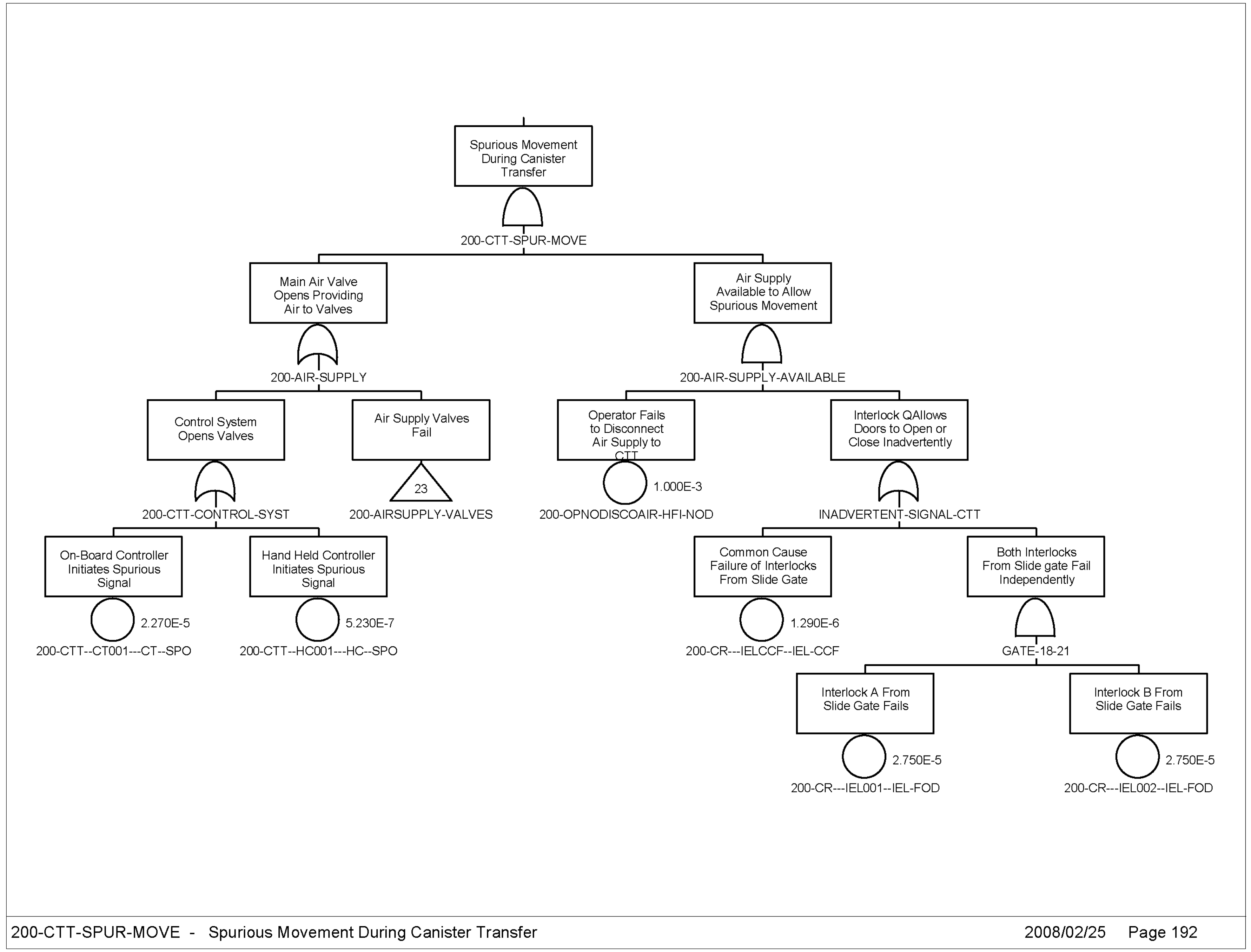
Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
200-CTT-SPUR-MOVE	99.91	2.928E-014	200-CR---IELCCF--IEL-CCF	Common Cause Failure of Interlocks From Slide Gate	1.290E-006
			200-CTT--CT001---CT--SPO	On-Board Controller Initiates Spurious Signal	2.270E-005
			200-OPNODISCOAIR-HFI-NOD	Operator Fails to Disconnect Air Supply to CTT	1.000E-003
Total	2.931E-014				

NOTE: CTT = cask transfer trolley; Prob. = probability.

Source: Original

B2.4.4.8 Fault Trees

The fault tree for "Spurious Movement of the CTT in the Canister Transfer Room" is shown in Figure B2.4-14. Note that the transfer gate 23 in Figure B2.4-14 refers to the fault tree in Figure B2.4-4.



Source: Original

Figure B2.4-14. Fault Tree for Spurious Movement of the CTT in the Cask Unloading Room

B3 LOADING/UNLOADING ROOM SHIELD DOOR AND SLIDE GATE FAULT TREE ANALYSIS

B3.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

B3.1.1 BSC (Bechtel SAIC Company) 2007. *Nuclear Facilities Equipment Shield Door Process and Instrumentation Diagram.* 000-M60-H000-00101-000 REV 00D. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071220.0024.

B3.1.2 BSC 2008. *Nuclear Facilities Slide Gate Process and Instrumentation Diagram.* 000-M60-H000-00201-000 REV 00E. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080123.0025.

B3.1.3 BSC 2007. *Receipt Facility General Arrangement Ground Floor Plan.* 200-P10-RF00-00102-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071212.0011.

B3.1.4 BSC 2007. *Receipt Facility General Arrangement Second Floor Plan.* 200-P10-RF00-00103-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071212.0012.

B3.2 SLIDE GATE AND SHIELD DOOR SYSTEM DESCRIPTION

B3.2.1 Overview

The shield doors and slide gates provide shielding during canister unloading and loading. They are considered important to safety (ITS) as they protect workers from radioactive material that is exposed while being handled in the Cask Unloading Room and Loading Room. There are two slide gates in the RF. One shields the unloading port between the Cask Unloading Room and the Canister Transfer Room and the other shields the loading port between the Loading Room and the Canister Transfer Room. Shield doors provide equipment access to the Loading Room and Cask Unloading Room. The Cask Unloading Room shield door provides access for the CTT to move from the Cask Preparation Room into the Cask Unloading Room. The Loading Room shield door provides a site transporter access to the Loading Room from the Lid Bolting Room ((Ref. B3.1.3) and (Ref. B3.1.4)).

B3.2.2 Operations Description

The Cask Unloading Room shield door is opened to allow the CTT to enter the room. Once equipment is positioned properly in the Cask Unloading Room, shield doors are shut in preparation for removing canisters from the cask. Once the shield doors are shut, the slide gate may be opened to allow the canister transfer machine (CTM) to perform cask unloading operations. Loading of the aging overpack is analogous to cask unloading operations. The slide gate may be opened to allow aging overpack loading access if the shield doors are closed. Once loading is complete and the slide gate is closed, the shield doors are opened to allow aging overpack removal.

B3.2.3 Physical Description

The shield doors consist of pairs of large heavy doors that are operated by individual motors with over-torque sensors to prevent crushing of an object. Each door has two position sensors to indicate either a closed or open door and an obstruction sensor prevents the doors from closing on an object. The obstruction sensor is also alarmed to provide operators indication when an object is between the shield doors. The shield doors and slide gates are interlocked to prevent one another from opening if the other is open. The shield doors are opened and closed via a hand lever that must be enabled by an enable/disable switch. An emergency open switch exists enabling the doors to be opened in case of an emergency situation.

Similar to the shield doors, the slide gates consist of two gates that close together between the Loading/Cask Unloading Rooms and the Canister Transfer Room. The gates are operated by individual motors that also have over-torque sensors. Each gate has limit switches to indicate open or closed gates. A CTM skirt-in-place switch is interlocked to the slide gate to prevent the gates from opening without the CTM in place. Slide gate operation is controlled by a hand switch coupled with an enable/disable switch and shield door interlocks prevent the slide gate from opening when the shield door is open. Open/closed and CTM in-place indicators exist to assist operators in their activities.

B3.2.4 Schematics

Schematics for the shield door and slide gate are available separately for review ((Ref. B3.1.1) and (Ref. B3.1.2)).

Additional shield door details are available in *Nuclear Facilities Slide Gate Process and Instrumentation Diagram* (Ref. B3.1.2), including slide gate instrumentation.

B3.3 DEPENDENCIES AND INTERACTIONS

Dependencies are broken down into five categories with respect to their interactions with structures, systems, and components. The five areas considered are addressed in Table B3.3-1 with the following dependencies:

1. Functional dependence
2. Environmental dependence
3. Spatial dependence

4. Human dependence
5. Failures based on external events.

Table B3.3-1. Dependencies and Interactions Analysis

Systems, Structures, Components	Dependencies and Interactions				
	Functional	Environmental	Spatial	Human	External Events
Door/gate motors	—	—	—	Inadvertent operation	—
Door/gate position limit switches	CTM	—	—	—	—
CTM	Gate position switches, obstruction sensor	—	—	—	—
Obstruction sensor	CTM	—	—	—	—

NOTE: CTM = canister transfer machine

Source: Original

B3.4 SLIDE GATE AND SHIELD DOOR FAILURE SCENARIOS

The slide gate and shield door system has three credible failure scenarios as follows:

1. Inadvertent opening of the shield door
2. Inadvertent opening of the slide gate
3. Shield door closes on conveyance.

B3.4.1 Inadvertent Opening of the Shield Door

B3.4.1.1 Description

Inadvertent opening of the shield door while a canister is being unloaded from a cask or loaded into an aging overpack can cause an exposure. For this situation to occur, the slide gate must be open for the CTM to be unloading/loading a canister. Interlocks between the slide gate and shield door prevent an operator from being able to open the shield door during canister unloading or loading. However, this situation can occur if the interlocks fail and an operator attempts to open the door, or a spurious open signal is received.

B3.4.1.2 Success Criteria

The success criteria for this failure scenario require that the interlocks between the slide gate and shield door prevent the shield door from opening when the slide gate is open.

B3.4.1.3 Design Requirements and Features

Redundant hardwired interlocks prevent the shield door from opening while the slide gate is open and vice versa. The shield door system does not have any test, maintenance, or other modes/settings that allows bypass of interlocks.

B3.4.1.4 Fault Tree Model

The top event in this fault tree is "Shield Door Inadvertently Opened While Unloading Cask." This is defined as an opening of the shield door during unloading operations while the cask is in a position that would result in a direct exposure to personnel outside of the unloading room. Faults considered in the evaluation of this top event include: failure of components in the control circuitry of the slide door and a human event that contribute to the inadvertent door opening. The fault tree is shown in Figure B3.4-3.

B3.4.1.5 Basic Event Data

Six basic events, as shown in Table B3.4-1, are used to model this failure scenario, including one human failure event (HFE), one common-cause failure, and one situational event.

The basic event, "Canister is Exposed During Mid-Unloading" represents the probability that the canister is removed from of the cask, but has not reached the CTM skirt yet. The screening value of 1.0 is used for this event.

Table B3.4-1. Basic Event Probabilities for Inadvertent Opening of Shield Door

Basic Event	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CR---IELCCF--IEL-CCF	Common-cause failure of interlocks from slide gate	1	1.290E-06	12900E-06	0.000E+00	0.000E+00
200-CR--- IEL001 —IEL-FOD	Interlock A from slide gate fails	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CR--- IEL002—IEL-FOD	Interlock B from slide gate fails	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CR---PLC001--PLC-SPO	Inadvertent signal sent due to PLC failure	3	3.650E-07	0.000E+00	3.650E-07	1.000E+00
200-CR-CASK-UNLOADING	Canister is exposed during mid-unloading	1	1.000E+00	1.000E+00	0.000E+00	0.000E+00
200-OPDIREXPOSE1-HFI-NOD	Operator mistakenly opens door	1	1.000E-01	1.000E-01	0.000E+00	0.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; PLC = programmable logic controller; Prob. = probability.

Source: Original

B3.4.1.5.1 Human Failure Events

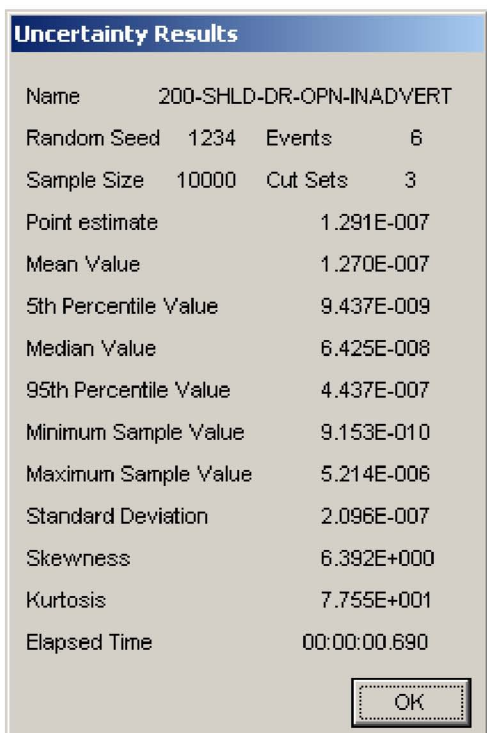
One HFE is modeled in the fault tree as an operator attempting to open the shield doors during a CTM loading or unloading operation. However, for the operator to open the shield door while the slide gate is open the interlock must fail. The screening value used for this HFE has a probability of 1.0E-01 (Table 6.4-1).

B3.4.1.5.2 Common-Cause Failures

One CCF scenario is modeled in the fault tree. The redundant interlocks that prevent the shield door from opening while the slide gate is open can both fail due to a common cause. The common-cause failure alpha factor for two of two successes is 0.047 (Attachment C) which is multiplied with the probability of failure of the component to establish the failure probability of the common-cause event associated with the two common-cause elements.

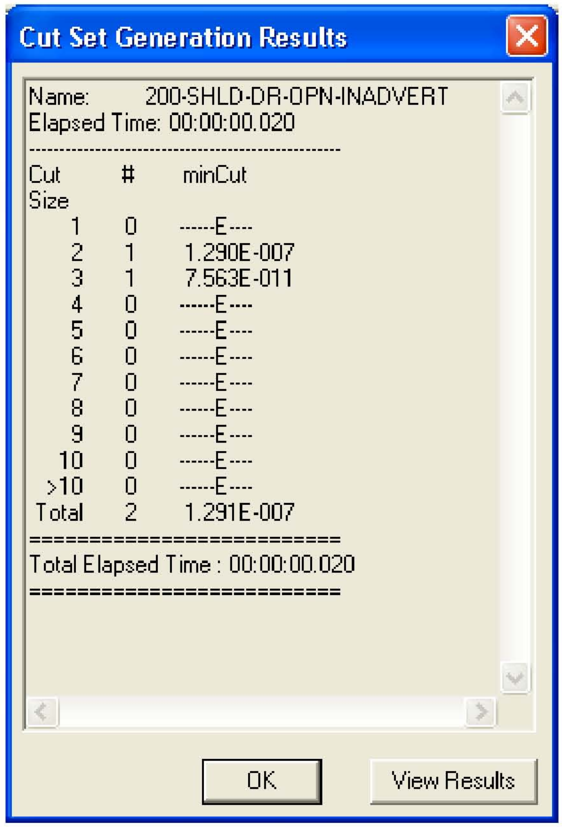
B3.4.1.6 Uncertainty and Cut Set Generation Results

Figure B3.4-1 contains the uncertainty results obtained from running the fault trees for "Inadvertent Opening of the Shield Door" while unloading cask, using a cutoff probability of 1E-15. Figure B3.4-2 provides the cut set generation results for the "Inadvertent Opening of the Shield Door" while unloading cask fault tree.



Source: Original

Figure B3.4-1. Uncertainty Results for the Shield Door Inadvertently Opened While Unloading Cask Fault Tree



Source: Original

Figure B3.4-2. Cut Set Generation Results for the Shield Door Inadvertently Opened While Unloading Cask Fault Tree

B3.4.1.7 Cut Sets

Cut sets for "Inadvertent Opening of Shield Door" are displayed in Table B3.4-2.

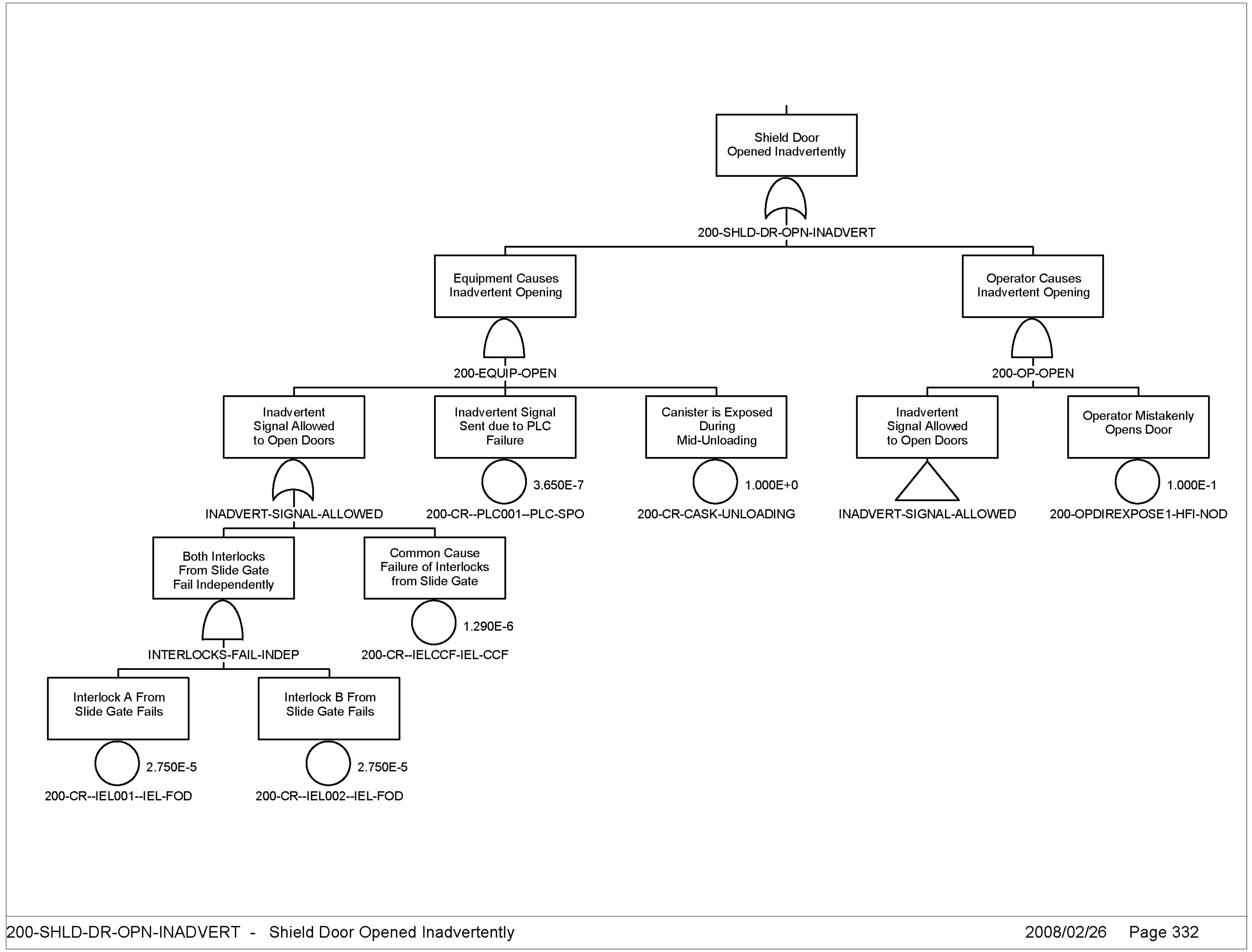
Table B3.4-2. Cut Sets for Inadvertent Opening of Shield Door

Fault Tree	% Cut Set	Prob./Freq.	Basic Event	Description	Event Prob.
200-SHLD-DR-OPN-INADVERT	99.94	1.290E-007	200-CR--IELCCF-IEL-CCF	Common Cause Failure of Interlocks from Slide Gate	1.290E-006
			200-OPDIREXPOSE1-HFI-NOD	Operator Mistakenly Opens Door	1.000E-001
	0.06	7.562E-011	200-CR--IEL001--IEL-FOD	Interlock A From Slide Gate Fails	2.750E-005
			200-CR--IEL002--IEL-FOD	Interlock B From Slide Gate Fails	2.750E-005
			200-OPDIREXPOSE1-HFI-NOD	Operator Mistakenly Opens Door	1.000E-001

NOTE: Freq. = frequency; PLC = programmable logic controller; Prob. = probability.

Source: Original

B3.4.1.8 Fault Trees



Source: Original

Figure B3.4-3. Fault Trees for Inadvertent Opening of the Shield Door

B3.4.2 Inadvertent Opening of Slide Gate Causing Direct Exposure

B3.4.2.1 Description

Inadvertent opening of a slide gate can result in an exposure if personnel are present in the Canister Transfer Room and a radiation source is exposed in the Loading or Cask Unloading Room. There are two ways that a slide gate may be inadvertently opened: (1) an operator mistakenly opens the slide gate or, (2) the control electronics spuriously opens the slide gate. Additionally, an interlock that prevents the slide gate from opening unless the CTM skirt is in place must also fail or be disabled. In this situation, the shield door may be closed; therefore the interlocks that prevent the slide gate from opening while the shield door is open do not prevent the slide gate from opening.

B3.4.2.2 Success Criteria

The success criteria for this failure scenario require that the shield bell slide gate not open during canister transfer operations unless the shield skirt is lowered.

B3.4.2.3 Design Requirements and Features

A single interlock is used to prevent the slide gate from opening when the CTM skirt is not in place.

B3.4.2.4 Fault Tree Model

The top event in this fault tree is "Inadvertent Opening of the Slide Gate Causing Direct Exposure." This is defined as an opening of the slide gate during unloading operations while the cask is in a position that would result in a direct exposure to personnel in the Canister Transfer Room. Faults considered in the evaluation of this top event include: failure of components in the control circuitry of the slide gate and a human event that contribute to the inadvertent gate opening. The fault tree is shown in Figure B3.4-6.

B3.4.2.5 Basic Event Data

Three basic events, as shown in Table B3.4-3, are used to model this failure scenario, including one human failure event and two hardware events.

Table B3.4-3. Basic Event Probabilities for Inadvertent Opening of Slide Gate Causing Direct Exposure

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CR---IEL001--IEL-FOD	Skirt interlock failed	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
2000-CR---PLC001--PLC-SPO	Inadvertent signal sent due to PLC failure	3	3.650E-07	0.000E+00	3.650E-07	1.000E+00
200-OPFAILRSTINT-HFI-NOM	Skirt interlock disabled	1	1.000E-02	1.000E-02	0.000E+00	0.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; PLC = programmable logic controller; Prob. = probability.

Source: Original

B3.4.2.5.1 Human Failure Events

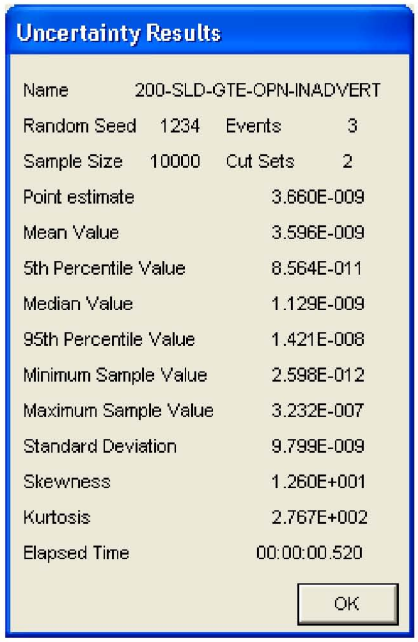
One HFE is modeled in the fault tree. This HFE is a combination of operator actions and interlock failures that can result in the slide gate being opened when the shield skirt is raised. The development of this event is presented in detail as part of the human reliability analysis in Section 6.4 (Table 6.4-1) and Attachment E.

B3.4.2.5.2 Common-Cause Failures

No CCFs are identified for this fault tree.

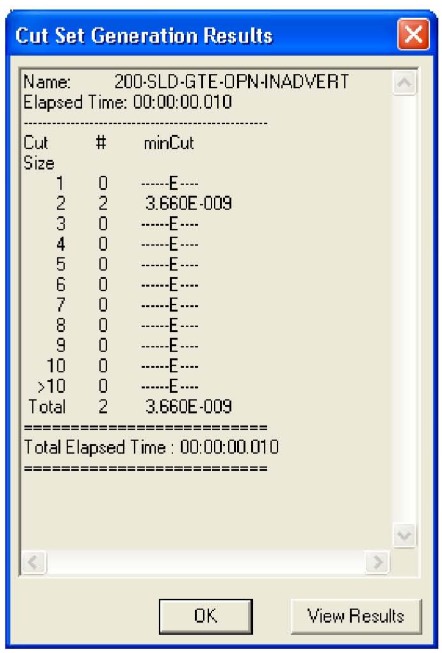
B3.4.2.6 Uncertainty and Cut Set Generation

Figure B3.4-4 contains the uncertainty results obtaining from running the fault tree for "Inadvertently Opening of the Slide Gate Causing Direct Exposure." Figure B3.4-5 provides the cut set generation results for the "Inadvertently Opening of the Slide Gate Causing Direct Exposure" fault tree.



Source: Original

Figure B3.4-4. Uncertainty Results for the Inadvertent Opening of the Slide Gate Causing Direct Exposure Fault Tree



Source: Original

Figure B3.4-5. Cut Set Generation Results for the Inadvertent Opening of the Slide Gate Causing Direct Exposure Fault Tree

B3.4.2.7 Cut Sets

Table B3.4-4 contains the cut sets for the "Inadvertent Opening of the Slide Gate Causing Direct Exposure" fault tree.

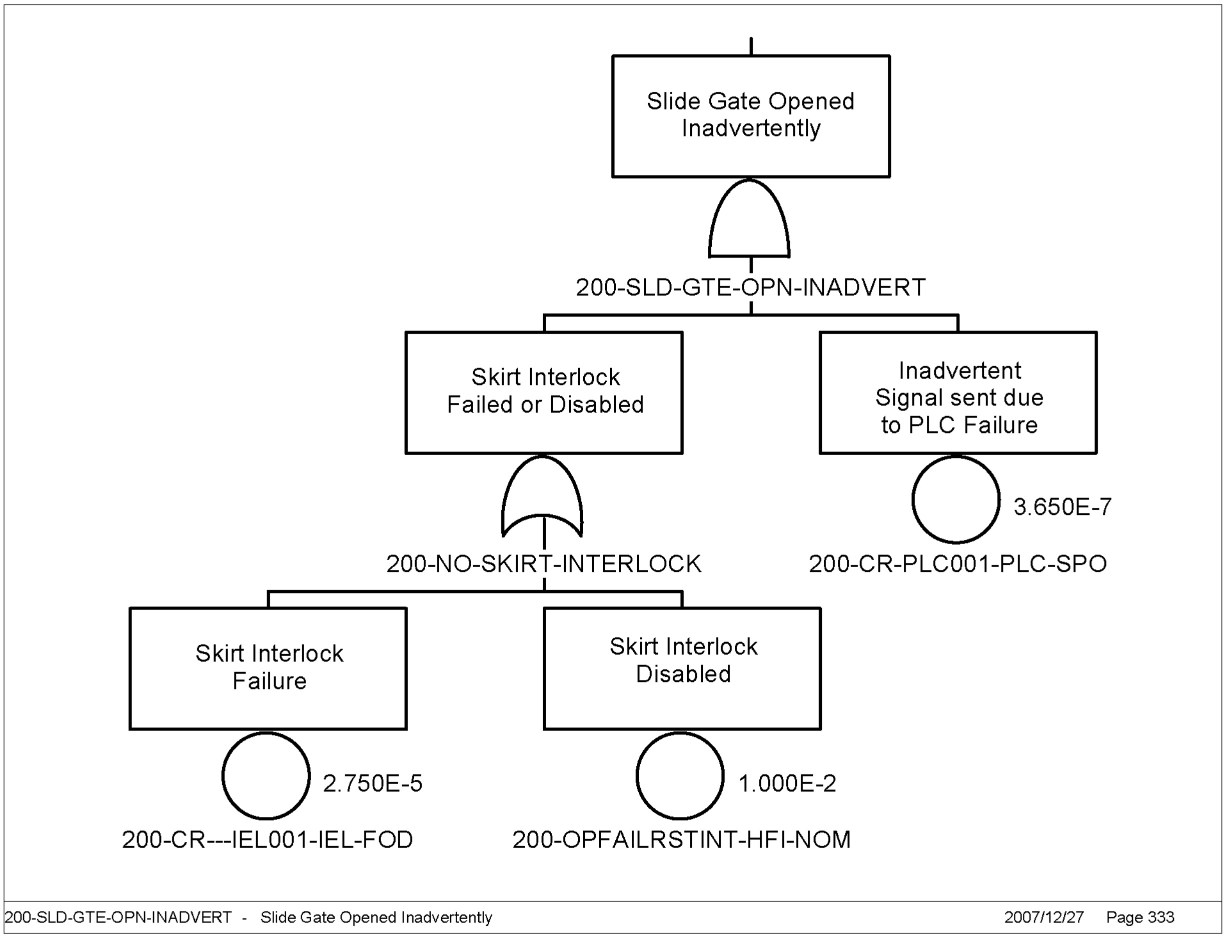
Table B3.4-4. Cut Sets for Inadvertent Opening of the Slide Gate Causing Direct Exposure

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
200-SLD-GTE-OPN-INADVERT	99.73	3.650E-009	200-CR-PLC001-PLC-SPO	Inadvertent Signal sent due to PLC Failure	3.650E-007
			200-OPFAILRSTINT-HFI-NOM	Skirt Interlock Disabled	1.000E-002
	0.27	1.004E-011	200-CR---IEL001-IEL-FOD	Skirt Interlock Failure	2.750E-005
			200-CR-PLC001-PLC-SPO	Inadvertent Signal sent due to PLC Failure	3.650E-007
		3.660E-009	= Total		

NOTE: PLC = programmable logic controller; Prob. = probability.

Source: Original

B3.4.2.8 Fault Trees



Source: Original

Figure B3.4-6. Fault Trees for Inadvertent Opening of the Slide Gate

B3.4.3. Shield Door Closes on Conveyance

B3.4.3.1 Description

If the shield doors to the Loading/Cask Unloading Rooms are closed as casks or aging overpacks are transferred to/from the Loading/Cask Unloading Rooms, a release may occur as a result. Measures are in place to ensure this situation does not occur, including the presence of an obstruction sensor and motor over-torque sensors.

B3.4.3.2 Success Criteria

The success criterion for this scenario is defined as the shield doors not causing a release due to closure on the conveyance. Specifically, success criteria are defined as follows:

- Obstruction sensor prohibits the initiation of shield door closure.
- In the event that the obstruction sensor fails and the shield doors do close on a conveyance, the motor over-torque sensors prevent excessive closure force, ensuring no release.

B3.4.3.3 Design Requirements and Features

Objects or obstructions are detected between the shield doors to prevent door closure initiation. Motor over-torque sensors prevent the shield doors from causing damage to casks or waste packages in the event of closure on a conveyance.

B3.4.3.4 Fault Tree Model

The top event in this fault tree is "Collision of Shield Door into Conveyance." This is defined as an inadvertent closure of the shield doors due to either operator action or component failure while the conveyance is in position to be hit by the doors. Faults considered in the evaluation of this top event include: failure of components in the control circuitry of the shield doors and human events that contribute to the inadvertent shield door closing. The fault tree is shown in Figure B3.4-9.

B3.4.3.5 Basic Event Data

Six basic events listed in Table B3.4-5 are used to model this failure scenario, including one HFE and one CCF.

Table B3.4-5. Basic Event Probabilities for Shield Door Closes on Conveyance

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-OPSDCLOSE001-HFI-NOD	Operator Collides Shield Door with CTT	1	1.000E+00	1.000E+00	0.000E+00	0.000E+00
200-SD---PLC001--PLC-SPO	Spurious signal from PLC closes door	3	3.650E-07	0.000E+00	3.650E-07	1.000E+00
200-SD---SRU001--SRU-FOH	Ultrasonic obstruction sensor fails	7	2.161E-03 [c]	0.000E+00	2.161E-03	1.000E+00
200-SD---TL000---TL--CCF	Common-cause failure of over-torque sensors	3	6.765E-04[b]	0.000E+00	3.780E-06	1.000E+00
200-SD---TL001---TL--FOH	Motor #1 over-torque sensor fails	3	1.435E-02[b]	0.000E+00	8.050E-05	1.000E+00
200-SD---TL002---TL--FOH	Motor #2 over-torque sensor fails	3	1.435E-02[b]	0.000E+00	8.050E-05	1.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
[b]Tau = 360 hours
[c]Tau = 45 hours.
PLC = programmable logic controller; Prob. = probability; ST = site transporter.

Source: Original

B3.4.3.5.1 Human Failure Events

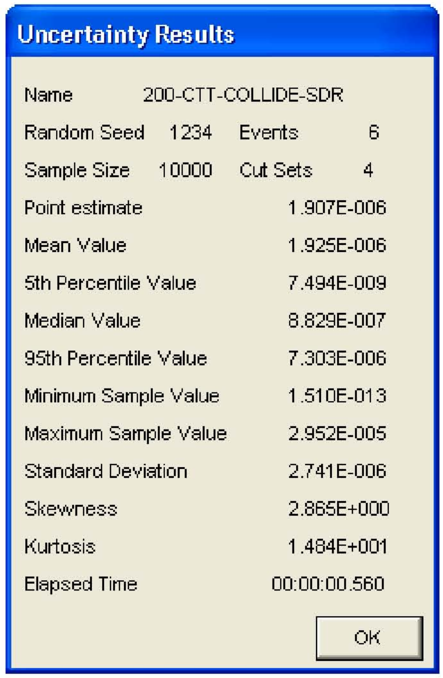
One HFE is modeled in the fault tree as an operator attempting to close the shield doors while a conveyance is between the doors. The screening value used for this HFE has a probability of 1.

B3.4.3.5.2 Common-Cause Failures

One CCF considered is the failure of the shield door over-torque sensors. This CCF allows the shield doors to continue to attempt to close once an obstruction, in this case the conveyance, is encountered.

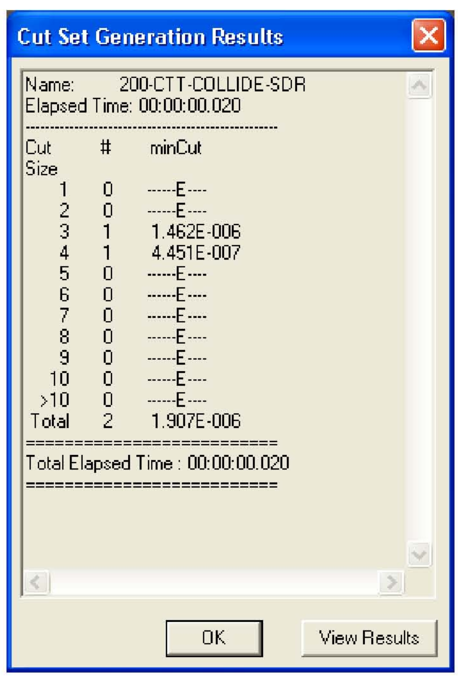
B3.4.3.6 Uncertainty and Cut set Generation

Figure B3.4-7 contains the uncertainty results obtaining from running the fault tree of “Shield Door Closes on Conveyance” using a cutoff probability of 1E-15. Figure B3.4-8 provides the cut set generation results for the “Shield Door Closes on Conveyance” fault tree. The fault tree and results for shield door closing on the CTT are identical with the fault tree and results for the shield door closing on the site transporter.



Source: Original

Figure B3.4-7. Uncertainty Results for the Shield Door Closes on Conveyance Fault Tree



Source: Original

Figure B3.4-8. Cut Set Generation Results for the Shield Door Closes on Conveyance Fault Tree

B3.4.3.7 Cut Sets

Table B3.4-6 contains the cut sets for spurious door closing on a conveyance.

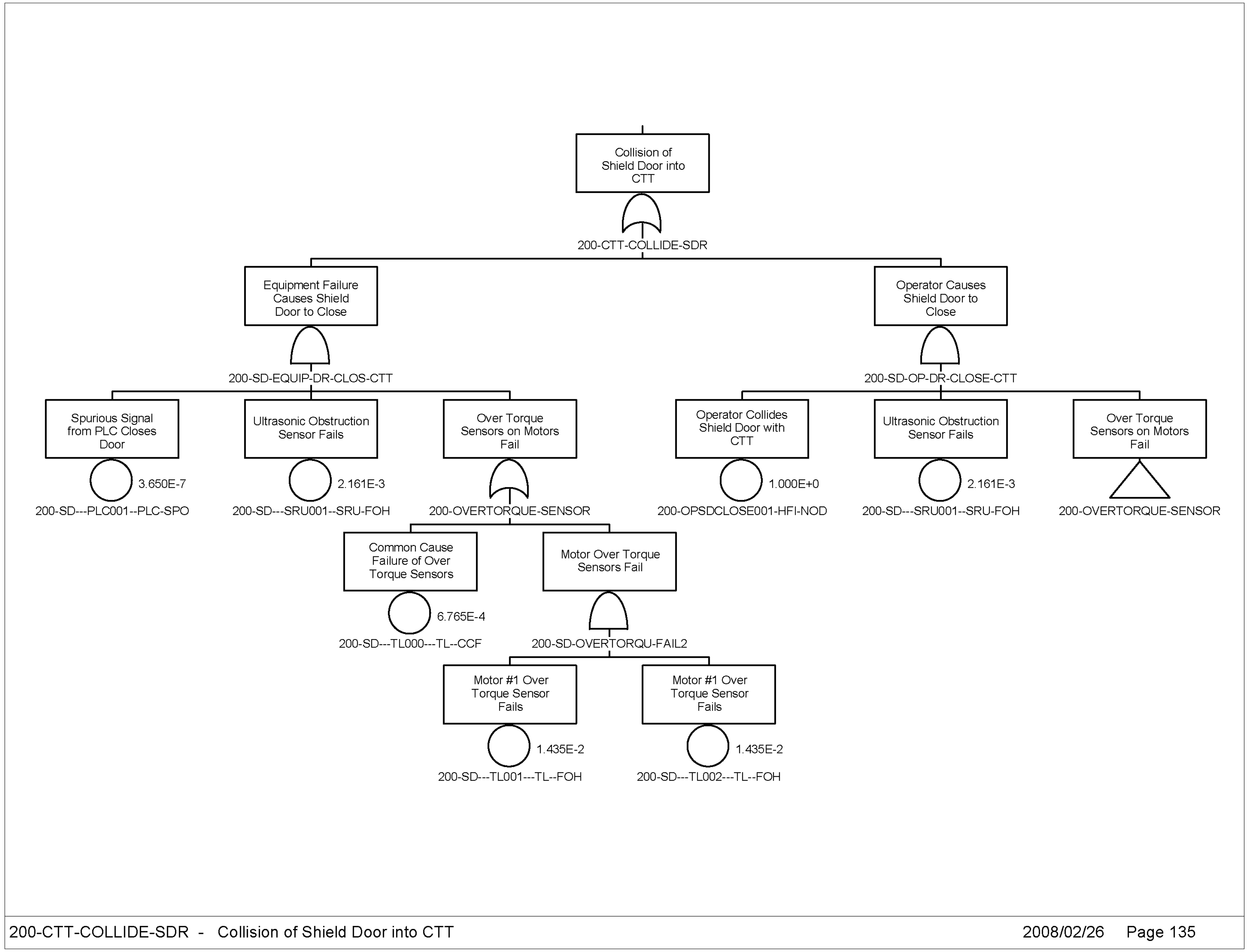
Table B3.4-6. Cut Sets for Shield Door Closes on Conveyance

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
200-CTT-COLLIDE-SDR	76.66	1.462E-006	200-OPSDCLOSE001-HFI-NOD	Operator Collides Shield Door with CTT	1.000E+000
			200-SD---SRU001--SRU-FOH	Ultrasonic Obstruction Sensor Fails	2.161E-003
			200-SD---TL000---TL--CCF	Common Cause Failure of Over Torque Sensors	6.765E-004
	23.34	4.451E-007	200-OPSDCLOSE001-HFI-NOD	Operator Collides Shield Door with CTT	1.000E+000
			200-SD---SRU001--SRU-FOH	Ultrasonic Obstruction Sensor Fails	2.161E-003
			200-SD---TL001---TL--FOH	Motor #1 Over Torque Sensor Fails	1.435E-002
			200-SD---TL002---TL--FOH	Motor #1 Over Torque Sensor Fails	1.435E-002
	0.00	5.337E-013	200-SD---PLC001--PLC-SPO	Spurious Signal from PLC Closes Door	3.650E-007
			200-SD---SRU001--SRU-FOH	Ultrasonic Obstruction Sensor Fails	2.161E-003
			200-SD---TL000---TL--CCF	Common Cause Failure of Over Torque Sensors	6.765E-004
	0.00	1.625E-013	200-SD---PLC001--PLC-SPO	Spurious Signal from PLC Closes Door	3.650E-007
			200-SD---SRU001--SRU-FOH	Ultrasonic Obstruction Sensor Fails	2.161E-003
			200-SD---TL001---TL--FOH	Motor #1 Over Torque Sensor Fails	1.435E-002
			200-SD---TL002---TL--FOH	Motor #1 Over Torque Sensor Fails	1.435E-002

NOTE: CTT = cask transfer trolley; Fail. = failure; PLC = programmable logic controller; Prob. = probability.

Source: Original

B3.4.3.8 Fault Trees



Source: Original

Figure B3.4-9. Fault Trees for Shield Door Closes on Conveyance

B4 CANISTER TRANSFER MACHINE FAULT TREE ANALYSIS

B4.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

B4.1.1 ASME NOG-1-2004. 2005. *Rules for Construction of Overhead and Gantry Cranes (Top Running Bridge, Multiple Girder)*. NEW YORK, NEW YORK: AMERICAN SOCIETY OF MECHANICAL ENGINEERS. ISBN: 0-7918-2923-1. TIC: 257672.

B4.1.2 BSC (Bechtel SAIC Company) 2007. *CRCF, RF, WHF, and IHF Canister Transfer Machine Process and Instrumentation Diagram Sheet 1 of 4*. 000-M60-HTC0-00101-000 REV 00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071218.0028.

B4.1.3 BSC 2007. *CRCF, RF, WHF and IHF Canister Transfer Machine Process and Instrumentation Diagram Sheet 2*. 000-M60-HTC0-00102-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071030.0022.

B4.1.4 BSC 2007. *CRCF, RF, WHF, and IHF CTM Canister Grapple Process and Instrumentation Diagram*. 000-M60-HTC0-00201-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071011.0008.

B4.1.5 BSC 2007. *Nuclear Facilities Shield Door Process and Instrumentation Diagram*. 000-M60-H000-00101-000 REV 00D. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071220.0024.

B4.1.6 BSC 2008. *CRCF, RF, WHF and IHF Canister Transfer Machine Process and Instrumentation Diagram Sheet 3*. 000-M60-HTC0-00103-000 REV 00D. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080103.0011.

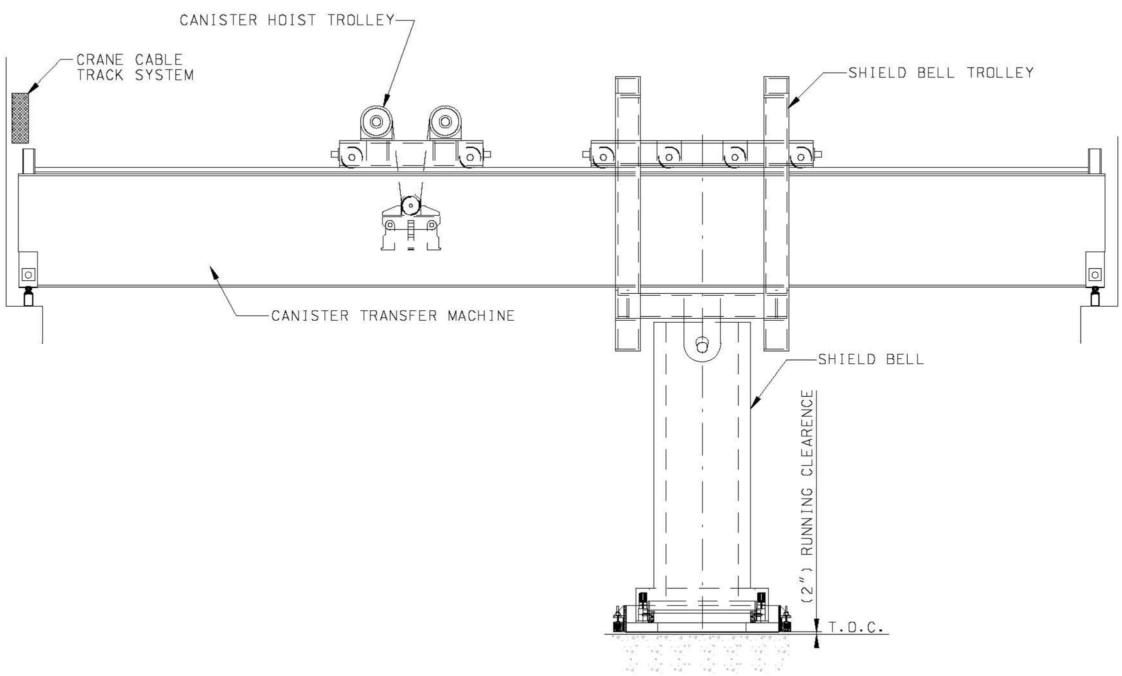
B4.1.7 BSC 2008. *Nuclear Facilities Slide Gate Process and Instrumentation Diagram*. 000-M60-H000-00201-000 REV 00E. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080123.0025.

B4.1.8 BSC 2008. *Mechanical Handling Design Report - Canister Transfer Machine*. 000-30R-WHS0-01900-000 REV 002. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080109.0022.

B4.2 CANISTER TRANSFER MACHINE DESCRIPTION

The CTM operates in the Canister Transfer Room of the RF. Its function is to transfer waste canisters from a cask on a CTT to an aging overpack on a site transporter in the Loading Room. The ports in the floor of the Canister Transfer Room provide access to the Cask Unloading Room and Loading Room.

The CTM is an overhead bridge crane with two trolleys as shown in Figure B4.2-1. The first is a canister hoist trolley with a grapple attachment and hoisting capacity of 70 tons. The second is a shield bell trolley that supports the shield bell. The shield bell is approximately 25 feet tall with an inside diameter of about 6 feet. The bottom end of the shield bell is attached to a larger chamber to accommodate cask lids with a diameter of up to 84 in. The CTM bottom plate assembly supports a 12-in. thick motorized slide gate. The slide gate, when closed, provides bottom shielding of the canister once the canister is inside the shield bell.



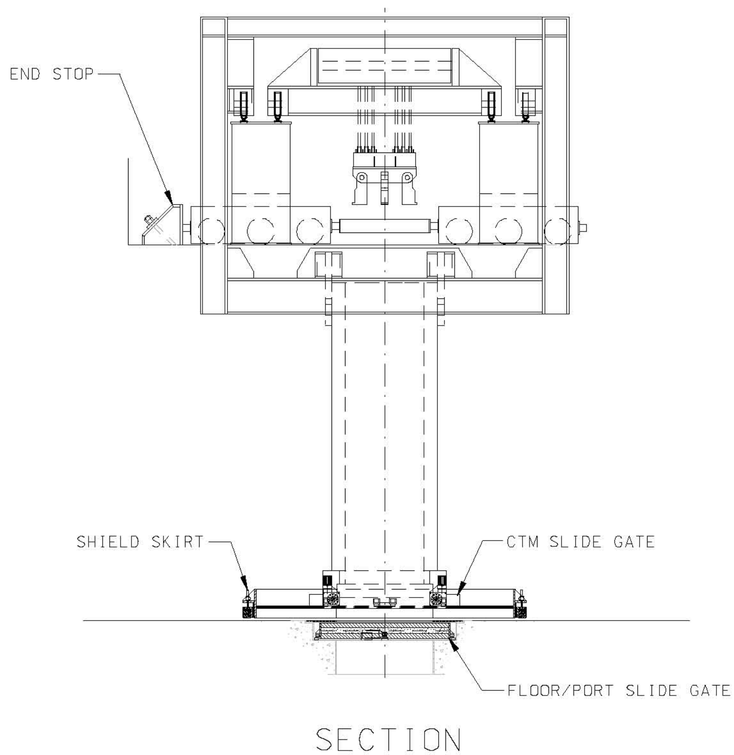
Source: Modified from Ref. B4.1.8.

Figure B4.2-1. Canister Transfer Machine Elevation

Around the perimeter of the bottom plate, a 9-in. thick shield skirt is provided which can be raised and lowered. The shield skirt is used to close any gap between the CTM bottom plate and floor surface to prevent lateral radiation shine during a canister transfer operation. The shield skirt in its lowered position is the only part of the CTM that touches the floor.

The CTM bridge is very similar to a typical crane bridge, with end trucks riding rails supported by wall corbels. Each bridge girder supports two sets of trolley rails; the two inner rails are for the canister hoist trolley and the two outer rails are for the shield bell trolley.

The CTM design allows for the two trolleys to move independently when required for maintenance but they are normally mechanically locked together and operate as a unit when performing a canister transfer operation. The hoist trolley with grapple is positioned over the shield bell and grapple center is aligned with the shield bell center as depicted in Figure B4.2-2.



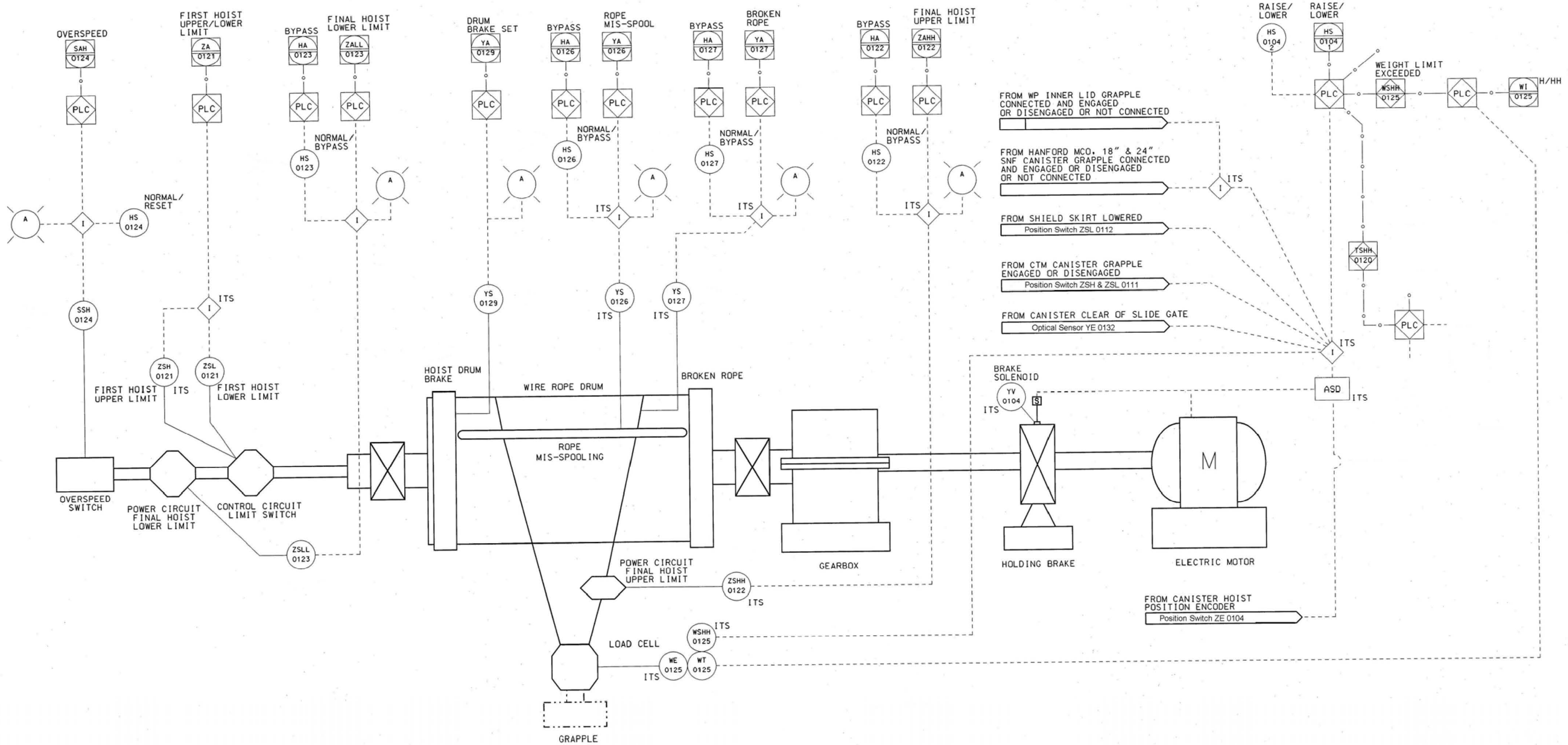
Source: Modified from Ref. B4.1.8.

Figure B4.2-2. Canister Transfer Machine Cross Section

Figures B4.2-3 through B4.2-6 show the ITS related instrumentation and controls incorporated into the CTM ((Ref. B4.1.2), (Ref. B4.1.3), and (Ref. B4.1.6)). Additional interlocks between the CTM and other systems (e.g., shield doors) are shown and described in *CRCF, RF, WHF, and IHF CTM Canister Grapple Process and Instrumentation Diagram* (Ref. B4.1.4), *Nuclear Facilities Shield Door Process and Instrumentation Diagram* (Ref. B4.1.5), and *Nuclear Facilities Slide Gate Process and Instrumentation Diagram Sheet 3* (Ref. B4.1.7). Hard-wired interlocks are provided to limit the possibility of operator error resulting in a CTM drop (of either a canister or any other object) or collision. While much of the operational controls are provided by programmable logic controllers (PLCs), the operation of these non-ITS devices are not credited in the system analysis. However, spurious operation of the PLCs are considered when such operation may contribute to a drop or collision event. Hard-wired interlocks are provided to:

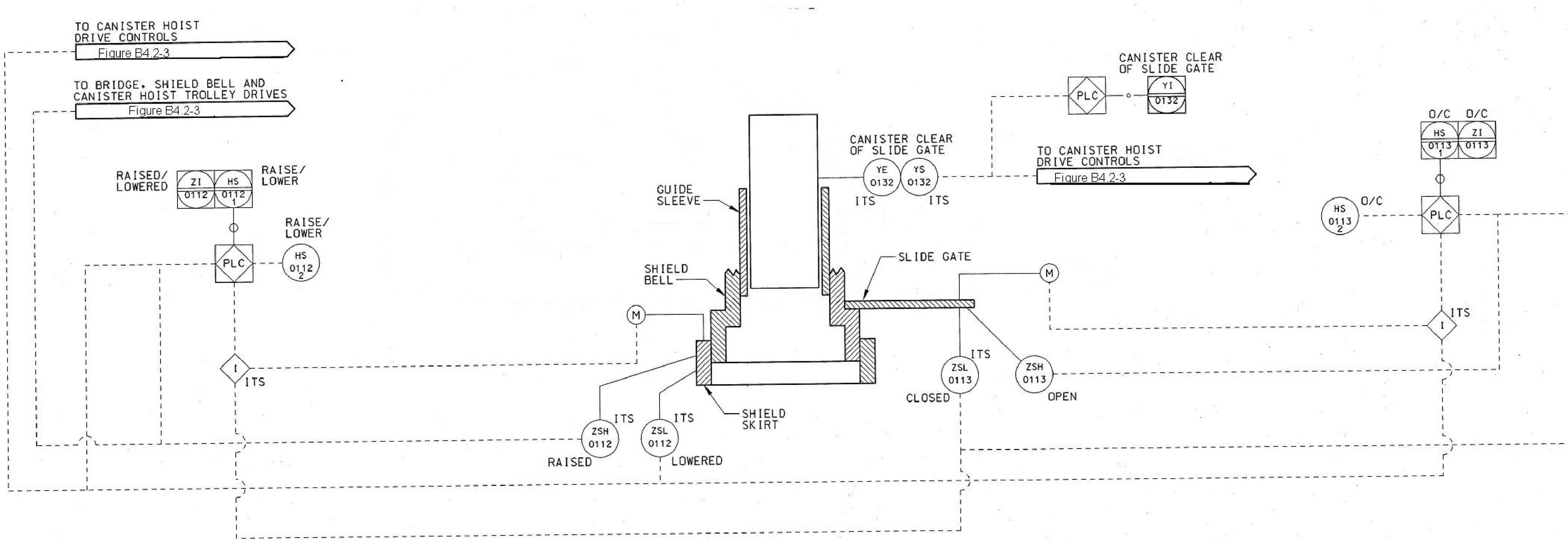
- Prevent bridge and trolley movement when the shield bell skirt is lowered.
- Prevent raising the shield bell skirt when the slide gate is open.
- Prevent hoist movement unless the grapple is fully engaged or disengaged.
- Stop the hoist and erase the lift command when a canister clears the shield bell slide gate.
- Stop a lift before upper lift heights are reached (two interlocks are provided for this function).
- Prevent opening of the port slide gate unless the shield bell skirt is lowered and in position.
- Prevent hoist movement unless the shield bell skirt is lowered.
- Prevent lifting of a load beyond the operational load limit of the CTM (load cells).

Some of these interlocks can be bypassed during maintenance. The most significant of these is the interlock between the shield skirt position and the position of the slide gate (shield skirt cannot be raised unless the slide gate is closed or the bypass is engaged). The design of the grapple interlock ensures that this interlock cannot be bypassed when the CTM is being used during operation.



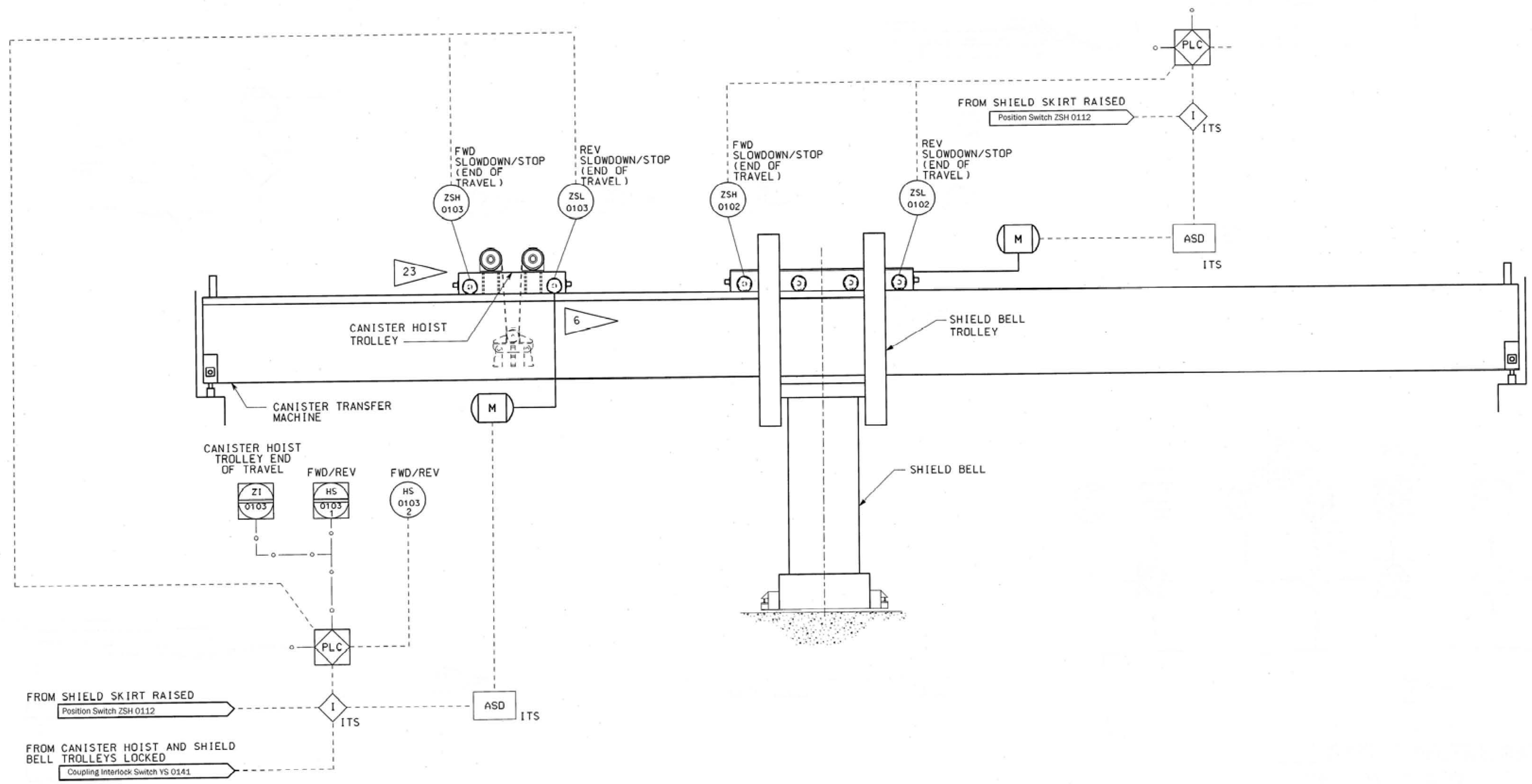
Source: Modified from Ref. B4.1.6.

Figure B4.2-3. Canister Hoist Instrumentation



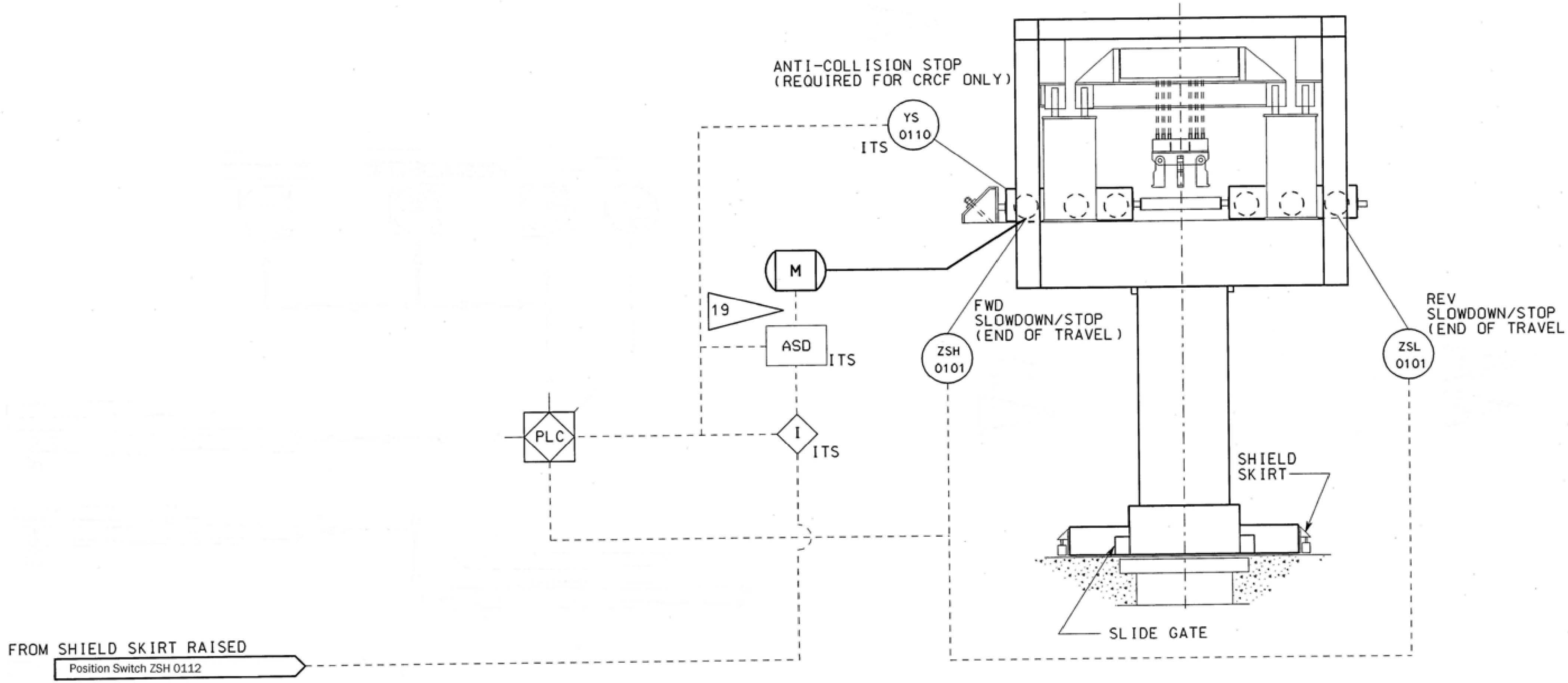
Source: Modified from Ref. B4.1.6.

Figure B4.2-4. Shield Skirt and Slide Gate Instrumentation



Source: Modified from (Ref. B4.1.3 DIRS 183763).

Figure B4.2-5. Trolley Instrumentation



Source: Modified from Ref. B4.1.3.

Figure B4.2-6. Bridge Instrumentation

B4.2.1 CTM Bridge

The bridge design meets the requirements of ASME NOG-1-2004 (Ref. B4.1.1) for a type I crane. The girder design resists the compression, bending, shear, torsion, and buckling loads induced by the fully-loaded trolley, crane dead weight, and impact loads due to seismic events. The end trucks are box section and of high strength design, minimizing deflection, and constraining horizontal crane skewing. The flame hardened wheels are attached to the end truck using wheel bearing capsules. Four seismic restraints are provided to prevent excessive horizontal and vertical uplifts.

Hoist, trolley, and bridge drive gearing are enclosed in sealed gear boxes and lubricated with oil of a high flash point, which will not support a flame and fire.

The electric power to the bridge is provided by a crane cable track system along the runway length and supported by the facility wall, as shown in Figure B4.2-1.

B4.2.2 Shield Bell Trolley

The shield bell trolley design meets the requirements of ASME NOG-1-2004 (Ref. B4.1.1) for a type I crane. During a seismic event, seismic restraints prevent the trolley from coming off the rails by limiting the amount of uplift. Electrical power to the trolley is provided through hard-wired connections using a cable track system.

B4.2.3 Canister Hoist Trolley

The hoist trolley design meets the requirements of ASME NOG-1-2004 (Ref. B4.1.1) for a type I crane and is also equipped with seismic restraints. The electrical power to the trolley is provided through hard-wired connections using a festoon system. The trolley incorporates a 70-ton hoist system that uses single-failure-proof technology. A canister grapple is supported by the lower block of the 70-ton hoist. The remotely operated grappling system utilizes limit switches to verify grapple engagement. The grapple utilizes a mechanism that includes a mechanical fail-safe drive that does not allow the grapple to disengage when a load is suspended from the canister grapple.

The hoist motor is designed to lift and lower the load at a nominal speed of 5 fpm. The hoist motor is controlled by an adjustable speed drive (ASD).

B4.2.4 ITS CTM Normal Operations

A typical CTM canister transfer operation is the transfer of a waste canister from a transportation cask to an aging overpack. For this operation a loaded transportation cask, secured in the cask transfer trolley, is positioned below the transfer port in the Cask Unloading Room. The cask lid is in place but unbolted. Similarly, an empty aging overpack secured by the site transporter is positioned under the adjacent transfer port in the Loading Room.

The CTM is moved to a position over the port above the loaded cask. The shield skirt is lowered to rest on the floor, and the port slide gate is opened. The CTM slide gate is opened and the canister grapple is lowered through the shield bell. The grapple engages a lift fixture on the cask lid. The cask lid is raised into the larger chamber of the CTM. The port slide gate is closed and the shield skirt is raised. The CTM is moved to a cask lid staging area, which is a recess in the floor of the Canister Transfer Room. The cask lid is lowered and placed in the staging area and the grapple is raised.

The CTM is moved over the port above the loaded cask, the CTM grapple is positioned and aligned for the canister pickup, and the shield skirt is lowered. The port slide gate is opened and the grapple is lowered to engage the canister lifting feature. The canister is raised into the shield bell and the hoist stops when a sensor detects that the bottom of the canister has cleared the CTM slide gate. The CTM slide gate and the port slide gate are closed, and the shield skirt is raised.

The CTM is moved to the port above the empty aging overpack and positioned for canister loading. The shield skirt is lowered and the port slide gate and CTM slide gate are opened. The canister is lowered and placed into the aging overpack and the grapple is disengaged from the canister.

The CTM canister grapple is used for handling large diameter canisters such as TAD canisters and DPCs. The CTM hoist is lowered through the shield bell until the CTM grapple is accessible in the room below for canister grapple attachment.

The CTM is normally controlled from the facility operations room, but a local control station is also provided.

B4.2.5 ITS CTM Off Normal Operations

Generally, under off normal conditions, the CTM is not in operation. Following a loss of AC offsite power, all power to the CTM motors (hoist, bridge, trolley, and bell trolley) is lost. If a transfer is underway when power is lost, all of the CTM motors would stop and the hoist holding brake engages. Operations would be suspended until power is restored and the load can be moved safely. Under other off normal conditions, transfer operations would be suspended and the CTM would remain idle.

B4.2.6 ITS CTM Testing and Maintenance

The CTM is operated, if not on a continual basis, regularly (e.g., once a shift). Most component functionality is verified during CTM operation. For those components that are not exercised during routine operations (e.g., bridge and trolley end-of-travel end stops, hoist upper limit position switches) routine verification of functionality is required.

B4.2.7 Testing and Maintenance

Requirements

Testing of components not exercised during routine operation of the CTM is performed annually at a minimum.

Features

Normal maintenance is performed in accordance with manufacture's recommendations; maintenance is performed only when the CTM is not in use.

B4.2.8 Fault Trees

Requirements

The fault tree model for the CTM only includes those components that have been declared as ITS. There is an exception: the spurious operation of PLCs is included in the fault tree model. Spurious operation can result in inadvertent CTM movements.

The mission time for the ITS CTM is set to one hour. Most lifts/transfers require less than one hour. When a transfer consists of several separate activities (e.g., auxiliary equipment movements, lifts, transfers, etc.) each of these activities require less than an hour, but all have been assigned a one hour mission time.

Features

Common-cause failures have been included for three events. Two are associated with position indication sensors: the two upper limit switches on the CTM hoist used to prevent raising a load too high (a two-blocking event), and the port gate position sensors (two gates, one sensor for each gate). Common-cause failure of the hoist cables is also considered.

Seven human error conditions are incorporated into the model. These are for drops initiated by the operator actions, inadvertent crane movements resulting in impacts, and a failure to restore interlocks allowing movement of the crane when the shield skirt is raised and the slide gates are open.

B4.3 DEPENDENCIES AND INTERACTIONS

Dependencies are broken down into five categories with respect to their interactions with systems, structures, and components. The five areas considered are addressed in Table B4.3-1 with the following dependencies:

1. Functional dependence
2. Environmental dependence
3. Spatial dependence
4. Human dependence
5. Failures based on external events.

Table B4.3-1. Dependencies and Interactions Analysis

Systems, Structures, Components	Dependencies and Interactions				
	Functional	Environmental	Spatial	Human	External Events
ASDs	Position sensors	—	—	—	—
	CTM hoist, bridge, and trolley motors control	—	—	—	—
CTM bridge	—	—	CTM bridge	—	—
CTM motors	ASDs, non-ITS power	—	—	Operational control	Off-site power
Port/slide gate position switches	ASDs	—	—	—	—
Grapple position (engaged /disengaged)	ASDs	—	—	—	—
Shield skirt position	ASDs	—	—	—	—
Non-ITS power	CTM motors	—	—	—	—
Obstruction sensor	Hoist motor ASD	—	—	—	—

NOTE: ASD = adjustable speed drive; CTM = canister transfer machine; ITS = important to safety.

Source Original

B4.4 CTM RELATED FAILURE SCENARIOS

The CTM has five credible failure scenarios:

1. The CTM drops a canister from a height below the design basis height for canister damage (this includes canister drops within the shield bell once the bell slide gate has been closed and drops through the Canister Transfer Room ports to the Loading/Cask Unloading Rooms that can occur before the bell slide gate is closed).

2. The CTM drops a canister from a height above the design basis height for canister damage.

3. The CTM drops an object onto a canister.

4. Canister impact. A collision between the canister and the shield bell or Canister Transfer Room floor from any cause during the lift, lateral movement, and lower portions of the canister transfer

5. CTM movement subjects canister to shearing forces. The CTM, while carrying a canister, moves in such a manner (e.g., spurious movements, exceeding bridge or trolley end of travel limits) as to cause an impact of the canister with the shield bell.

B4.4.1 Canister Drops from Below the Canister Design-Limit Drop Height

B4.4.1.1 Description

Transfer operations using the CTM entail the possibility of inadvertent drops of the canisters. These drops have been divided into two classes: drops from heights below the design basis drop height of the canister and drops from heights above the design basis drop height of the canister. The fault tree for canister drops addresses the first of these two scenarios.

B4.4.1.2 Success Criteria

The success criterion for the CTM is the prevention of a canister drop from any cause during the lift, lateral movement, and lowering portions of the canister transfer.

B4.4.1.3 Design Requirements and Features

Requirements

Hard-wired interlocks are used to prevent inadvertent actions during CTM transfer operations. These include the following:

- An optical sensor at the bottom of the shield bell that, once it is cleared, stops the hoist and erases the lift command (can only lower hoist). This interlock is used only when lifting a canister.
- Above the ASD stop point is an upper limit switch which, when reached, stops the hoist from lifting. This first limit switch (first hoist upper limit) effectively erases the lift command (the hoist still has power) and the operator can only lower the hoist. Roughly one foot above that limit switch is another limit switch (final hoist upper limit) that, when reached, cuts off the power to the CTM hoist.
- An interlock between the shield skirt and port gate which requires the shield skirt to be lowered in order for the port gate to open. There is a bypass for this interlock.
- An interlock between the CTM bridge/trolley travel and shield skirt position. Neither the CTM bridge nor the trolley can travel while the skirt is lowered.
- An interlock between the slide gate and shield skirt – the shield skirt cannot be raised unless the slide gate is closed. This interlock can be bypassed to allow the CTM to move with the slide gate open during lid removal.
- Interlocks preventing improper hoist movement. The hoist cannot move unless the shield skirt is lowered. This interlock is based on hoist movement, not position, so movement with the hoist too low is not precluded.
- The load cells which cut off power to the hoist when the crane capacity is exceeded.

- An interlock between the grapple position (fully engaged or fully disengaged) and hoist movement. The grapple automatically engages/disengages with a given object. The grapple must be positively engaged for the grapple engagement indicator to give a positive indication.

Features

Bridge and trolley motors are sized to limit lateral travel to less than 20 fpm, sufficient to ensure that in the event of an impact, impact forces are below the design limits of the canister.

The shield bell slide gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

The floor port gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

Hard-wired interlocks used to prevent inadvertent actions during CTM transfer operations are ITS; PLCs are not ITS equipment.

The end stops for both the bridge and trolley end of travel end are capable of stopping the bridge/trolley at their maximum speed and preclude impact with any permanent structure.

The interlock between the grapple position and the operation of the hoist motor cannot be bypassed during CTM canister transfer operations.

B4.4.1.4 Fault Tree Model

The top event in this fault tree is “CTM Drop All Heights”. This is defined as a drop of a canister during transfer operations. Faults considered in the evaluation of this top event include: human events that contribute to a drop (considered in conjunction with the interlocks intended to prevent the erroneous human action) and mechanical (structural) failures of the CTM components (Figures B4.4-3 to B4.4-15). The interlocks and safety features (position controls, load cells, and drum and holding brakes) intended to either prevent CTM failure or given failure of the CTM to prevent a load drop are included in the model.

Structural failures of components, including the hoist cables, sheaves, drum, and grapples, can result in canister drops. Operator events are addressed for actions, including improper grapple connections, misalignments of the hoist and the canister, improper hoist activities, and improper lateral movement of the CTM. Protection from these actions are provided by hard-wired interlocks keyed to the position of the CTM (both hoist position and CTM lateral position), slide and port gate doors, and the shield bell skirt. Also considered in the analysis is a canister drop initiated by improper operation of the shield bell slide gates and the port slide gates. While the gate motors are sized to prevent damage to the canister in the event of an inadvertent closure of the gates, the possibility that the gates would close above the canister during a lift blocking the lift and causing a canister drop was considered.

Failures specifically considered are:

- Electro-mechanical failures that occur as a result of the random catastrophic failure of hoisting components, such as the grapple of the canister transfer machine, or the redundant wire ropes failing independently, or by common-cause.

- Electro-mechanical failures that occur as a result of the conveyance, from which the canister is being extracted, moving spuriously during the transfer. In response, a misalignment can develop that may result in the canister getting caught on the edge of the shield bell; tension can develop in the wire ropes, conceivably leading to their failure. A load control safety system is capable of detecting such abnormal tension and reacts by stopping the transfer operations and applying brakes to retain the canister in a safe position. Failure of this system is considered to cause the drop of the canister.

- Electro-mechanical failures that occur as a result of a slide gate spuriously closing during transfer of a canister. There are two types of slide gates: one that closes the port between the lower and the upper floor in the Canister Transfer Room, and another that closes the bottom part of the shield transfer bell. When the canister is lifted from its container, a spurious slide gate closure can result in the canister getting caught up against the gate; tension can develop in the wire ropes, conceivably leading to their failure. The load control safety system detects such abnormal tension and reacts by stopping the transfer operations and applying brakes to retain the canister in a safe position. Failure of this system is considered to cause the drop of the canister.

- Electro-mechanical failures that occur as a result of a spurious movement of the canister transfer machine. The canister transfer machine has several trolleys that govern lateral movements, one controls the movement of the CTM bridge, one controls the movement of the shield bell, while another one controls the movement of the load being transferred inside the shield bell (these last two are physically locked together during transfer operations. Interlocks ensure coordination between the trolley movements. Spurious actuation of a trolley motor after the grapple is attached to a canister but before the canister is raised above the Canister Transfer Room floor can result in tension developing in the wire ropes, conceivably leading to their failure. Because the load control safety system does not control lateral movements of the canister transfer machine, it is not capable of stopping operations in this case.

- Human related actions associated with the operator inappropriately closing a slide gate during vertical canister movement. As for the spurious electro-mechanical slide gate closure discussed previously, tension in the wire ropes can develop as a result of this event, conceivably leading to their failure. The load control safety system detects such abnormal tension and reacts by stopping the transfer operations and applying brakes to retain the canister in a safe position. Failure of this system is considered to cause the drop of the canister. The human error probability assigned to this human failure event is a screening value of 0.001 (i.e., it is a conservative estimate based upon predetermined characteristics of the human failure event) (Table 6.4-1).

- Human related actions associated with the operator causing a drop of a canister, from a low height, during its extraction from its container. The human error probability for this event required a detailed analysis, entailing an examination of human failure scenarios that account for interactions and error-forcing context resulting from the combination of equipment conditions and human factor. The result of this analysis was condensed into a single basic event whose probability embeds the combination of both human and equipment failures necessary to cause a drop, which explains its relatively low value (5×10^{-7}) (Table 6.4-1).

B4.4.1.5 Basic Event Data

Table B4.4-1 contains a list of basic events used in the "CTM Canister Drop from Below Canister Drop Height" fault trees. Included are the human failure events and the CCF events identified in the previous two sections. There are no maintenance-related failures associated with the CTM. The CTM is not in service while undergoing maintenance. Sensor failures that could be associated with the failure to restore from maintenance are not expected to contribute significantly to the overall sensor availability.

The canister drop probability modeled by the fault tree is evaluated over a mission time of one hour. This mission time encompasses vertical lifting, lateral movement, and vertical lowering of the canister by the canister transfer machine. A longer mission time is also considered for specific components. For example, the fault tree accounts for the failure of standby components whose potential malfunction would remain hidden until they are put into operation. They are consequently evaluated over the interval of time between their actuation, considered to be the duration of a shift, (i.e., eight hours). In another example, brakes are also analyzed over a mission time of eight hours. This duration is deemed to encompass the time required to revert to normal transfer operations after a malfunction that would have caused a safety system of the canister transfer machine to cease transfer activities.

Table B4.4-1. Basic Event Probability for the CTM Canister Drop from Below Canister Drop Height Limit Fault Tree

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200--DRUM001-DM--FOD	CTM Drum Failure on Demand	1	4.000E-08	4.000E-08	0.000E+00	0.000E+00
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--CBL0001-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0002-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	3	9.400E-08	9.400E-08	9.400E-08	0.000E+00
200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--HOISTMT-MOE-FTR	CTM Hoist Motor (Electric) Fails to Run	3	6.500E-06	0.000E+00	6.500E-06	1.000E+00
200-CTM--HOLDBRK-BRK-FOD	Brake Failure on Demand	1	1.460E-06	1.460E-06	0.000E+00	0.000E+00
200-CTM--HOLDBRK-BRK-FOH	CTM Holding Brake (Electric) Failure to hold	3	3.520E-05	0.000E+00	4.400E-06	8.000E+00
200-CTM--IMEC125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--IMEC125-ZS-FOD	CTM Load Cell Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--LOWERBL-BLK-FOD	CTM Lower Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--MISSPOOL-DM-MSP	CTM Mis-spool event spool event	3	6.860E-07	0.000E+00	6.860E-07	0.000E+00
200-CTM--OVERSP--ZS--FOD	CTM Hoist motor speed Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--PORTGT1-MOE-SPO	Spurious port gate1 motor operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--PORTGT1-PLC-SPO	Port Gage PCL Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM--PORTGT2-MOE-SPO	Port Gate Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--PORTGT2-PLC-SPO	Port Gage PCL Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM--SLIDEGT-MOE-SPO	CTM Slide Gate Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--SLIDEGT-PLC-SPO	CTM Slide Gate PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM--SLIDGT2-IEL-FOD	CTM Slide Gate Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--TROLLY-MOE-SPO	CTM Trolley Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--UPPERBL-BLK-FOD	CTM Upper Sheaves failure	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	1	3.990E-03	3.990E-03	0.000E+00	0.000E+00
200-CTM--WTSW125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--WTSW125-ZS--FOD	CTM Load Cell Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00

Table B4.4-1. Basic Event Probability for the CTM Canister Drop from Below Canister Drop Height Limit Fault Tree (Continued)

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM--YS01129-ZS--FOD	CTM Drum Brake control circuit Limit Switch 1129 Failure	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--ZSH0111-ZS--SPO	CTM grapple engaged Limit Switch Spurious Operation	3	1.280E-06	0.000E+00	1.280E-06	1.000E+00
200-CTM-ASD0122#-CTL-FOD	CTM Hoist ASD Controller fails	1	2.030E-03	2.030E-03	0.000E+00	8.000E+00
200-CTM-BRDGEMTR-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	0.000E+00
200-CTM-DRTM-CT-FOD	CTM Drive Train Protection and Fail Det. Controller Failure	1	4.000E-06	4.000E-06	0.000E+00	0.000E+00
200-CTM-DRUMBRK-BRP-FOH	CTM Drum Brake (Pneumatic) Failure to Hold	3	6.704E-05	0.000E+00	8.380E-06	8.000E+00
200-CTM-HSTTRLLY-MOE-SPO	Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM-SBELTRLY-MOE-SPO	Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	0.000E+00
200-CTM-SLIDGT2-SRX-FOD	CTM slide Gate Position Sensor Fails on Demand	1	1.100E-03	1.100E-03	0.000E+00	0.000E+00
200-CTM-ZSL0111-ZS--SPO	CTM Grapple engaged Limit Switch Spurious Operation	3	1.280E-06	0.000E+00	1.280E-06	0.000E+00
200-DRUMBRK-BRP-FOH	CTM Drum Brake (Pneumatic) Failure on Demand	3	8.380E-06	0.000E+00	8.380E-06	0.000E+00
200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1	1.000E-03	1.000E-03	0.000E+00	0.000E+00
200-OPCTMDROP002-HFI-COD	Operator causes drop of less than design height limit	1	5.000E-07	5.000E-07	0.000E+00	0.000E+00
200CTM-PLC0101#-PLC-SPO	CTM Bridge Motor PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200CTM-PLC0102#-PLC-SPO	CTM Shield Bell Trolley PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200CTM-PLC0103#-PCL-SPO	CTM Hoist Trolley PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200--DRUM001-DM--FOD	CTM Drum Failure on Demand	1	4.000E-08	4.000E-08	0.000E+00	0.000E+00
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--CBL0001-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0002-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	3	9.400E-08	9.400E-08	9.400E-08	0.000E+00
200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--HOISTMT-MOE-FTR	CTM Hoist Motor (Electric) Fails to Run	3	6.500E-06	0.000E+00	6.500E-06	1.000E+00

Table B4.4-1. Basic Event Probability for the CTM Canister Drop from Below Canister Drop Height Limit Fault Tree (Continued)

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM--HOLDBRK-BRK-FOD	Brake Failure on Demand	1	1.460E-06	1.460E-06	0.000E+00	0.000E+00
200-CTM--HOLDBRK-BRK-FOH	CTM Holding Brake (Electric) Failure to hold	3	3.520E-05	0.000E+00	4.400E-06	8.000E+00
200-CTM--IMEC125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--IMEC125-ZS-FOD	CTM Load Cell Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--LOWERBL-BLK-FOD	CTM Lower Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
ASD = adjustable speed drive; Calc. = calculation; CCF = common-cause failure; CTM = canister transfer machine; CTT = cask transfer trolley; Fail. = failure; Miss. = mission; PLC = programmable logic controller; Prob. = probability.

Source: Original

B4.4.1.5.1 Human Failure Events

Two basic events are associated with human error (Table B4.4-2). These are for drops initiated by the operator actions and an operator action to close the shield or slide gate doors while a CTM lift is being performed.

Table B4.4-2. Human Failure Events

Name	Description
200-OPCTMDROP002-HFI-COD	Operator causes drop of less than design height limit
200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close

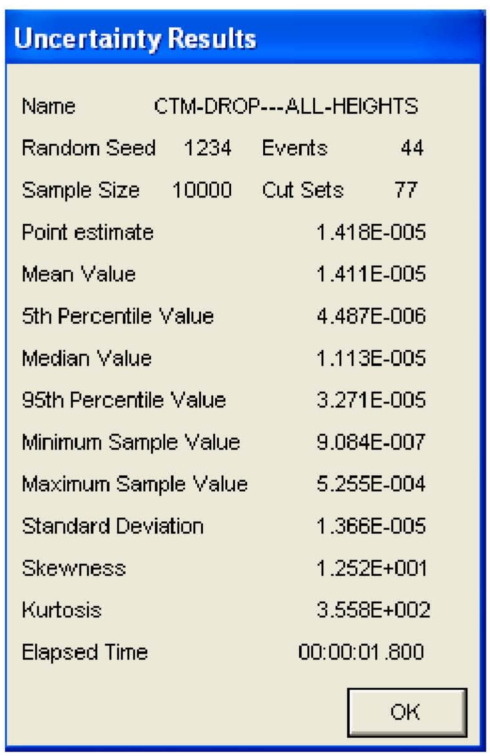
Source: Original

B4.4.1.5.2 Common-Cause Failures

One CCF event considered in the evaluation of this top event is the CCF of the hoist cables.

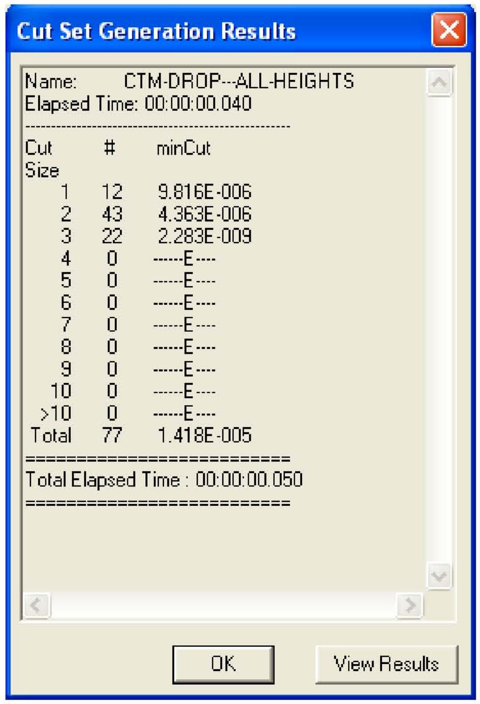
B4.4.1.6 Uncertainty and Cut Set Generation

Figure B4.4-1 contains the uncertainty results obtaining from running the fault trees for the CTM canister drop with a cutoff probability of 1E-15. Figure B4.4-2 provides the cut set generation results for the CTM canister drop fault tree.



Source: Original

Figure B4.4-1. Uncertainty Results of the CTM Canister Drop Fault Tree



Source: Original

Figure B4.4-2. Cut Set Generation Results for the CTM Canister Drop Fault Tree

B4.4.1.7 Cut Sets

Table B4.4-3 contains the top 20 cut sets for the CTM Canister Drop Fault Tree.

Table B4.4-3. Dominant Cut Sets for the CTM Canister Drop

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
28.14	28.14	3.990E-06	200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-3
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-3
37.17	9.03	1.280E-06	200-CTM--ZSH0111-ZS--SPO	CTM grapple engaged Limit Switch Spurious Operation	1.280E-6
46.20	9.03	1.280E-06	200-CTM-ZSL0111-ZS--SPO	CTM Grapple engaged Limit Switch Spurious Operation	1.280E-6
54.31	8.11	1.150E-06	200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1.150E-6
62.42	8.11	1.150E-06	200-CTM--UPPERBL-BLK-FOD	CTM Upper Sheaves failure	1.150E-6
70.53	8.11	1.150E-06	200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1.150E-6
78.64	8.11	1.150E-06	200-CTM--LOWERBL-BLK-FOD	CTM Lower Sheaves Failure on Demand	1.150E-6
83.39	4.75	6.740E-07	200-CTM-BRDGEMTR-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation	6.740E-7

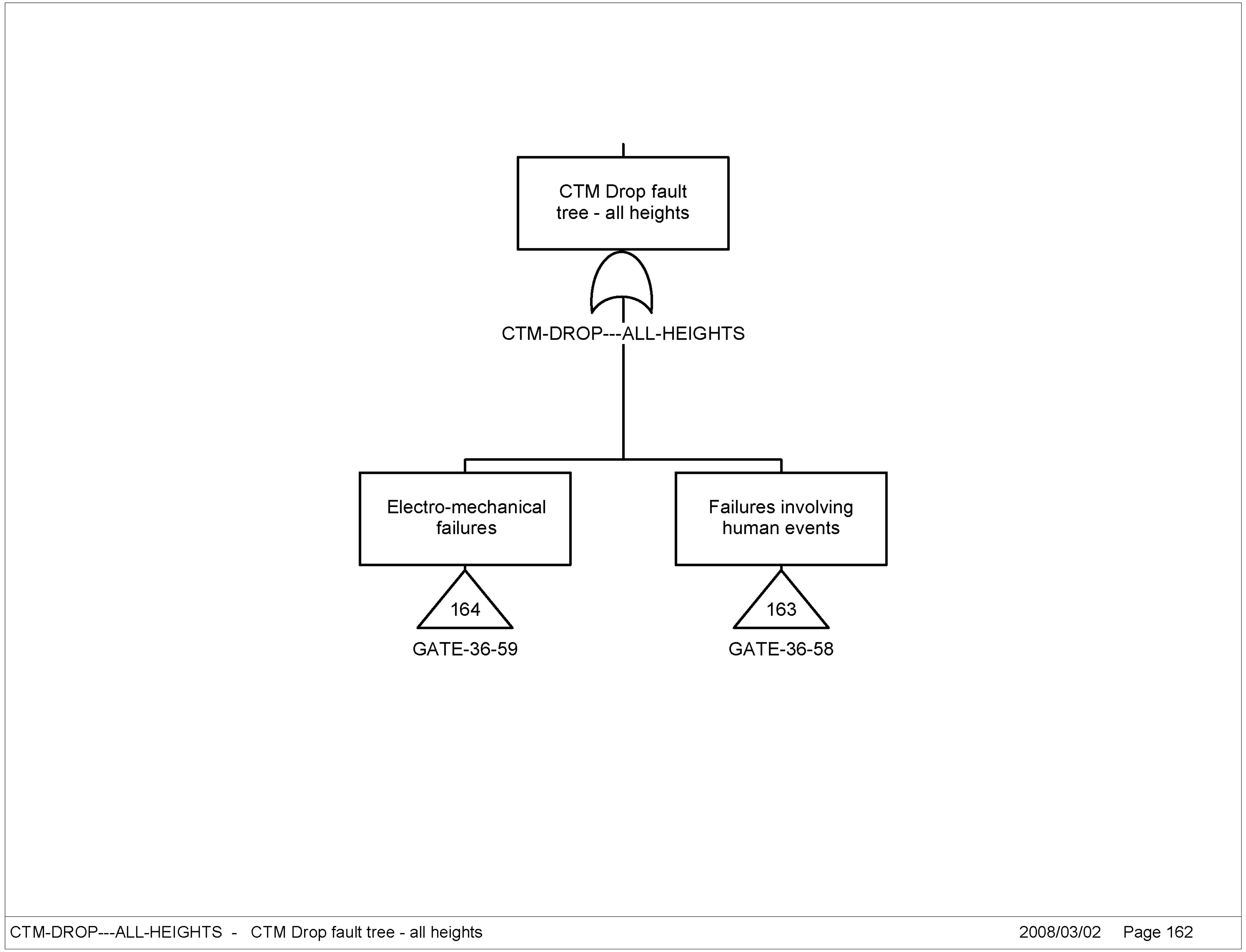
Table B4.4-3. Dominant Cut sets for the CTM Canister Drop (Continued)

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
88.14	4.75	6.740E-07	200-CTM-HSTTRLLY-MOE-SPO	Motor (Electric) Spurious Operation	6.740E-7
92.89	4.75	6.740E-07	200-CTM-SBELTRLY-MOE-SPO	Motor (Electric) Spurious Operation	6.740E-7
96.42	3.53	5.000E-07	200-OPCTMDROP002-HFI-COD	Operator causes drop of less than design height limit	5.000E-7
98.49	2.07	2.930E-07	200-CTM--WTSW125-ZS--FOD	CTM Load Cell Limit Switch Failure on Demand	2.930E-4
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-3
99.15	0.66	9.400E-08	200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	9.400E-8
99.43	0.28	4.000E-08	200--DRUM001-DM--FOD	CTM Drum Failure on Demand	4.000E-8
99.68	0.25	3.520E-08	200-CTM--HOLDBRK-BRK-FOH	CTM Holding Brake (Electric) Failure to hold	3.520E-5
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-3
99.87	0.19	2.750E-08	200-CTM--IMEC125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	2.750E-5
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-3
99.89	0.02	2.689E-09	200-CTM--PORTGT1-MOE-SPO	Spurious port gate1 motor operation	6.740E-7
			200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-3
99.91	0.02	2.689E-09	200-CTM--PORTGT2-MOE-SPO	Port Gate Motor (Electric) Spurious Operation	6.740E-7
			200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-3
99.93	0.02	2.689E-09	200-CTM--SLIDEGT-MOE-SPO	CTM Slide Gate Motor (Electric) Spurious Operation	6.740E-7
			200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-3
99.95	0.02	2.689E-09	200-CTM--TROLLY-MOE-SPO	CTM Trolley Motor (Electric) Spurious Operation	6.740E-7
			200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-3
28.14	28.14	3.990E-06	200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-3

NOTE: Calc. = calculation; CCF = common-cause failure; CTM = canister transfer machine; CTT = cask transfer trolley; Fail. = failure; Miss. = mission; Prob. = probability.

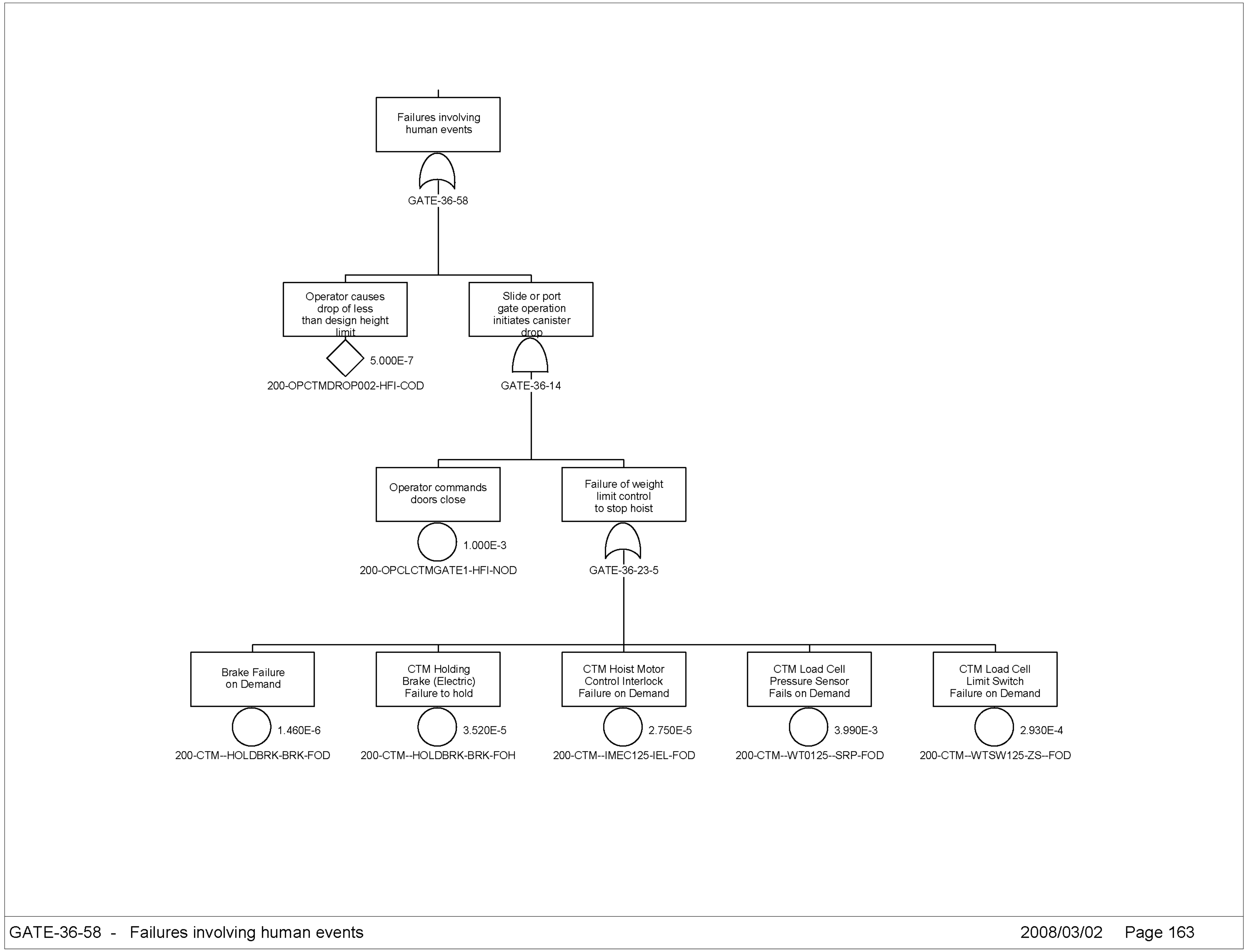
Source: Original

B4.4.1.8 Fault Trees



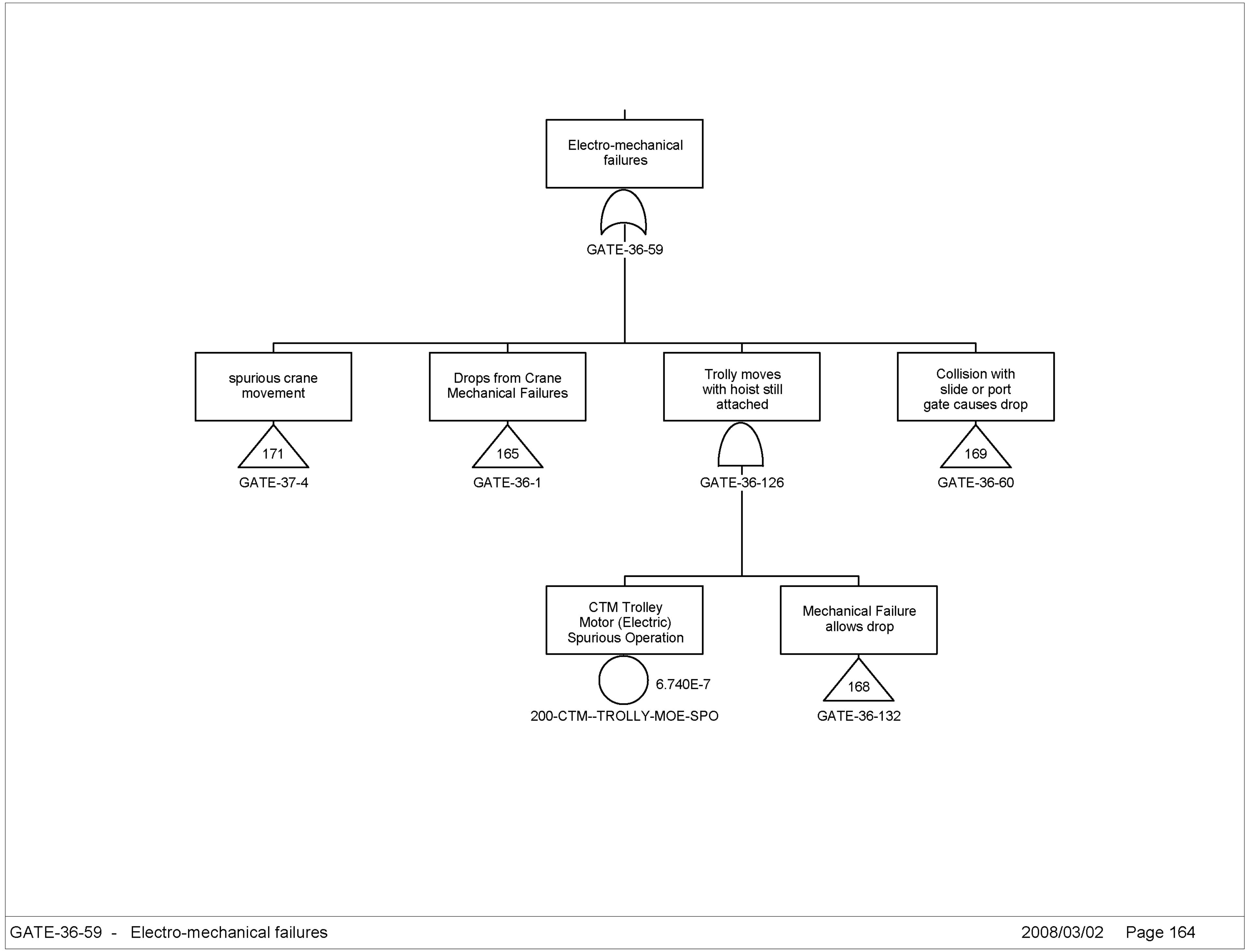
Source: Original

Figure B4.4-3. CTM Drop Fault Tree Sheet 1



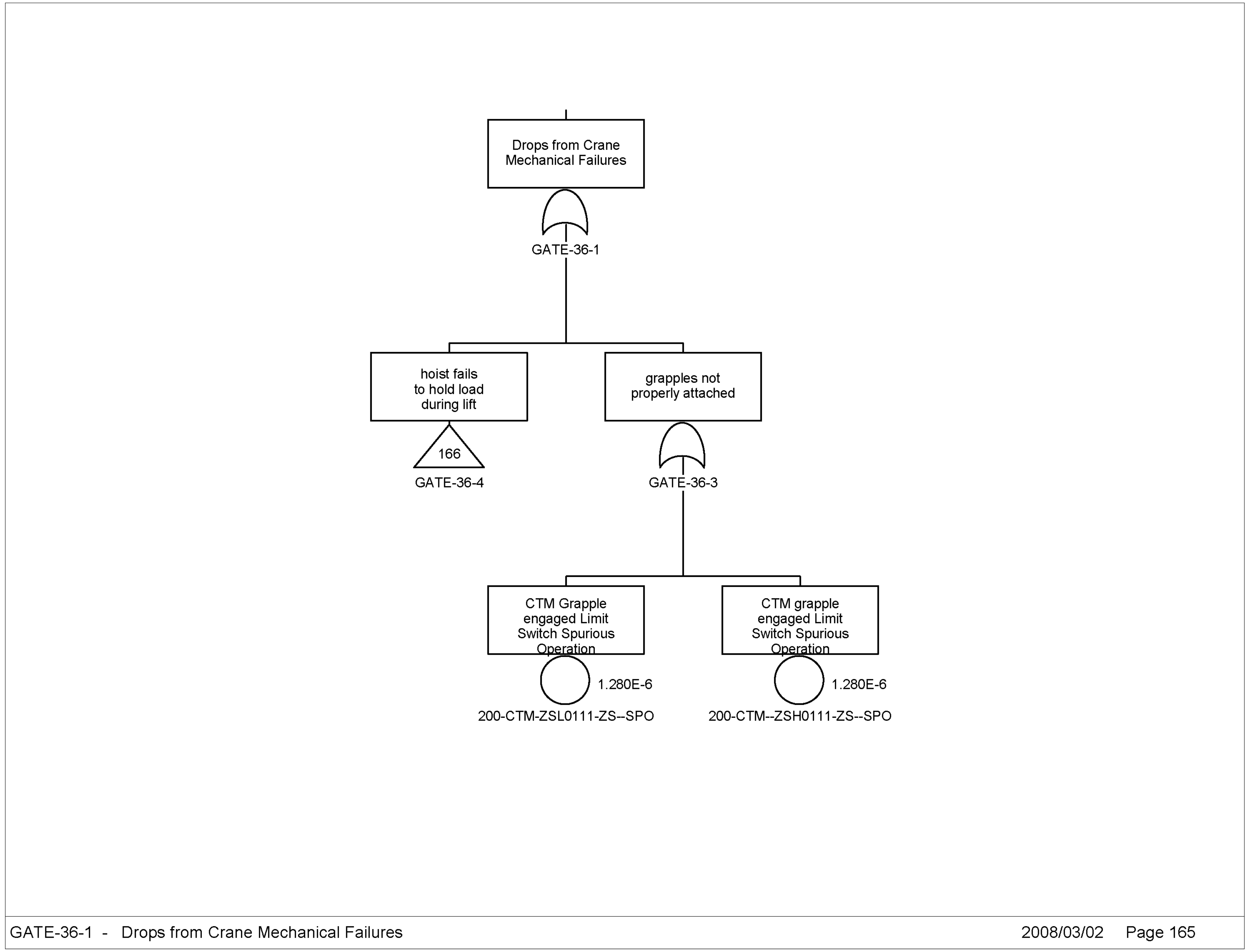
Source: Original

Figure B4.4-4. CTM Drop Fault Tree Sheet 2



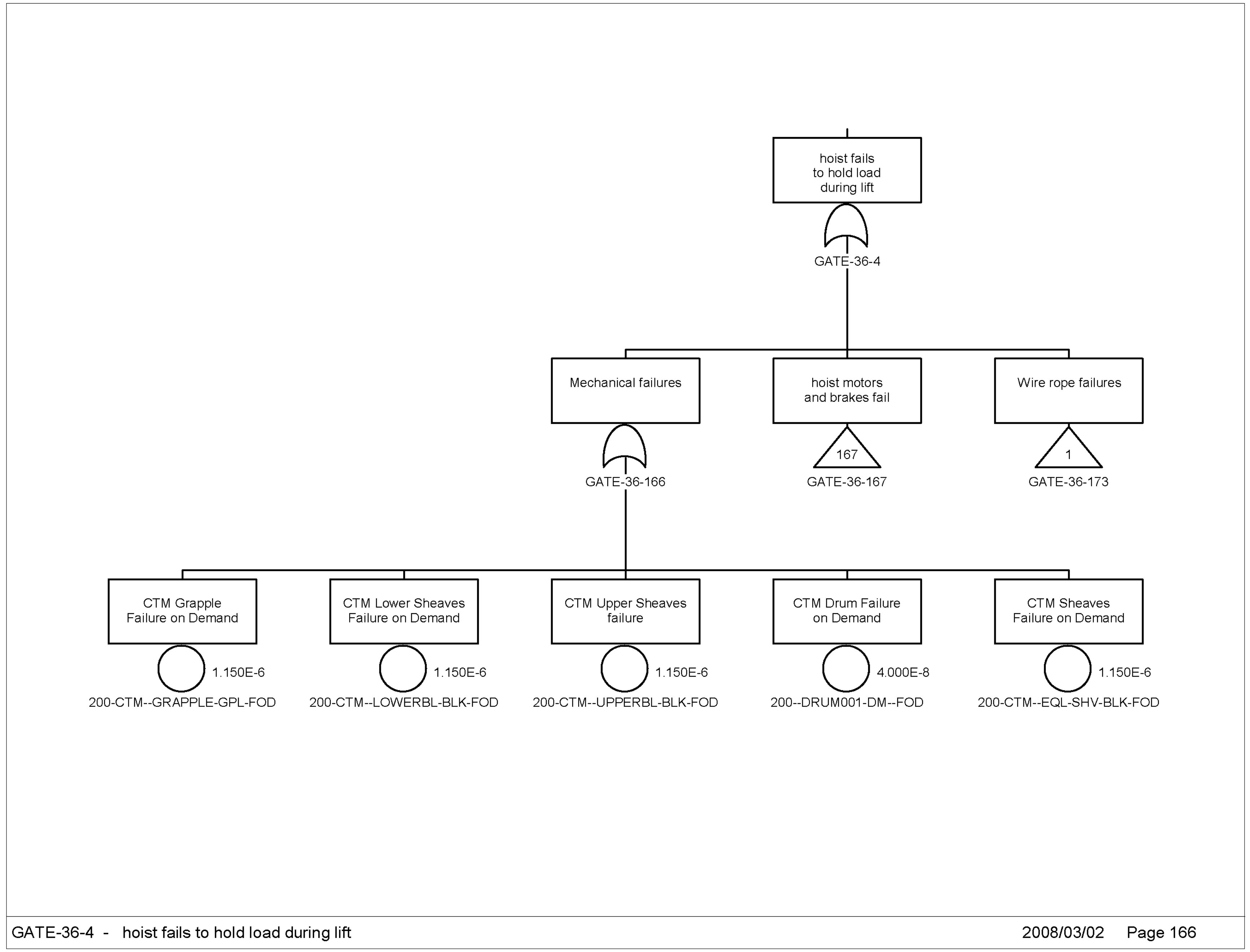
Source: Original

Figure B4.4-5. CTM Drop Fault Tree Sheet 3



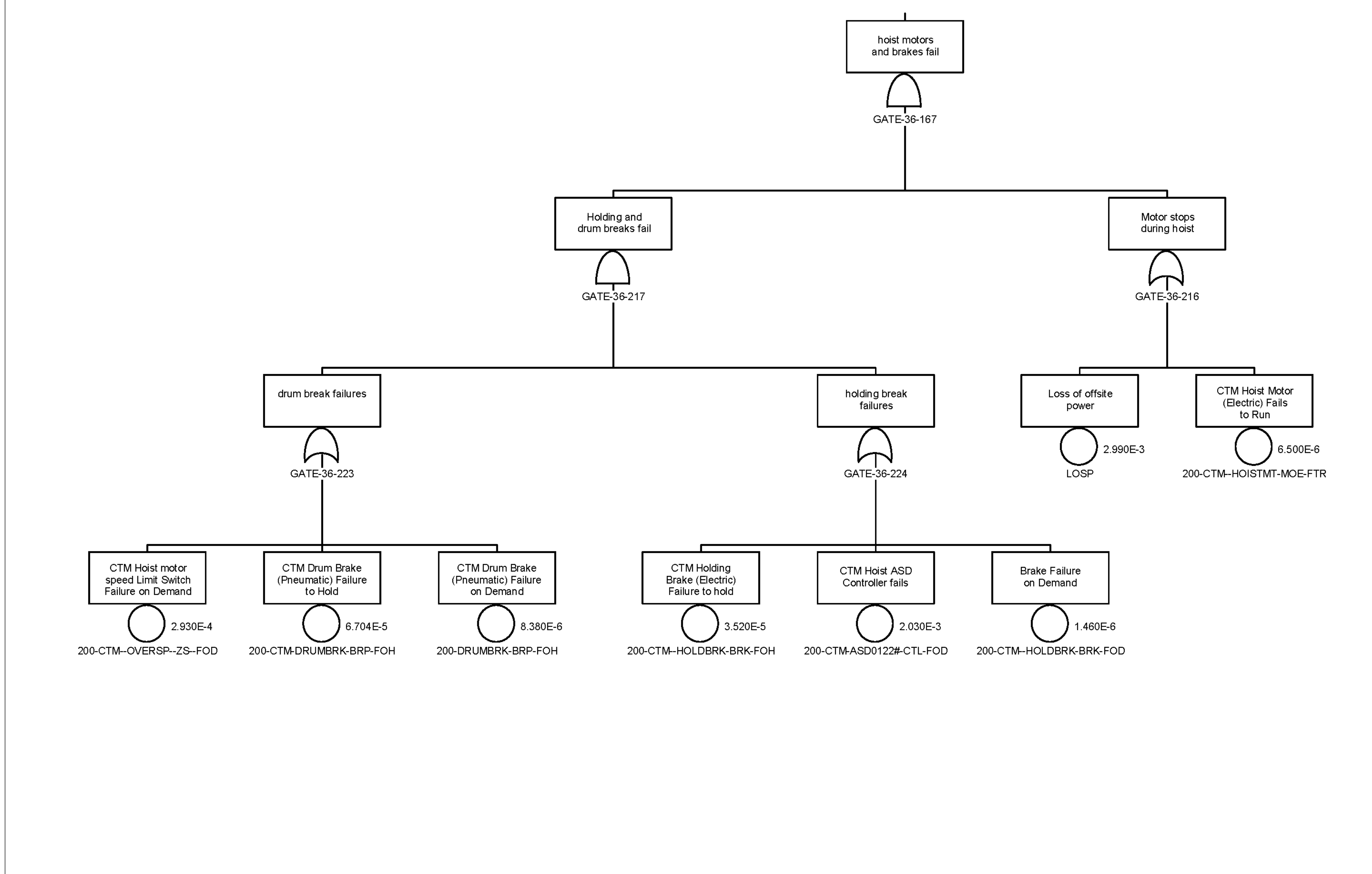
Source: Original

Figure B4.4-6. CTM Drop Fault Tree Sheet 4



Source: Original

Figure B4.4-7. CTM Drop Fault Tree Sheet 5



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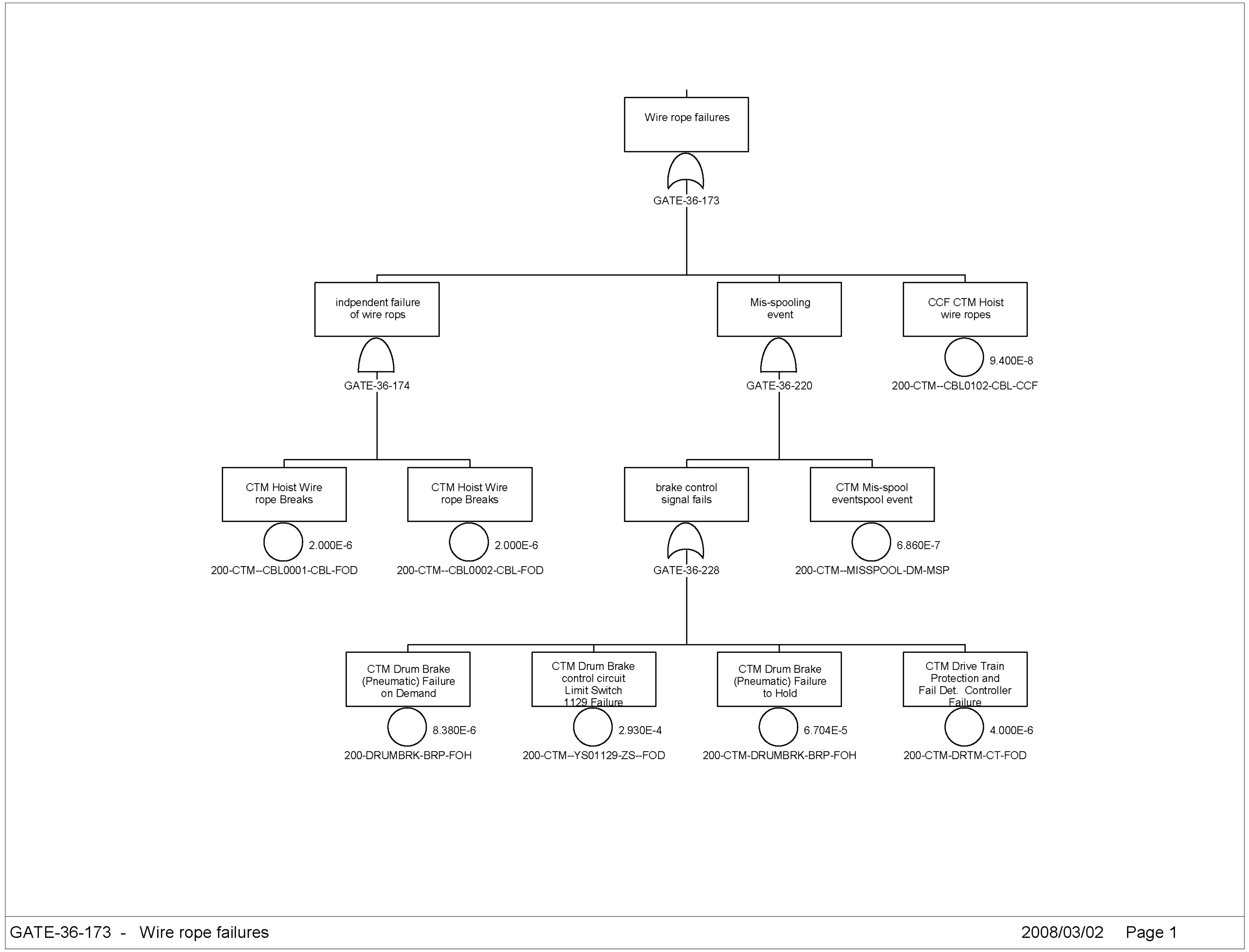
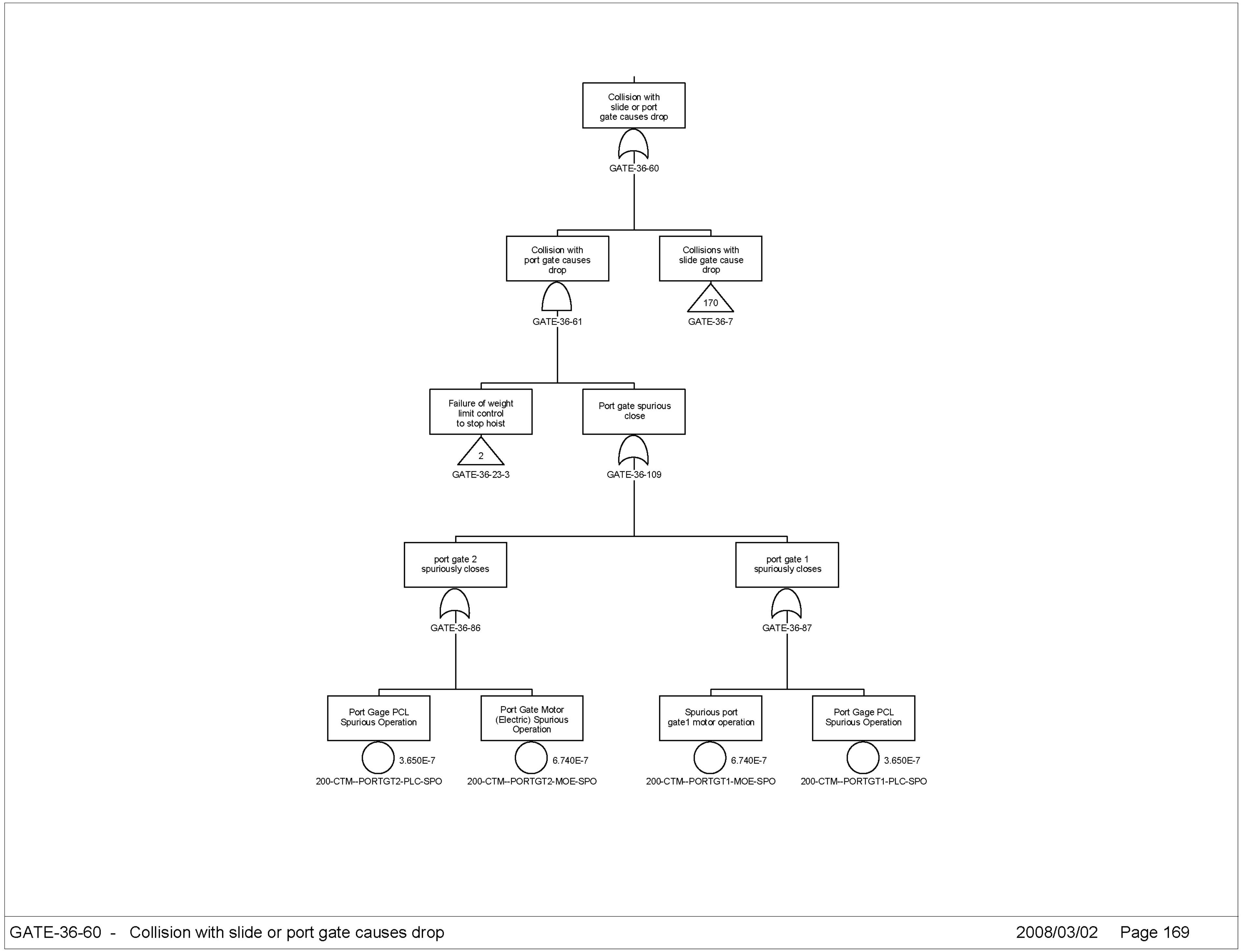
Source: Original

Figure B4.4-8. CTM Drop Fault Tree Sheet 6



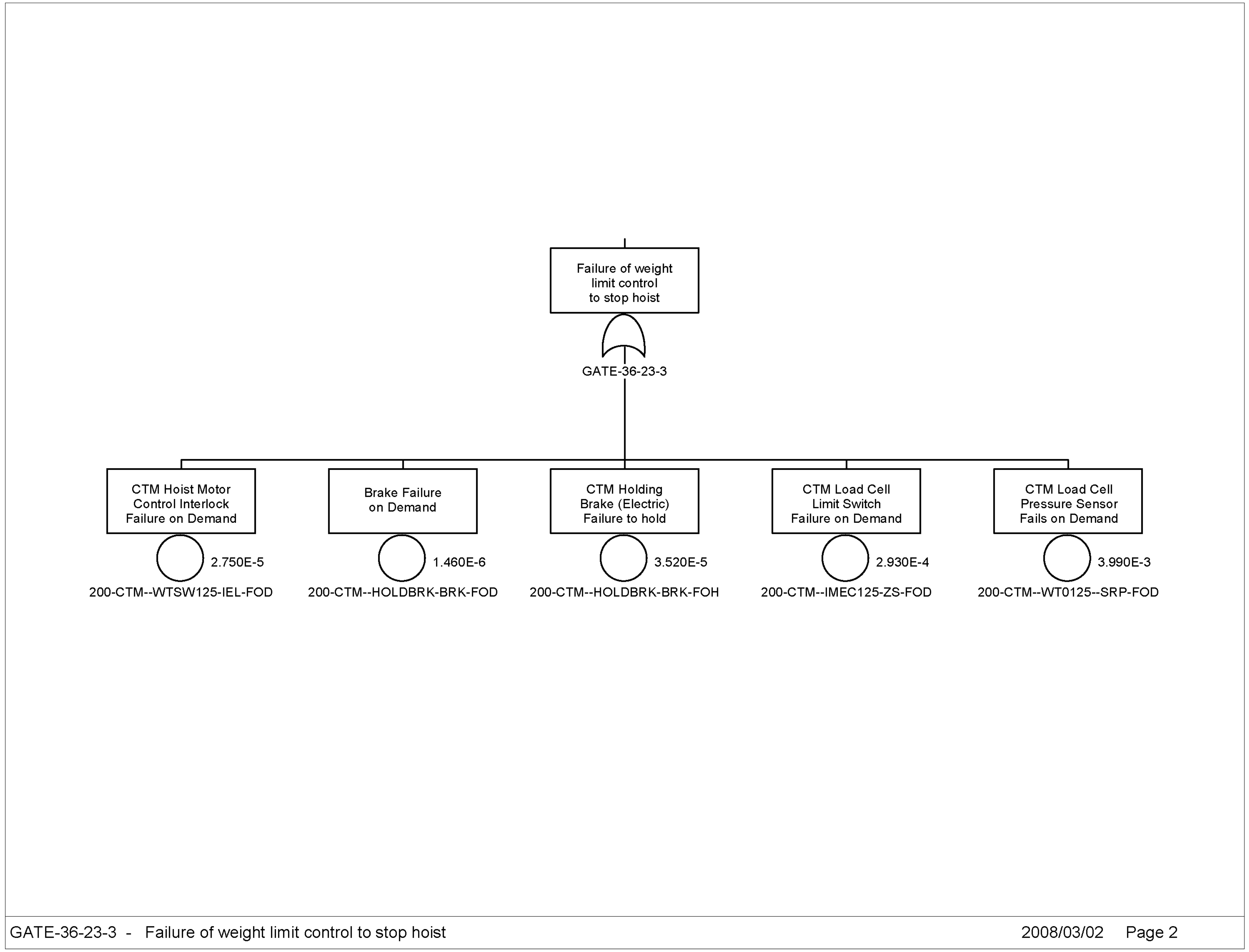
Source: Original

Figure B4.4-9. CTM Drop Fault Tree Sheet 7



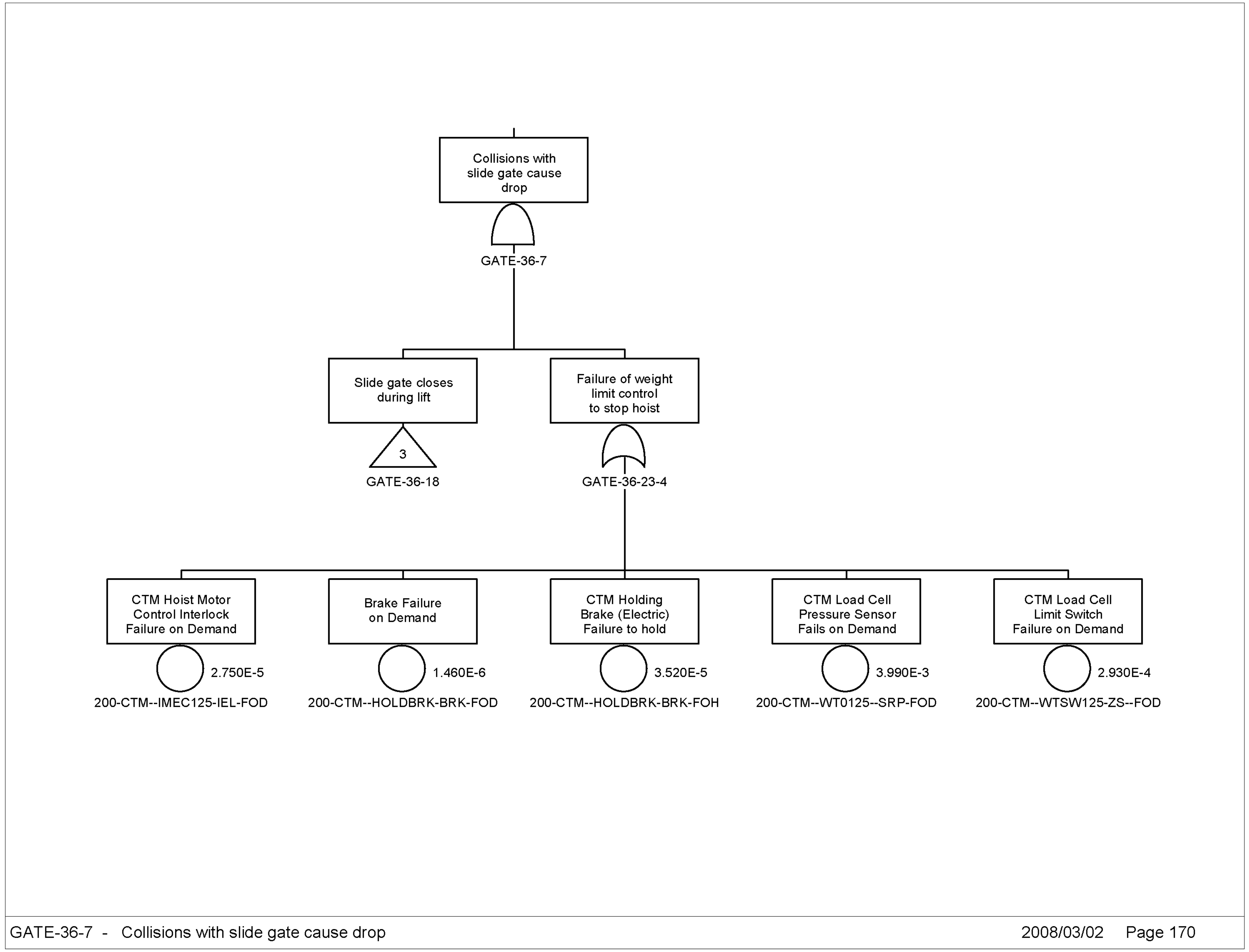
Source: Original

Figure B4.4-10. CTM Drop Fault Tree Sheet 8



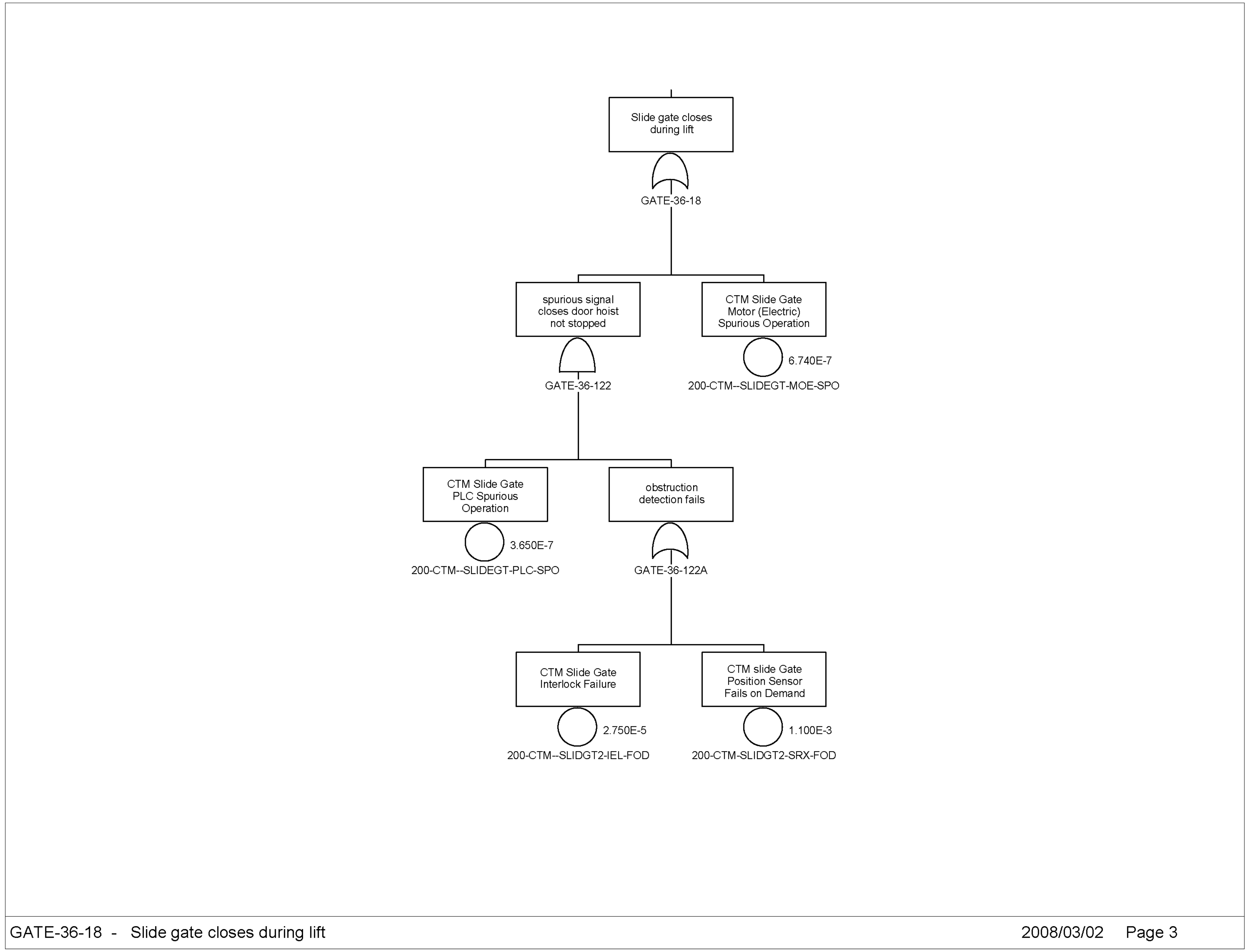
Source: Original

Figure B4.4-11. CTM Drop Fault Tree Sheet 9



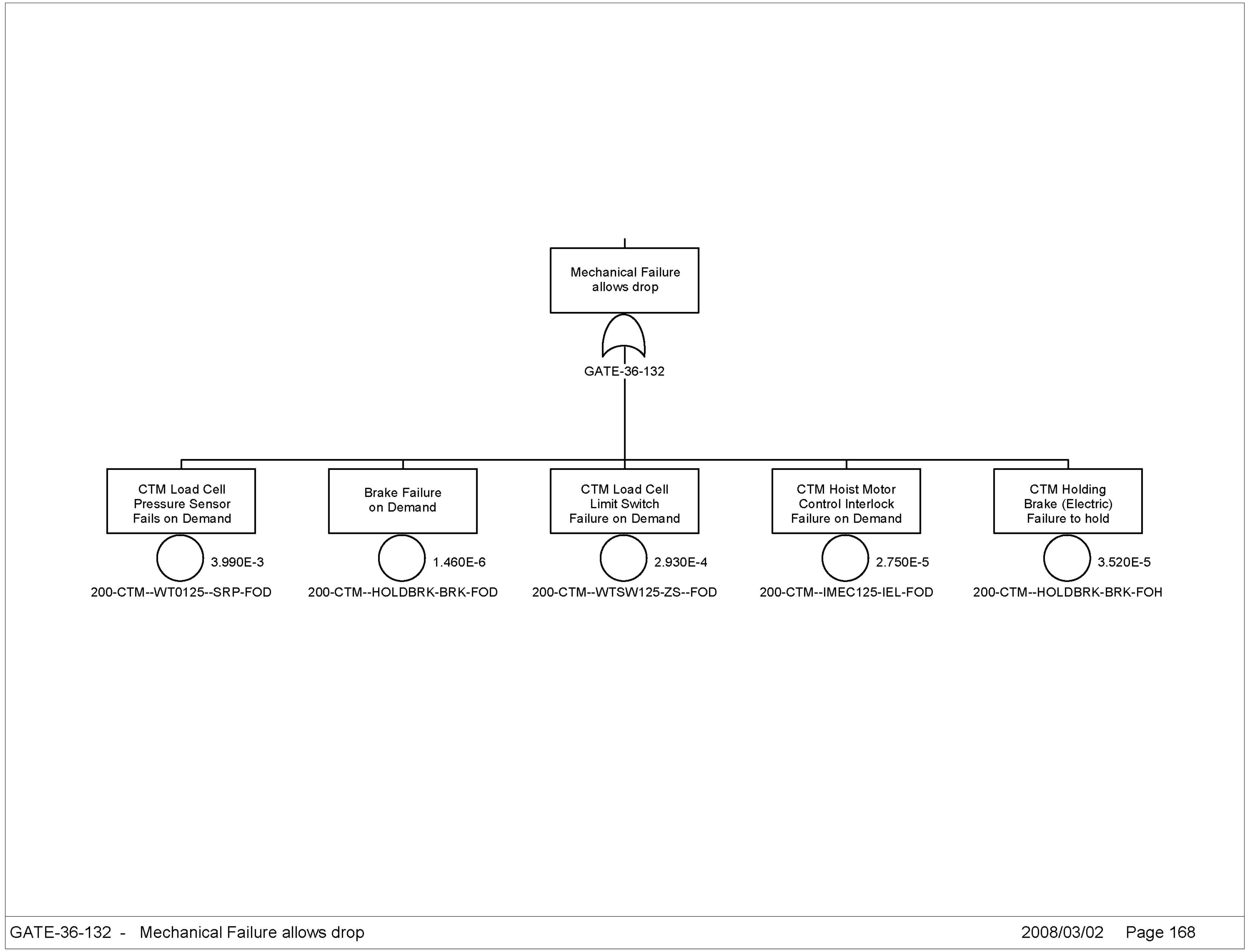
Source: Original

Figure B4.4-12. CTM Drop Fault Tree Sheet 10



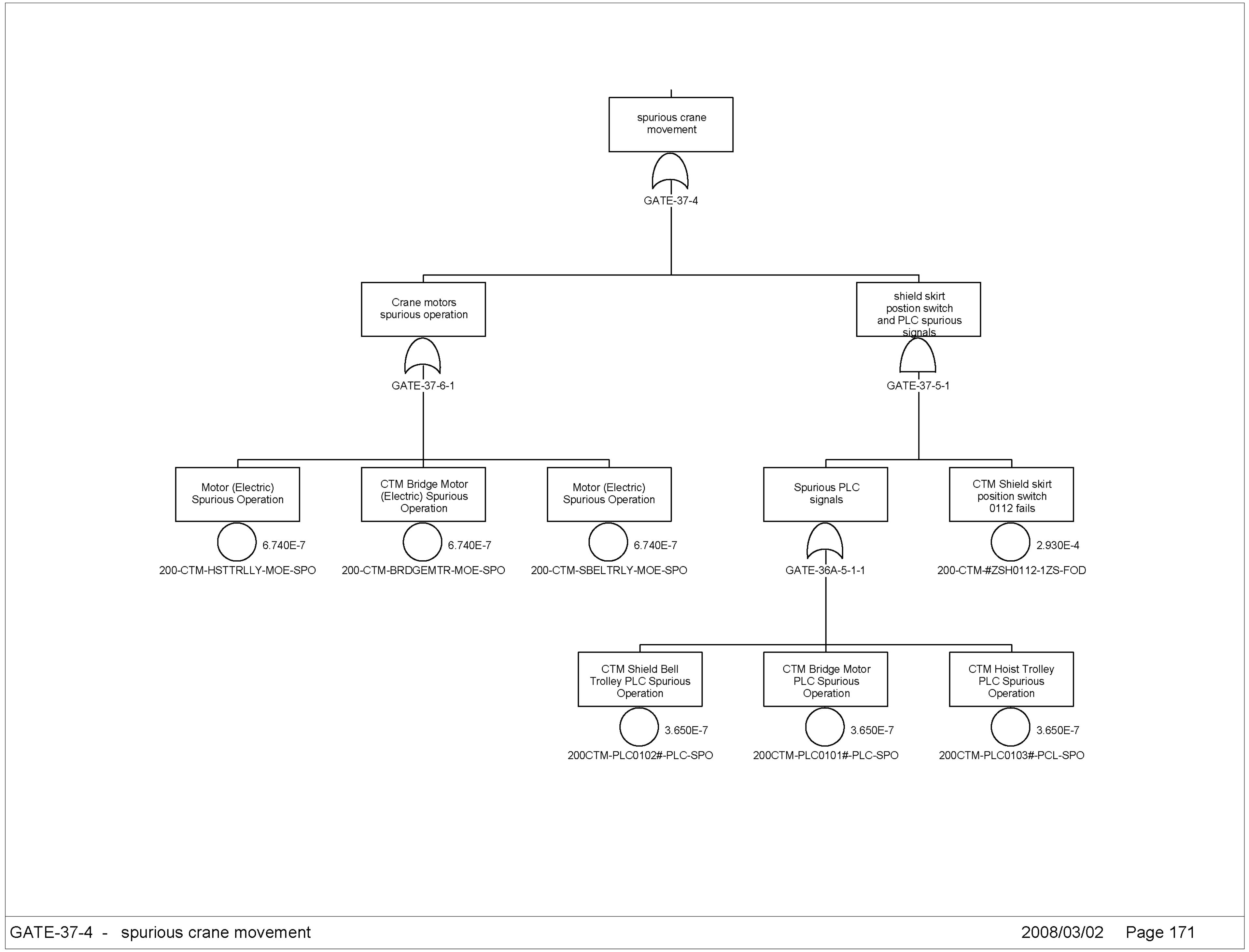
Source: Original

Figure B4.4-13. CTM Drop Fault Tree Sheet 11



Source: Original

Figure B4.4-14. CTM Drop Fault Tree Sheet 12



Source: Original

Figure B4.4-15. CTM Drop Fault Tree Sheet 13

B4.4.2 Canister Drops from Above the Canister Design Limit Drop Height

B4.4.2.1 Description

Transfer operations using the CTM entail the possibility of inadvertent drops of the canisters. These drops have been divided into two classes: drops from heights below the design basis drop height of the canister and drops from heights above the design basis drop height of the canister. This fault tree for canister drops addresses the second of these two scenarios.

B4.4.2.2 Success Criteria

Success criteria for the CTM is the prevention of a canister drop from above the canister design limit drop height from any cause during the lift, lateral movement, and lower portions of the canister transfer.

B4.4.2.3 Design Requirements and Features

Requirements

Hard-wired interlocks are used to prevent inadvertent actions during CTM transfer operations. These include the following:

- An optical sensor at the bottom of the shield bell that, once it is cleared, stops the hoist and erases the lift command (can only lower hoist). This interlock is used only when lifting a canister.

- Above the ASD stop point is an upper limit switch which, when reached, stops the hoist from lifting. This first limit switch (first hoist upper limit) effectively erases the lift command (the hoist still has power) and the operator can only lower the hoist. Roughly one foot above that limit switch is another limit switch (final hoist upper limit) that, when reached, cuts off the power to the CTM hoist.

- An interlock between the shield skirt and port gate which requires the shield skirt to be lowered in order for the port gate to open. There is a bypass for this interlock.

- An interlock between the CTM bridge/trolley travel and shield skirt position. Neither the CTM bridge nor the trolley can travel while the skirt is lowered.

- An interlock between the slide gate and shield skirt – the shield skirt cannot be raised unless the slide gate is closed. This interlock can be bypassed, to allow the CTM to move with the slide gate open during lid removal.

- Interlocks preventing improper hoist movement. The hoist cannot move unless the shield skirt is lowered. This interlock is based on hoist movement, not position, so movement with the hoist too low is not precluded.

- The load cells which cut off power to the hoist when the crane capacity is exceeded.

- An interlock between the grapple position (fully engaged or fully disengaged) and hoist movement. The grapple automatically engages/disengages with a given object. The grapple must be positively engaged for the grapple engagement indicator to give a positive indication.

Features

Bridge and trolley motors are sized to limit lateral travel to less than 20 fpm, sufficient to ensure that in the event of an impact, impact forces are below the design limits of the canister.

The shield bell slide gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

The floor port gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

Hard wired interlocks used to prevent inadvertent actions during CTM transfer operations are ITS; PLCs are not ITS equipment.

The end stops for both the bridge and trolley end of travel end stops are capable of stopping the bridge/trolley at their maximum speed and preclude impact with any permanent structure.

The interlock between the grapple position and the operation of the hoist motor cannot be bypassed during CTM canister transfer operations.

B4.4.2.4 Fault Tree Model

The top event in this fault tree is "CTM High Drops from Two Blocking Events." This is defined as a drop of a canister from a height above the design limit height for the canister during transfer operations. (The two-block designation refers to the condition where the object being lifted is raised to the point where the upper and lower blocks of the crane come into contact. Attempts to continue to lift the load at this point places additional strains on the CTM components.) For this event to occur the canister must be lifted above the normal heights associated with a lift and the features designed to limit the drop height must fail. During normal operation, once the canister clears the optical sensor in the shield bell, the shield bell slide gate is closed. Provided the gate is closed at this time, the potential drop height for the canister never exceeds the canister design limit drop height. Faults considered in the evaluation of this top event include: component and human events (considered in conjunction with the interlocks intended to prevent the erroneous human action) that contribute to raising the canister too high (Figures B4.4-18, B4.4-19 and B4.4-20). The model does not credit CTM features that could mitigate the consequences of a two-block event. All two-block events are modeled to result in a drop.

B4.4.2.5 Basic Event Data

Table B4.4-4 contains a list of basic events used in the "CTM High Drops from Two Blocking Events" fault tree. Included are the human failure events and the CCF events identified in the following two sections. There are no maintenance-related failures associated with the CTM. The CTM is not in service while undergoing maintenance. Sensor failures that could be associated with the failure to restore from maintenance are not expected to contribute significantly to the overall sensor availability.

The canister drop probability modeled by the fault tree is evaluated over a mission time of one hour. This mission time encompasses vertical lifting, lateral movement, and vertical lowering of the canister by the CTM. A longer mission time is also considered for specific components. For example, the fault tree accounts for the failure of standby components whose potential malfunction would remain hidden until they are put into operation. They are consequently evaluated over the interval of time between their actuation, considered to be the duration of a shift (i.e., eight hours).

Table B4.4-4. Basic Event Probability for the CTM High Drops from Two Blocking Events Fault Tree

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1	1.380E-05	1.380E-05	0.000E+00	0.000E+00
200-CTM--330121--ZS--FOD	CTM hoist first upper limit switch 0121 failure on demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--330122--ZS--FOD	CTM final hoist upper limit switch 0122 failure on demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM-ASD0122#-CTL-FOD	CTM hoist ASD controller fails	1	2.030E-03	2.030E-03	0.000E+00	8.000E+00
200-CTM-HOISTMTR-MOE-FSO	CTM hoist motor (electric) fails to shut off	3	1.350E-08	0.000E+00	1.350E-08	1.000E+00
200-CTM-OPSENSOR-SRX-FOH	Canister above CTM slide gate optical sensor fails	3	4.700E-06	0.000E+00	4.700E-06	1.000E+00
200-OPCTMDRINT01-HFI-COD	Operator raises load too high - two block	1	1.000E+00	1.000E+00	0.000E+00	0.000E+00
200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1	1.380E-05	1.380E-05	0.000E+00	0.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
ASD = adjustable speed drive; Calc. = calculation; CCF = common-cause failure; CTM = canister transfer machine; CTT = cask transfer trolley; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

B4.4.2.5.1 Human Failure Events

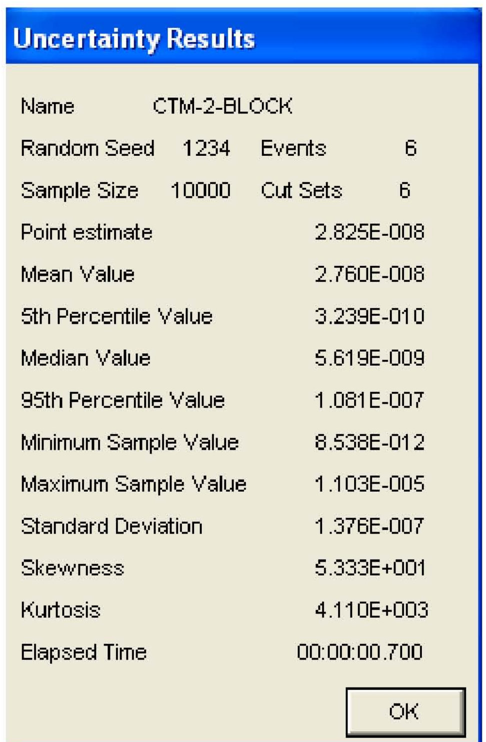
One basic event is associated with human error: 200-OPCTMDRINT01-HFI-COD (Operator Raises Load Too High - Two Block). This event models the combination of operator actions and interlock failures required to allow the operator to raise a load above design limits, and action that can lead to a two blocking failure.

B4.4.2.5.2 Common-Cause Failures

One CCF event was considered in the evaluation of this fault tree. There are two upper limit switches intended to prevent raising a load too high. The CCF of these switches was considered.

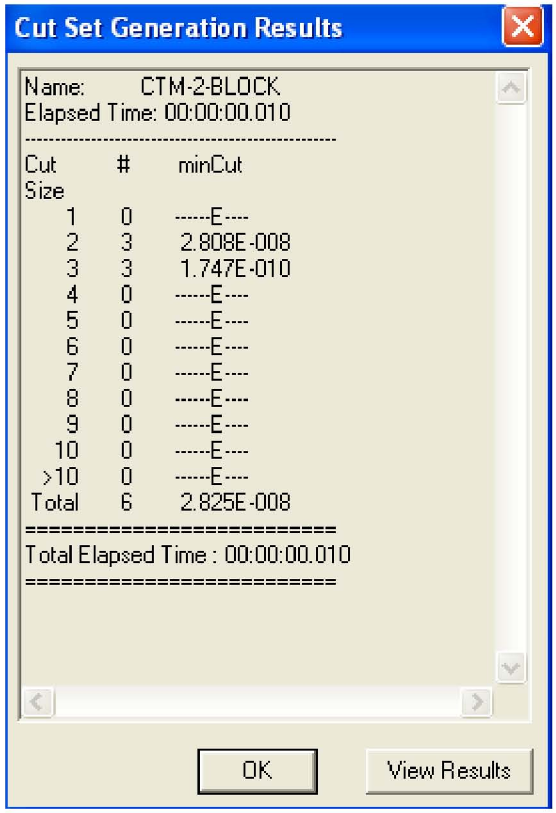
B4.4.2.6 Uncertainty and Cut Set Generation Results

Figure B4.4-16 contains the uncertainty results obtaining from running the fault tree for CTM two blocking with a cutoff probability of 1E-15. Figure B4.4-17 provides the cut set generation results for the CTM two-blocking fault tree.



Source: Original

Figure B4.4-16. Uncertainty Results of the CTM Canister Drop Two-Block Fault Tree



Source: Original

Figure B4.4-17. Cut Set Generation Results for the CTM Canister Drop Two-Block Fault Tree

B4.4.2.7 Cut Sets

Table B4.4-5 contains the top six cut sets for the canister drop two-blocking fault tree.

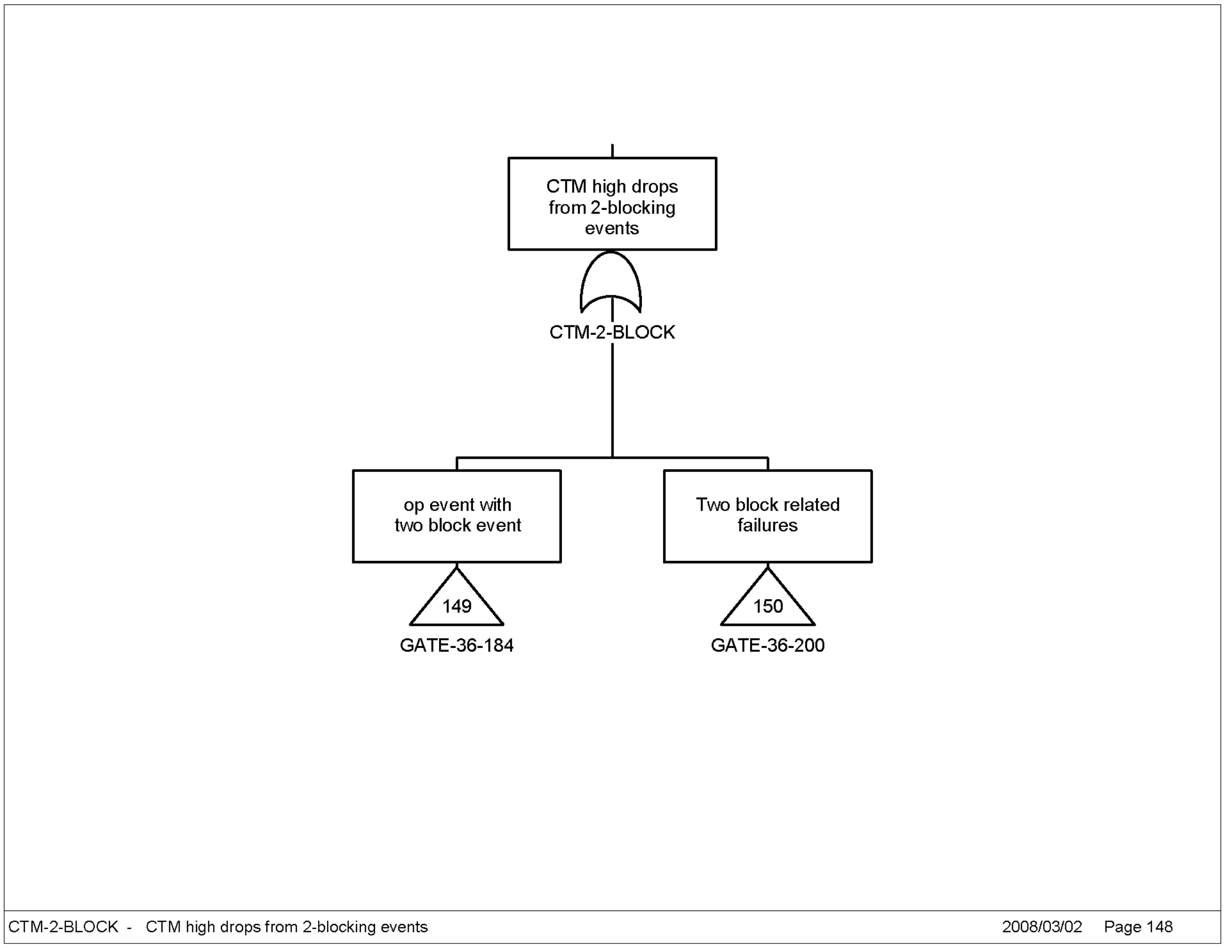
Table B4.4-5. Dominant Cut Sets for the CTM Canister Drop from Above the Canister Design Height Limit

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
99.15	99.15	2.801E-08	200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1.380E-05
			200-CTM-ASD0122#-CTL-FOD	CTM Hoist ASD Controller fails	2.030E-03
99.77	0.62	1.743E-10	200-CTM--330121--ZS--FOD	CTM Hoist First Upper Limit Switch 0121 Failure on Demand	2.930E-04
			200-CTM--330122--ZS--FOD	CTM Final Hoist Upper Limit Switch 0122 Failure on Demand	2.930E-04
			200-CTM-ASD0122#-CTL-FOD	CTM Hoist ASD Controller fails	2.030E-03
100.00	0.23	6.486E-11	200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1.380E-05
			200-CTM-OPSENSOR-SRX-FOH	Canister above CTM slide gate optical sensor fails	4.700E-06
100.00	0.00	4.035E-13	200-CTM--330121--ZS--FOD	CTM Hoist First Upper Limit Switch 0121 Failure on Demand	2.930E-04
			200-CTM--330122--ZS--FOD	CTM Final Hoist Upper Limit Switch 0122 Failure on Demand	2.930E-04
			200-CTM-OPSENSOR-SRX-FOH	Canister above CTM slide gate optical sensor fails	4.700E-06
100.00	0.00	1.863E-13	200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1.380E-05
			200-CTM-HOISTMTR-MOE-FSO	CTM Hoist Motor (Electric) Fails to Shut Off	1.350E-08
100.00	0.00	1.159E-15	200-CTM--330121--ZS--FOD	CTM Hoist First Upper Limit Switch 0121 Failure on Demand	2.930E-04
			200-CTM--330122--ZS--FOD	CTM Final Hoist Upper Limit Switch 0122 Failure on Demand	2.930E-04
			200-CTM-HOISTMTR-MOE-FSO	CTM Hoist Motor (Electric) Fails to Shut Off	1.350E-08

NOTE: ASD = adjustable speed drive; CCF = common-cause failure; CTM = canister transfer machine; Prob. = probability.

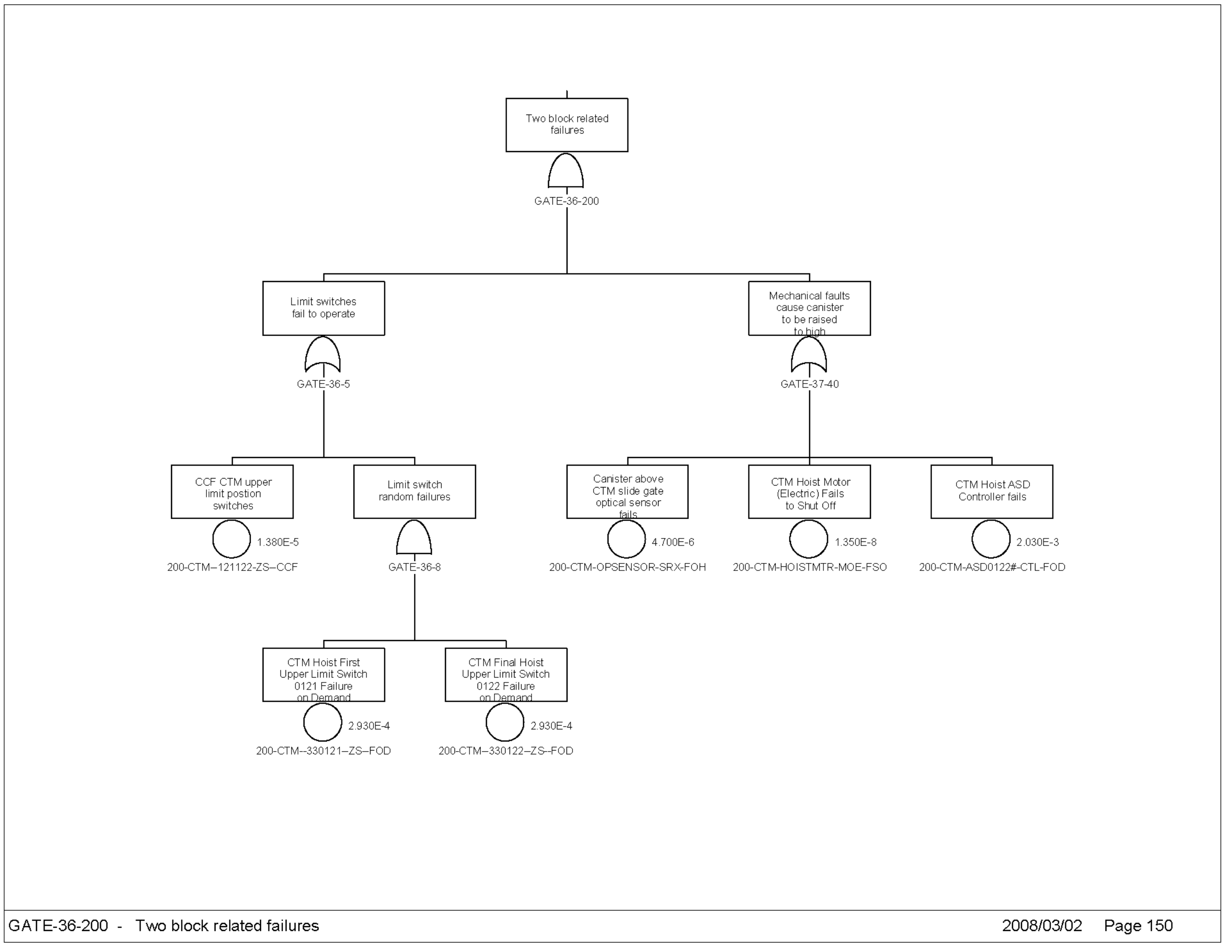
Source: Original

B4.4.2.4.8 Fault Trees



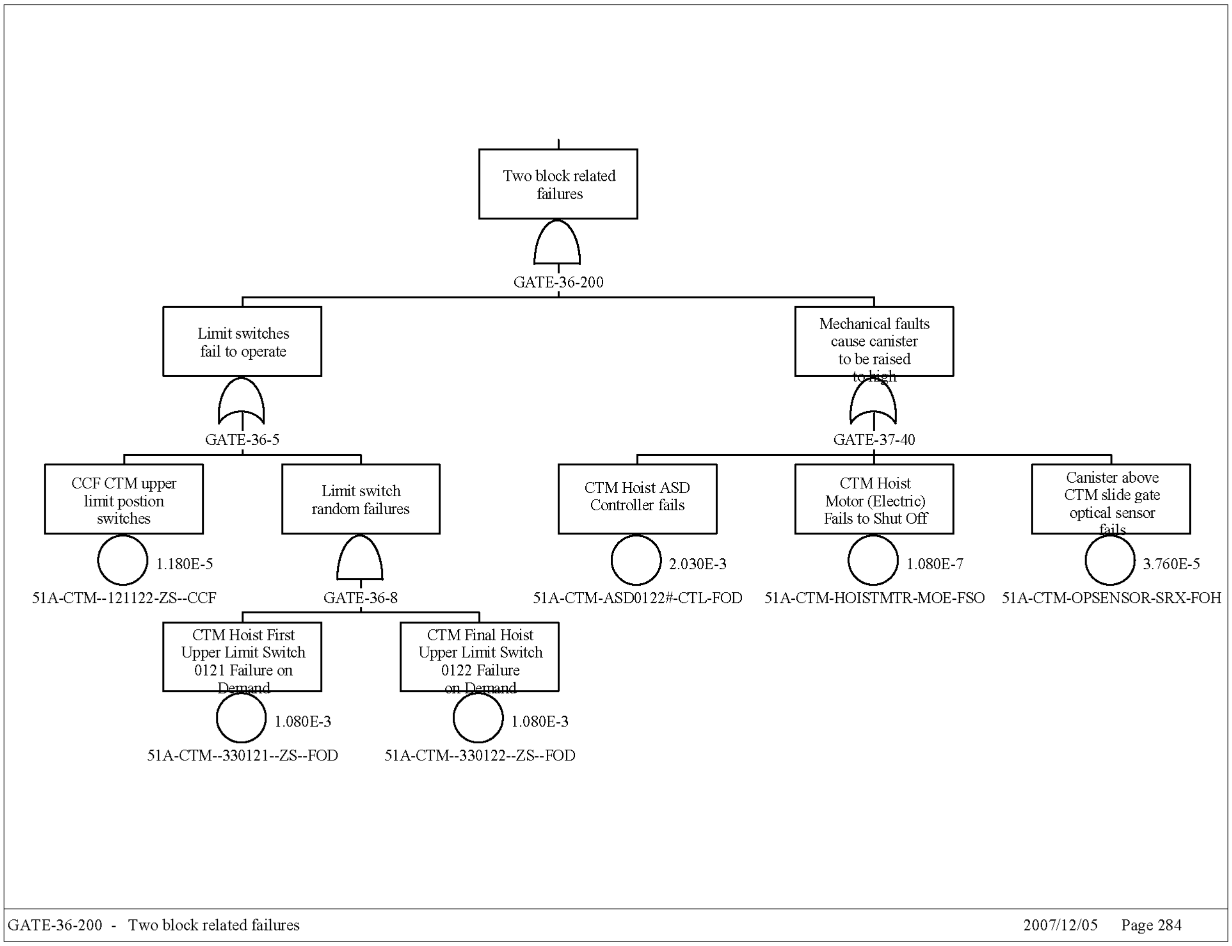
Source: Original

Figure B4.4-18. CTM High Drops from Two-Blocking Event (Sheet 1)



Source: Original

Figure B4.4-19. CTM High Drops from Two-Blocking Event (Sheet 2)



Source: Original

Figure B4.4-20. CTM High Drops from Two-Blocking Event (Sheet 3)

B4.4.3 Drop of Object onto Canister

B4.4.3.1 Description

Transfer operations using the CTM entail the possibility of inadvertent drops of objects onto canisters. Cask lids, handling equipment, and auxiliary grapples are handled during the canister transfer process. At times these objects are over the canister and could be dropped onto the canister.

B4.4.3.2 Success Criteria

The success criterion for the CTM is the prevention of a drop of any object onto the canister from any cause during the lift, lateral movement, and lowering portions of the canister transfer.

B4.4.3.3 Design Requirements and Features

Requirements

Hard-wired interlocks are used to prevent inadvertent actions during CTM transfer operations. These include the following:

- An optical sensor at the bottom of the shield bell that, once it is cleared, stops the hoist and erases the lift command (can only lower hoist). This interlock is used only when lifting a canister.

- Above the ASD stop point is an upper limit switch which, when reached, stops the hoist from lifting. This first limit switch (first hoist upper limit) effectively erases the lift command (the hoist still has power) and the operator can only lower the hoist. Roughly a foot above that limit switch is another limit switch (final hoist upper limit) that, when reached, cuts off the power to the CTM hoist.

- An interlock between the shield skirt and port gate which requires the shield skirt to be lowered in order for the port gate to open. There is a bypass for this interlock.

- An interlock between the CTM bridge/trolley travel and shield skirt position. Neither the CTM bridge nor the trolley can travel while the skirt is lowered.

- An interlock between the slide gate and shield skirt–the shield skirt cannot be raised unless the slide gate is closed. This interlock can be bypassed to allow the CTM to move with the slide gate open during lid removal.

- Interlocks preventing improper hoist movement. The hoist cannot move unless the shield skirt is lowered. This interlock is based on hoist movement, not position, so movement with the hoist too low is not precluded.

- The load cells cut off power to the hoist when the crane capacity is exceeded.

- An interlock between the grapple position (fully engaged or fully disengaged) and hoist movement. The grapple automatically engages/disengages with a given object. The grapple must be positively engaged for the grapple engagement indicator to give a positive indication.

Features

Bridge and trolley motors are sized to limit lateral travel to less than 20 feet per minute, sufficient to ensure that in the event of an impact, impact forces are below the design limits of the canister.

The shield bell slide gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

The floor port gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

Hard wired interlocks used to prevent inadvertent actions during CTM transfer operations are ITS; PLCs are not ITS equipment.

The end stops for both the bridge and trolley end of travel end stops are capable of stopping the bridge/trolley at their maximum speed and preclude impact with any permanent structure.

The interlock between the grapple position and the operation of the hoist motor cannot be bypassed during CTM canister transfer operations.

B4.4.3.4 Fault Tree Model

The top event in this fault tree is "Drop of Object onto Canister." This is defined as a drop of an object onto a canister during transfer operations. Faults considered in the evaluation of this top event include: human events that contribute to a drop (considered in conjunction with the interlocks intended to prevent the erroneous human action) and mechanical (structural) failures of the CTM components (Figures B4.4-23 to B4.4-34). The interlocks and safety features (position controls, load cells, and drum and holding brakes) intended to either prevent CTM failure or given failure of the CTM to prevent a load drop are included in the model.

Structural failures of components including the hoist cables, sheaves, drum, and grapples can result in canister drops. Operator events are addressed for actions including improper grapple connections, misalignments of the hoist and the canister, improper hoist activities and improper lateral movement of the CTM. Protection from these actions are provided by hard-wired interlocks keyed to the position of the CTM (both hoist position and CTM lateral position), slide and port gate doors, and the shield bell skirt. Also considered in the analysis is a canister drop initiated by improper operation of the shield bell slide gates and the port slide gates. While the gate motors are sized to prevent damage to the canister in the event of an inadvertent closure of the gates, the possibility that the gates would close above the canister during a lift blocking the lift and causing a canister drop was considered.

B4.4.3.5 Basic Event Data

Table B4.4-6 contains a list of basic events used in the "CTM Drop of Object onto Canister" fault tree. Included are the human failure events and the CCF events identified in the previous two sections. There are no maintenance-related failures associated with the CTM. The CTM is not in service while undergoing maintenance. Sensor failures that could be associated with the failure to restore from maintenance are not expected to contribute significantly to the overall sensor availability.

The object drop probability modeled by the fault tree is evaluated over a mission time of one hour. This mission time encompasses vertical lifting, lateral movement, and vertical lowering of the canister by the CTM. A longer mission time is also considered for specific components. For example, the fault tree accounts for the failure of standby components whose potential malfunction would remain hidden until they are put into operation. They are consequently evaluated over the interval of time between their actuation, considered to be the duration of a shift, i.e., eight hours. In another example, brakes are also analyzed over a mission time of twenty-four hours. This duration is deemed sufficient to encompass the time required to revert to normal transfer operations, after a malfunction that would have caused a safety system of the CTM to cease transfer activities.

Table B4.4-6. Basic Event Probability for the CTM Drop of Objects onto Canister Fault Tree

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200--DRUM001-DM--FOD	CTM Drum Failure on Demand	1	4.000E-08	4.000E-08	0.000E+00	0.000E+00
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1	1.380E-05	1.380E-05	0.000E+00	0.000E+00
200-CTM--330121--ZS--FOD	CTM Hoist First Upper Limit Switch 0121 Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--330122--ZS--FOD	CTM Final Hoist Upper Limit Switch 0122 Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--CBL0001-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0002-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	3	9.400E-08	9.400E-08	9.400E-08	0.000E+00
200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--HOISTMT-MOE-FTR	CTM Hoist Motor (Electric) Fails to Run	3	6.500E-06	0.000E+00	6.500E-06	1.000E+00
200-CTM--HOLDBRK-BRK-FOD	Brake Failure on Demand	1	1.460E-06	1.460E-06	0.000E+00	0.000E+00
200-CTM--HOLDBRK-BRK-FOH	CTM Holding Brake (Electric) Failure to hold	3	3.520E-05	0.000E+00	4.400E-06	8.000E+00
200-CTM--IMEC125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--IMEC125-ZS-FOD	CTM Load Cell Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--LOWERBL-BLK-FOD	CTM Lower Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--MISSPOOL-DM-MSP	CTM Mis-spool event	3	6.860E-07	0.000E+00	6.860E-07	0.000E+00
200-CTM--OVERSP--ZS--FOD	CTM Hoist motor speed Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--PORTGT1-MOE-SPO	Spurious port gate1 motor operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--PORTGT1-PLC-SPO	Port Gage PCL Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM--PORTGT2-MOE-SPO	Port Gate Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--PORTGT2-PLC-SPO	Port Gage PCL Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM--SLIDEGT-MOE-SPO	CTM Slide Gate Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM--SLIDEGT-PLC-SPO	CTM Slide Gate PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM--SLIDGT2-IEL-FOD	CTM Slide Gate Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--UPPERBL-BLK-FOD	CTM Upper Sheaves failure	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	1	3.990E-03	3.990E-03	0.000E+00	0.000E+00

Table B4.4-6. Basic Event Probability for the CTM Drop of Objects onto Canister Fault Tree (Continued)

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM--WTSW125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM--WTSW125-ZS--FOD	CTM Load Cell Limit Switch Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--YS01129-ZS--FOD	CTM Drum Brake control circuit Limit Switch 1129 Failure	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--ZSH0111-ZS--SPO	CTM grapple engaged Limit Switch Spurious Operation	3	1.280E-06	0.000E+00	1.280E-06	1.000E+00
200-CTM-ASD0122#-CTL-FOD	CTM Hoist ASD Controller fails	1	2.030E-03	2.030E-03	0.000E+00	8.000E+00
200-CTM-BRDGEMTR-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	0.000E+00
200-CTM-BRIDGMTR-IEL-FOD	CTM Shield Skirt-Bridge motor Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM-DRTM-CT-FOD	CTM Drive Train Protection and Fail Det. Controller Failure	1	4.000E-06	4.000E-06	0.000E+00	0.000E+00
200-CTM-DRUMBRK-BRP-FOH	CTM Drum Brake (Pneumatic) Failure to Hold	3	6.704E-05	0.000E+00	8.380E-06	8.000E+00
200-CTM-HOISTMTR-MOE-FSO	CTM Hoist Motor (Electric) Fails to Shut Off	3	1.350E-08	0.000E+00	1.350E-08	1.000E+00
200-CTM-HSTTRLLY-IEL-FOD	CTM shield skirt Hoist Trolley motor Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM-HSTTRLLY-MOE-SPO	Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	1.000E+00
200-CTM-OPSENSOR-SRX-FOH	Canister above CTM slide gate optical sensor fails	3	4.700E-06	0.000E+00	4.700E-06	1.000E+00
200-CTM-SBELTRLY-IEL-FOD	CTM Shield Bell Trolley Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
200-CTM-SBELTRLY-MOE-SPO	Motor (Electric) Spurious Operation	3	6.740E-07	0.000E+00	6.740E-07	0.000E+00
200-CTM-SLIDGT2-SRX-FOD	CTM slide Gate Position Sensor Fails on Demand	1	1.100E-03	1.100E-03	0.000E+00	0.000E+00
200-CTM-ZSL0111-ZS--SPO	CTM Grapple engaged Limit Switch Spurious Operation	3	1.280E-06	0.000E+00	1.280E-06	0.000E+00
200-DRUMBRK-BRP-FOH	CTM Drum Brake (Pneumatic) Failure on Demand	3	8.380E-06	0.000E+00	8.380E-06	0.000E+00
200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1	1.000E-03	1.000E-03	0.000E+00	0.000E+00
200-OPCTMDRINT01-HFI-COD	Operator raises load too high - two block	1	1.000E+00	1.000E+00	0.000E+00	0.000E+00
200-OPCTMDROP001-HFI-COD	Operator causes drop of object onto canister	1	4.000E-07	4.000E-07	0.000E+00	0.000E+00
200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	1	4.000E-08	4.000E-08	0.000E+00	0.000E+00
200CTM-PLC0101#-PLC-SPO	CTM Bridge Motor PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200CTM-PLC0102#-PLC-SPO	CTM Shield Bell Trolley PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200CTM-PLC0103#-PCL-SPO	CTM Hoist Trolley PLC Spurious Operation	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200--DRUM001-DM--FOD	CTM Drum Failure on Demand	1	4.000E-08	4.000E-08	0.000E+00	0.000E+00
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00

Table B4.4-6. Basic Event Probability for the CTM Drop of Objects onto Canister Fault Tree (Continued)

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1	1.380E-05	1.380E-05	0.000E+00	0.000E+00
200-CTM--330121--ZS--FOD	CTM Hoist First Upper Limit Switch 0121 Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--330122--ZS--FOD	CTM Final Hoist Upper Limit Switch 0122 Failure on Demand	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM--CBL0001-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0002-CBL-FOD	CTM Hoist Wire rope Breaks	1	2.000E-06	2.000E-06	0.000E+00	1.000E+00
200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	3	9.400E-08	9.400E-08	9.400E-08	0.000E+00
200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1	1.150E-06	1.150E-06	0.000E+00	0.000E+00
200-CTM--HOISTMT-MOE-FTR	CTM Hoist Motor (Electric) Fails to Run	3	6.500E-06	0.000E+00	6.500E-06	1.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
ASD = adjustable speed drive; Calc. = calculation; CCF = common-cause failure; CTM = canister transfer machine; CTT = cask transfer trolley; Fail. = failure; Miss. = mission; PLC = programmable logic controller; Prob. = probability.

Source: Original

B4.4.3.5.1 Human Failure Events

Four basic events are associated with human error (Table B4.4-7). These are for drops initiated by operator actions, drops caused by the operator initiating a two-block event, a failure to restore interlocks allowing movement of the crane when the shield skirt is raised and the slide gates are open, and the operator closing the slide or port gates during a lift. The quantification of these events includes operator actions and the failures of interlocks intended to prevent such operator action.

Table B4.4-7. Human Failure Events

Name	Description
200-OPCTMDRINT01-HFI-COD	Operator raises load too high - two block
200-OPCTMDROP001-HFI-COD	Operator causes drop of object onto canister
200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close
200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor

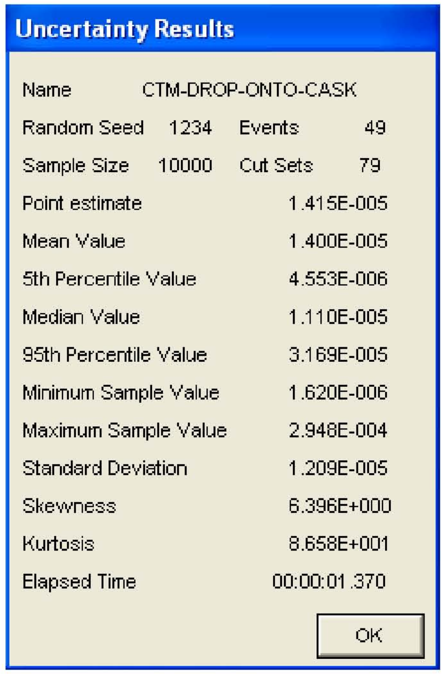
Source: Original

B4.4.3.5.2 Common-Cause Failures

Three common-cause events were considered in the evaluation of this fault tree. Common cause failure of the two upper limit sensors on the hoist used to prevent a two-block event is considered. The second CCF event considered is the CCF of the hoist cables.

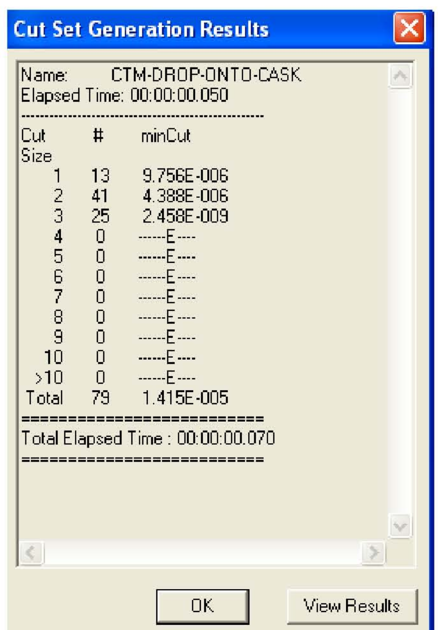
B4.4.3.6 Uncertainty and Cut Set Generation

Figure B4.4-21 contains the uncertainty results obtaining from running the fault trees for the CTM Drop onto Canister with a cutoff probability of 1E-15. Figure B4.4-22 provides the cut set generation results for the CTM Drop onto Canister fault tree.



Source: Original

Figure B4.4-21. Uncertainty Results of the CTM Drop onto Canister Fault Tree



Source: Original

Figure B4.4-22. Cut Set Generation Results for the CTM Drop onto Canister Fault Tree

B4.4.3.7 Cut Sets

Table B4.4-8 contains the top 20 cut sets for the CTM Drop onto Canister fault tree.

Table B4.4-8. Dominant Cut Sets for the CTM Drop onto Canister Fault Tree

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
28.21	28.21	3.990E-06	200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-03
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-03
37.26	9.05	1.280E-06	200-CTM--ZSH0111-ZS--SPO	CTM grapple engaged Limit Switch Spurious Operation	1.280E-06
46.31	9.05	1.280E-06	200-CTM-ZSL0111-ZS--SPO	CTM Grapple engaged Limit Switch Spurious Operation	1.280E-06
54.44	8.13	1.150E-06	200-CTM--EQL-SHV-BLK-FOD	CTM Sheaves Failure on Demand	1.150E-06
62.57	8.13	1.150E-06	200-CTM--UPPERBL-BLK-FOD	CTM Upper Sheaves failure	1.150E-06
70.70	8.13	1.150E-06	200-CTM--GRAPPLE-GPL-FOD	CTM Grapple Failure on Demand	1.150E-06
78.83	8.13	1.150E-06	200-CTM--LOWERBL-BLK-FOD	CTM Lower Sheaves Failure on Demand	1.150E-06
83.59	4.76	6.740E-07	200-CTM-BRDGEMTR-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation	6.740E-07
88.35	4.76	6.740E-07	200-CTM-HSTTRLLY-MOE-SPO	Motor (Electric) Spurious Operation	6.740E-07
93.11	4.76	6.740E-07	200-CTM-SBELTRLY-MOE-SPO	Motor (Electric) Spurious Operation	6.740E-07
95.94	2.83	4.000E-07	200-OPCTMDROP001-HFI-COD	Operator causes drop of object onto canister	4.000E-07
98.01	2.07	2.930E-07	200-CTM--IMEC125-ZS-FOD	CTM Load Cell Limit Switch Failure on Demand	2.930E-04
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-03
98.67	0.66	9.400E-08	200-CTM--CBL0102-CBL-CCF	CCF CTM Hoist wire ropes	9.400E-08
98.95	0.28	4.000E-08	200--DRUM001-DM--FOD	CTM Drum Failure on Demand	4.000E-08
99.23	0.28	4.000E-08	200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
99.48	0.25	3.520E-08	200-CTM--HOLDBRK-BRK-FOH	CTM Holding Brake (Electric) Failure to hold	3.520E-05
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-03
99.68	0.20	2.801E-08	200-CTM--121122-ZS--CCF	CCF CTM upper limit position switches	1.380E-05
			200-CTM-ASD0122#-CTL-FOD	CTM Hoist ASD Controller fails	2.030E-03
99.87	0.19	2.750E-08	200-CTM--WTSW125-IEL-FOD	CTM Hoist Motor Control Interlock Failure on Demand	2.750E-05
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-03

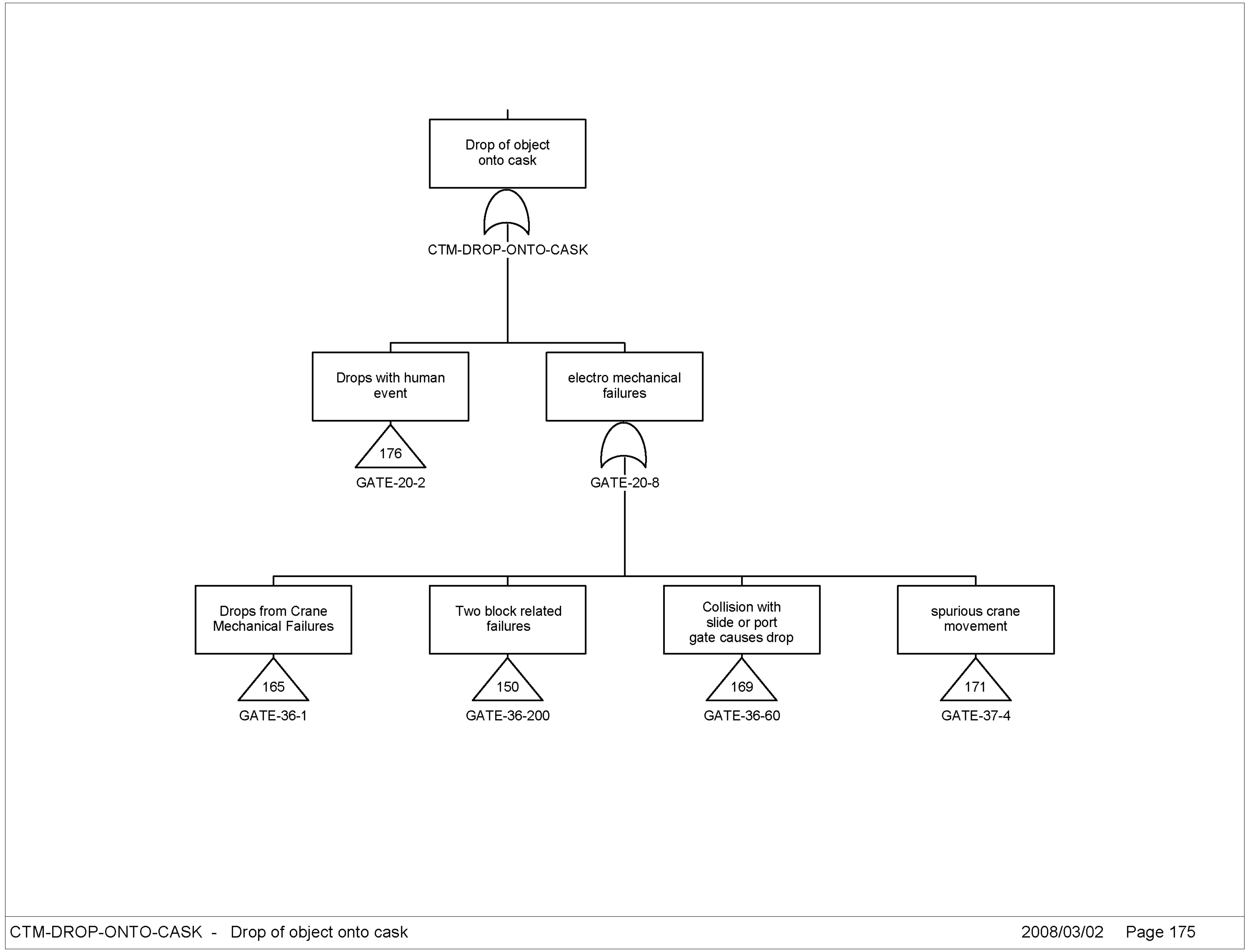
Table B4.4-8. Dominant Cut sets for the CTM Drop onto Canister Fault Tree (Continued)

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
99.89	0.02	2.689E-09	200-CTM--PORTGT1-MOE-SPO	Spurious port gate1 motor operation	6.740E-07
			200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-03
99.91	0.02	2.689E-09	200-CTM--PORTGT2-MOE-SPO	Port Gate Motor (Electric) Spurious Operation	6.740E-07
			200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-03
28.21	28.21	3.990E-06	200-CTM--WT0125--SRP-FOD	CTM Load Cell Pressure Sensor Fails on Demand	3.990E-03
			200-OPCLCTMGATE1-HFI-NOD	Operator commands doors close	1.000E-03
37.26	9.05	1.280E-06	200-CTM--ZSH0111-ZS--SPO	CTM grapple engaged Limit Switch Spurious Operation	1.280E-06

NOTE: CCF = common-cause failure; CTM = canister transfer machine; HRA = human reliability analysis; Prob. = probability.

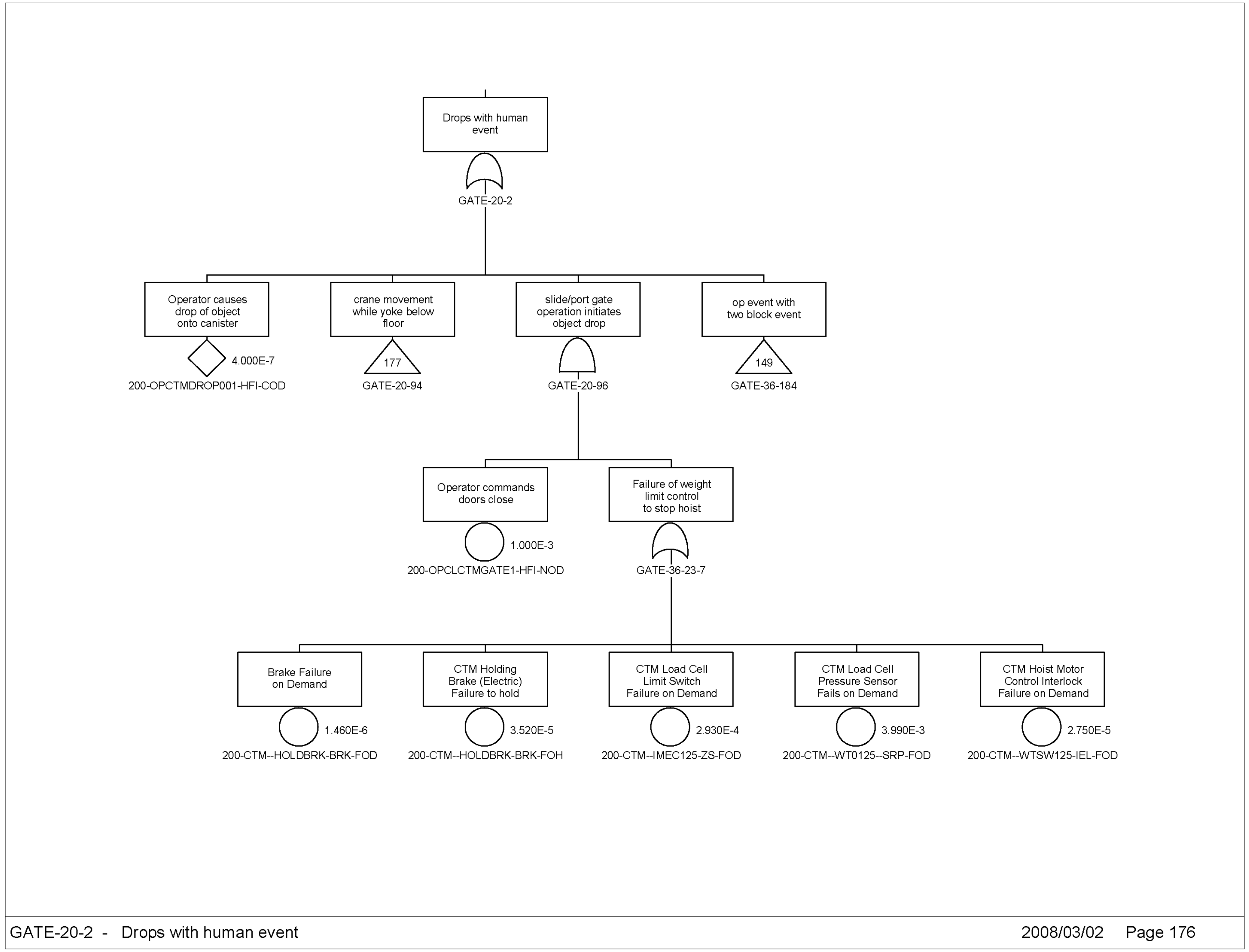
Source: Original

B4.4.3.8 Fault Trees



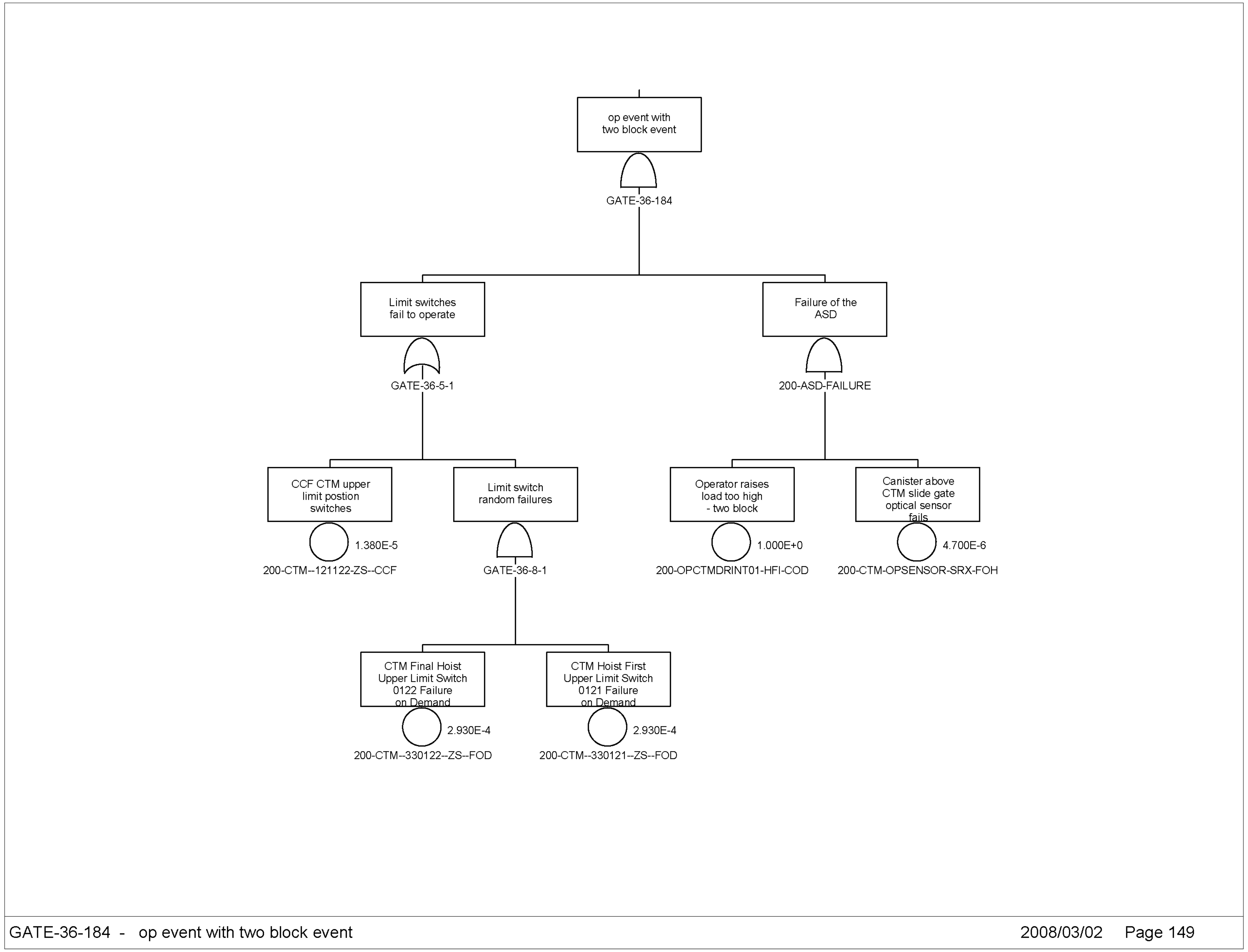
Source: Original

Figure B4.4-23. Drop of Object onto Cask (Sheet 1)



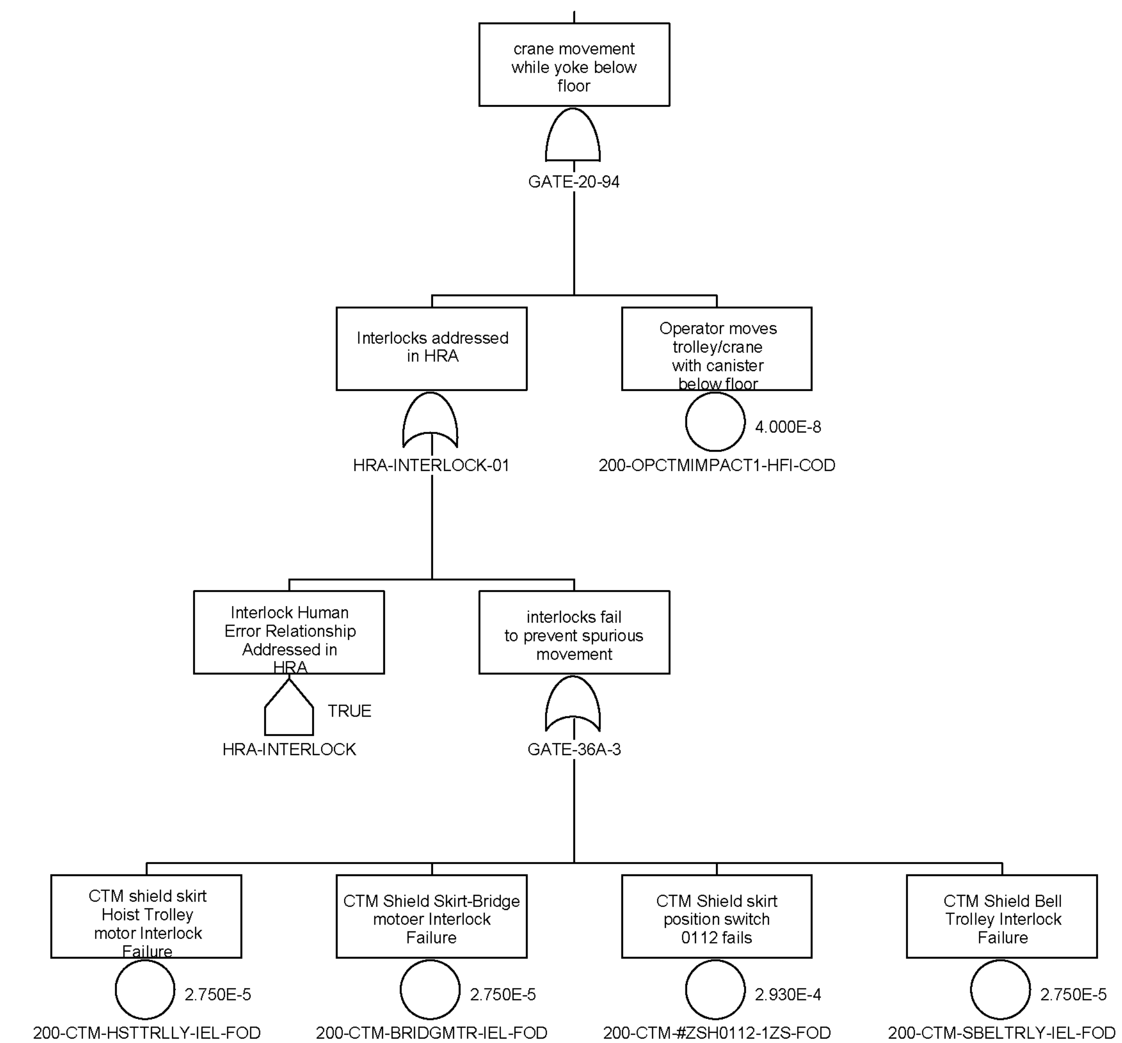
Source: Original

Figure B4.4-24. Drop of Object onto Cask (Sheet 2)



Source: Original

Figure B4.4-25. Drop of Object onto Cask (Sheet 3)



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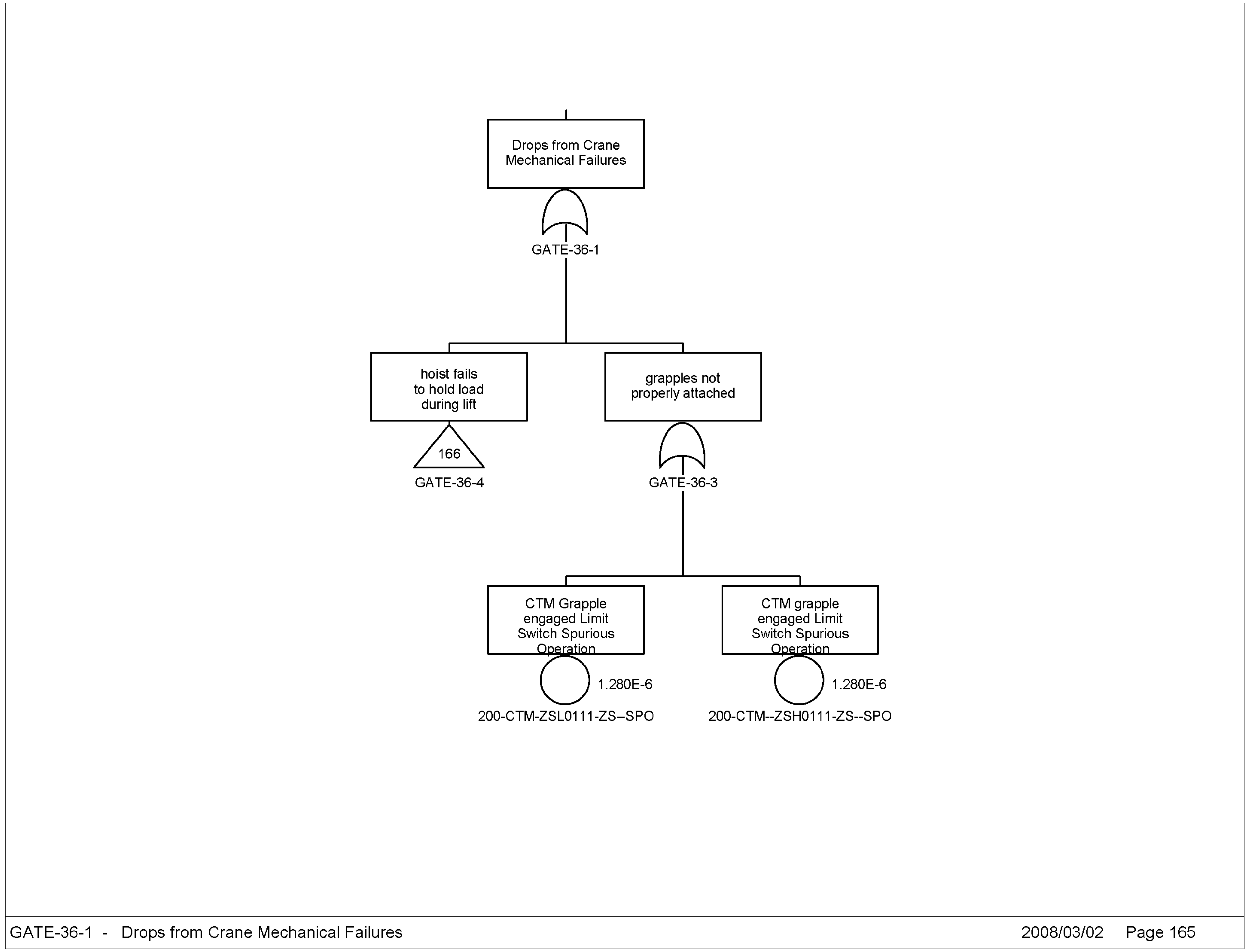
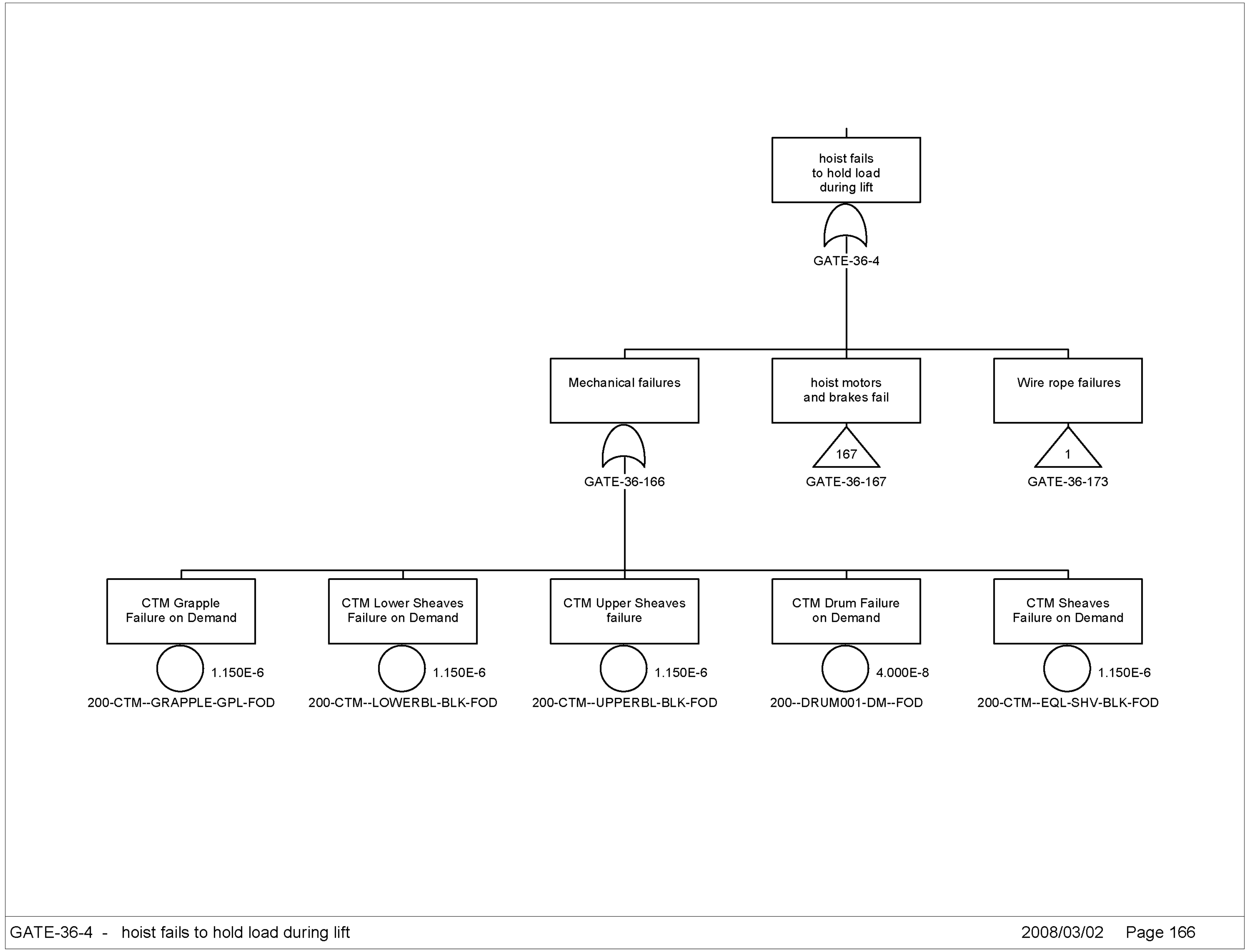
Source: Original

Figure B4.4-26. Drop of Object onto Cask (Sheet 4)



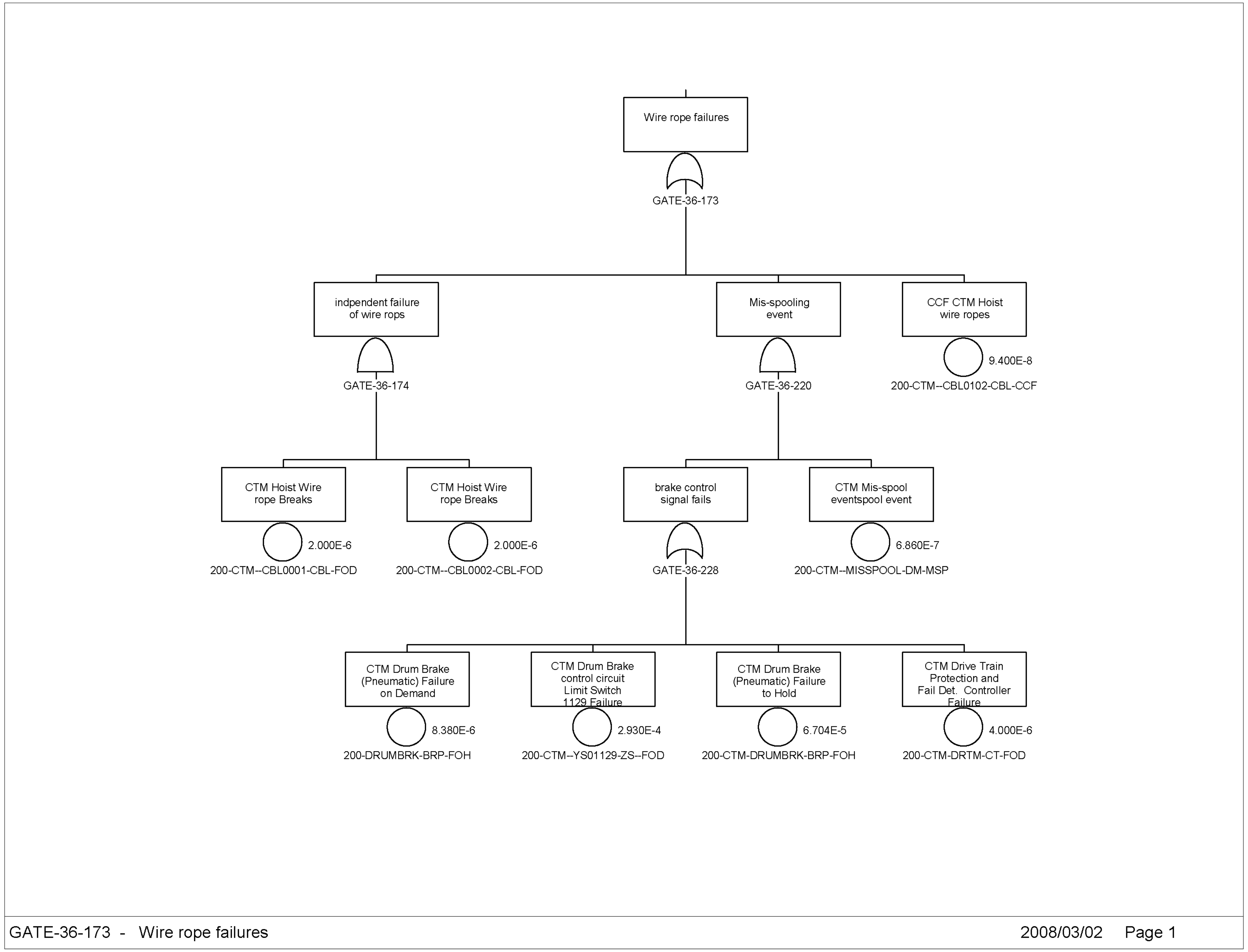
Source: Original

Figure B4.4-27. Drop of Object onto Cask (Sheet 5)



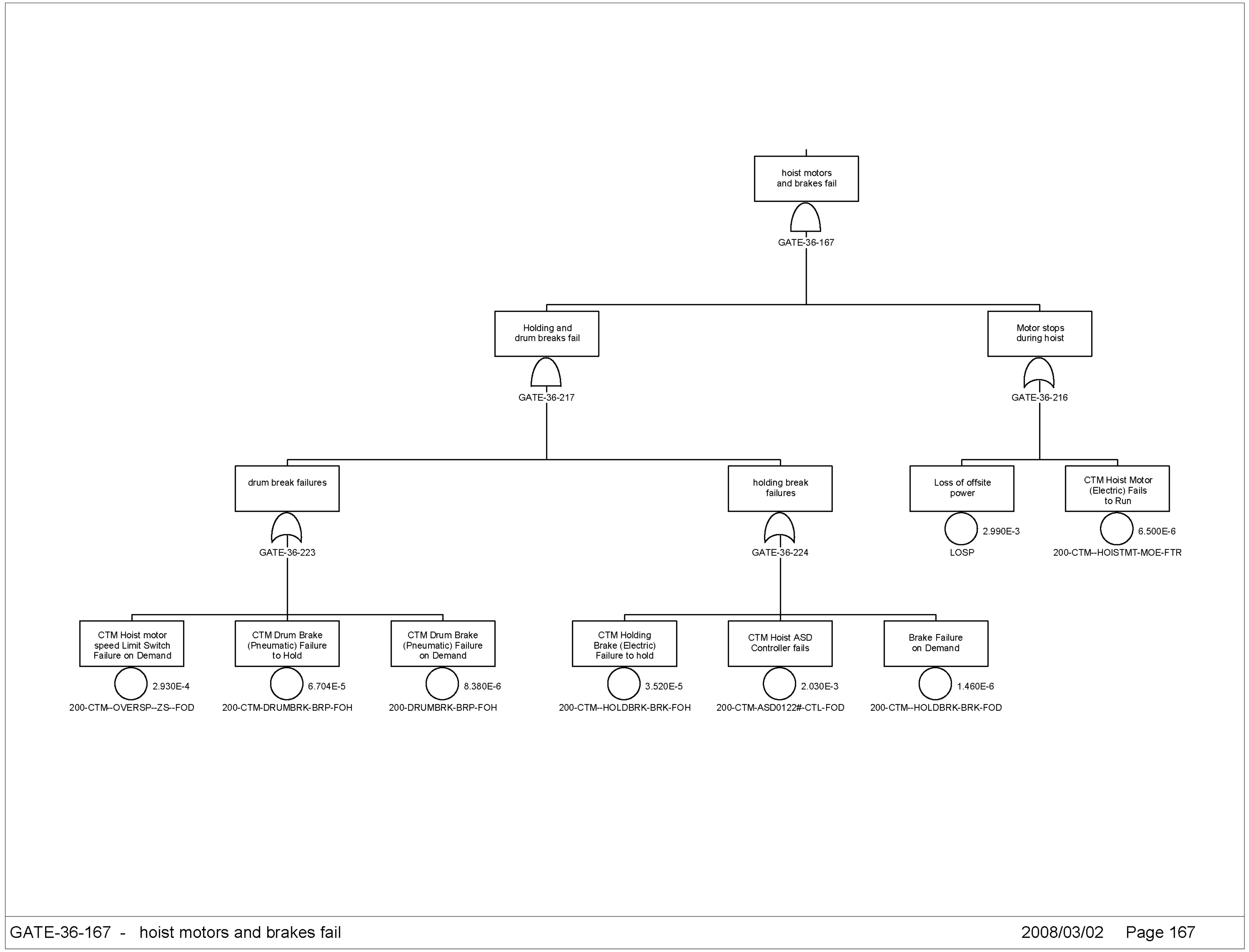
Source: Original

Figure B4.4-28. Drop of Object onto Cask (Sheet 6)



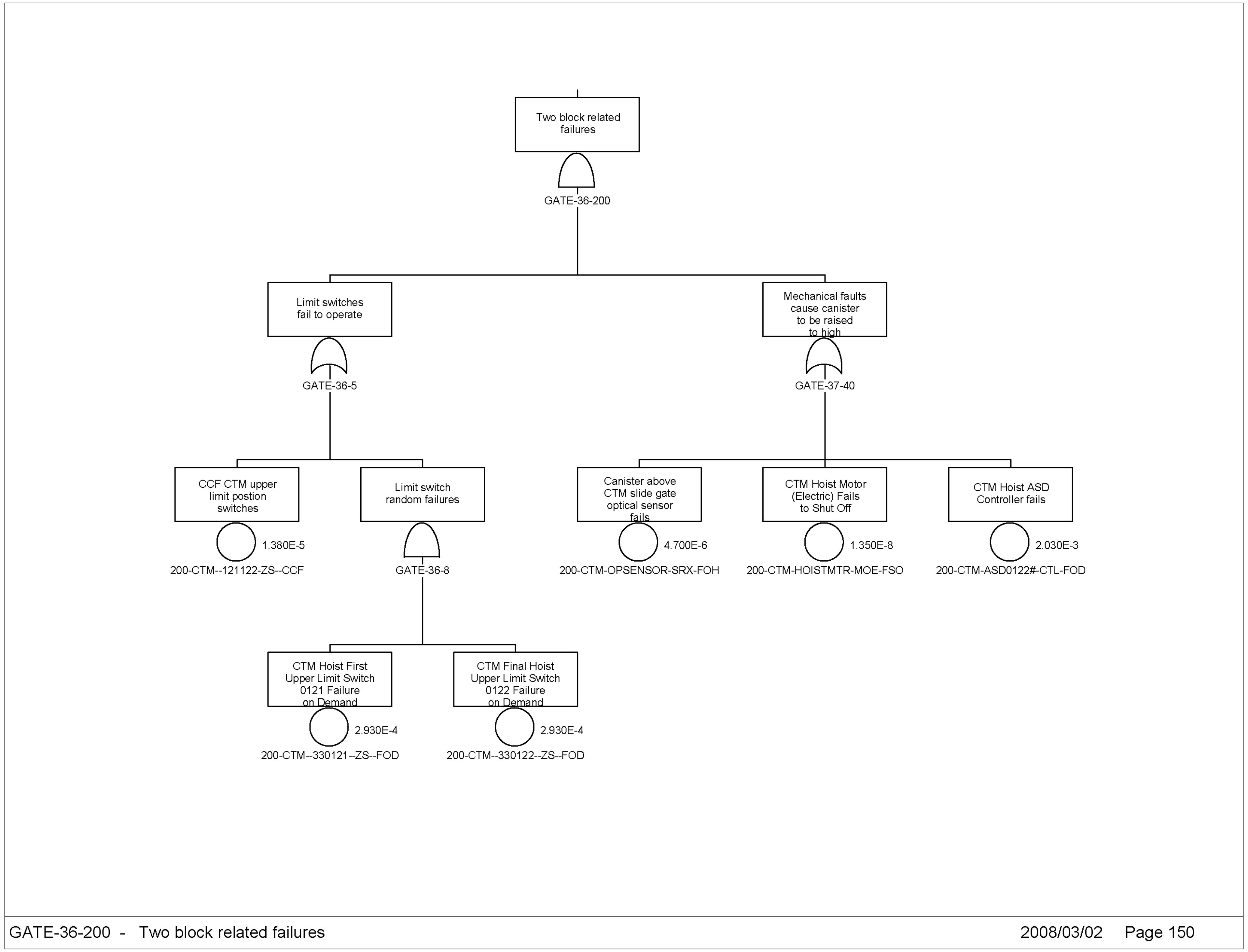
Source: Original

Figure B4.4-29. Drop of Object onto Cask (Sheet 7)



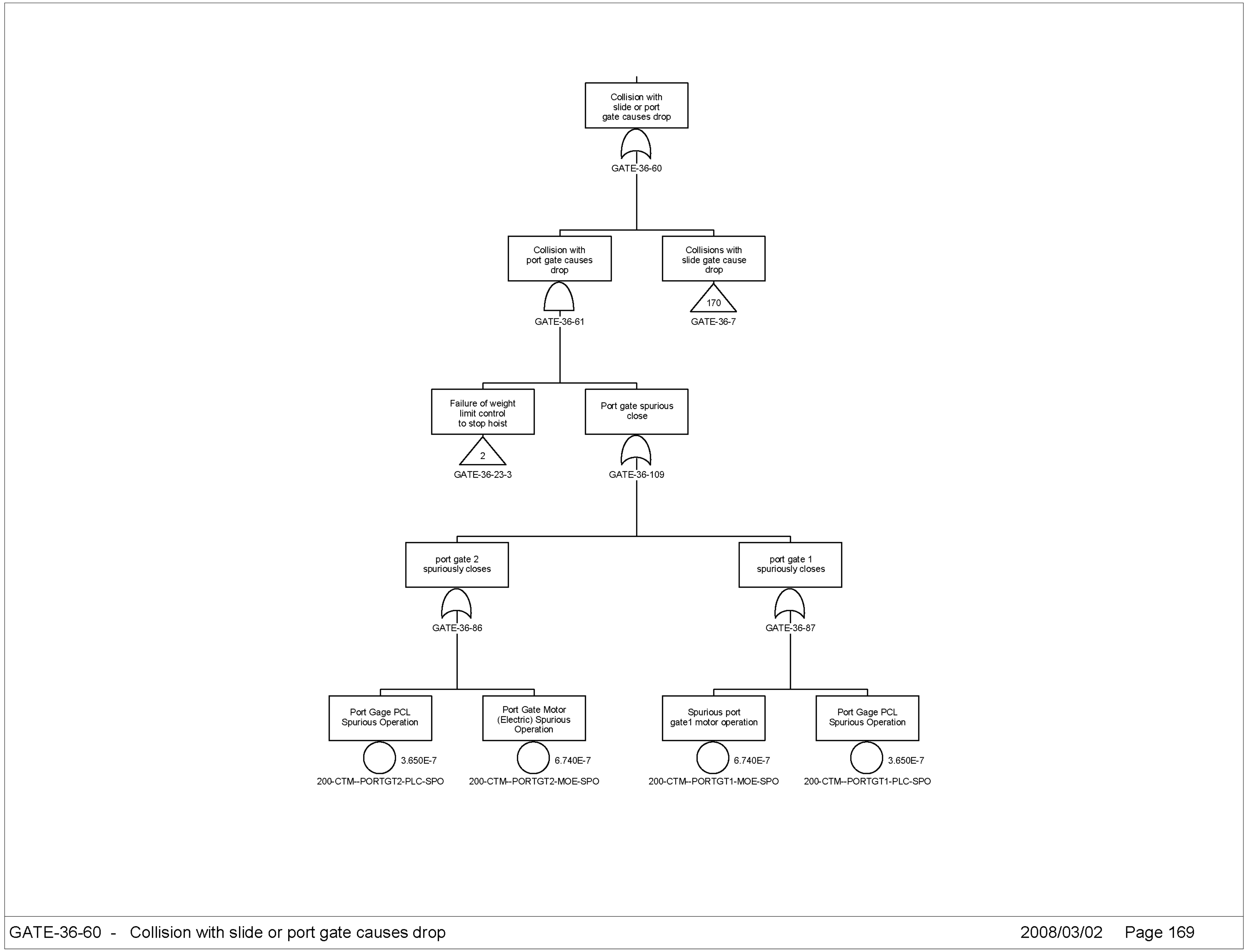
Source: Original

Figure B4.4-30. Drop of Object onto Cask (Sheet 8)



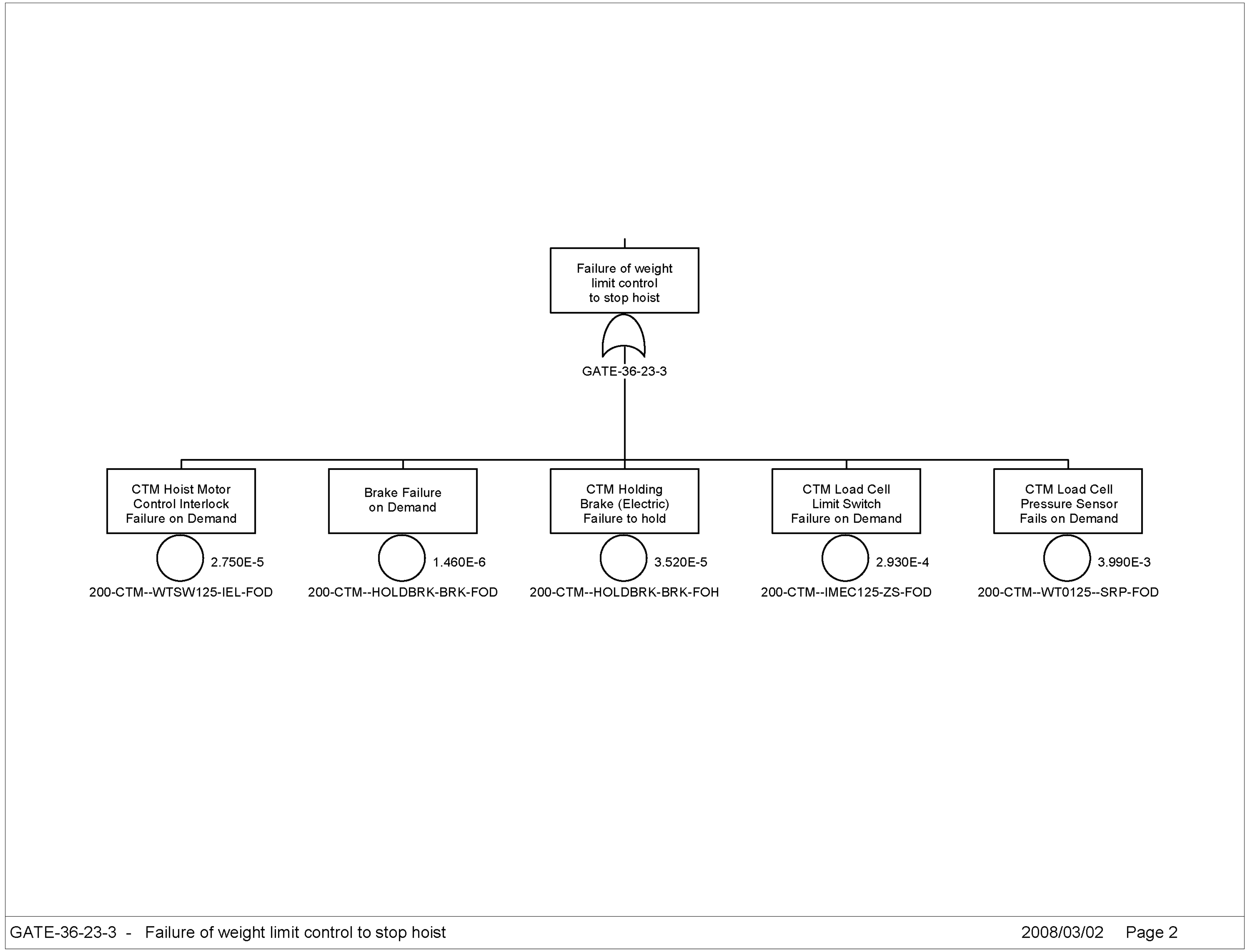
Source: Original

Figure B4.4-31. Drop of Object onto Cask (Sheet 9)



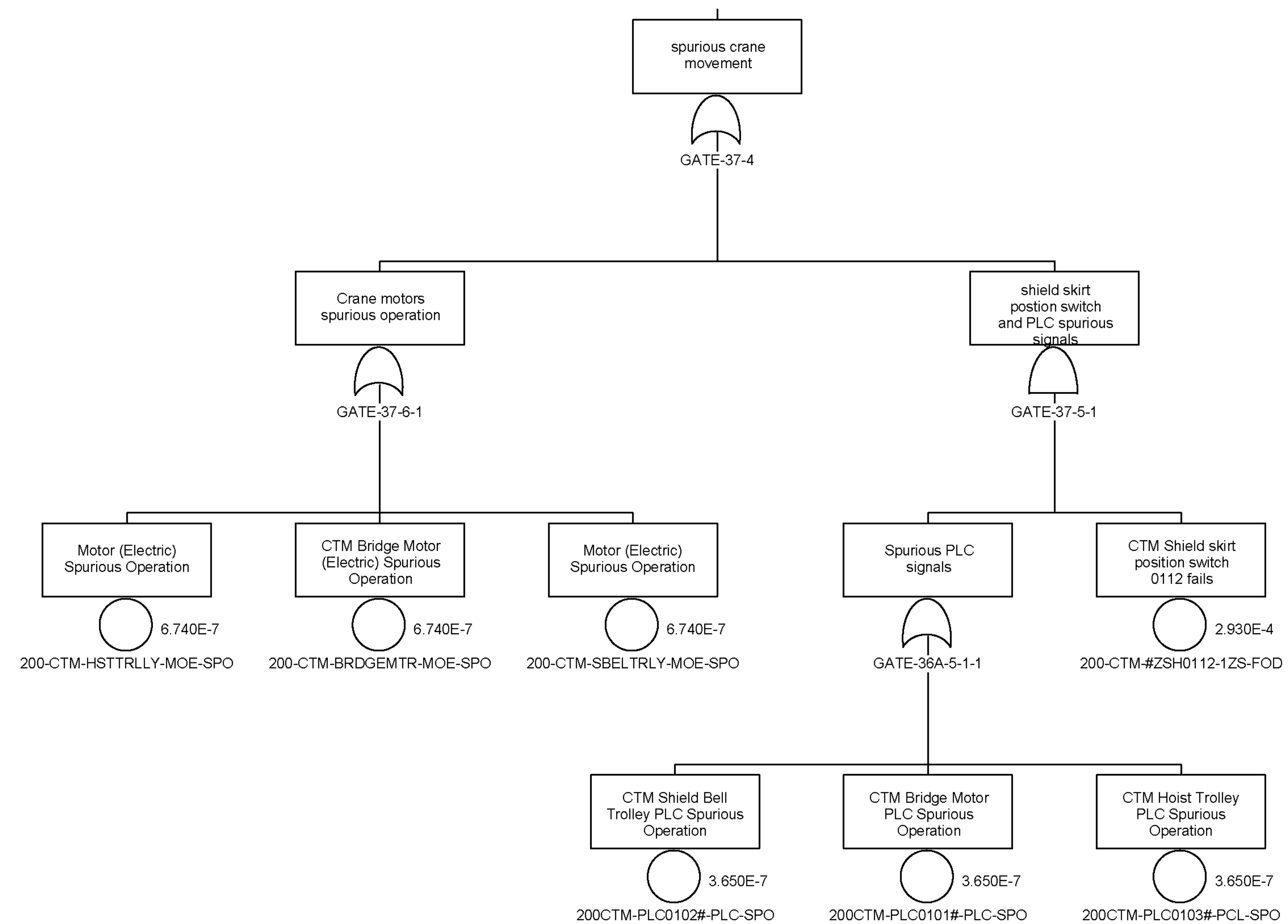
Source: Original

Figure B4.4-32. Drop of Object onto Cask (Sheet 10)



Source: Original

Figure B4.4-33. Drop of Object onto Cask (Sheet 11)



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Source: Original

Figure B4.4-34. Drop of Object onto Cask (Sheet 12)

B4.4.4 Canister Impact

B4.4.4.1 Description

Two fault trees were developed to address the potential for impacts to the canister. CTM movements that could result in a collision were modeled. Collisions between the CTM and a permanent structure were considered. Also, sudden spurious movements with the canister in a partially raised position were addressed.

B4.4.4.2 Success Criteria

Success criteria for the CTM is the prevention of a collision between the canister and the shield bell or Canister Transfer Room floor from any cause during the lift, lateral movement, and lower portions of the canister transfer.

B4.4.4.3 Design Requirements and Features

Requirements

Hard-wired interlocks are used to prevent inadvertent actions during CTM transfer operations. These include the following:

- An optical sensor at the bottom of the shield bell that, once it is cleared, stops the hoist and erases the lift command (can only lower hoist). This interlock is used only when lifting a canister.

- Above the ASD stop point is an upper limit switch which, when reached, stops the hoist from lifting. This first limit switch (first hoist upper limit) effectively erases the lift command (the hoist still has power) and the operator can only lower the hoist. Roughly one foot above that limit switch is another limit switch (final hoist upper limit) that, when reached, cuts off the power to the CTM hoist.

- An interlock between the shield skirt and port gate, which requires the shield skirt to be lowered in order for the port gate to open. There is a bypass for this interlock.

- An interlock between the CTM bridge/trolley travel and shield skirt position. Neither the CTM bridge nor the trolley can travel while the skirt is lowered.

- An interlock between the slide gate and shield skirt – the shield skirt cannot be raised unless the slide gate is closed. This interlock can be bypassed to allow the CTM to move with the slide gate open during lid removal.

- Interlocks preventing improper hoist movement. The hoist cannot move unless the shield skirt is lowered. This interlock is based on hoist movement, not position, so movement with the hoist too low is not precluded.

- The load cells which cut off power to the hoist when the crane capacity is exceeded.

- An interlock between the grapple position (fully engaged or fully disengaged) and hoist movement. The grapple automatically engages/disengages with a given object. The grapple must be positively engaged for the grapple engagement indicator to give a positive indication.

Features

Bridge and trolley motors are sized to limit lateral travel to less than 20 fpm, sufficient to ensure that in the event of an impact, impact forces are below the design limits of the canister.

The shield bell slide gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

The floor port gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

Hard wired interlocks used to prevent inadvertent actions during CTM transfer operations are ITS; PLCs are not ITS equipment.

The end stops for both the bridge and trolley end of travel end stops are capable of stopping the bridge/trolley at their maximum speed and preclude impact with any permanent structure.

The interlock between the grapple position and the operation of the hoist motor cannot be bypassed during CTM canister transfer operations.

B4.4.4.4 Fault Tree Model

The top event in this fault tree is "CTM Collision." The CTM collision fault tree addresses potential end of run over travel events and collisions between the CTM. Faults considered in the evaluation of this top event include: human events that contribute to a collision and mechanical (structural) failures of the CTM components (Figures B4.4-37 to B4.4-40). The interlocks intended to prevent improper CTM movement are included in the model.

B4.4.4.5 Basic Event Data

Table B4.4-9 contains a list of basic events used in the CTM fault tree. Included are the human failure events and the CCF events identified in the previous two sections. There are no maintenance-related failures associated with the CTM. The CTM is not in service while undergoing maintenance. Sensor failures that could be associated with the failure to restore from maintenance are not expected to contribute significantly to the overall sensor availability.

Table B4.4-9. Basic Event Probability for the CTM Fault Tree

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM-BREDGMTR-#CT-FOD	CTM Hand Held Radio Remote Controller Fails	1	4.000E-006	4.000E-006	0.000E+000	0.000E+000
200-CTM-BRIDGETR-#PR-FOH	CTM Bridge Passive restraint (end stops) Failure	3	1.949E-006	0.000E+000	4.450E-010	4.380E+003
200-CTM-BRIDGETR-MOE-FSO	CTM Bridge motor fails to stop	3	1.350E-008	0.000E+000	1.350E-008	1.000E+000
200-CTM-BRIDGMTR-IEL-FOD	CTM Shield Skirt-Bridge motor Interlock Failure	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-CTM-HSTTRLLY-IEL-FOD	CTM shield skirt Hoist Trolley motor Interlock Failure	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-CTM-SBELTRLY-IEL-FOD	CTM Shield Bell Trolley Interlock Failure	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-CTM-SKRTCTCT-SRP-FOD	CTM Skirt floor contact sensors fail	1	3.990E-003	3.990E-003	0.000E+000	0.000E+000
200-CTM-TROLLEYT-MOE-FSO	CTM Trolley motor fails to stop	3	1.350E-008	0.000E+000	1.350E-008	1.000E+000
200-CTM-TROLLYTR-#PR-FOH	CTM Trolley end run stops Failure	3	1.949E-006	0.000E+000	4.450E-010	4.380E+003
200-CTM-TROLYCNT-#HC-FOD	CTM trolley motor hand controller fails	1	1.740E-003	1.740E-003	0.000E+000	0.000E+000
200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	1	4.000E-008	4.000E-008	0.000E+000	0.000E+000
200-OPCTMIMPACT5-HFI-COD	Operator over runs travel - collides into end stop	1	1.000E+000	1.000E+000	0.000E+000	0.000E+000
200-CTM-BREDGMTR-#CT-FOD	CTM Hand Held Radio Remote Controller Fails	1	4.000E-006	4.000E-006	0.000E+000	0.000E+000
200-CTM-BRIDGETR-#PR-FOH	CTM Bridge Passive restraint (end stops) Failure	3	1.949E-006	0.000E+000	4.450E-010	4.380E+003
200-CTM-BRIDGETR-MOE-FSO	CTM Bridge motor fails to stop	3	1.350E-008	0.000E+000	1.350E-008	1.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; CTM = canister transfer machine; Fail. = failure; Miss. = mission; Prob. = probability.

Source: Original

The canister impact modeled by the fault tree is evaluated over a mission time of one hour. This mission time encompasses vertical lifting, lateral movement, and vertical lowering of the canister by the CTM. A longer mission time is also considered for specific components. For example, the fault tree accounts for the failure of standby components whose potential malfunction would remain hidden until they are tested. They are consequently evaluated over the interval of time between their test (mission time set to the average fault exposure time, one-half the test interval).

B4.4.4.5.1 Human Failure Events

Two basic events are associated with human error (Table B4.4-10). One addresses the movement of the CTM during a lift and the second addresses the potential overrun of the CTM (either the bridge trolley or the hoist/shield skirt trolley). The quantification of these events includes the probability of operator actions and the failure of ITS related interlocks intended to prevent such operator actions.

Table B4.4-10. Human Failure Events

Name	Description
200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor
200-OPCTMIMPACT5-HFI-COD	Operator over runs travel - collides into end stop

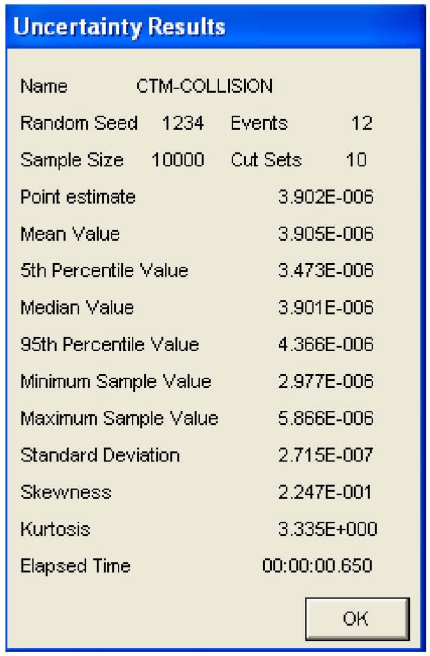
Source: Original

B4.4.4.5.2 Common-Cause Failures

There are no CCFs modeled in the CTM collision fault tree.

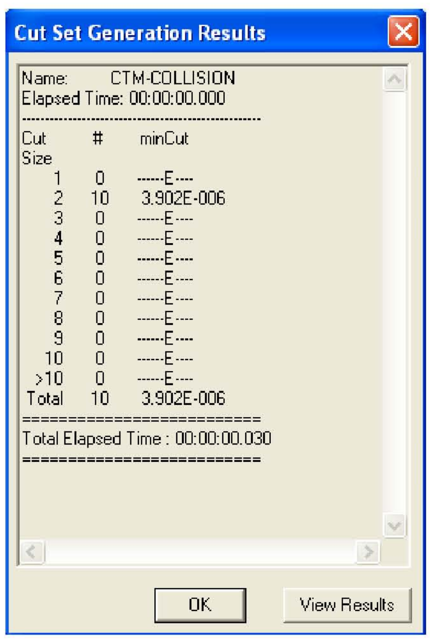
B4.4.4.6 Uncertainty and Cut Set Generation

Figure B4.4-35 contains the uncertainty results obtained from running the fault trees for the CTM Collision with a cutoff probability of 1E-15. Figure B4.4-36 provides the cut set generation results for the CTM Collision fault tree.



Source: Original

Figure B4.4-35. Uncertainty Results of the CTM Collision Fault Tree



Source: Original

Figure B4.4-36. Cut Set Generation Results for the CTM Collision Fault Tree

B4.4.4.7 Cut Sets

Table B4.4-11 contains the cut sets for the CTM collision fault tree.

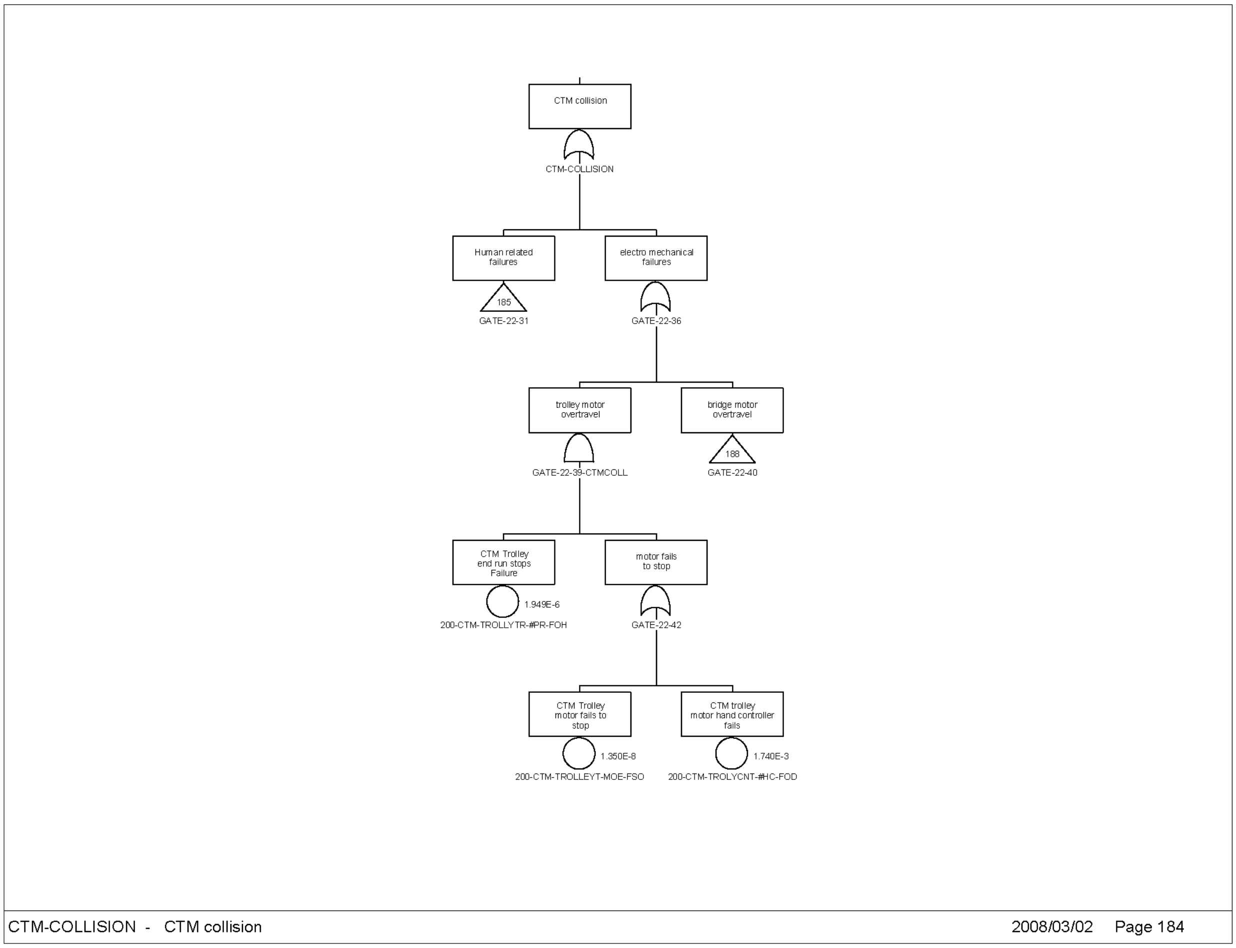
Table B4.4-11. Dominant Cut sets for the CTM Collision Fault Tree

% Total	% Cut set	Prob./ Frequency	Basic Event	Description	Event Prob.
49.95	49.95	1.949E-06	200-CTM-TROLLYTR-#PR-FOH	CTM Trolley end run stops Failure	1.949E-06
			200-OPCTMIMPACT5-HFI-COD	Operator over runs travel - collides into end stop	1.000E+00
99.90	49.95	1.949E-06	200-CTM-BRIDGETR-#PR-FOH	CTM Bridge Passive restraint (end stops) Failure	1.949E-06
			200-OPCTMIMPACT5-HFI-COD	Operator over runs travel - collides into end stop	1.000E+00
99.99	0.09	3.391E-09	200-CTM-TROLLYTR-#PR-FOH	CTM Trolley end run stops Failure	1.949E-06
			200-CTM-TROLYCNT-#HC-FOD	CTM trolley motor hand controller fails	1.740E-03
99.99	0.00	1.596E-10	200-CTM-SKRTCTCT-SRP-FOD	CTM Skirt floor contact sensors fail	3.990E-03
			200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
99.99	0.00	7.796E-12	200-CTM-BREDGMTR-#CT-FOD	CTM Hand Held Radio Remote Controller Fails	4.000E-06
			200-CTM-BRIDGETR-#PR-FOH	CTM Bridge Passive restraint (end stops) Failure	1.949E-06
99.99	0.00	1.100E-12	200-CTM-BRIDGMTR-IEL-FOD	CTM Shield Skirt-Bridge motor Interlock Failure	2.750E-05
			200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
99.99	0.00	1.100E-12	200-CTM-HSTTRLLY-IEL-FOD	CTM shield skirt Hoist Trolley motor Interlock Failure	2.750E-05
			200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
99.99	0.00	1.100E-12	200-CTM-SBELTRLY-IEL-FOD	CTM Shield Bell Trolley Interlock Failure	2.750E-05
			200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
99.99	0.00	2.631E-14	200-CTM-TROLLEYT-MOE-FSO	CTM Trolley motor fails to stop	1.350E-08
			200-CTM-TROLLYTR-#PR-FOH	CTM Trolley end run stops Failure	1.949E-06
99.99	0.00	2.631E-14	200-CTM-BRIDGETR-#PR-FOH	CTM Bridge Passive restraint (end stops) Failure	1.949E-06
			200-CTM-BRIDGETR-MOE-FSO	CTM Bridge motor fails to stop	1.350E-08

NOTE: CTM = canister transfer machine; Prob. = probability.

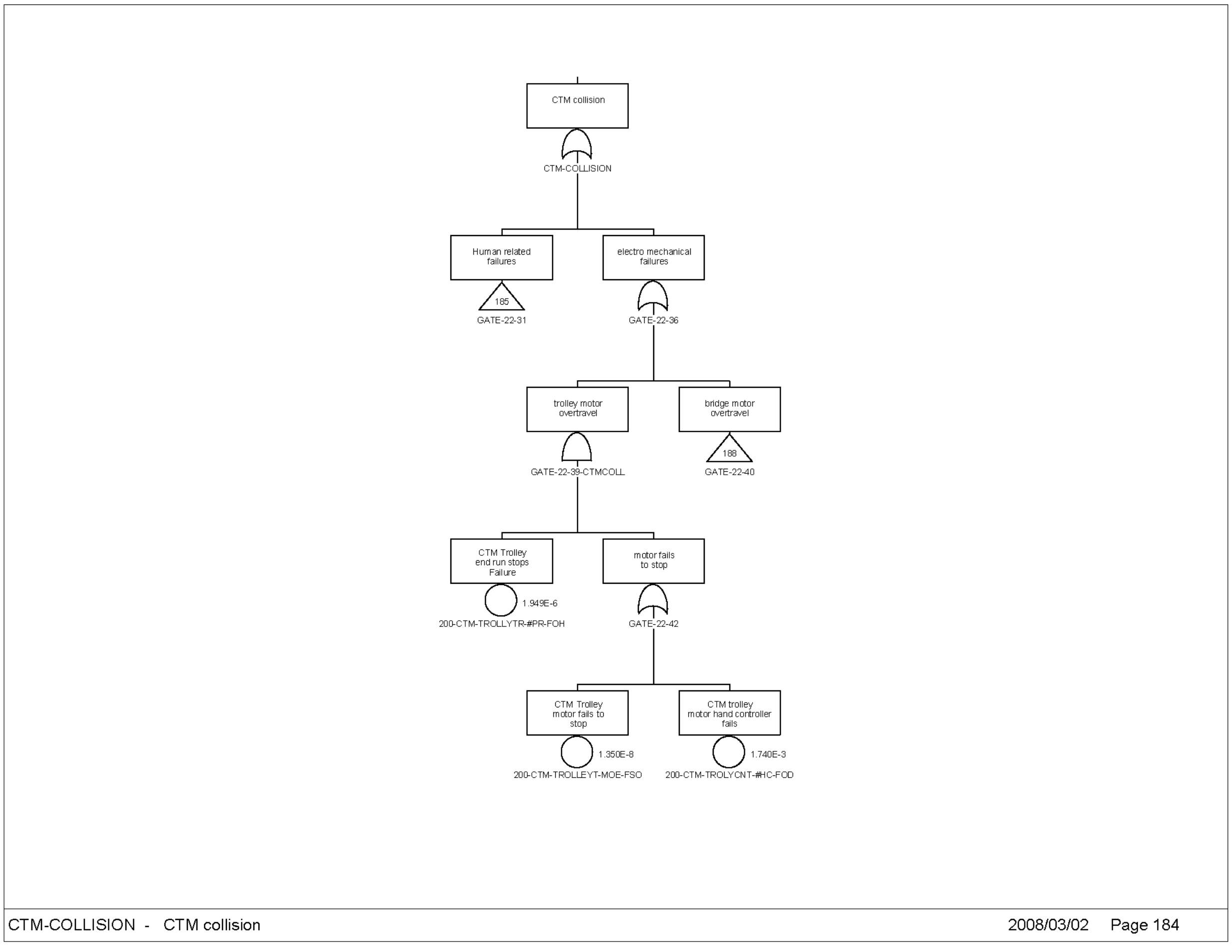
Source: Original

B4.4.4.8 Fault Trees



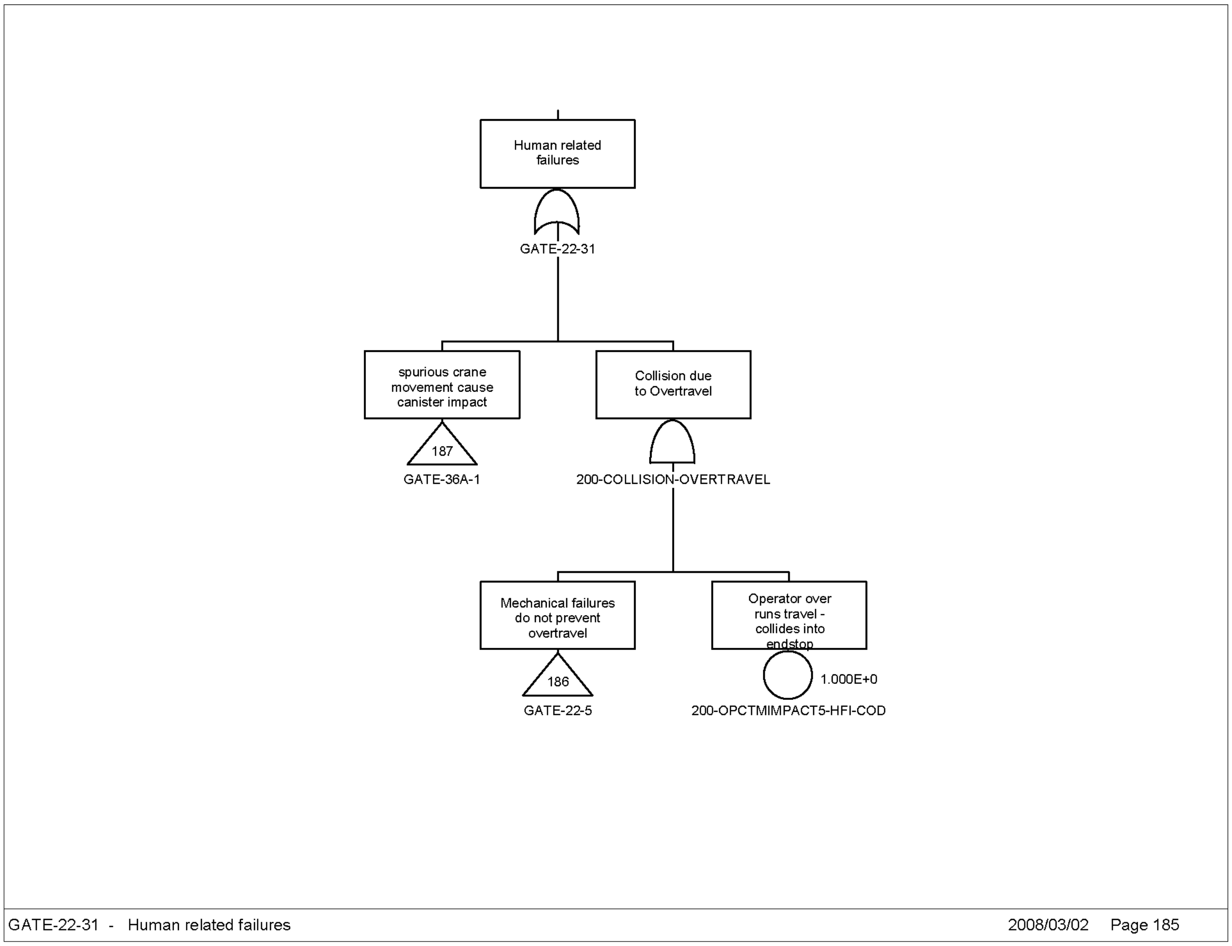
Source: Original

Figure B4.4-37. CTM Collision (Sheet 1)



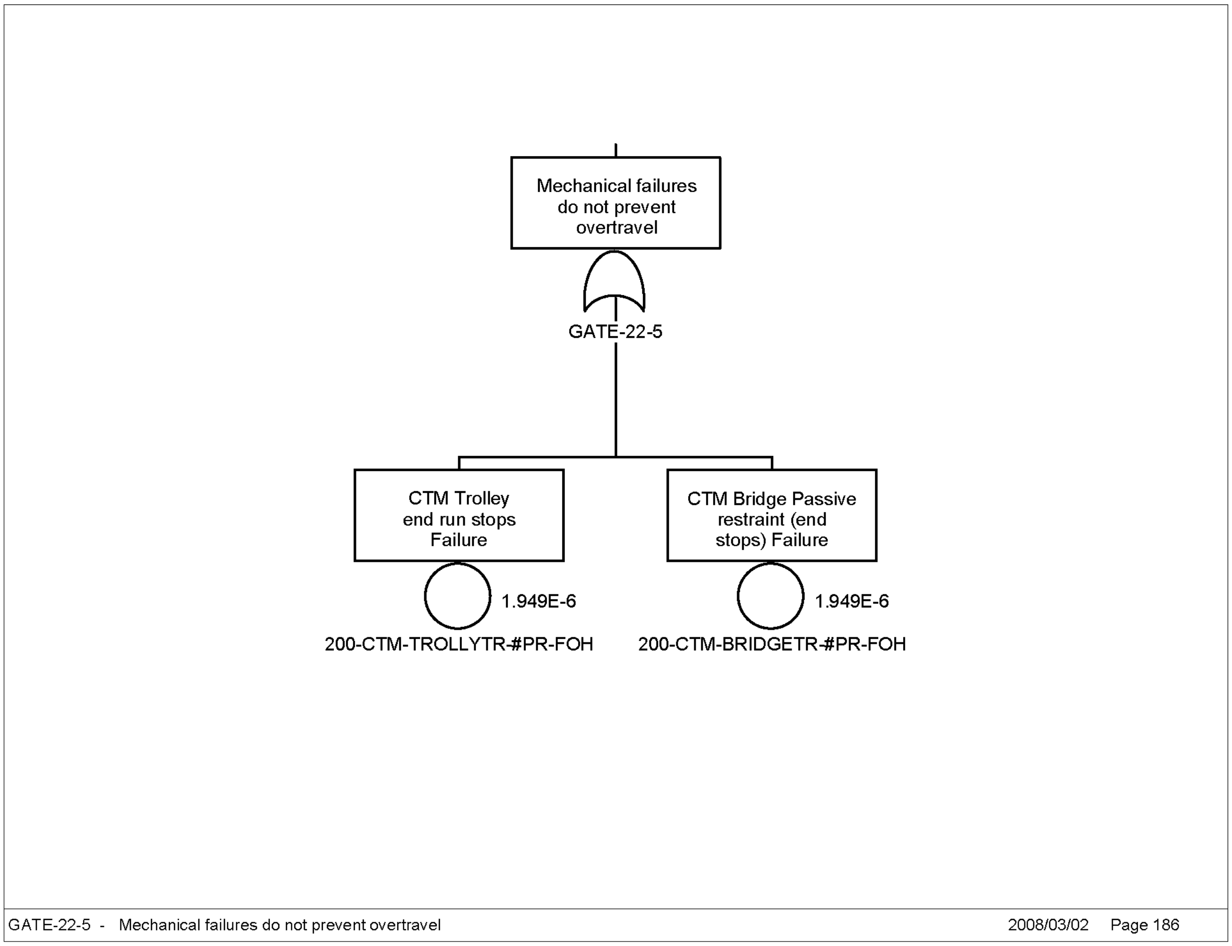
Source: Original

Figure B4.4-38. CTM Collision (Sheet 2)



Source: Original

Figure B4.4-39. CTM Collision (Sheet 3)



Source: Original

Figure B4.4-40. CTM Collision (Sheet 4)

B4.4.5 CTM Movement Subjects Canister to Shearing Forces

B4.4.5.1 Description

A fault tree was developed to address the potential for movement of the CTM when the canister being transferred is being lifted and is between the RF floors. Movement initiated by the bridge or trolley motors could result in shear forces being applied to the canister should it be lifted when the CTM moves away from the floor port opening.

B4.4.5.2 Success Criteria

Success criteria for the CTM is the prevention of CTM movement that could result in a shearing force being applied to the canister when the canister is being lifted and is between the first and second floors of the RF during the lift portions of the canister transfer.

B4.4.5.3 Design Requirements and Features

Requirements

Hard-wired interlocks are used to prevent inadvertent actions during CTM transfer operations. These include the following:

- An optical sensor at the bottom of the shield bell that, once it is cleared, stops the hoist and erase the lift command (can only lower hoist). This interlock is used only when lifting a canister.

- Above the ASD stop point is an upper limit switch which, when reached, stops the hoist from lifting. This first limit switch (first hoist upper limit) effectively erases the lift command (the hoist still has power) and the operator can only lower the hoist. Roughly a foot above that limit switch is another limit switch (final hoist upper limit) that, when reached, cuts off the power to the CTM hoist.

- An interlock between the shield skirt and port gate which requires the shield skirt to be lowered in order for the port gate to open. There is a bypass for this interlock.

- An interlock between the CTM bridge/trolley travel and shield skirt position. Neither the CTM bridge nor the trolley can travel while the skirt is lowered.

- An interlock between the slide gate and shield skirt – the shield skirt cannot be raised unless the slide gate is closed. This interlock can be bypassed to allow the CTM to move with the slide gate open during lid removal.

- Interlocks preventing improper hoist movement. The hoist cannot move unless the shield skirt is lowered. This interlock is based on hoist movement, not position, so movement with the hoist too low is not precluded.

- The load cells cut off power to the hoist when the crane capacity is exceeded.

- An interlock between the grapple position (fully engaged or fully disengaged) and hoist movement. The grapple automatically engages/disengages with a given object. The grapple must be positively engaged for the grapple engagement indicator to give a positive indication.

Features

Bridge and trolley motors are sized to limit lateral travel to less than 20 fpm, sufficient to ensure that in the event of an impact, impact forces are below the design limits of the canister.

The shield bell slide gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

The floor port gate motors are sized so that they are incapable of exerting sufficient force to damage any canister given an inadvertent closure of the gate when a canister is suspended in the gate closure path.

Hard wired interlocks used to prevent inadvertent actions during CTM transfer operations are ITS; PLCs are not ITS equipment.

The end stops for both the bridge and trolley end of travel end stops are capable of stopping the bridge/trolley at their maximum speed and preclude impact with any permanent structure.

The interlock between the grapple position and the operation of the hoist motor cannot be bypassed during CTM canister transfer operations.

B4.4.5.4 Fault Tree Model

The top event in this fault tree is "CTM Movement Causes Canister Shear." The fault tree includes events (mechanical control failures and human actions, considered in conjunction with the interlocks intended to prevent the erroneous human action) that can initiate a spurious movement of the CTM trolley or bridge while the canister is between the first and second floors of the RF (Figures B4.4-43, B4.4-44 and B4.4-45).

B4.4.5.5 Basic Event Data

Table B4.4-12 contains a list of basic events used in the CTM shear fault tree. Included are the human failure events and the CCF events identified in the following two sections. There are no maintenance-related failures associated with the CTM. The CTM is not in service while undergoing maintenance. Sensor failures that could be associated with the failure to restore from maintenance are not expected to contribute significantly to the overall sensor availability.

Table B4.4-12. Basic Event Probability for the CTM Fault Trees

Name	Description	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM-BIDGMTR-#TL-FOH	CTM Bridge motor Torque limiter Failure	3	2.856E-02	0.000E+00	8.050E-05	3.600E+02
200-CTM-BRIDGMTS-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation -shear	3	3.370E-08	0.000E+00	6.740E-07	5.000E-02
200-CTM-HSTTRLLS-MOE-SPO	CTM Hoist Trolley Motor (Electric) Spurious Operation m- shear	3	3.370E-08	0.000E+00	6.740E-07	5.000E-02
200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motor Torque limiter Failure	3	2.856E-02	0.000E+00	8.050E-05	3.600E+02
200-CTM-PLC0101S-PLC-SPO	CTM Bridge Motor PLC Spurious Operation - shear	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM-PLC0102S-PLC-SPO	CTM Shield Bell Trolley PLC Spurious Operation - shear	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM-PLC0103S-PLC-SPO	CTM Hoist Trolley PLC Spurious Operation -shear	3	3.650E-07	0.000E+00	3.650E-07	0.000E+00
200-CTM-SBELTRLS-MOE-SPO	Motor (Electric) Spurious Operation	3	6.740E-08	0.000E+00	6.740E-07	1.000E-01
200-CTM-SBELTRLY-#TL-FOH	CTM Shield Bell Motor Torque limiter Failure	3	2.856E-02	0.000E+00	8.050E-05	3.600E+02
200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	1	4.000E-08	4.000E-08	0.000E+00	0.000E+00
200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	1	2.930E-04	2.930E-04	0.000E+00	0.000E+00
200-CTM-BIDGMTR-#TL-FOH	CTM Bridge motor Torque limiter Failure	3	2.856E-02	0.000E+00	8.050E-05	3.600E+02
200-CTM-BRIDGMTS-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation -shear	3	3.370E-08	0.000E+00	6.740E-07	5.000E-02
200-CTM-HSTTRLLS-MOE-SPO	CTM Hoist Trolley Motor (Electric) Spurious Operation m- shear	3	3.370E-08	0.000E+00	6.740E-07	5.000E-02

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; CTM = canister transfer machine; CTM = canister transfer machine; Fail. = failure; Miss. = mission; PLC = programmable logic controller; Prob. = probability.

Source: Original

The shear impact probability modeled by the fault tree is evaluated over a mission time of one-tenth of an hour (limited to the time the canister is being lifted and is between the first and second floors). A longer mission time is also considered for specific components. For example, the fault tree accounts for the failure of standby components whose potential malfunction would remain hidden until they are tested. They are consequently evaluated over the interval of time between their tests, and the mission time is assigned a value of the average fault exposure time, half the test interval.

B4.4.5.5.1 Human Failure Events

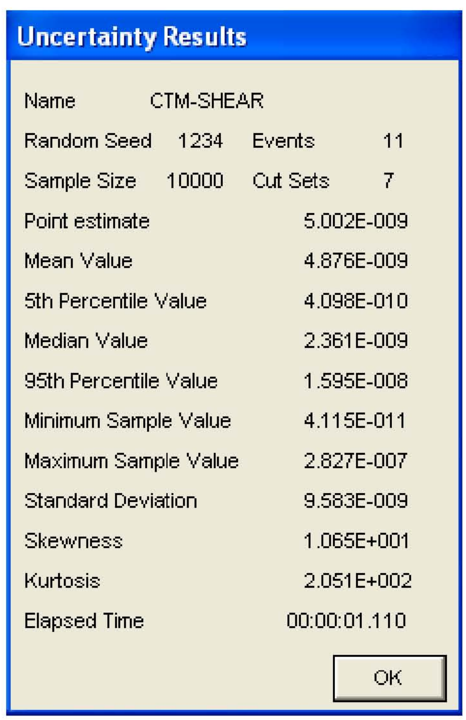
One basic event is associated with human error: 200-OPCTMIMPACT1-HFI-COD (operator moves trolley/crane with canister below floor). This event addresses the possible operator initiated movement of the bridge or trolleys while a canister is being lifted and is between RF floors.

B4.4.5.5.2 Common-Cause Failures

No CCFs apply to this fault tree.

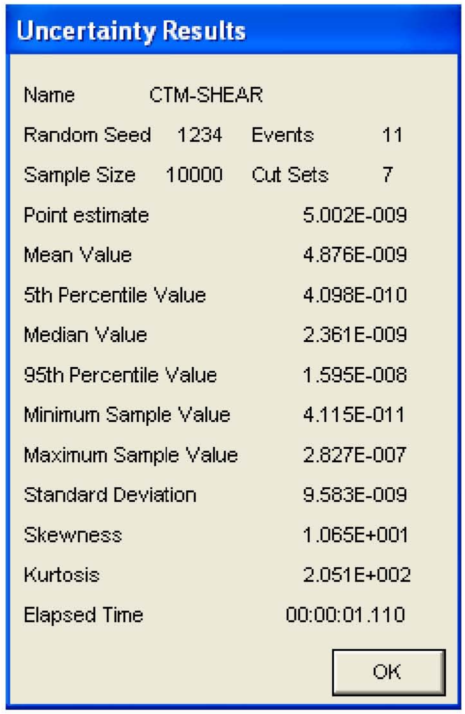
B4.4.5.6 Uncertainty and Cut Set Generation

Figure B4.4-41 contains the uncertainty results obtained from running the fault trees for CTM-SHEAR, with a cutoff probability of 1E-15. Figure B4.4-42 provides the cut set generation results for the CTM-SHEAR fault tree.



Source: Original

Figure B4.4-41. Uncertainty Results of the CTM Shear Fault Tree



Source: Original

Figure B4.4-42. Cut Set Generation Results for the CTM Shear Fault Tree

B4.4.4.7 Cut Sets

Table B4.4-13 contains the cut sets for the CTM Shear fault tree.

Table B4.4-13. Dominant Cut Sets for the CTM Collision Fault Tree

% Total	% Cut set	Prob./ Frequency	Basic Event	Description	Event Prob.
38.49	38.49	1.925E-09	200-CTM-SBELTRLS-MOE-SPO	Motor (Electric) Spurious Operation	6.740E-08
			200-CTM-SBELTRLY-#TL-FOH	CTM Shield Bell Motor Torque limiter Failure	2.856E-02
61.33	22.84	1.143E-09	200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motor Torque limiter Failure	2.856E-02
			200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
80.57	19.24	9.626E-10	200-CTM-BIDGMTR-#TL-FOH	CTM Bridge motor Torque limiter Failure	2.856E-02
			200-CTM-BRIDGMTS-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation -shear	3.370E-08
99.81	19.24	9.626E-10	200-CTM-HSTTRLLS-MOE-SPO	CTM Hoist Trolley Motor (Electric) Spurious Operation m- shear	3.370E-08
			200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motor Torque limiter Failure	2.856E-02
99.87	0.06	3.055E-12	200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	2.930E-04
			200-CTM-PLC0102S-PLC-SPO	CTM Shield Bell Trolley PLC Spurious Operation -shear	3.650E-07
			200-CTM-SBELTRLY-#TL-FOH	CTM Shield Bell Motor Torque limiter Failure	2.856E-02
99.93	0.06	3.055E-12	200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	2.930E-04
			200-CTM-BIDGMTR-#TL-FOH	CTM Bridge motor Torque limiter Failure	2.856E-02
			200-CTM-PLC0101S-PLC-SPO	CTM Bridge Motor PLC Spurious Operation - shear	3.650E-07
99.99	0.06	3.055E-12	200-CTM-#ZSH0112-1ZS-FOD	CTM Shield skirt position switch 0112 fails	2.930E-04
			200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motor Torque limiter Failure	2.856E-02
			200-CTM-PLC0103S-PLC-SPO	CTM Hoist Trolley PLC Spurious Operation -shear	3.650E-07
38.49	38.49	1.925E-09	200-CTM-SBELTRLS-MOE-SPO	Motor (Electric) Spurious Operation	6.740E-08
			200-CTM-SBELTRLY-#TL-FOH	CTM Shield Bell Motor Torque limiter Failure	2.856E-02

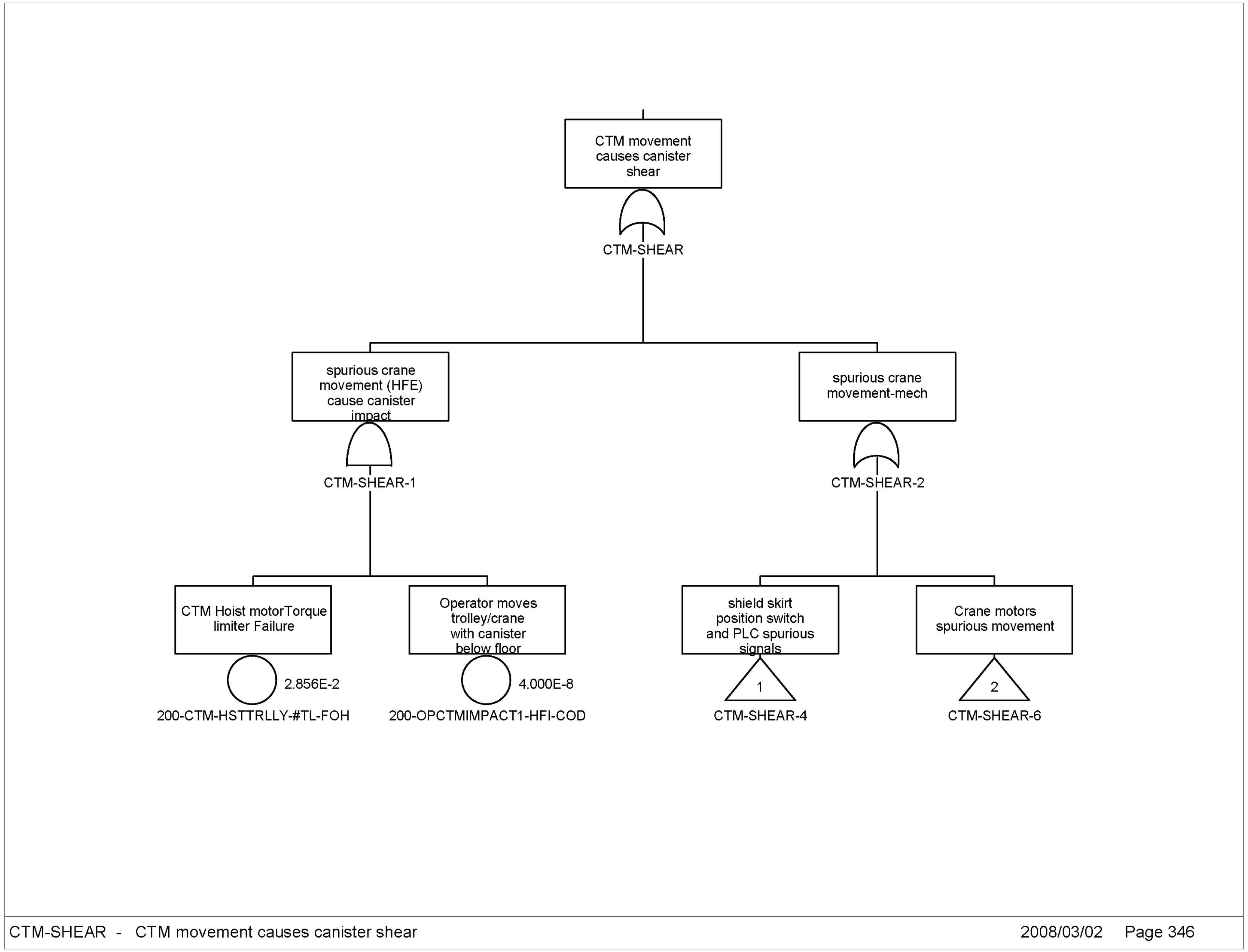
Table B4.4-13. Dominant Cut Sets for the CTM Collision Fault Tree (Continued)

% Total	% Cut set	Prob./ Frequency	Basic Event	Description	Event Prob.
61.33	22.84	1.143E-09	200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motor Torque limiter Failure	2.856E-02
			200-OPCTMIMPACT1-HFI-COD	Operator moves trolley/crane with canister below floor	4.000E-08
80.57	19.24	9.626E-10	200-CTM-BIDGMTR-#TL-FOH	CTM Bridge motor Torque limiter Failure	2.856E-02
			200-CTM-BRIDGMTS-MOE-SPO	CTM Bridge Motor (Electric) Spurious Operation -shear	3.370E-08
99.81	19.24	9.626E-10	200-CTM-HSTTRLLS-MOE-SPO	CTM Hoist Trolley Motor (Electric) Spurious Operation m- shear	3.370E-08
			200-CTM-HSTTRLLY-#TL-FOH	CTM Hoist motor Torque limiter Failure	2.856E-02

NOTE: CTM = canister transfer machine; HRA = human reliability analysis; PLC = programmable logic controller; Prob. = probability.

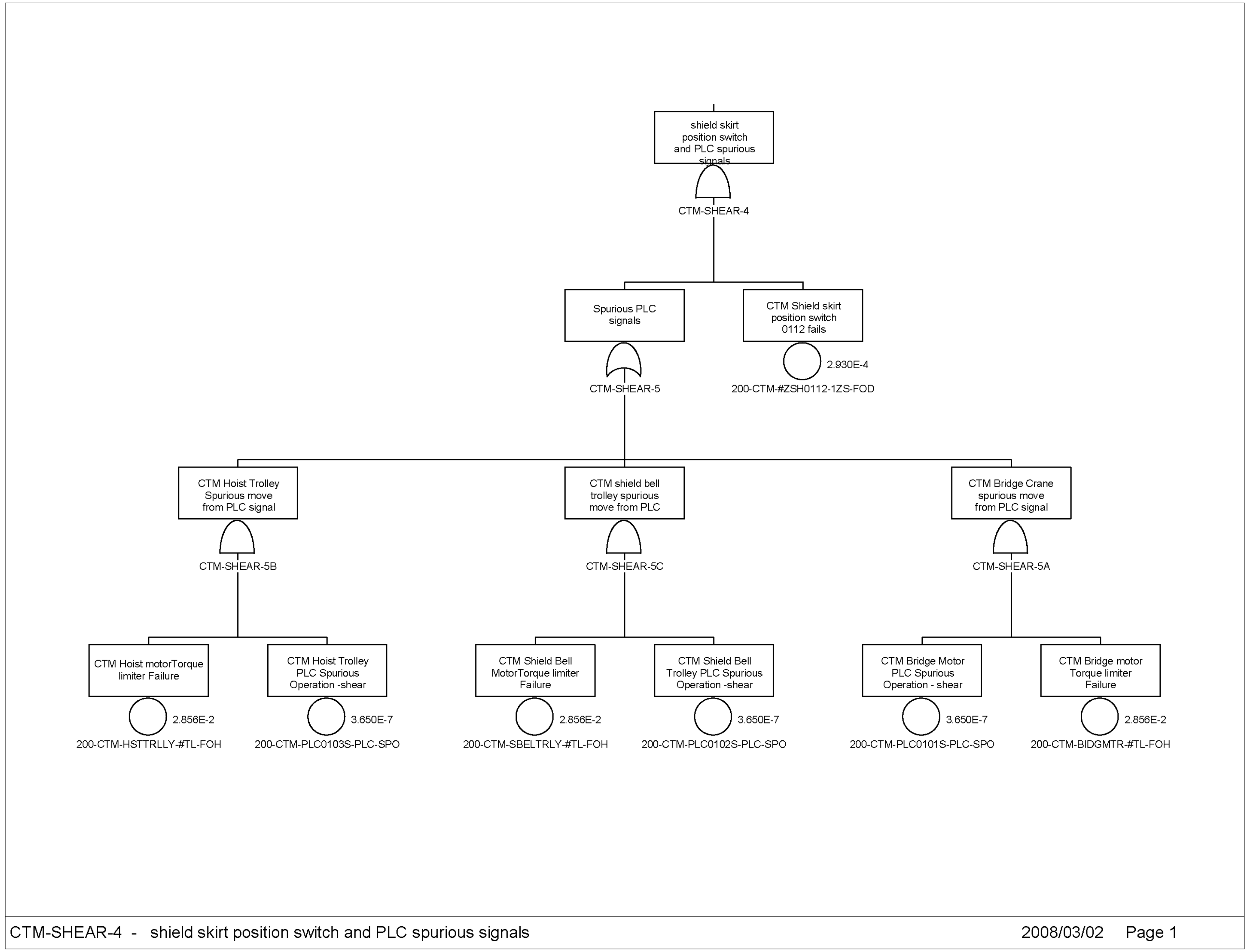
Source: Original

B4.4.5.8 Fault Tree



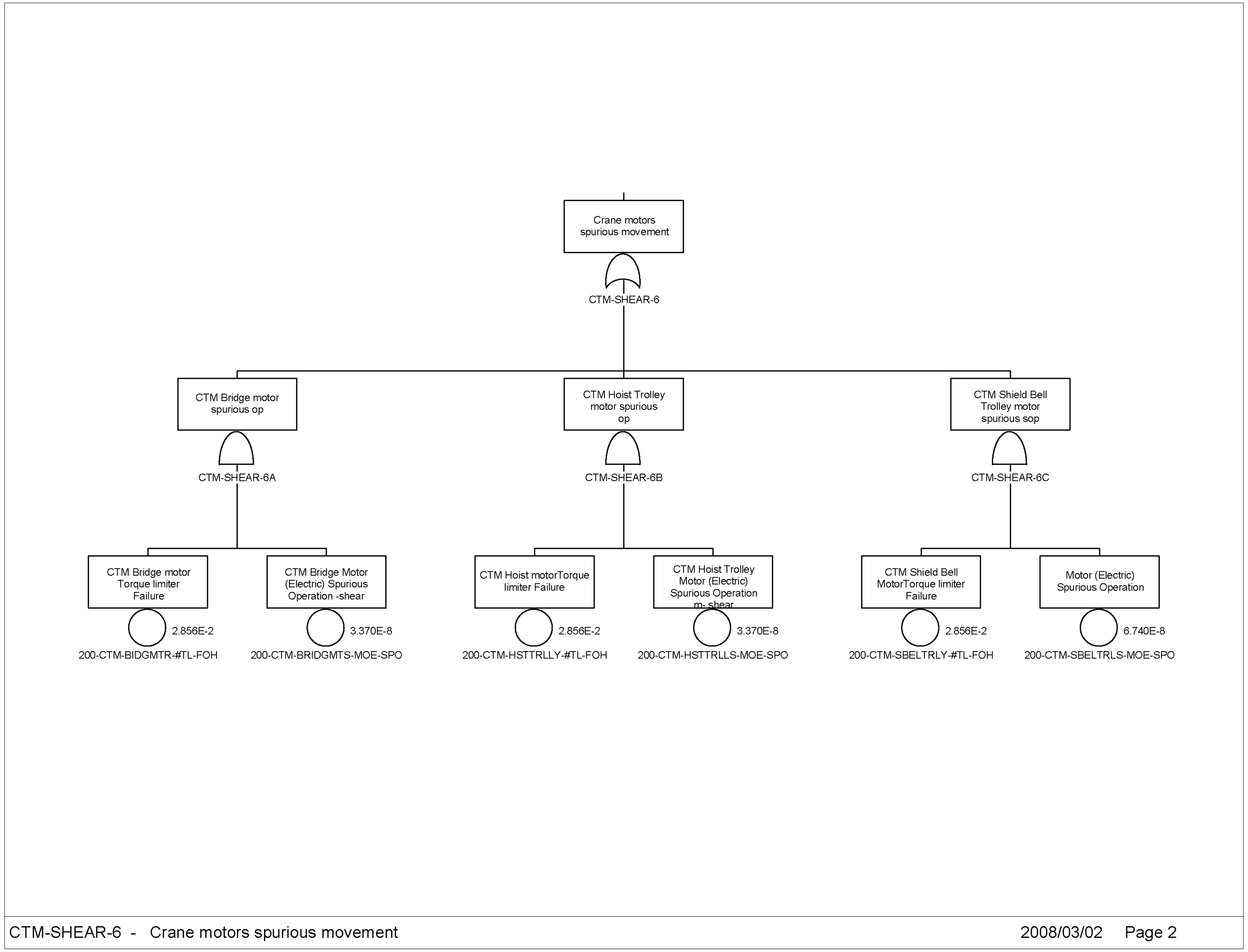
Source: Original

Figure B4.4-43. CTM Shear (Sheet 1)



Source: Original

Figure B4.4-44. CTM Shear (Sheet 2)



Source: Original

Figure B4.4-45. CTM Shear (Sheet 3)

B5 CASK TRACTOR AND CASK TRANSFER TRAILER FAULT TREE ANALYSIS

B5.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

B5.1.1 BSC (Bechtel SAIC Company) 2007. *Aging Facility Cask Transfer Trailers Mechanical Equipment Envelope.* 170-MJ0-HAT0-00201-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070518.0002.

B5.1.2 BSC 2007. *Yucca Mountain Project Engineering Specification for Cask Tractor and Cask Transfer Trailers.* 000-3PS-HAT0-00300-000 REV 000. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071006.0004.

B5.2 HORIZONTAL CASK TRACTOR AND TRAILER DESCRIPTION

B5.2.1 Overview

The cask tractor and the cask transfer trailer are collectively called the horizontal cask tractor and trailer (HCTT). This equipment provides the following functions as described in Section 3.1.1 of *Yucca Mountain Project Engineering Specification for Cask Tractor and Cask Transfer Trailers* (Ref. B5.1.2):

The function of the cask tractor coupled with the cask transfer trailer is to:

- Move a transportation cask loaded with a horizontal DPC from the RF to a horizontal aging module (HAM) located on aging pad 17R.
- Retrieve a horizontal DPC from the HAM, place it into the horizontal shielded transfer cask, and transport it to the WHF.

For fault tree models in SAPHIRE, the cask tractor and cask transfer trailer are collectively referred to in the code as an HCTT.

B5.2.2 Physical Description

The cask tractor is a large, four-wheel drive diesel tractor designed specifically for pulling the cask transfer trailer. The cask tractor has redundant brakes in addition to having a fail-safe emergency brake. The cask trailer has non-driven hydraulic pendular axles with a minimum of four tires per axle to ensure the cask remains level during transportation across uneven terrain. In addition to the pendular axles, the trailer has three other hydraulic systems: (1) stabilizing jacks, (2) a cask support skid and positioning system, and (3) a hydraulic ram. The cask tractor and cask transfer trailer are depicted in *Aging Facility Cask Transfer Trailers Mechanical Equipment Envelope* (Ref. B5.1.1).

B5.3 DEPENDENCE AND INTERACTIONS ANALYSIS

Dependencies are broken down into five categories with respect to their interactions with SSCs. The five areas considered are addressed in Table B5.3-1 with the following dependencies:

1. Functional dependence
2. Environmental dependence
3. Spatial dependence
4. Human dependence
5. Failures based on external events.

Table B5.3-1. Dependencies and Interactions Analysis

Systems, Structures, and Components	Dependencies and Interactions				
	Functional	Environmental	Spatial	Human	External Events
Hydraulic pendular axles	Vertical support and leveling during transport and load/unload	—	—	—	—
Hydraulic stabilizing jacks	Redundant vertical support during load/unload	—	—	—	—
Tractor brakes	Sufficient to stop conveyance with failed trailer brakes	—	—	—	—
Cask transfer trailer brakes	Sufficient to stop conveyance on failed tractor brakes	—	—	—	—
Vehicle steering, control, and speed limiter	Tractor/trailer control	—	—	—Collision —Overspeed	—

Source: Original

B5.4 HORIZONTAL CASK TRACTOR AND TRAILER FAILURE SCENARIOS

A cask tractor and cask transfer trailer collision is the only failure scenario modeled. A rollover scenario was also considered, but is screened-out per Attachment E.

B5.4.1 Horizontal Cask Tractor and Trailer Collision

B5.4.1.1 Description

There are two situations modeled where a cask tractor and cask transfer trailer collision may occur and each has a unique vehicle configuration: (1) during the loading and unloading of the DPCs (the trailer is unhitched from the tractor), and (2) during transport between the facilities and HAMs when the tractor is pulling the trailer.

B5.4.1.2 Success Criteria

A collision is defined as any undesired contact of the cask tractor and cask transfer trailer with another vehicle or facility structure or equipment. Any of the steering, braking, and hydraulic system can cause this to occur, in addition to operator error.

B5.4.1.3 Design Requirements and Features

The tractor brakes are a redundant–brake design and include a backup system with a split master cylinder and an indicator light inside the cabin to warn an operator if one of the systems fails (Ref. B5.1.2, Section 3.9.1.8.b).

- The parking brakes are fail safe – The parking brakes are designed as spring-applied, with hydraulically released calipers mounted on each axle input (Ref. B5.1.2, Section 3.9.1.9.b).
- The tractor and trailer brakes are redundant – either are capable of stopping the conveyance.
- The stabilizing jacks and pendular axles are redundant vertical support systems during loading and unloading operations.
- The trailer has four pendular axles and eight axle hydraulic actuators. The pendular axle hydraulic system can sustain one actuator failure and still function properly.
- There are four stabilizing jacks, failure of any one stabilizing jack results in the failure of the stabilizing jack system.

B5.4.1.4 Fault Tree Model

The top event in this fault tree is “Horizontal Cask Tractor Trailer Collision.” This is defined as an undesired contact at any speed between the cask tractor and/or cask transfer trailer with another vehicle, facility structures or equipment. Faults modeled in this tree include axle and stabilizing jack hydraulic failures and vehicle control failures (Figures B5.4-3 thru B5.4-7).

B5.4.1.5 Basic Event Data

A number of basic events are used in this fault tree, including two common-cause failure events and two human failure events as listed in Table B5.4-1.

Table B5.4-1. Basic Event Probabilities for Collision of Cask Tractor and Cask Transfer Trailer

Name	Description	Calc. Type	Calc Prob.	Fail. Prob.	Lambda	Tau	Miss. Time
200-CRWT-BRK001--BRK-FOD	Tractor brake A fails	1	1.46E-06	1.46E-06	0.00E+00	0.00E+00	0.00E+00
200-CRWT-BRK002--BRK-FOD	Tractor brake B fails	1	1.46E-06	1.46E-06	0.00E+00	0.00E+00	0.00E+00
200-CRWT-BRK003--BRK-FOD	Trailer brakes fail	1	1.46E-06	1.46E-06	0.00E+00	0.00E+00	0.00E+00
200-CRWT-BRKCCF--BRK-CCF	CCF of both tractor brakes	1	6.86E-08	6.86E-08	0.00E+00	0.00E+00	0.00E+00
200-CRWT-LPATH--ATH--CCF	CCF of pendular axle hydraulics during load/unload	1	8.83E-05	8.83E-05	0.00E+00	0.00E+00	0.00E+00
200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LSJATH1-ATH-FOH	Stabilizing jack 1 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LSJATH2-ATH-FOH	Stabilizing jack 2 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LSJATH3-ATH-FOH	Stabilizing jack 3 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-LSJATH4-ATH-FOH	Stabilizing jack 4 failure	3	1.78E-03	0.00E+00	8.91E-04	0.00E+00	2.00E+00
200-CRWT-TRCT-STEER-FAIL	Tractor steering system failure	3	1.84E-05	0.00E+00	1.84E-05	0.00E+00	1.00E+00
200-CRWT-TRLR-STEER-FAIL	Trailer steering system failure	3	1.84E-05	0.00E+00	1.84E-05	0.00E+00	1.00E+00
200-HTTCOLLIDE---G65-FOH	Speed limiter fails	3	1.16E-05	0.00E+00	1.16E-05	0.00E+00	1.00E+00
200-OPHTCOLLIDE1-HFI-NOD	Operator causes collision of HTT while leaving the RF	1	3.00E-03	3.00E-03	0.00E+00	0.00E+00	0.00E+00
200-OPHTINTCOL01-HFI-NOD	Operator causes collision of HTT due to over speed	1	1.00E+00	1.00E+00	0.00E+00	0.00E+00	0.00E+00

NOTE: Calc. = calculation; CCF = common-cause failure; Fail. = failure; HTT = the cask tractor and cask transfer trailer referred to as the HCTT in Section 6.2; Miss. = mission; Prob. = probability; RF = Receipt Facility.

Source: Original

B5.4.1.5.1 Human Failure Events

Two human failure events are modeled in the cask tractor and cask transfer trailer collision failure scenario as follows:

1. Operator causes collision of cask tractor and cask transfer trailer while leaving the RF.
2. Operator causes collision of cask tractor and cask transfer trailer due to overspeed.

Further description of these events can be found in Attachment E.

B5.4.1.5.2 Common-Cause Failures

Two common-cause failure events are modeled in the cask tractor and cask transfer trailer collision failure scenario as follows:

1. Common-cause failure of the primary and redundant tractor brakes.
2. Common-cause failure of two or more pendular axle hydraulics.

B5.4.1.6 Uncertainty and Cut Set Generation Results

Figure B5.4-1 contains the uncertainty results obtained from running the fault trees for cask tractor and cask transfer trailer collision. Figure B5.4-2 provides the cut set generation results for the Cask Tractor and cask Transfer Trailer Collision tree.

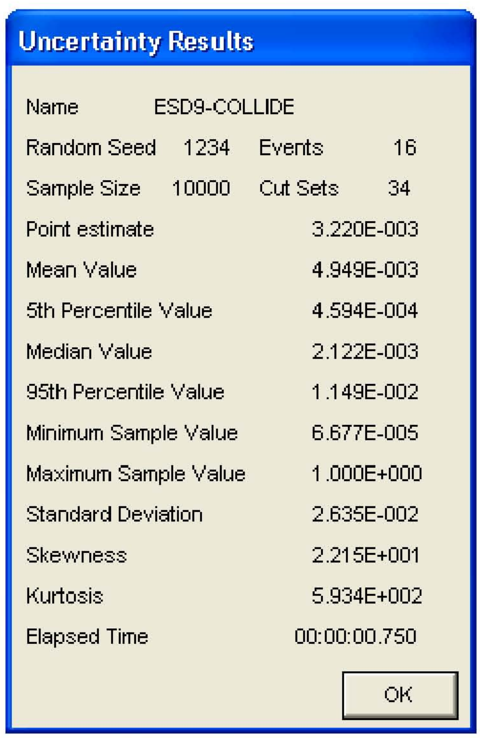


Figure B5.4-1. Uncertainty Results for the Cask Tractor and Cask Transfer Trailer Collision Fault Tree

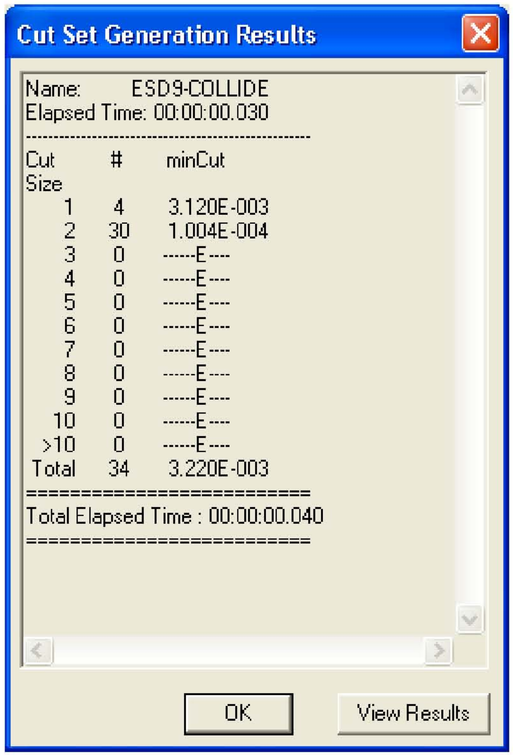


Figure B5.4-2. Cut Set Generation Results

B5.4.1.7 Cut Sets

Table B5.4-2 contains the cut sets for the collision of the cask tractor and cask transfer trailer.

Table B5.4-2. Cut Set for Collision of Cask Tractor and Cask Transfer Trailer

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
200-HCTT-COLLISION	93.16	3.000E-003	200-OPHTCOLLIDE1-HFI-NOD	Operator causes collision of HTT while leaving the RF	3.000E-003
	2.60	8.380E-005	200-CRWT-LPATH--ATH--CCF	CCF of pendular axle hydraulics during load/unload	8.380E-005
	0.57	1.840E-005	200-CRWT-TRCT-STEER-FAIL	Tractor steering system failure	1.840E-005
	0.57	1.840E-005	200-CRWT-TRLR-STEER-FAIL	Trailer steering system failure	1.840E-005
	0.36	1.160E-005	200-HTTCOLLIDE---G65-FOH	Speed limiter fails	1.160E-005
			200-OPHTINTCOL01-HFI-NOD	Operator causes collision of HTT due to over speed	1.000E+000
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003

Table B5.4-2. Cut Set for Collision of Horizontal Cask Tractor and Trailer (Continued)

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
	0.10	3.170E-006	200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
			200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
			200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
			200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003
			200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
			200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
			200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
			200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
			200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003
			200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
			200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003

Table B5.4-2. Cut Set for Collision of Horizontal Cask Tractor and Trailer (Continued)

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
	0.10	3.170E-006	200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
			200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
			200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
			200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
			200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
			200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH1--ATH-FOH	Pendular axle hydraulic 1 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH2--ATH-FOH	Pendular axle hydraulic 2 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH3--ATH-FOH	Pendular axle hydraulic 3 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH4--ATH-FOH	Pendular axle hydraulic 4 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003

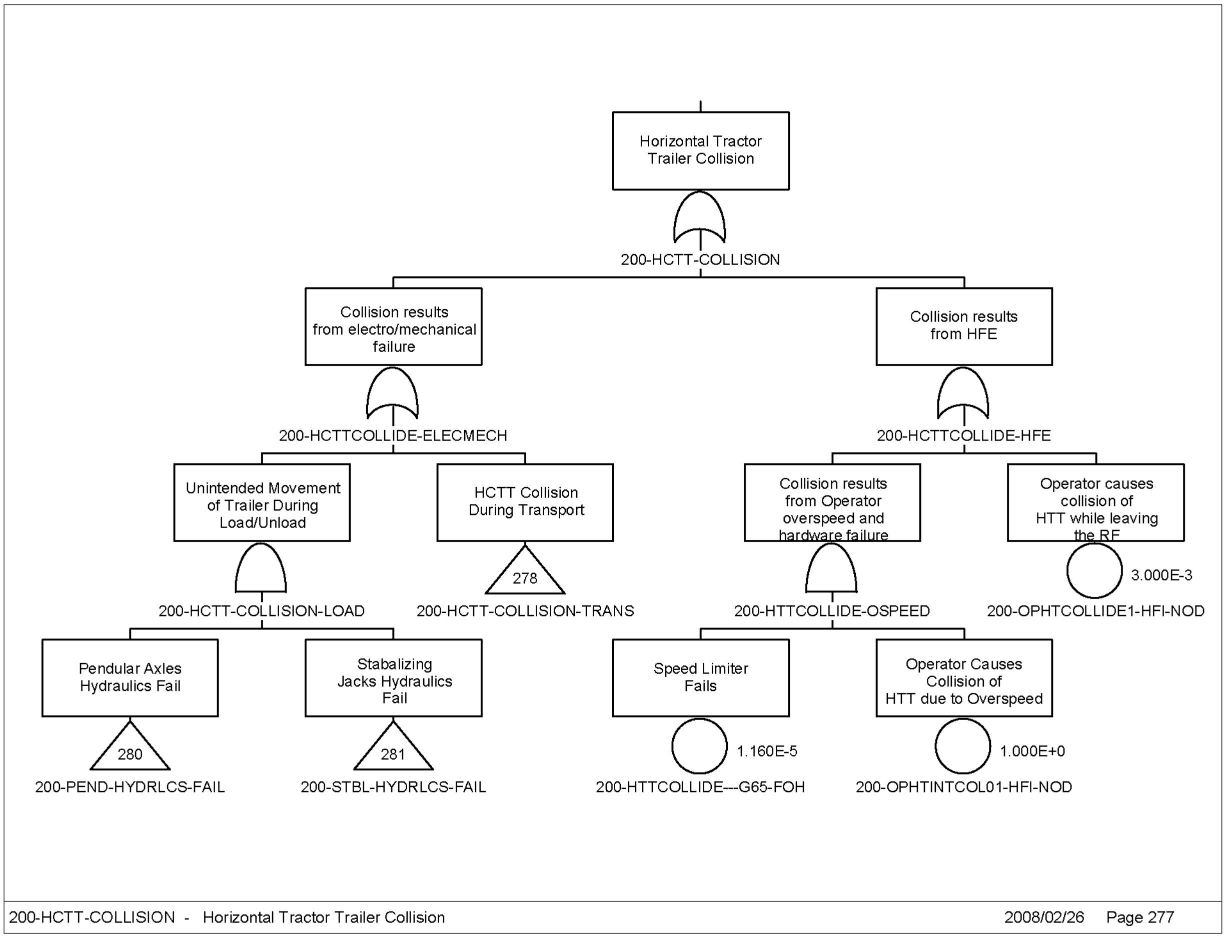
Table B5.4-2. Cut Set for Collision of Horizontal Cask Tractor and Trailer (Continued)

Fault Tree	% Cut Set	Prob./Frequency	Basic Event	Description	Event Prob.
	0.10	3.170E-006	200-CRWT-LPATH5--ATH-FOH	Pendular axle hydraulic 5 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH6--ATH-FOH	Pendular axle hydraulic 6 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003
	0.10	3.170E-006	200-CRWT-LPATH7--ATH-FOH	Pendular axle hydraulic 7 failure	1.780E-003
			200-CRWT-LPATH8--ATH-FOH	Pendular axle hydraulic 8 failure	1.780E-003
	0.00	1.002E-013	200-CRWT-BRK003--BRK-FOD	Trailer brakes fail	1.460E-006
			200-CRWT-BRKCCF--BRK-CCF	CCF of both tractor brakes	6.860E-008

NOTE: CCF = common-cause failure; HTT = the cask tractor and cask transfer trailer referred to as the HCTT in Section 6.2; No. = number; Prob. = probability; RF = Receipt Facility.

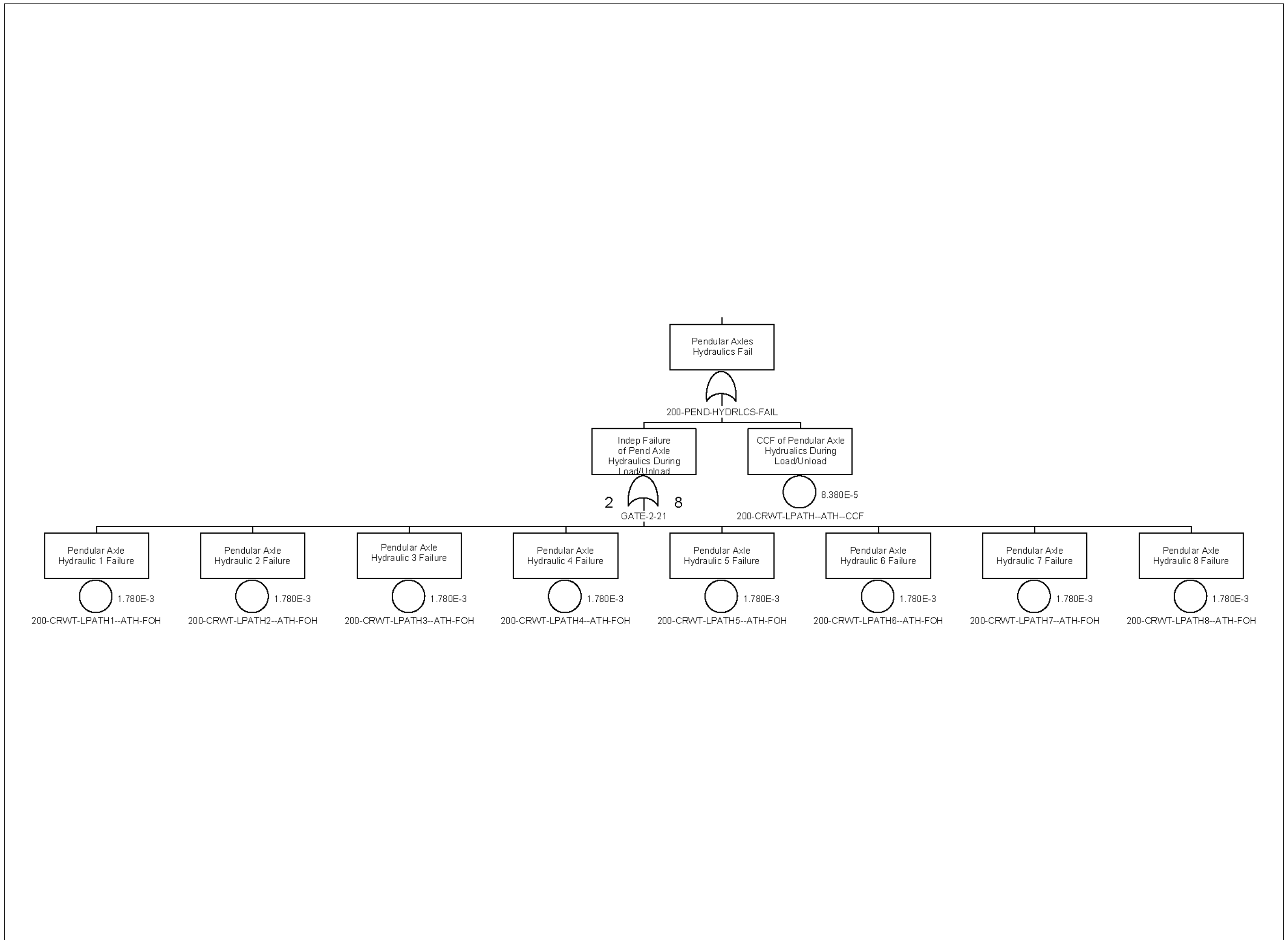
Source: Original

B5.4.1.8 Fault Trees



Source: Original

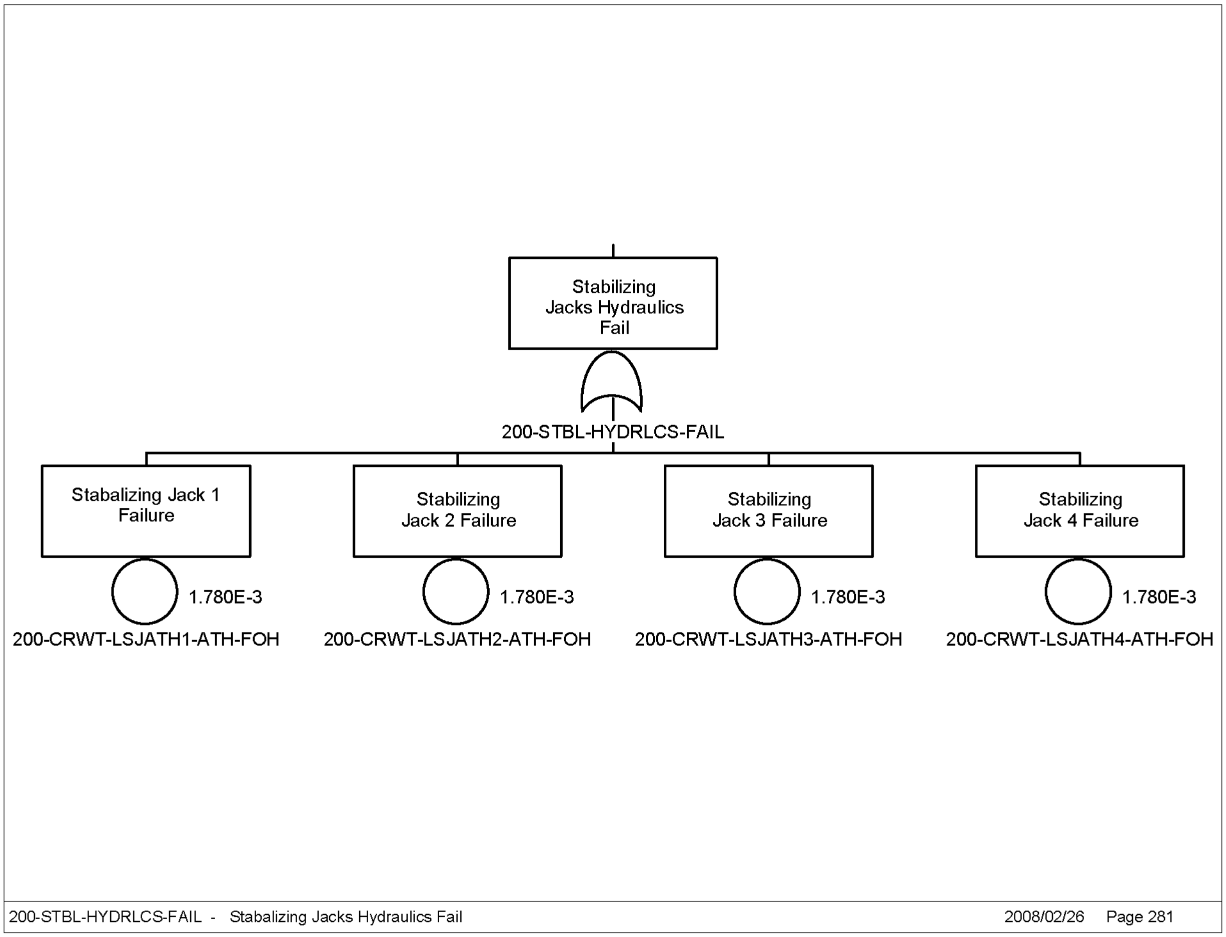
Figure B5.4-3. Fault Tree for Cask Tractor and Cask Transfer Trailer Collision



200-PEND-HYDRLCS-FAIL - Pendular Axles Hydraulics Fail 2008/02/26 Page 280

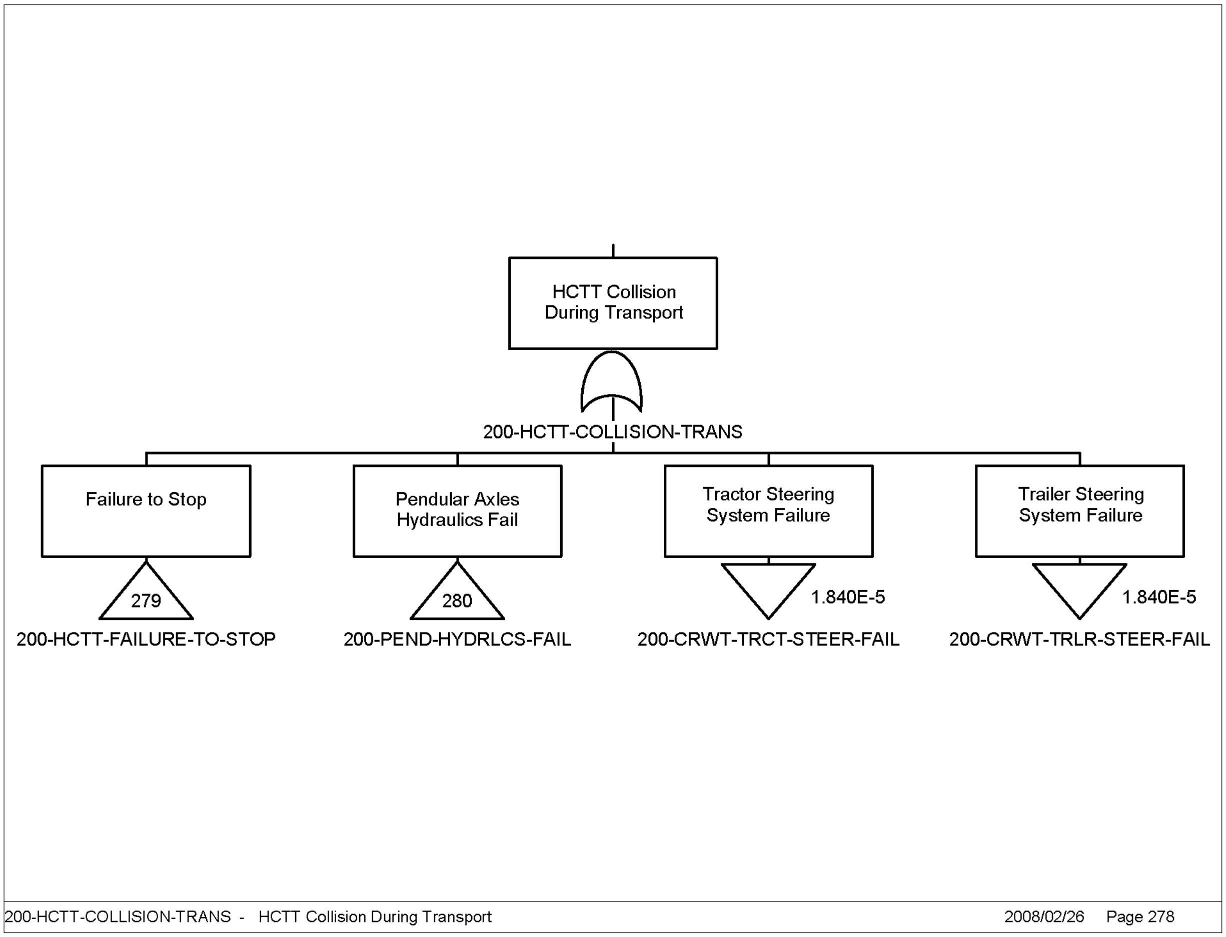
Source: Original

Figure B5.4-4. Fault Tree for Pendular Axles Hydraulics Fail



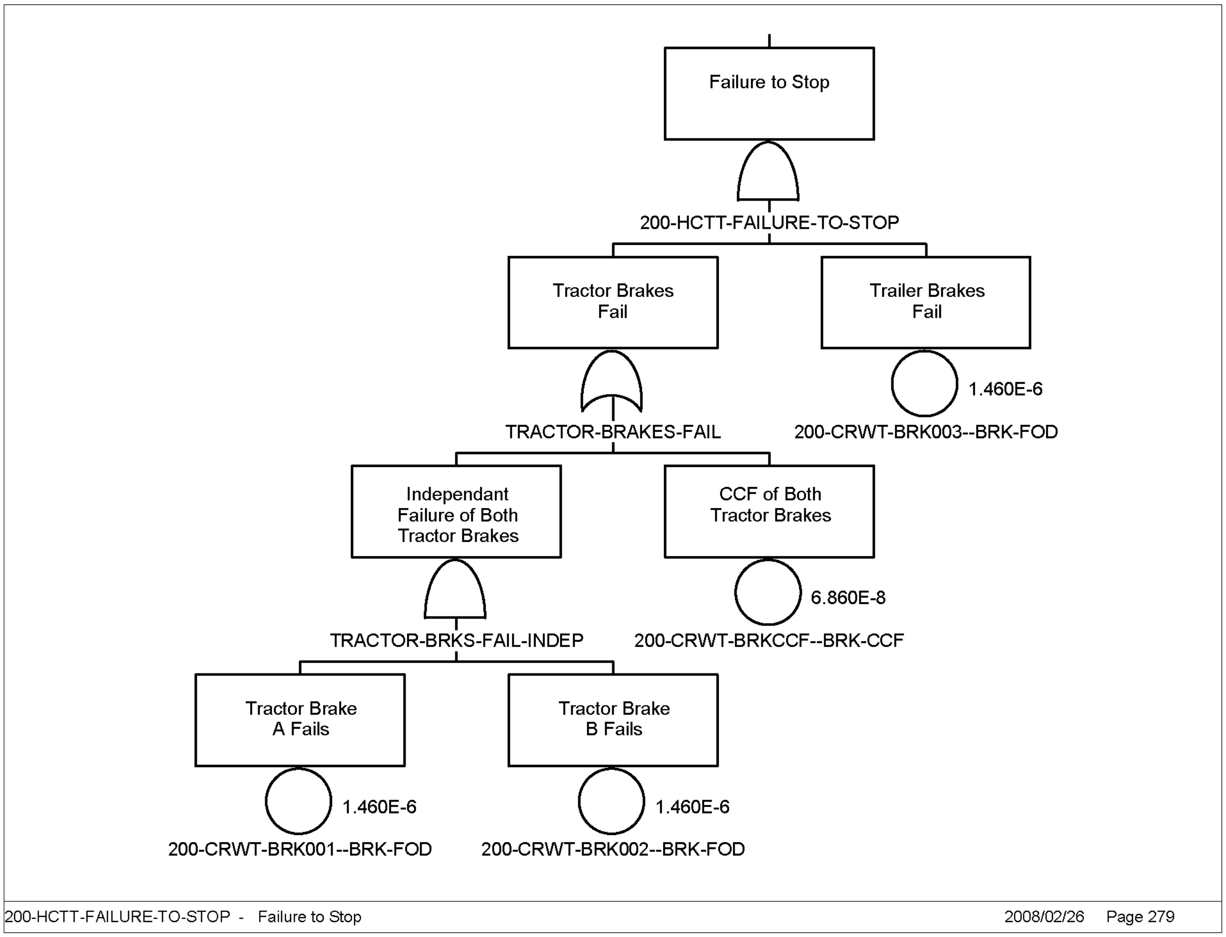
Source: Original

Figure B5.4-5. Fault Tree for Stabilizing Jacks Hydraulics Fail



Source: Original

Figure B5.4-6. Fault Tree for Cask Tractor and Cask Transfer Trailer Collision during Transport



Source: Original

Figure B5.4-7. Fault Tree for Failure to Stop

B6 SITE TRANSPORTER FAULT TREE ANALYSIS

B6.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

B6.1.1 BSC (Bechtel SAIC Company) 2007. *Mechanical Handling Design Report - Site Transporter.* 170-30R-HAT0-00100-000-000. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0015.

B6.1.2 BSE 2007. *Exhibit D, Statement of Work for Mechanical Handling Equipment Design.* 000-3SW-MGR0-00100-000 Rev. 003. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070904.0031.

B6.1.3 Morris Material Handling 2007. *P&ID Site Transporter.* V0-CY05-QHC4-00459-00049-001 Rev. 004. Oak Creek, Wisconsin: Morris Material Handling. ACC: ENG.20071022.0012.

B6.2 SITE TRANSPORTER DESCRIPTION

The site transporter is a diesel/electric self-propelled tracked vehicle that is designed to transport a cylindrical concrete and steel ventilated aging overpack. The transport occurs both Intra-Site and within the CRCF, the WHF, and the RF[1]. In the RF, the site transporter is only used during the loading of aging overpacks with a DPC or TAD canister, and for removing the loaded aging overpack from the facility.

Movement of the site transporter within the RF is limited to the Loading Room, Lid Bolting Room, and the Site Transporter Vestibule.

B6.2.1 Overview

The interface between the site transporter and the aging overpack is via two parallel rectangular lift slots that pass through the containers near their lower ends. Orientation of the aging overpack is such that the axis of the aging overpack is vertical with lid, at the top. Access to the top of the aging overpack is unobstructed.

[1] Variations in the use of the site transporter for Intra-Site, WHF and CRCF are addressed in their respective volumes.

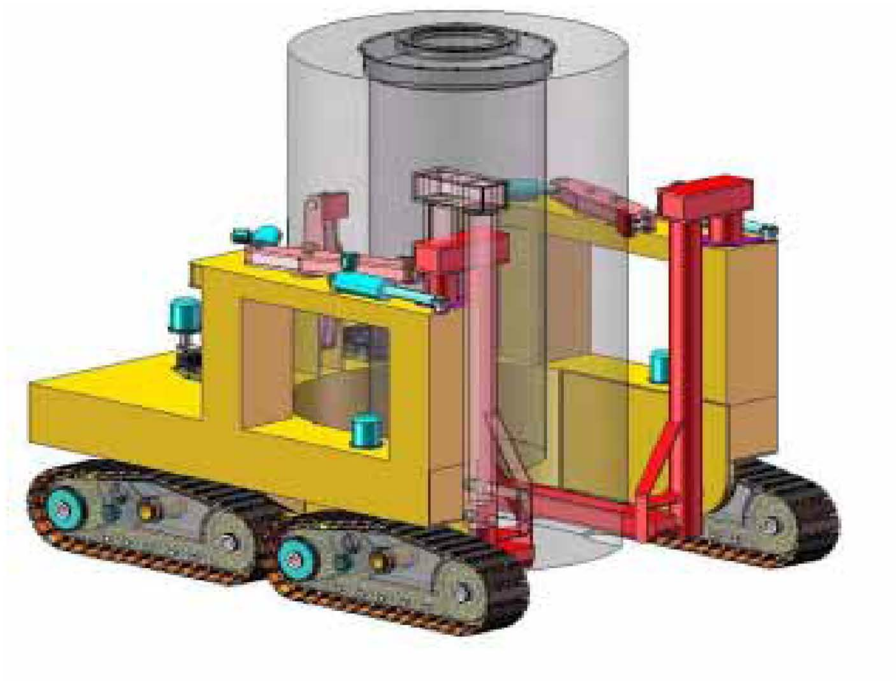
An integrated diesel powered electric generator provides the electricity to operate the site transporter outside the facility building. Inside the facility buildings the site transporter is electrically driven via an umbilical cable from the facility main electrical supply (Ref. B6.1.1, Section 2.1).

The site transporter is a track driven vehicle with four synchronized tracks (two on each side of the site transporter). The components of the drive system (i.e., tumblers, idlers, rollers) are not included in this analysis since these components are not ITS.

A rear fork assembly consists of a pair of arms that extend to the front of the site transporter. These forks move up and down for the purpose of raising, lowering, and supporting the aging overpack during movement. A pair of support arms is located at the front of the site transporter which is moved into position around the forks to provide support and assistance during the lifting and lowering of the aging overpack.

A passive restraint system stabilizes the aging overpack during movement. There are two mechanisms that control aging overpack movement on the pitch and roll axis. These restraints are not engaged until the aging overpack has been raised to the desired height. Once engaged, three pins are inserted, one in each restraint arm, that keep the restraints in place should there be a failure of the electromechanical assembly used to position and secure the restraint device. Properly installed, they also serve as an interlock that prevent movement of a loaded site transporter.

Control of the site transporter is provided by a wireless remote control or a wired pendant. Although these devices only provide a subset of the controls and indicators that are available on the control console located on the site transporter, they do contain all the necessary controls and indicators to perform and monitor the operation state of the site transporter during normal operations. The site transporter is shown in Figure B6.2-1.



Source: Ref. B6.1.1

Figure B6.2-1. Site Transporter

The site transporter system is composed of six subsystems:

1. Crawler Tracks Subsystem—four crawlers, two on each side of the site transporter, are used to move the vehicle. These crawlers use tracks with chamfered flat steel plates mounted to double grouser shoes on a continuous chain.

2. Power Plant Subsystem—a diesel engine, generator, and diesel fuel tank are enclosed in the back of the site transporter. During Intra-Site Operation activities, the diesel engine will drive the generator, which provides the required 480V 3-phase/60 Hz power to the vehicle. During facility operations, the diesel engine is disabled and facility 480V 3-phase/60 Hz power is supplied to operate the vehicle.

3. Rear Lift Fork Subsystem—the site transporter contains a pair of arms that extend forward from the site transporter through slots in the aging overpack. The lift/lower drive system utilizes an ACME type nut that changes the elevation of the fork as the screw lift mechanism turns through the ACME nut. A lift synchronizer controls the lift/lower operation.

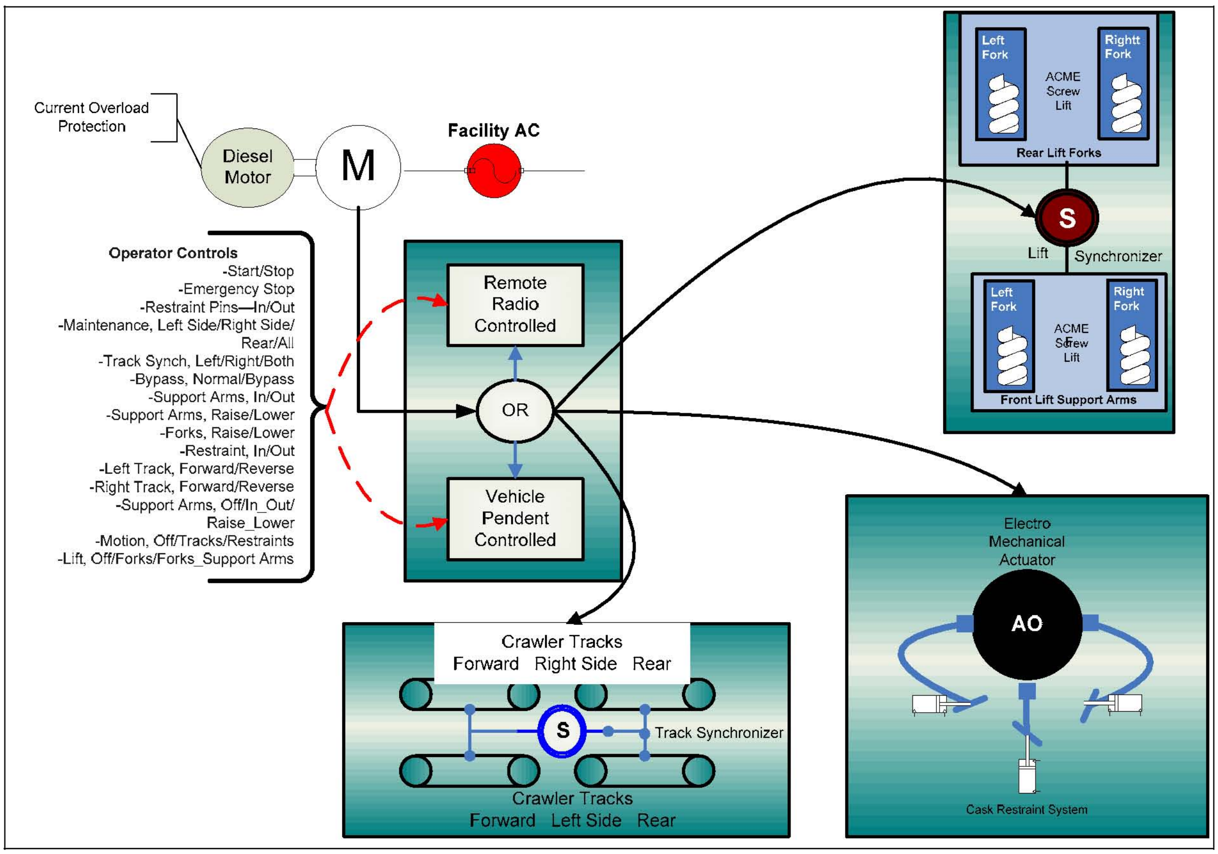
4. Lift Support Arms Subsystem—two support arms with electromechanical actuators are located on the front of the site transporter. These support arms are rotated 90 degrees to provide support and stabilization for the lift forks during lifting/lowering/moving operations. ACME nuts are used on these arms and synchronized with the lift forks during lifting/lowering/moving.

5. Restraint Subsystem—a two axis restraint system is incorporated to stabilize the aging overpack during site transporter movement. The restraints are emplaced/retracted with electromechanical actuators. These restraints, when positioned against the aging overpack will be secured with a locking pin. The three pins serve as an interlock and must be properly installed before the site transporter can be moved.

6. Vehicle Controls Subsystem—there are two modes of control provided on the site transporter. Operators can control every operation on the site transporter with either a remote (wireless) controller or through a pendant connected to the site transporter.

Note: In addition to the six subsystems identified above, *Mechanical Handling Design Report – Site Transporter* (Ref. B6.1.1) also includes a description of the site transporter "car body." Events associated with car body failure are screened from this analysis based on the results of the stress analysis contained in this reference.

A simplified block diagram of the functional subsystems on the site transporter is shown in Figure B6.2-2.



NOTE: AO = aging overpack.

Source: Original

Figure B6.2-2. Simplified Block Diagram of the Site Transporter Subsystems

B6.2.1.1 Site Transporter Crawler Tracks Subsystem Description

The site transporter moves by four tracks mounted on the crawler frames, with two on each side of the vehicle to increase stability when traversing terrain that includes sudden changes in elevation such as a drainage trough or curb. The site transporter is designed to negotiate roadways with a 5% grade and up to a 2% cross-slope (Ref. B6.1.2, Section 7.2.2-11). Special pads are included on the tracks to reduce the wear and tear on concrete or roadways.

Each track is driven by its own electric motor (50 hp @ 900 rpm) through its own gear reduction and final chain drive reduction. During forward operations, motors on both sides of the machine drive are synchronized. During turns the outside tracks are driven faster, and for very sharp turns the tracks are counter-rotated to turn the site transporter about its own vertical centerline (Ref. B6.1. 1, Section 2.1.2).

B6.2.1.2 Power Plant Subsystem Description

The power plant subsystem supplies the site transporter with 480V AC, 3-phase power at 60 Hz. Because of the risk of contamination from their various fluids, there are no storage batteries or capacitors in the system. The generator is sized approximately at 110% of than the highest power requirement for the vehicle.

The 150kW generator is sized for seven hours of continuous operation with a fuel tank containing approximately 100 gallons of diesel fuel (Ref. B6.1.1, Section 2.2.3). The fuel tank capacity is sized to minimize the amount of fuel taken inside the facilities but sufficient to transport a loaded aging overpack three miles and return to the site transporter's point of origin without refueling (Ref. B6.1.2, Section 7.2.2-2)

When entering a building the generator is shut down and a power source from the building is plugged into the site transporter integral receptacle to allow the site transporter to operate inside the building without a source of combustion.

The motor drive and current over load protection system prevents the site transporter from exceeding 2.5 mph (Ref. B6.1.1, Section 3.2.1).

B6.2.1.3 Rear Lift Forks Subsystem Description

The rear forks are only capable of moving up or down. Each fork is driven by its own gear reduction and 16 hp, 900 rpm electric motor. The output of the drive is a rotating ACME type screw, which, as it turns inside the rear forklift tube, drives an ACME nut that raises or lowers the fork. The height of the rear lift fork is controlled by limit switches as well as being mechanically unable to lift an aging overpack higher than 12 in. above the floor or ground (Ref. B6.1.1, Sections 2.1.4 and 2.2).

B6.2.1.4 Lift Support Arms Subsystem Description

The front support arms have constrained movement which consists of a clockwise/counterclockwise rotation and up and down movement. The right and left assemblies are mirror images of one another and move as a synchronous pair although they are each driven by their own gear reduction and 20 hp, 900 rpm electric motor (Ref. B6.1.1, Section 2.1.5).

The operator positions the lift support arms around the lifting forks. After the site transporter has been positioned properly around the aging overpack, the rear forks are raised to contact the bottom of the aging overpack's lifting slots. Limit and position switches ensure the lift support arms are in the correct position. Additional limit switches prevent the support arms from exceeding the 12 in. lift.

B6.2.1.5 Restraints Subsystem Description

When the load on the site transporter is ready to be lifted, the three arms of the restraint system are activated and moved to a location "near" the aging overpack. This location is determined by a combination of operator observation and integral limit switches.

After the aging overpack has been raised to the specified transportation height, the restraint arms are engaged to hold the aging overpack in place during movement. The arms are moved by linear electromechanical actuators. In addition, a locking pin is utilized to take extreme loads as well as serve as an interlock device. The three restraint arms must be properly pinned before the interlock will allow the site transporter to be moved (Ref. B6.1.3, Sheet 1 of 3).

B6.2.1.6 Vehicle Controls Subsystem Description

The site transporter can be operated in two modes: a remote (wireless) control and an operator controlled pendant (Ref. B6.1.1, Section 2.1.7). Both of these devices have the same capability. Table B6.2-1 contains a list of controls that are available on the controller and the corresponding activation device (Ref. B6.1.3, Sheet 3 of 3).

Table B6.2-1. Site Transporter Remote or Pendant Controls

Site Transporter Operation	Activation Device on Controller
Start/Stop	Pushbutton
Emergency stop	Palm button
Restraint pin—engage (in)/disengage (out)	Selector switch
Maintenance—left side/right side/rear/all	Keyed selector switch
Track synch—left/right/both	Selector switch
Bypass—normal/bypass	Keyed selector switch
Support arms—in/out	Induction pushbutton
Support arms—raise/lower	Induction pushbutton
Forks—raise/lower	Induction pushbutton
Restraint—in/out	Induction pushbutton
Left Track—forward/reverse	Induction pushbutton
Right Track—forward/reverse	Induction pushbutton
Support Arms—off/in_out/raise_lower	Selector switch
Motion—off/tracks/restraints	Selector switch
Lift—off/forks/forks_support arms	Selector switch

Source: Original

All safety interlocks and controls of the site transporter are hard wired between the specific relays, drives, circuit breakers, and other electrical equipment. No PLC or computer is used to control the machine.

B6.2.2 Normal Operations

Once the lift has been completed, the operator performs the final positioning of the upper restraint arms and inserts a pin in each arm. When the pins are properly installed, the site transporter can move.

The operator trails behind the site transporter during movement using the remote control to drive the site transporter to the desired location. Once the site transporter arrives at the facility, the operator stops the vehicle outside the Site Transporter Vestibule and turns off the diesel generator. An electrical umbilical cord is manually retrieved from inside the building and attached to the site transporter. The site transporter is never operated inside the RF on diesel power.

Once inside the building, the operator positions the site transporter in the Loading Room. When work is being performed on the aging overpack, the site transporter operator will remove the pins from the restraint arms and disengage them from the aging overpack. The movement interlock is engaged when the pins are removed. The operator will then lower the aging overpack to the floor. The procedure is reversed when it is necessary to move the site transporter again inside the facility or to transport the aging overpack to some other location. Once outside the RF, the operator shuts down the site transporter and removes the electrical cable. Subsequent activities are addressed in the Intra-Site Operations analysis.

The operations used to move an unloaded aging overpack are identical but not considered in this analysis.

B6.2.3 Site Transporter Off-Normal Operations

There are four off normal conditions that could occur during the movement of an aging overpack in the RF. When any of these occur, the operator response encompasses only those actions to return the aging overpack to a safe state. These are:

1. Lowering the forks without electrical power
2. Rotating the lift support arms without electrical power
3. On-board generator fails to operate
4. Track belt fails.

In the event of a loss of power, the site transporter is designed to stop, retain its load and enter a locked mode. Upon the restoration of power the site transporter will stay in the locked mode until operator action is taken (Ref. B6.1.2, Section 7.2.3-5).

B6.2.4 Site Transporter Testing and Maintenance

Testing and maintenance of the site transporter is done on a periodic basis and does not affect the normal operations of the site transporter. Testing and/or maintenance are not performed on a site transporter loaded with an aging overpack. A site transporter that has malfunctioned or has a lighted warning light will be deemed unserviceable and turned in for maintenance. Unserviceable vehicles will not be used.

If an unserviceable state is identified during a lift/lower or movement activity, the site transporter shall immediately be placed in a safe state (as quickly as possible) and recovery actions for the site transporter will be invoked.

B6.2.5 Site Transporter System/Pivotal Event Success Criteria

A site transporter failure is the initiating event in five event sequences in the RF as shown in Table B6.2-2.

Table B6.2-2. Site Transporter Initiating Events by ESD

Site Transporter Initiating Event	Affected ESD
Site transporter spurious movement	ESD-06 Lifting and lowering a canister during transfer in CTM
Site transporter collision	ESD-07 Assembly and closure of aging overpack
Site transporter collision Site transporter rollover Site transporter load drop	ESD-08 Export of aging overpack from RF

NOTE: CTM = canister transfer machine; ESD = event sequence diagram; RF = Receipt Facility.

Source: Original

Spurious movement of the site transporter is prevented by the inherent design constraints of the site transporter. There is only sufficient electrical power to perform one type of operation at a time. For example, it is not possible to command a lift/lower of the aging overpack when the site transporter is moving. Spurious signals can not be generated when primary power is removed from the site transporter (i.e., diesel engine shut down and/or facility electrical power cord disconnected). There are no batteries or capacitors in the site transporter that can store electrical energy.

Requirements

Two means of stopping the site transporter are incorporated in the controllers. One is the normal stop button and the other consists of an emergency stop that is the equivalent of a deadman switch.

On the loss of AC power from the facility, the site transporter immediately enters the lock mode state. The lock mode state is not reversible without specific operator action.

There is no maintenance or testing permitted on a site transporter loaded with an aging overpack.

Since the dominant contributor to site transporter collision in the facility is human error, no priority is given to either the remote or the pendant controllers.

Design Features

Stopping the site transporter is accomplished by pushing the "stop" button on the remote or pendant controller. The site transporter, upon receiving a stop command from either control source, will immediately respond by removing power from the propulsion system.

The site transporter can only perform one function at any time. It can lift a aging overpack or it can move it, but it can not perform both functions at the same time. This feature is accomplished by interlock and by power limitations inherent in the sizing of the power plant that ensures a limited amount of power for each of the electromechanical devices and drive system.

B6.3 DEPENDENCIES AND INTERACTIONS ANALYSIS

Dependencies are broken down into five categories with respect to their interactions with system, structures, and components. The five areas considered are addressed in Table B6.3-1 with the following dependencies:

1. Functional dependence
2. Environmental dependence
3. Spatial dependence
4. Human dependence
5. Failures based on external events.

Table B6.3-1. Dependencies and Interactions Analysis

Systems, Structures, Components	Dependencies & Interactions				
	Functional	Environ-mental	Spatial	Human	External Events
Lift booms	-Material failure -ACME screw/nut	—	—	—	—
Lift support arms	-Material failure -ACME screw/nut	—	—	—	—
Restraint arms	-Material failure	—	—	—	—
Power plant	-Current overload protection fails -Safe state on	—	—	-Failure to stop -Failure to remove power cable	—
Remote control	-Spurious commands	—	—	-Improper command	-Collide with crane rigging
Tracks	—	—	—	-Failure to stop	—

Source: Original

B6.4 RELATED FAILURE SCENARIOS

There are four basic site transporter fault trees developed for the RF. The top events for these fault trees and the variations are:

1. Site transporter collides with RF structures.
2. Site transporter load drop during lift/movement.
3. Site transporter tipover.
4. Site transporter spurious movement.

B6.4.1 Site Transporter Collides with RF Structures (ESD-07, -08)

B6.4.1.1 Description

The fault trees for the collision events are identical. Collisions can occur as a result of human error or hardware failures (i.e., human error events are uniquely identified but all have the same screening value of 3E-3 with a lognormal error factor of 5). Hardware failures leading to a collision consist of: the site transporter fails to stop when commanded, the site transporter exceeding a safe speed, or the site transporter moves in the wrong direction.

B6.4.1.2 Success Criteria

The success criteria for preventing a collision includes safety design features incorporated in the site transporter for hardware failures and the operator maintaining situational awareness and proper control of the movement of the site transporter. To avoid collisions, the site transporter must stop when commanded, be prevented from entering a runaway situation, or respond correctly to a site transporter movement command.

The site transporter is designed to stop whenever commanded to stop or when there is a loss of power. The operator can stop the site transporter by either commanding a stop from the start/stop button or by releasing the palm switch which initiates an emergency stop. At anytime there is a loss of power detected, the site transporter will immediately stop all movement and enter into lock mode safe state. The site transporter will remain in this locked mode until power is returned and the operator restarts the site transporter.

Runaway situations on the site transporter are prevented by hardware constraints. The maximum speed of the site transporter is limited by motor current overload protection (Ref. B6.1.1, Section 3.2.1). The site transporter motor speed and gearing prevents the site transporter from exceeding 2.5 mph.

The prevention of site transporter movements in the wrong direction is prevented by the limitation of the power plant that prevents simultaneous operations.

B6.4.1.3 Design Requirements and Features

The site transporter has two off-equipment control devices that have complete control over the site transporter.

The drive system consists of electric motors and a transmission constraint which will limit the maximum speed of the site transporter to 2.5 mph.

B6.4.1.4 Fault Tree Model

The fault tree model for "Site Transporter Collides with RF Structures" in the RF accounts for both human error and/or site transporter hardware problems that could result in collision. Movement within the facility is restricted and even at low speeds a collision can occur.

The fault tree considers mechanical failures that fail to stop the site transporter, events that could cause the site transporter to exceed safe speed, and events that could cause the site transporter to move in the wrong direction.

B6.4.1.5 Basic Event Data

Table B6.4-1 lists the basic events used in the site transporter collision fault tree. Uncertainty and cut set results are provide in Figures B6.4-1 and B6.4-2 respectively.

Table B6.4-1. Basic Event Probability for Site Transporter Collides with RF Structures

Basic Events Probability Report					
Project: Yucca-Mountain		Case: Current			
ST Collision in Facility		Units: Per Hour			
Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-OPSTCOLLIDE1-HFI-NOD	1	3.00E-03	3.00E-03	0.00E+00	0.00E+00
200-ST---BRK001--BRK-FOD	3	1.46E-06	1.46E-06	0.00E+00	0.00E+00
200-ST---CBP004-CBP--OPC	3	9.13e-08	0.00E+00	9.13E-08	1.00E+00
200-ST---CBP004-CBP--SHC	3	1.88E-08	0.00E+00	1.88E-08	1.00E+00
200-ST---CT000---CT--FOD	1	4.00E-06	4.00E-06	0.00E+00	0.00E+00
200-ST---CT002---CT--FOH	3	6.88E-05	0.00E+00	6.88E-05	1.00E+00
200-ST---HC001--HC--FOD	1	1.74E-03	1.74E-03	0.00E+00	0.00E+00
200-ST---HC002---HC--SPO	3	5.23E-05	0.00E+00	5.23E-05	1.00E+00
200-ST---MOE000--MOE-FSO	3	1.35E-08	0.00E+00	1.35E-08	1.00E+00
200-ST---MOE021--MOE-FSO	3	1.35E-08	0.00E+00	1.35E-08	1.00E+00
200-ST---SC021---SC--FOH	3	1.28E-04	0.00E+00	1.28E-04	1.00E+00
200-ST---SC021---SC--SPO	3	3.20E-05	0.00E+00	3.20E-05	1.00E+00
200-ST---SEL021--SEL-FOH	3	4.16E-06	0.00E+00	4.16E-06	1.00E+00
LOSP-4	1	4.16E-06	4.16E-06	0.00E+00	0.00E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail =failure; Miss. = mission; Prob. = probability; ST = site transporter.

Source: Original

B6.4.1.5.1 Human Failure Events

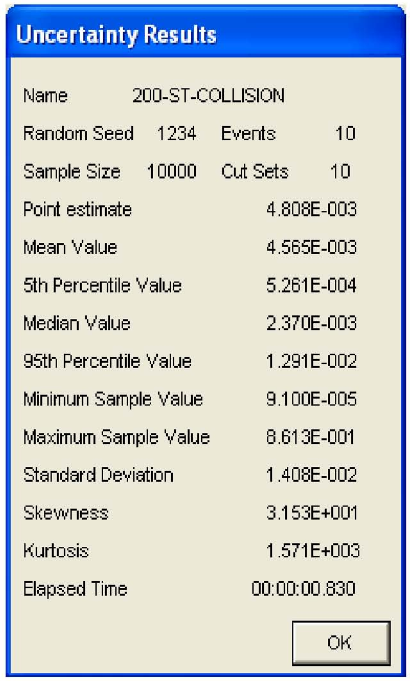
There is one human event in the collision trees for the site transporter and accounts for the site transporter operator causing the collision. This human error is set at the screening value of 3E-03 for all four ESD events.

B6.4.1.5.2 Common-Cause Failures

There are no common-cause events identified for the site transporter collision events.

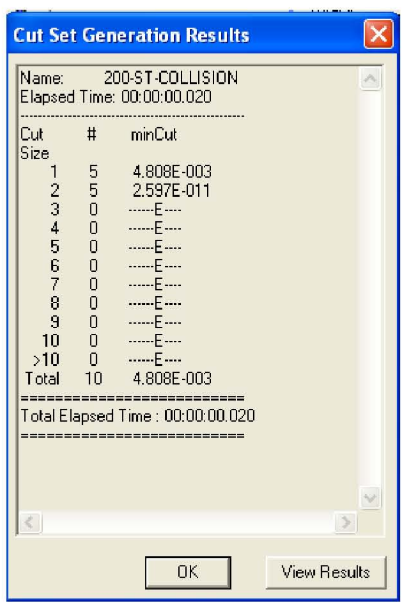
B6.4.1.6 Uncertainty and Cut Set Generation

Figures B6.4-1 and B6.4-2 contain the uncertainty and the cut set generation results for the "Site Transporter Collides with RF Structures" fault tree. The fault trees are shown in Figures B6.4-3 through B6.4-5.



Source: Original

Figure B6.4-1. Uncertainty Results for the Site Transporter Collides with RF Structures Fault Tree



Source: Original

Figure B6.4-2. Cut Set Generation Results for the Site Transporter Collides with RF Structures Fault Tree

B6.4.1.7 Cut Sets

Table B6.4-2 contains the cut sets for the “Site Transporter Collides with RF Structures” fault tree.

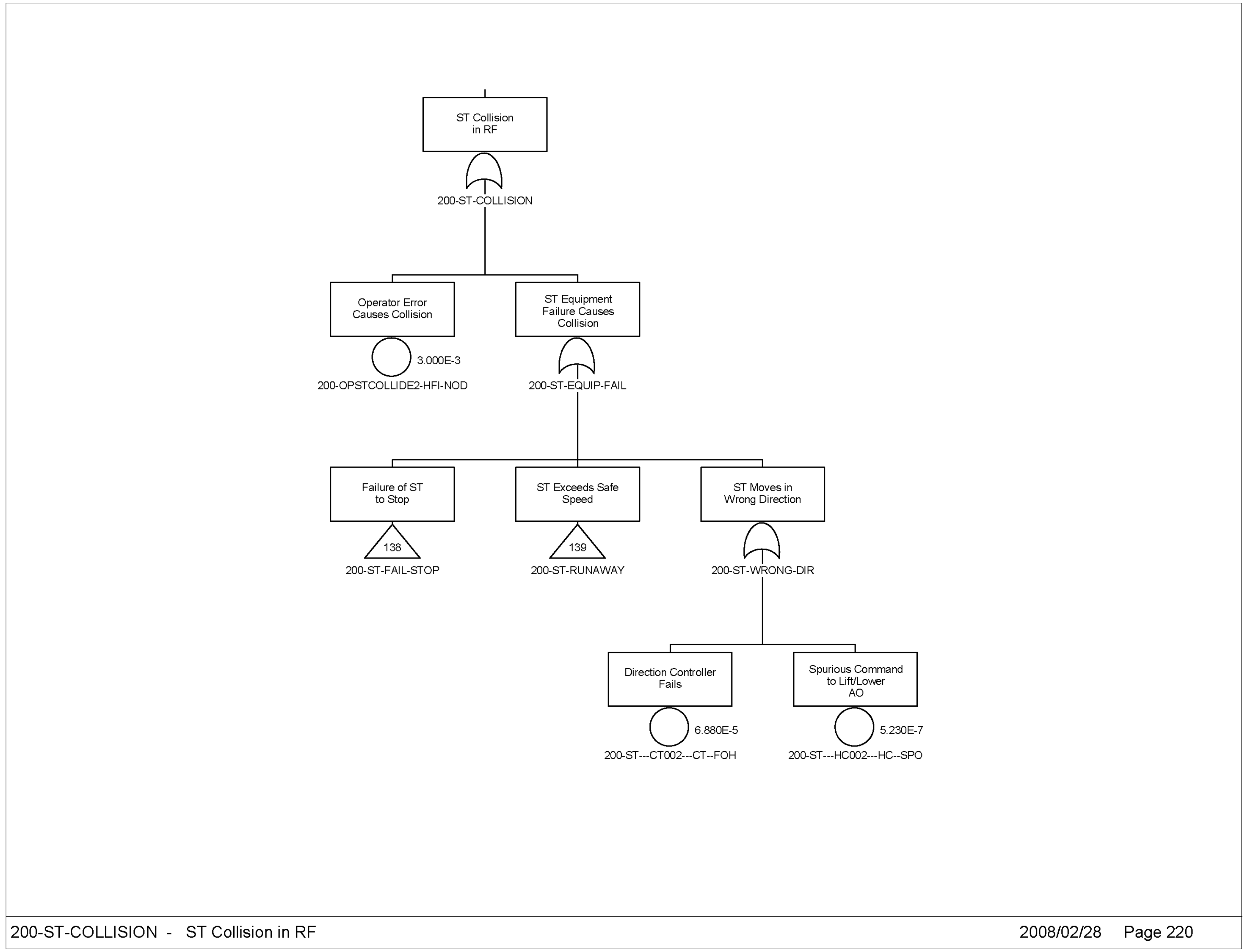
Table B6.4-2. Cut Sets for the Site Transporter Collision in Facility

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
200-ST-COLLISION	62.40	3.000E-003	200-OPSTCOLLIDE2-HFI-NOD	Operator error causes collision	3.0E-003
	36.19	1.740E-003	200-ST---HC001--HC--FOD	Remote control transmits wrong signal	1.7E-003
	1.43	6.880E-005	200-ST---CT002---CT--FOH	Direction controller fails	6.9E-005
	0.08	4.000E-006	200-ST---CT000---CT--FOD	ST primary stop switch fails	4.0E-006
	0.01	5.230E-007	200-ST---HC002---HC--SPO	Spurious command to lift/lower AO	5.2E-007
	0.00	5.986E-012	200-ST---BRK001--BRK-FOD	ST fails to stop on loss of power	4.4E-006
			LOSP-4	Failure of off site power	5.7E-006
	0.00	1.33E-013	200-ST---BRK001--BRK-FOD	ST fails to stop on loss of power	4.4E-006
			200-ST---CBP004-CBP--OPC	ST power cable–open circuit	1.5E-007
	0.00	2.745E-014	200-ST---MOE000--MOE-FSO	ST lock mode state fails on loss of power	1.4E-008
			LOSP-4	Failure of off site power	5.7E-006
	0.00	5.532E-014	200-ST---BRK001--BRK-FOD	ST fails to stop on loss of power	4.4E-006
			200-ST---CBP004-CBP--SHC	ST power cable short circuit	3.2E-008
	0.00	1.233E-015	200-ST---CBP004-CBP--OPC	ST power cable–open circuit	1.5E-007
			200-ST---MOE000--MOE-FSO	ST lock mode state fails on loss of power	1.4E-008
		4.808E-003	= Total		

NOTE: AO = aging overpack; ST = site transporter.

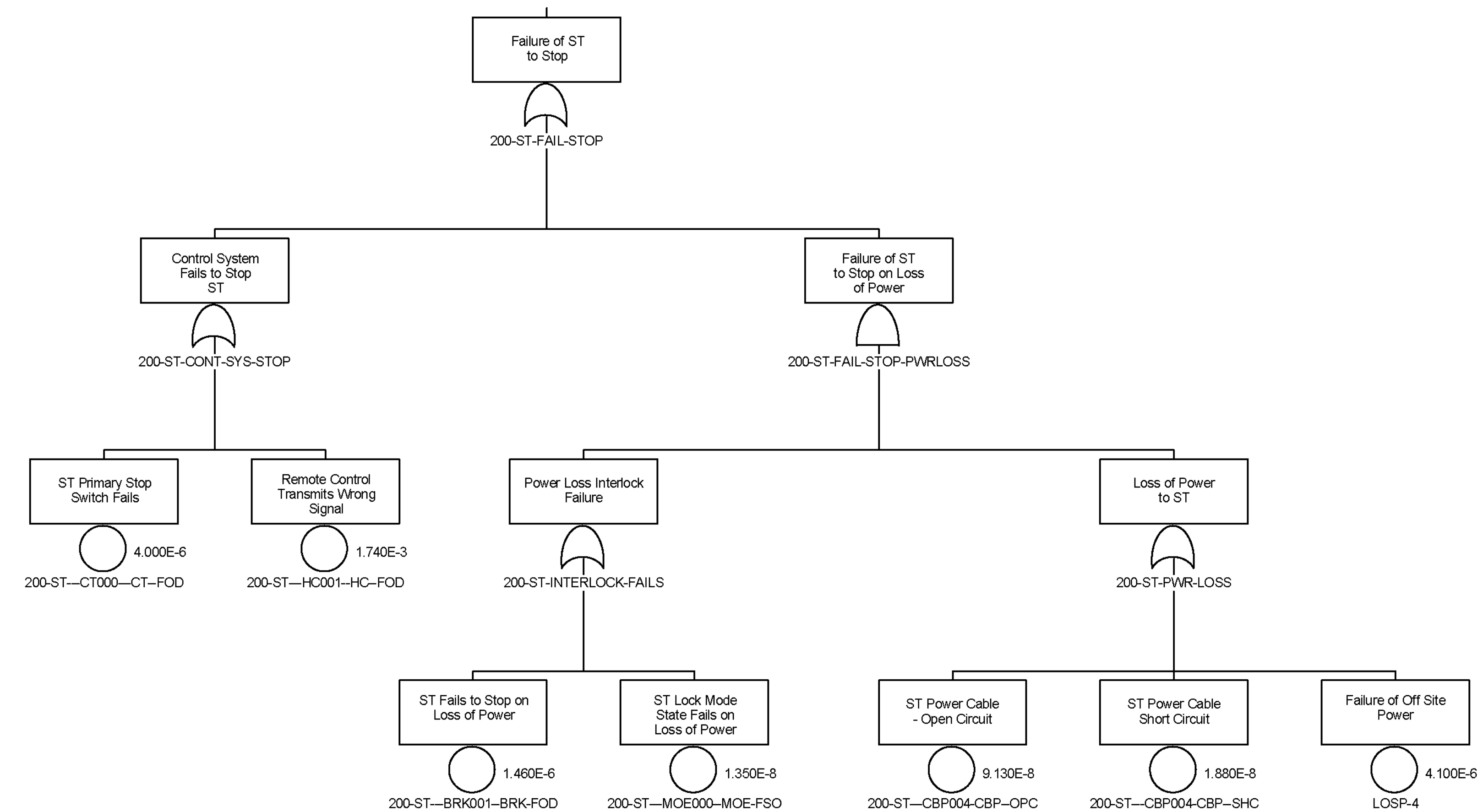
Source: Original

B6.4.1.8 Fault Tree



Source: Original

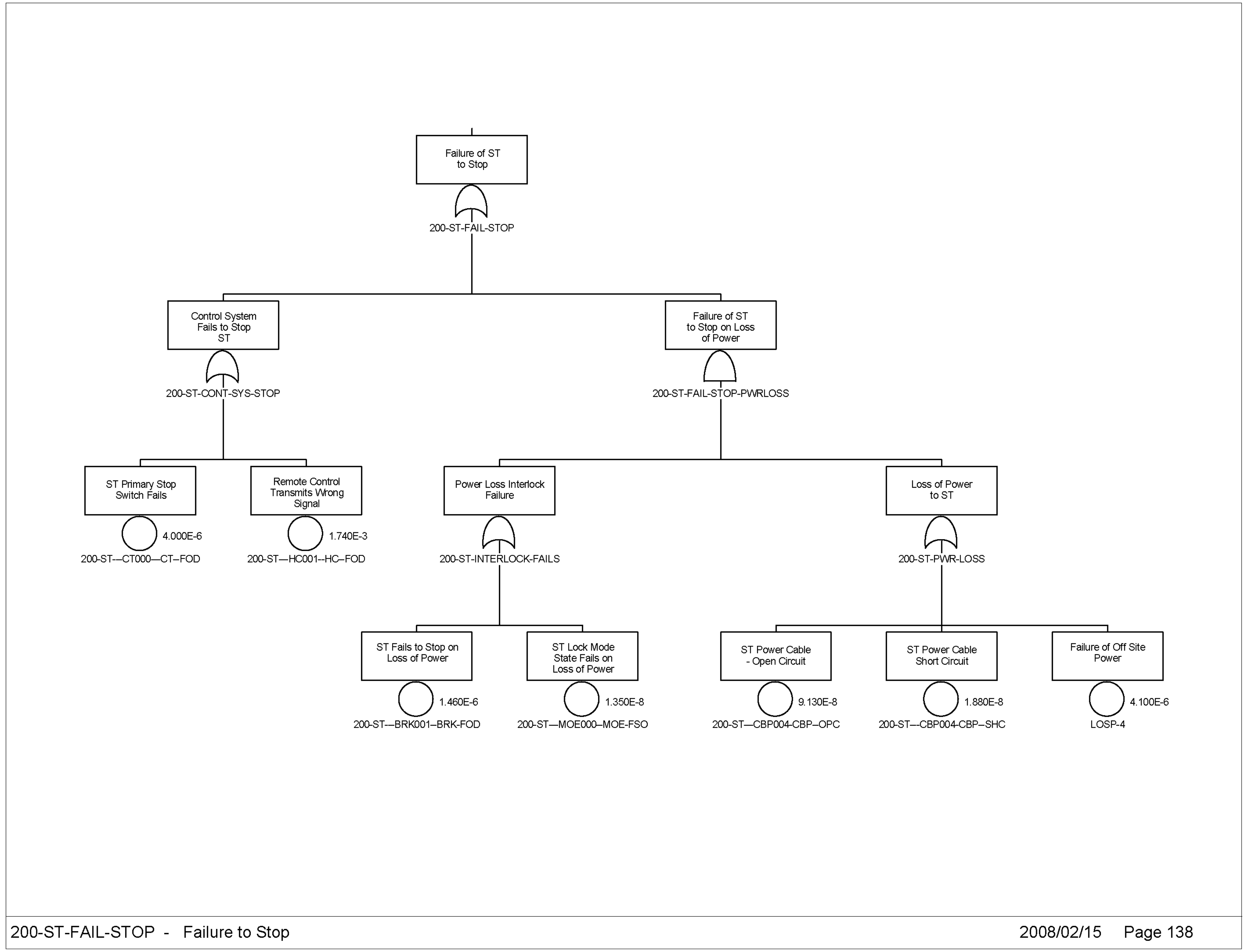
Figure B6.4-3. Site Transporter Collision in the RF



200-ST-FAIL-STOP - Failure to Stop 2008/02/15 Page 138

Source: Original

Figure B6.4-4. Failure to Stop



Source: Original

Figure B6.4-5. Site Transporter Exceeds Safe Speed

B6.4.2 Site Transporter Load Drop during Lift/Movement (ESD-08)

B6.4.2.1 Description

The site transporter conducts lift/lowering and movement operations at the aging pads and inside the facilities. Since the site transporter is only capable of performing one operation at a time it is not possible to move an aging overpack while it is being lifted/lowered. For activities associated with this ESD, there are four distinct failure modes. Those associated with electrical failures, site transporter controller failures, mechanical failures during lifting and lowering, and mechanical failures during movement.

B6.4.2.2 Success Criteria

The potential for a load drop exists when there is a loss of site transporter power, a hardware failure of the lift/lowering devices, aging overpack restraint device failure during movement, or a failure of the site transporter control system during these operations.

If there is a failure of the electrical system during lifting/lower or movement, the ACME screw/nuts prevent the rear forks and the lift support arms from moving. There is a potential for a common-cause failure of the forks.

The ACME devices also serve to prevent a load drop when there is a lift boom failure. There are four of these devices: one on each of the rear forks and one on each of the lift support arms.

The aging overpack restraint system is engaged after the lift has been accomplished and released prior to performing a lowering operation. These devices restrict the movement of the aging overpack during transport. There are three of these restraints that prevent/restrict movement in the X-Y-axis. Pins are used in these devices that prevent the release of the restraint in the advent of an electromechanical failure that controls the position of these devices.

There is an interlock built-in to the restraint system. Movement of the site transporter is prevented until the three pins in the restraint system have been properly installed. These pins also preclude an inadvertent release of the restraint system since they have to be physically removed by the operator before the restraints can be released.

The receipt of inadvertent command signals is also prevented in that the site transporter can only perform one operation at a time due to the limitations in the power plant.

B6.4.2.3 Design Requirements and Features

Requirements

Facility power is removed from the site transporter when it has been properly position within the Loading Room.

On the loss or removal of AC power derived from the facility, the site transporter performs a controlled stop. Once stopped the site transporter enters the “lock mode” safe state. The “lock mode” safe state is not reversible without specific operator action.

Features

There are no electrical storage devices in the design of the site transporter. When the facility AC power cable is removed, the site transporter is incapable of movement.

Two operators have the capability of stopping any operation performed by the site transporter when it is inside a facility.

B6.4.2.4 Fault Tree Model

The fault tree model for site transporter drop load during lift and movement addresses:

- Electrical failures including motor and distribution events and the failure to enter a lock mode safe state.
- A load drop during the lifting or lowering of the aging overpack which includes mechanical failure of the lifting booms and restraint/lifting arms.
- Failure of the aging overpack restraint subsystem during the lift/lowering/moving of the site transporter.
- Failure of the site transporter control subsystem.

NOTE: The fault tree defines the movement of the aging overpack in a three axis system as:

1. A roll movement side-to-side as the "R-axis."
2. A pitch movement front-to-back as the "P-axis."
3. A drop movement as the "D-axis."

B6.4.2.5 Basic Events Data

Table B6.4-3 lists the basic events used in the "Site Transporter Drop Load during Lift/Movement" fault tree. Uncertainty and cut set results are provided in Figures B6.4-6 and B6.4-7 respectively.

Table B6.4-3. Basic Event Probability for the Load Drop during Lift/Movement

Basic Events Probability Report					
Project: Yucca-Mountain		Case: Current			
ST Load Drop Lift/Movement		Units: Per Hour			
Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CRWT-ATB1001-AT--FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATB1011-AT--FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATB2002-AT--FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATB222-AT--FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATD0002-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000

Table B6.4-3. Basic Event Probability for the Load Drop during Lift/Movement (Continued)

Basic Events Probability Report					
Project: Yucca-Mountain		**Case: Current**			
ST Load Drop Lift/Movement		**Units: Per Hour**			
Name	**Calc. Type[a]**	**Calc. Prob.**	**Fail. Prob.**	**Lambda**	**Miss. Time[a]**
200-CRWT-ATD001-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATD03-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATD04-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATP002-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATR10002-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-ATR2004-AT-FOH	3	7.540E-005	0.000E+000	7.540E-005	1.000E+000
200-CRWT-BEA#1-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEA22-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEAB202-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEAD003-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEAD006-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEAP02-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEAR103-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-BEAR204-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-CBP0000-CBP-OPC	3	9.130E-008	0.000E+000	9.130E-008	1.000E+000
200-CRWT-CON0000-CON-FOH	3	7.140E-005	0.000E+000	7.140E-005	1.000E+000
200-CRWT-CTSHC000-CT-SPO	3	2.270E-005	0.000E+000	2.270E-005	1.000E+000
200-CRWT-DROP11-BEA-BRK	3	2.400E-008	0.000E+000	2.400E-008	1.000E+000
200-CRWT-ECP0000-ECP-FOH	3	1.790E-006	0.000E+000	1.790E-006	1.000E+000
200-CRWT-ELEC-MOE-FOD	1	6.000E-005	6.000E-005	0.000E+000	0.000E+000
200-CRWT-IEL0001-IEL-FOD	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-CRWT-LC000011-LC-FOD	1	6.250E-004	6.250E-004	0.000E+000	0.000E+000
200-CRWT-LVRD01-LVR-FOH	3	2.100E-006	0.000E+000	2.100E-006	1.000E+000
200-CRWT-LVRD02-LVR-FOH	3	2.100E-006	0.000E+000	2.100E-006	1.000E+000
200-CRWT-PIND004-PIN-BRK	3	2.120E-009	0.000E+000	2.120E-009	1.000E+000
200-CRWT-PIND005-PIN-BRK	3	2.120E-009	0.000E+000	2.120E-009	1.000E+000
200-CRWT-PINP04-PIN-BRK	3	2.120E-009	0.000E+000	2.120E-009	1.000E+000
200-CRWT-PINR103-PIN-BRK	3	2.120E-009	0.000E+000	2.120E-009	1.000E+000
200-CRWT-PINR202-PIN-BRK	3	2.120E-009	0.000E+000	2.120E-009	1.000E+000
200-CRWT-SJKB011-SJK-FOH	3	8.140E-006	0.000E+000	8.140E-006	1.000E+000
200-CRWT-SJKB101-SJK-FOH	3	8.140E-006	0.000E+000	8.140E-006	1.000E+000
200-CRWT-SJKB202-SJK-FOH	3	8.140E-006	0.000E+000	8.140E-006	1.000E+000

Table B6.4-3. Basic Event Probability for the Load Drop during Lift/Movement (Continued)

Basic Events Probability Report					
Project: Yucca-Mountain		**Case: Current**			
ST Load Drop Lift/Movement		**Units: Per Hour**			
Name	**Calc. Type[a]**	**Calc. Prob.**	**Fail. Prob.**	**Lambda**	**Miss. Time[a]**
200-CRWT-SJKB22-SJK-FOH	3	8.140E-006	0.000E+000	8.140E-006	1.000E+000
200-CRWT-ZSD00005-ZS-FOD	1	2.930E-004	2.930E-004	0.000E+000	0.000E+000
200-CRWT-ZSD0006-ZS-FOD	1	2.930E-004	2.930E-004	0.000E+000	0.000E+000
200-CRWT-ZSP00003-ZS-FOD	1	2.930E-004	2.930E-004	0.000E+000	0.000E+000
200-CRWT-ZSR00005-ZS-FOD	1	2.930E-004	2.930E-004	0.000E+000	0.000E+000
200-ST-MOE0001-MOE-FSO	3	1.350E-008	0.000E+000	1.350E-008	1.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; ST = site transporter.

Source: Original

B6.4.2.5.1 Human Failure Events

There are two human error events incorporated in the tree. These are:

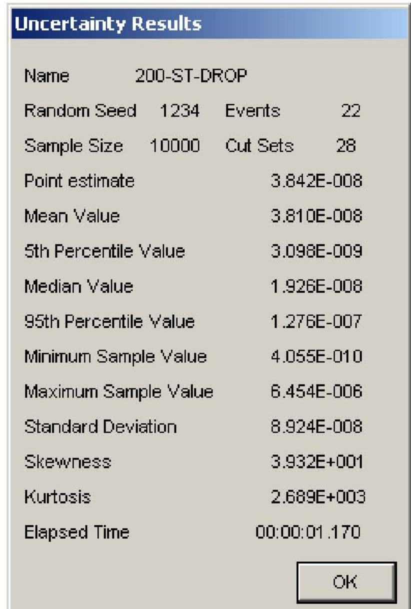
- Operator action which results in a load drop. This is set to a screening value of 1E-03.
- Operator sends wrong command which results in a load drop. This event is also set to a screening value of 1E-03.

B6.4.2.5.2 Common-Cause Failures

There are no CCFs identified in this fault tree.

B6.4.2.6 Uncertainty and Cut Set Generation

Figures B6.4-6 and B6.4-7 contain the uncertainty and the cut set generation results for site transporter load drop during lift and movement. The fault trees are shown in Figures B6.4-8 through B6.4-19.



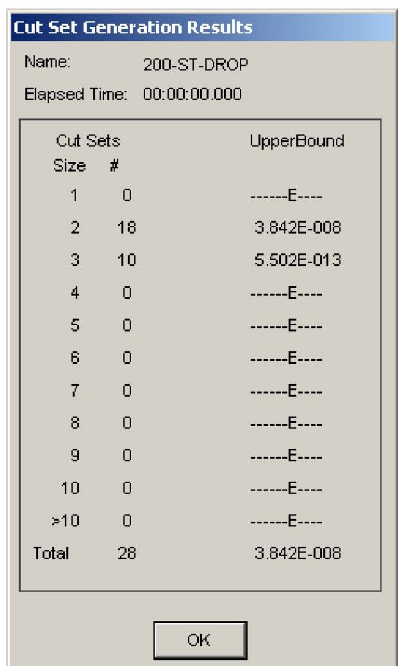
Uncertainty Results

Name	200-ST-DROP		
Random Seed	1234	Events	22
Sample Size	10000	Cut Sets	28
Point estimate			3.842E-008
Mean Value			3.810E-008
5th Percentile Value			3.098E-009
Median Value			1.926E-008
95th Percentile Value			1.276E-007
Minimum Sample Value			4.055E-010
Maximum Sample Value			6.454E-006
Standard Deviation			8.924E-008
Skewness			3.932E+001
Kurtosis			2.689E+003
Elapsed Time			00:00:01.170

OK

Source: Original

Figure B6.4-6. Uncertainty Results for the Site Transporter Load Drop during Lift and Movement Fault Tree



Cut Set Generation Results

Name: 200-ST-DROP

Elapsed Time: 00:00:00.000

Cut Sets Size	#	UpperBound
1	0	------E----
2	18	3.842E-008
3	10	5.502E-013
4	0	------E----
5	0	------E----
6	0	------E----
7	0	------E----
8	0	------E----
9	0	------E----
10	0	------E----
>10	0	------E----
Total	28	3.842E-008

OK

Source: Original

Figure B6.4-7. Cut Set Generation Results for the Site Transporter Load Drop during Lift and Movement Fault Tree

B6.4.2.7 Cut Sets

Table B6.4-4 contains the cut sets for the "Site Transporter Load Drop during Lift and Movement" fault tree.

Table B6.4-4. Cut Sets for the Site Transporter Load Drop during Lift and Movement Fault Tree

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
200-ST-DROP	36.92	1.419E-008	200-CRWT-CTSHC000-CT-SPO	Spurious Command to Raise/Lower AO	2.3E-005
			200-CRWT-LC000011-LC-FOD	ST Lift/Lower Selector Level Fails	6.2E-004
	20.97	8.058E-009	200-CRWT-IEL0001-IEL-FOD	Restraint System Interlock Failure	2.8E-005
			200-CRWT-ZSD00005-ZS-FOD	ST D-Axis Position Switch Failure Movement	2.9E-004
	20.97	8.058E-009	200-CRWT-IEL0001-IEL-FOD	Restraint System Interlock Failure	2.8E-005
			200-CRWT-ZSP00003-ZS-FOD	ST P-Axis Position Switch Failure During Movement	2.9E-004
	20.97	8.058E-009	200-CRWT-IEL0001-IEL-FOD	Restraint System Interlock Failure	2.8E-005
			200-CRWT-ZSR00005-ZS-FOD	ST R-Axis Position Switch Failure Movement	2.9E-004
	0.11	4.063E-011	200-CRWT-CTSHC000-CT-SPO	Spurious Command to Raise/Lower AO	2.3E-005
			200-CRWT-ECP0000-ECP-FOH	ST Restraint Arms Position Selector Fails	1.8E-006
	0.02	7.032E-012	200-CRWT-BEAB202-BEA-BRK	Boom#2 Fails During Cask Movement	2.4E-008
			200-CRWT-ZSD0006-ZS-FOD	ST D-Axis Position Switch Failure Lift/Lower	2.9E-004
	0.02	7.032E-012	200-CRWT-DROP11-BEA-BRK	Boom#1 Fails During Cask Lift	2.4E-008
			200-CRWT-ZSD0006-ZS-FOD	ST D-Axis Position Switch Failure Lift/Lower	2.9E-004
	0.00	1.810E-012	200-CRWT-ATD0002-AT-FOH	ST D-Axis Electrical Actuator #2 Fails Lift/Lower	7.5E-005
			200-CRWT-BEAB202-BEA-BRK	Boom#2 Fails During Cask Movement	2.4E-008
	0.00	1.810E-012	200-CRWT-ATD0002-AT-FOH	ST D-Axis Electrical Actuator #2 Fails Lift/Lower	7.5E-005
			200-CRWT-DROP11-BEA-BRK	Boom#1 Fails During Cask Lift	2.4E-008
	0.00	1.810E-012	200-CRWT-ATD001-AT-FOH	ST D-Axis Electrical Actuator #1 Fails Lift/Lower	7.5E-005
			200-CRWT-BEAB202-BEA-BRK	Boom#2 Fails During Cask Movement	2.4E-008
	0.00	1.810E-012	200-CRWT-ATD001-AT-FOH	ST D-Axis Electrical Actuator #1 Fails Lift/Lower	7.5E-005
			200-CRWT-DROP11-BEA-BRK	Boom#1 Fails During Cask Lift	2.4E-008
	0.00	9.639E-013	200-CRWT-CON0000-CON-FOH	Electrical Power Dist Connectors Fail on ST	7.1E-005
			200-ST-MOE0001-MOE-FSO	ST Lock Mode State Fails on Loss of Power	1.4E-008
	0.00	8.100E-013	200-CRWT-ELEC-MOE-FOD	ST Electric Motor Failure	6.0E-005
			200-ST-MOE0001-MOE-FSO	ST Lock Mode State Fails on Loss of Power	1.4E-008
	0.00	1.798E-013	200-CRWT-ATB1011-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.5E-005

Table B6.4-4. Cut Sets for the Site Transporter Load Drop during Lift and Movement Fault Tree (Continued)

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
			200-CRWT-SJKB011-SJK-FOH	Screw Lift on Boom #1 Fails	8.1E-006
			200-CRWT-ZSD0006-ZS-FOD	ST D-Axis Position Switch Failure Lift/Lower	2.9E-004
	0.00	1.798E-013	200-CRWT-ATB2002-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.5E-005
			200-CRWT-SJKB202-SJK-FOH	Screw Lift on Boom #2 Fails	8.1E-006
			200-CRWT-ZSD0006-ZS-FOD	ST D-Axis Position Switch Failure Lift/Lower	2.9E-004
	0.00	5.040E-014	200-CRWT-BEAB202-BEA-BRK	Boom#2 Fails During Cask Movement	2.4E-008
			200-CRWT-LVRD01-LVR-FOH	ST D-Axis Actuator Structural Arm #1 Failure	2.1E-006
	0.00	5.040E-014	200-CRWT-BEAB202-BEA-BRK	Boom#2 Fails During Cask Movement	2.4E-008
			200-CRWT-LVRD02-LVR-FOH	ST D-Axis Actuator Structural Arm #2 Failure	2.1E-006
	0.00	5.040E-014	200-CRWT-DROP11-BEA-BRK	Boom#1 Fails During Cask Lift	2.4E-008
			200-CRWT-LVRD01-LVR-FOH	ST D-Axis Actuator Structural Arm #1 Failure	2.1E-006
	0.00	5.040E-014	200-CRWT-DROP11-BEA-BRK	Boom#1 Fails During Cask Lift	2.4E-008
			200-CRWT-LVRD02-LVR-FOH	ST D-Axis Actuator Structural Arm #2 Failure	2.1E-006
	0.00	4.627E-014	200-CRWT-ATB1011-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.5E-005
			200-CRWT-ATD0002-AT-FOH	ST D-Axis Electrical Actuator #2 Fails Lift/Lower	7.5E-005
			200-CRWT-SJKB011-SJK-FOH	Screw Lift on Boom #1 Fails	8.1E-006
	0.00	4.627E-014	200-CRWT-ATB1011-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.5E-005
			200-CRWT-ATD001-AT-FOH	ST D-Axis Electrical Actuator #1 Fails Lift/Lower	7.5E-005
			200-CRWT-SJKB011-SJK-FOH	Screw Lift on Boom #1 Fails	8.1E-006
	0.00	4.627E-014	200-CRWT-ATB2002-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.5E-005
			200-CRWT-ATD0002-AT-FOH	ST D-Axis Electrical Actuator #2 Fails Lift/Lower	7.5E-005
			200-CRWT-SJKB202-SJK-FOH	Screw Lift on Boom #2 Fails	8.1E-006
	0.00	4.627E-014	200-CRWT-ATB2002-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.5E-005
			200-CRWT-ATD001-AT-FOH	ST D-Axis Electrical Actuator #1 Fails Lift/Lower	7.5E-005
			200-CRWT-SJKB202-SJK-FOH	Screw Lift on Boom #2 Fails	8.1E-006
	0.00	1.289E-015	200-CRWT-ATB1011-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.5E-005
			200-CRWT-LVRD01-LVR-FOH	ST D-Axis Actuator Structural Arm #1 Failure	2.1E-006
			200-CRWT-SJKB011-SJK-FOH	Screw Lift on Boom #1 Fails	8.1E-006

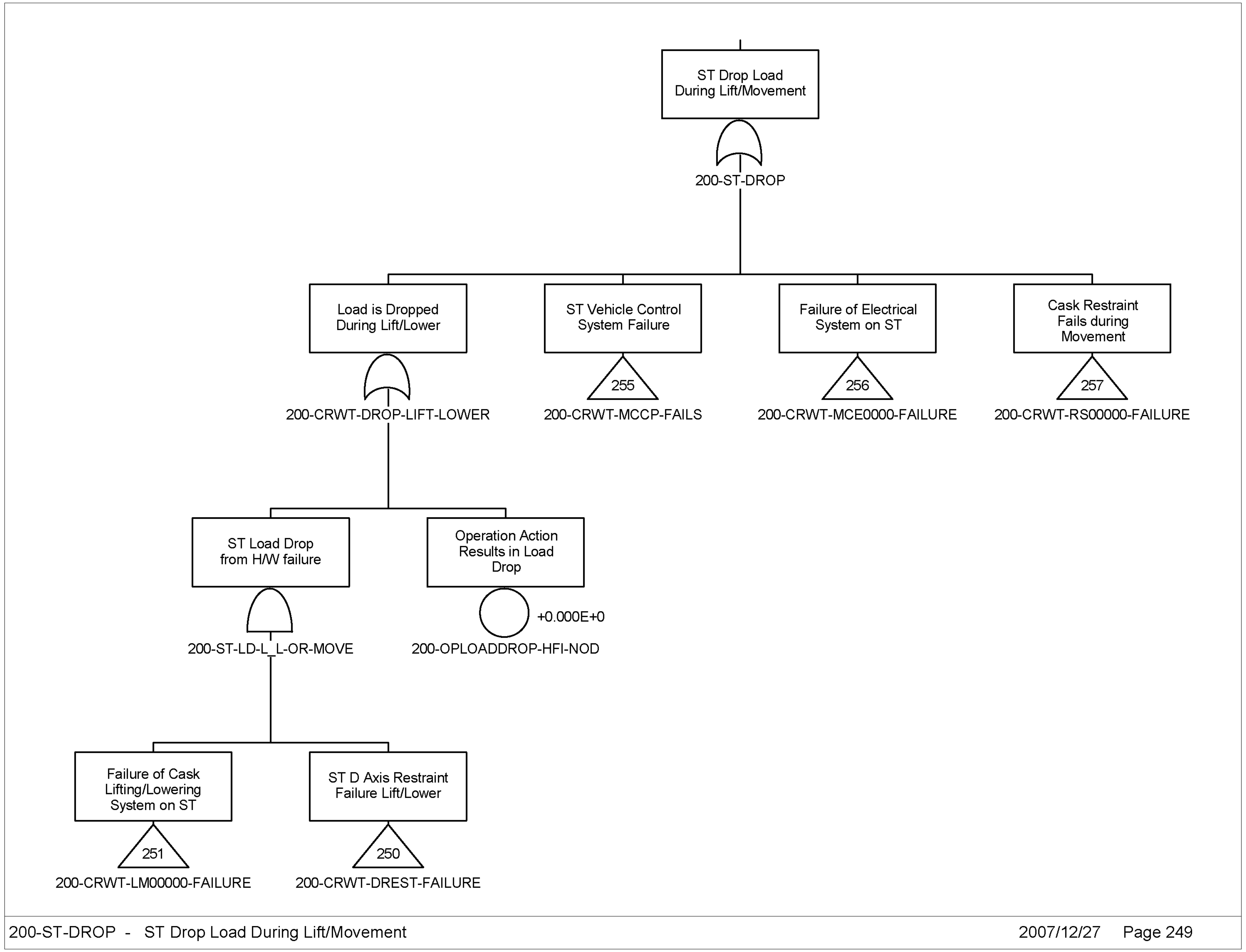
Table B6.4-4. Cut Sets for the Site Transporter Load Drop during Lift and Movement Fault Tree (Continued)

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
	0.00	1.289E-015	200-CRWT-ATB1011-AT--FOH	Screw Actuator Mechanism on Lift Boom #1 Fails	7.5E-005
			200-CRWT-LVRD02-LVR-FOH	ST D-Axis Actuator Structural Arm #2 Failure	2.1E-006
			200-CRWT-SJKB011-SJK-FOH	Screw Lift on Boom #1 Fails	8.1E-006
	0.00	1.289E-015	200-CRWT-ATB2002-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.5E-005
			200-CRWT-LVRD01-LVR-FOH	ST D-Axis Actuator Structural Arm #1 Failure	2.1E-006
			200-CRWT-SJKB202-SJK-FOH	Screw Lift on Boom #2 Fails	8.1E-006
	0.00	1.289E-015	200-CRWT-ATB2002-AT--FOH	Screw Actuator Mechanism on Lift Boom #2 Fails	7.5E-005
			200-CRWT-LVRD02-LVR-FOH	ST D-Axis Actuator Structural Arm #2 Failure	2.1E-006
			200-CRWT-SJKB202-SJK-FOH	Screw Lift on Boom #2 Fails	8.1E-006
	0.00	1.233E-015	200-CRWT-CBP0000-CBP-OPC	Electrical Power Dist Cable Failure on ST	9.1E-008
			200-ST-MOE0001-MOE-FSO	ST Lock Mode State Fails on Loss of Power	1.4E-008
		3.842E-008	= Total		

NOTE: AO = aging overpack; CCF = common-cause failure; ST = site transporter.

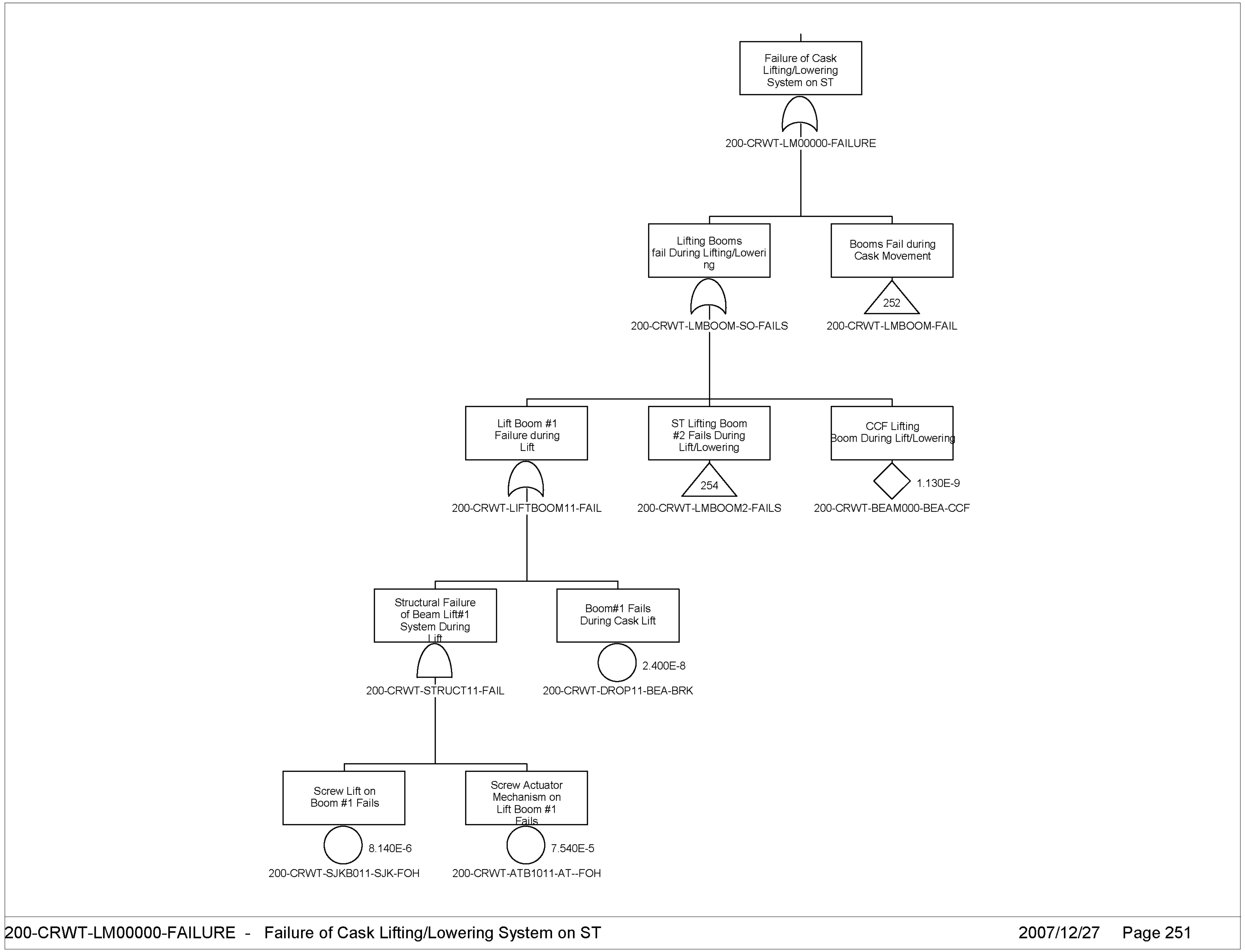
Source: Original

B6.4.2.8 Fault Tree



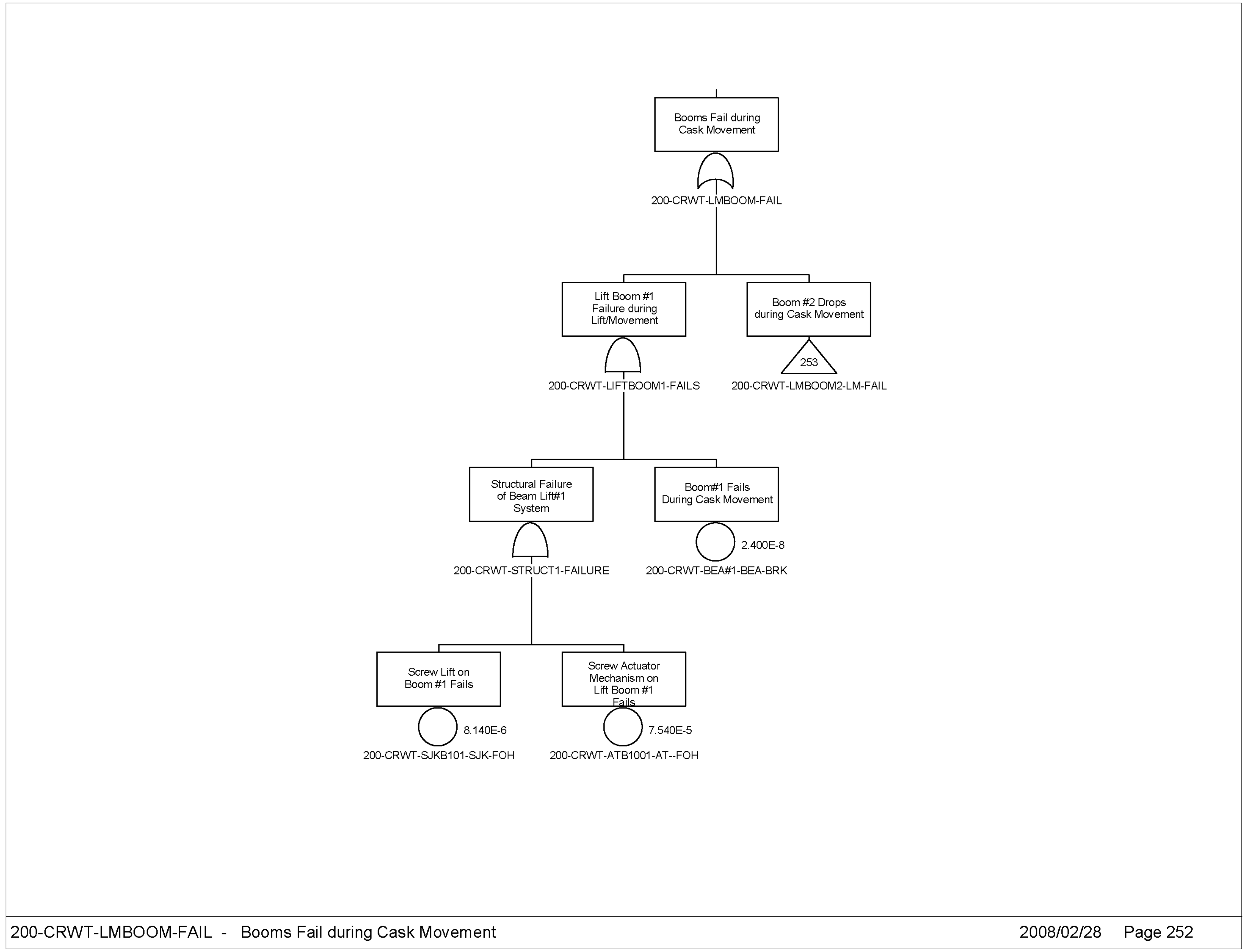
Source: Original

Figure B6.4-8. Site Transporter Drop Load During Lift/Movement



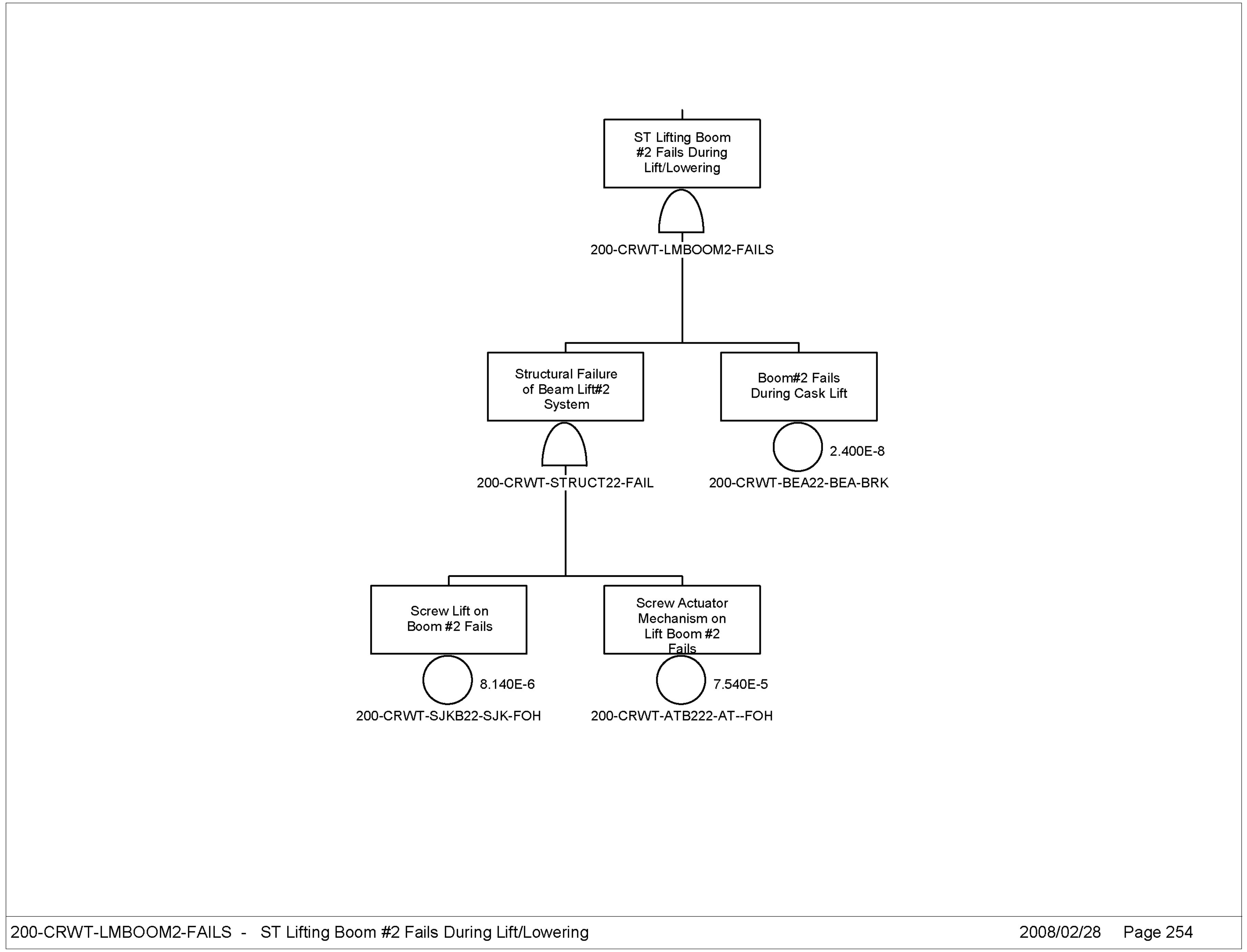
Source: Original

Figure B6.4-9. Failure of Cask Lifting/Lowering System on Site Transporter



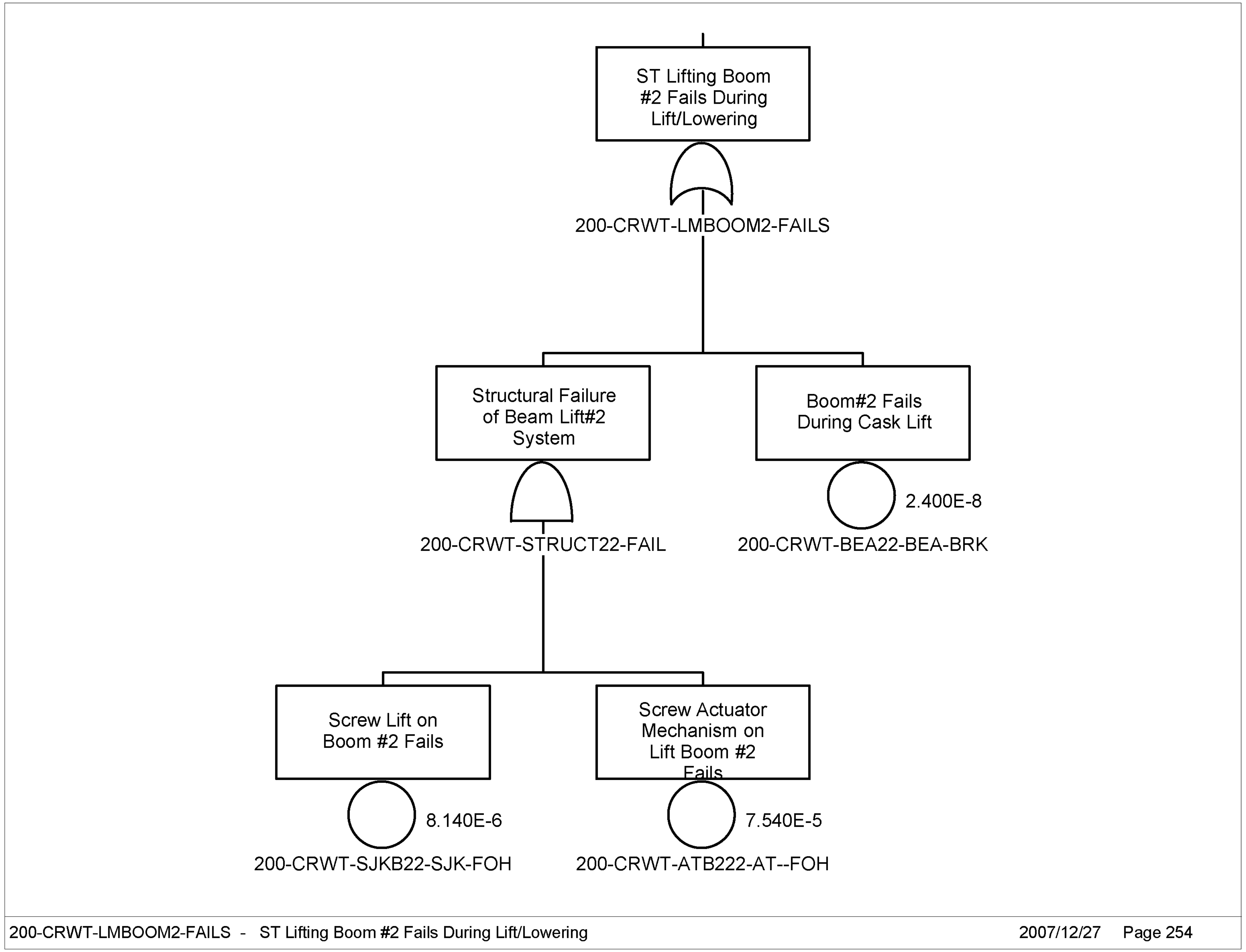
Source: Original

Figure B6.4-10. Booms Fail during Cask Movement



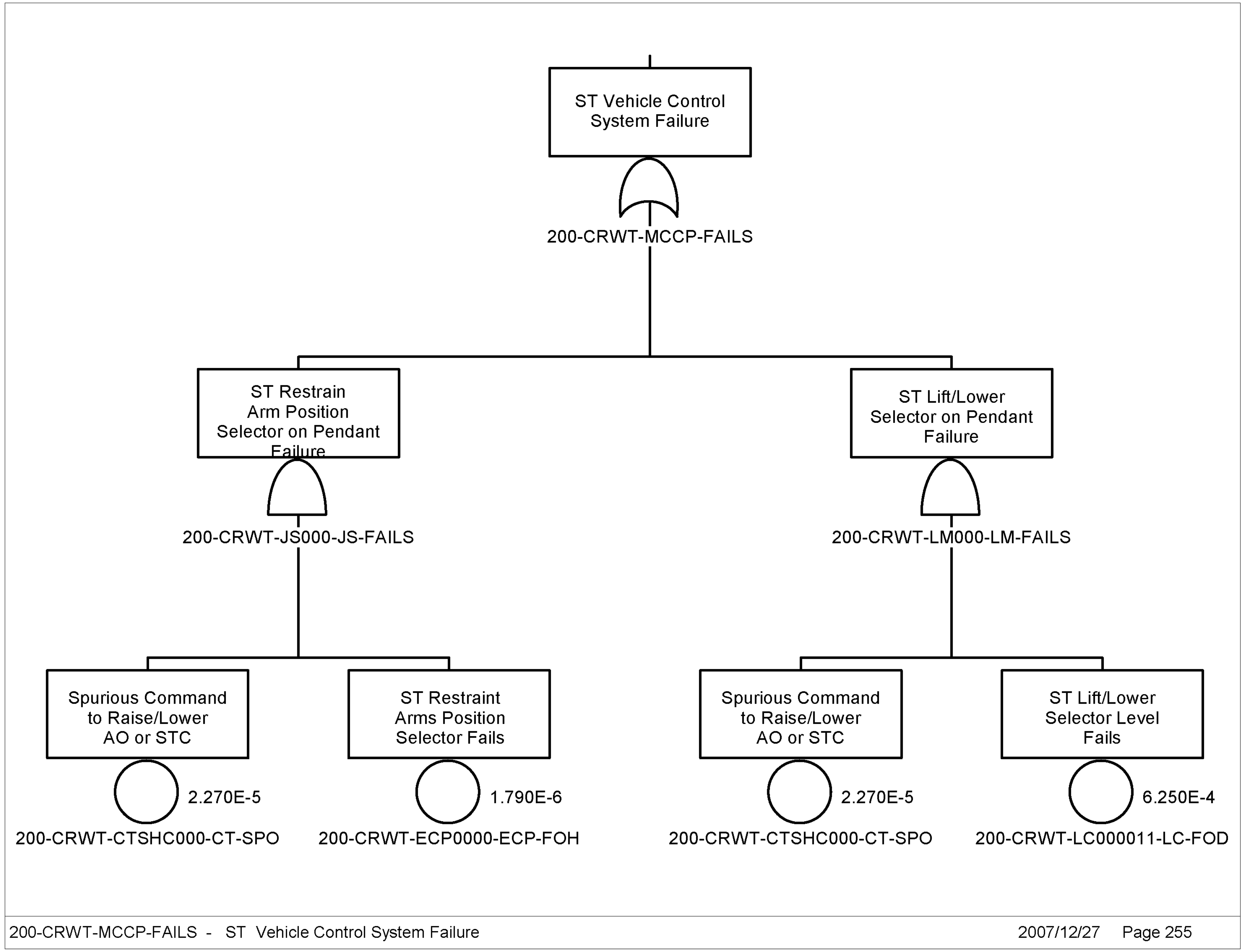
Source: Original

Figure B6.4-11. Boom #2 Drops during Cask Movement



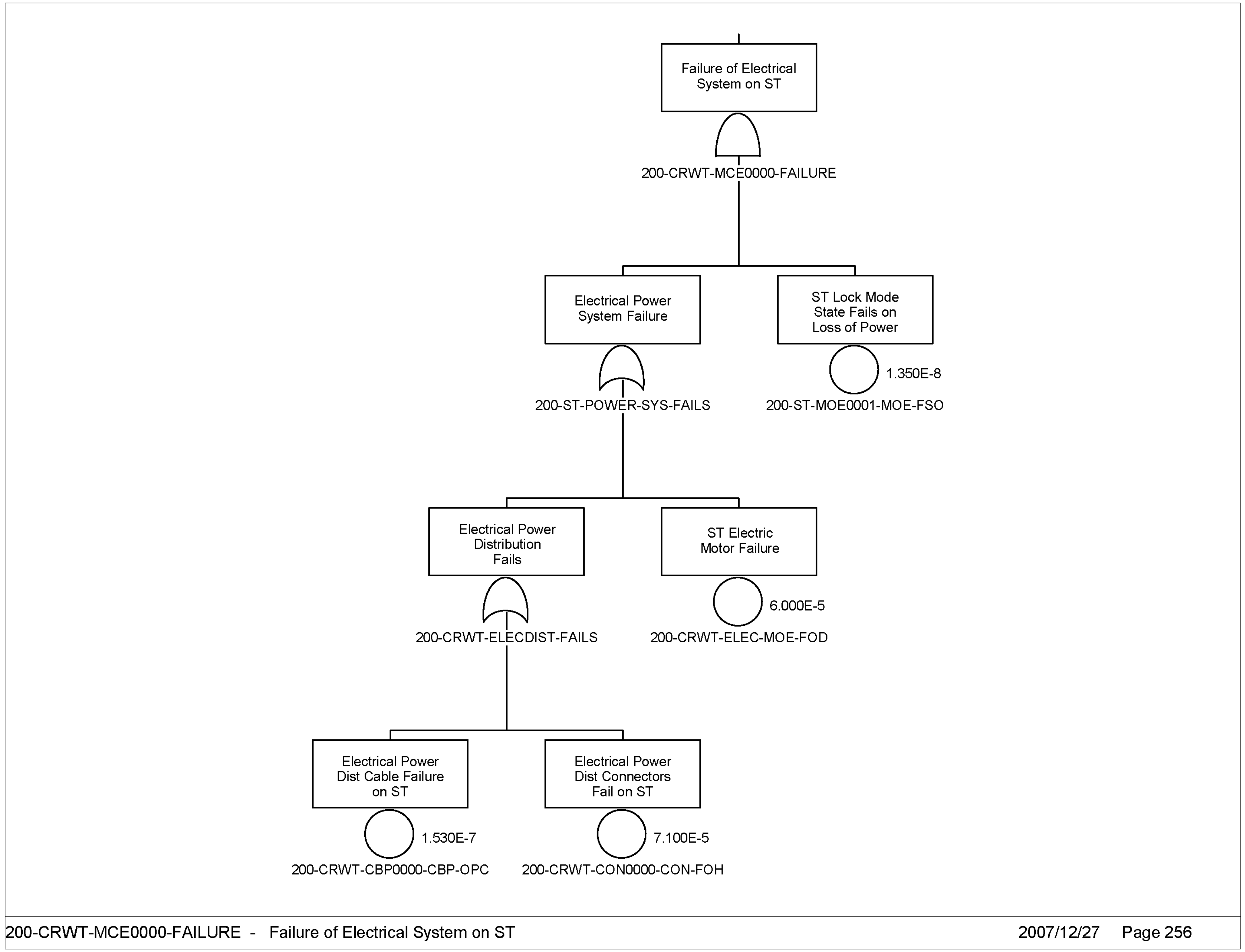
Source: Original

Figure B6.4-12. Site Transporter Lifting Boom #2 Fails During Lift/Lowering



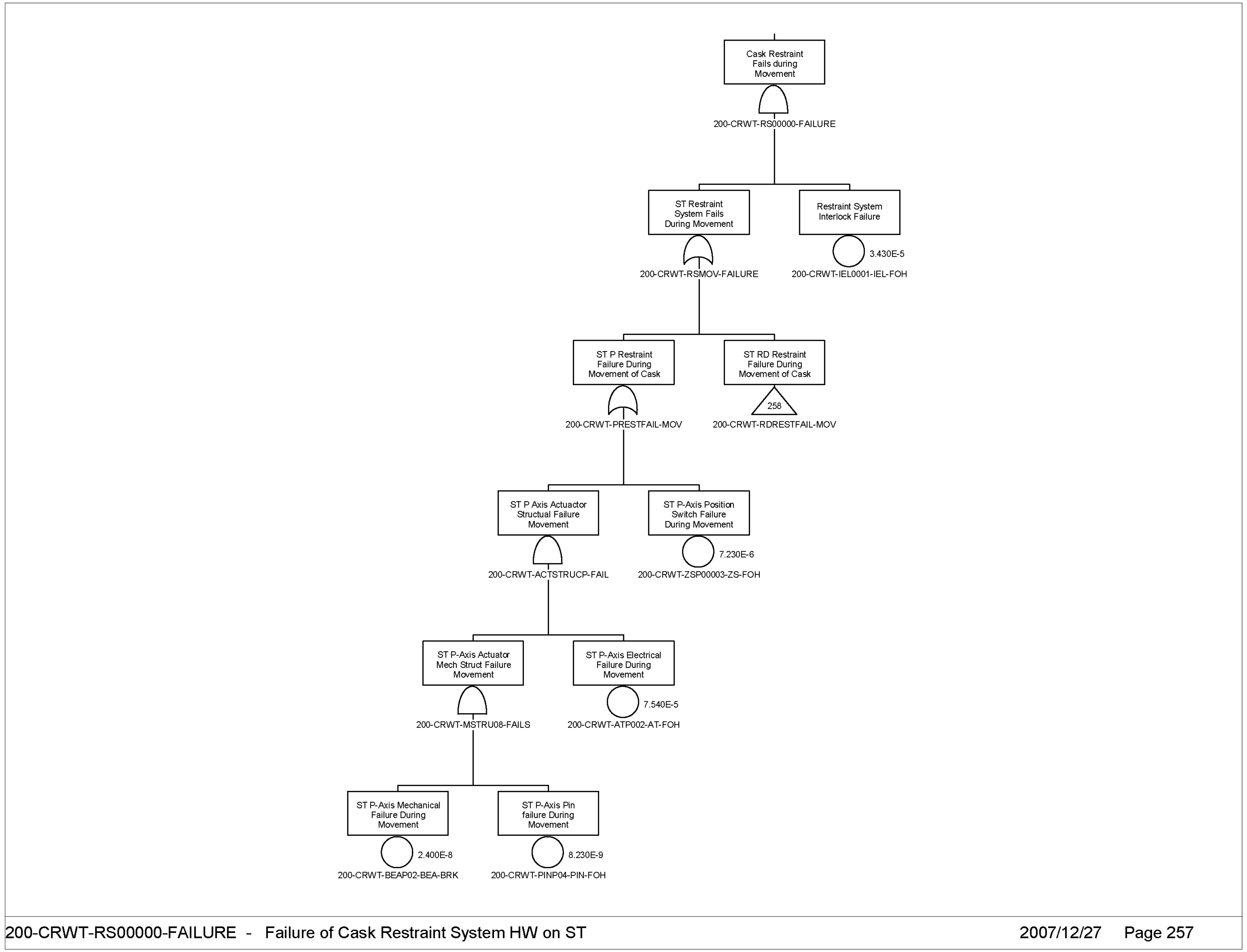
Source: Original

Figure B6.4-13. Site Transporter Vehicle Control System Failure



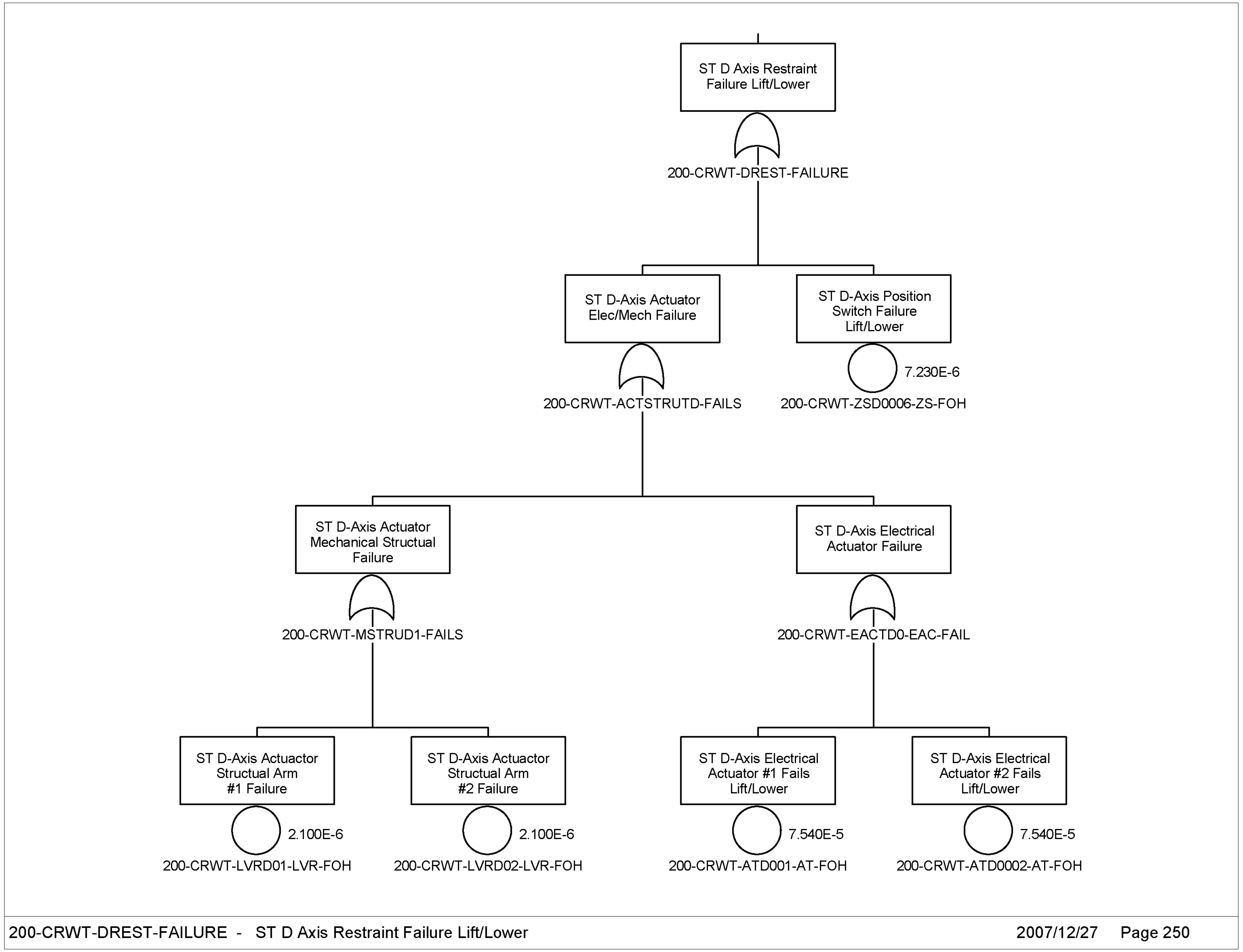
Source: Original

Figure B6.4-14. Failure of Electrical System on Site Transporter



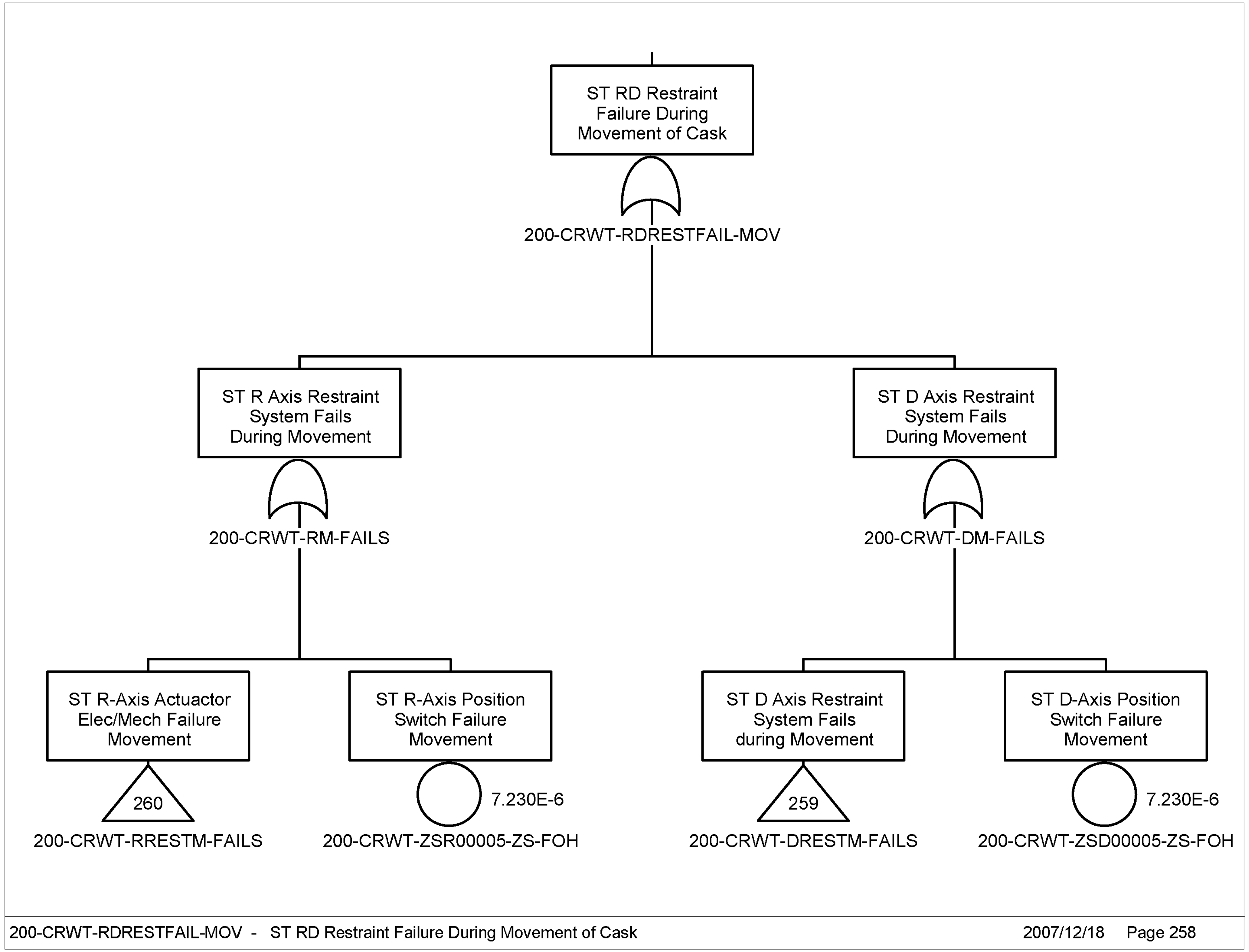
Source: Original

Figure B6.4-15. Cask Restraint Fails During Movement



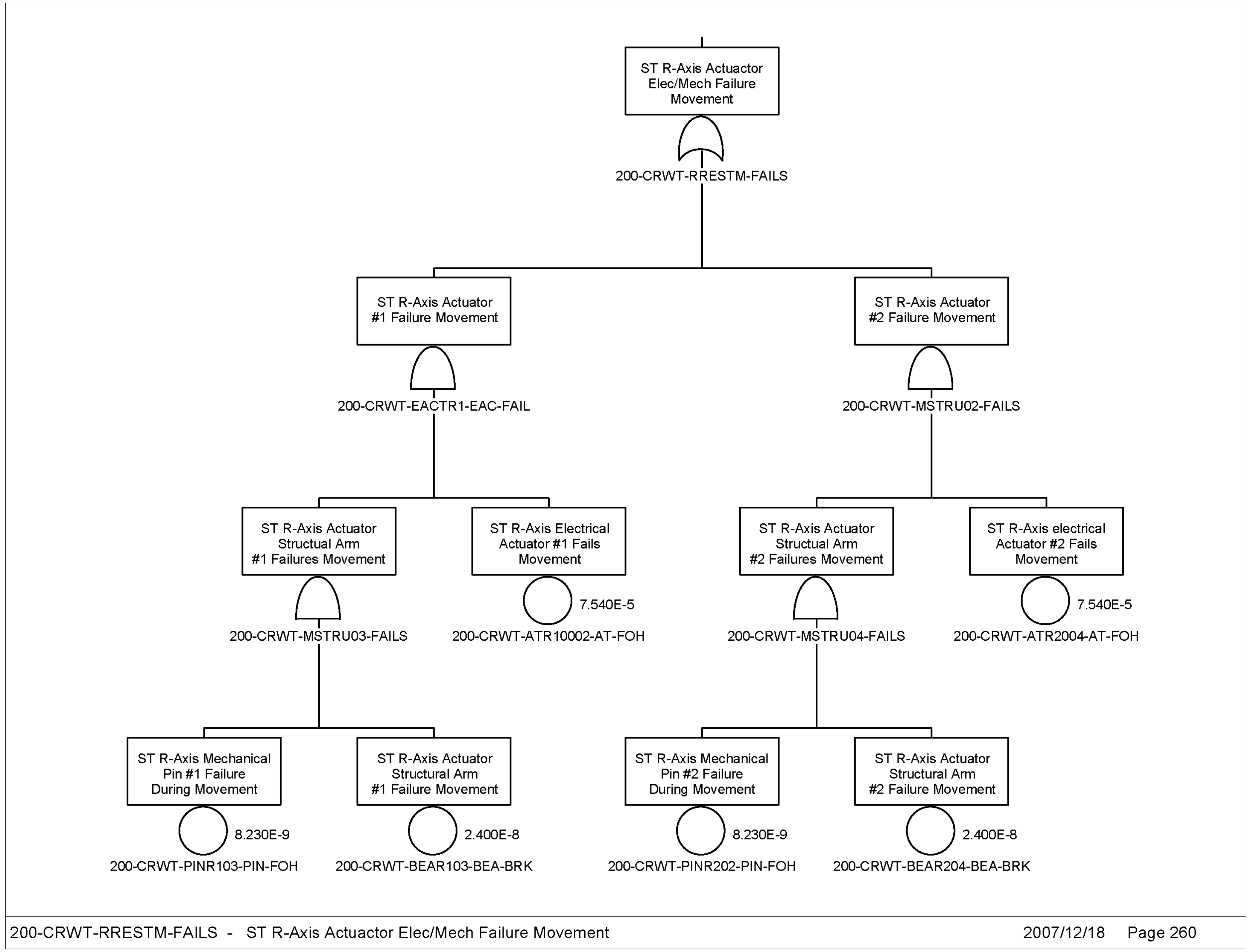
Source: Original

Figure B6.4-16. Site Transporter D-Axis Restraint Failure Lift/Lower



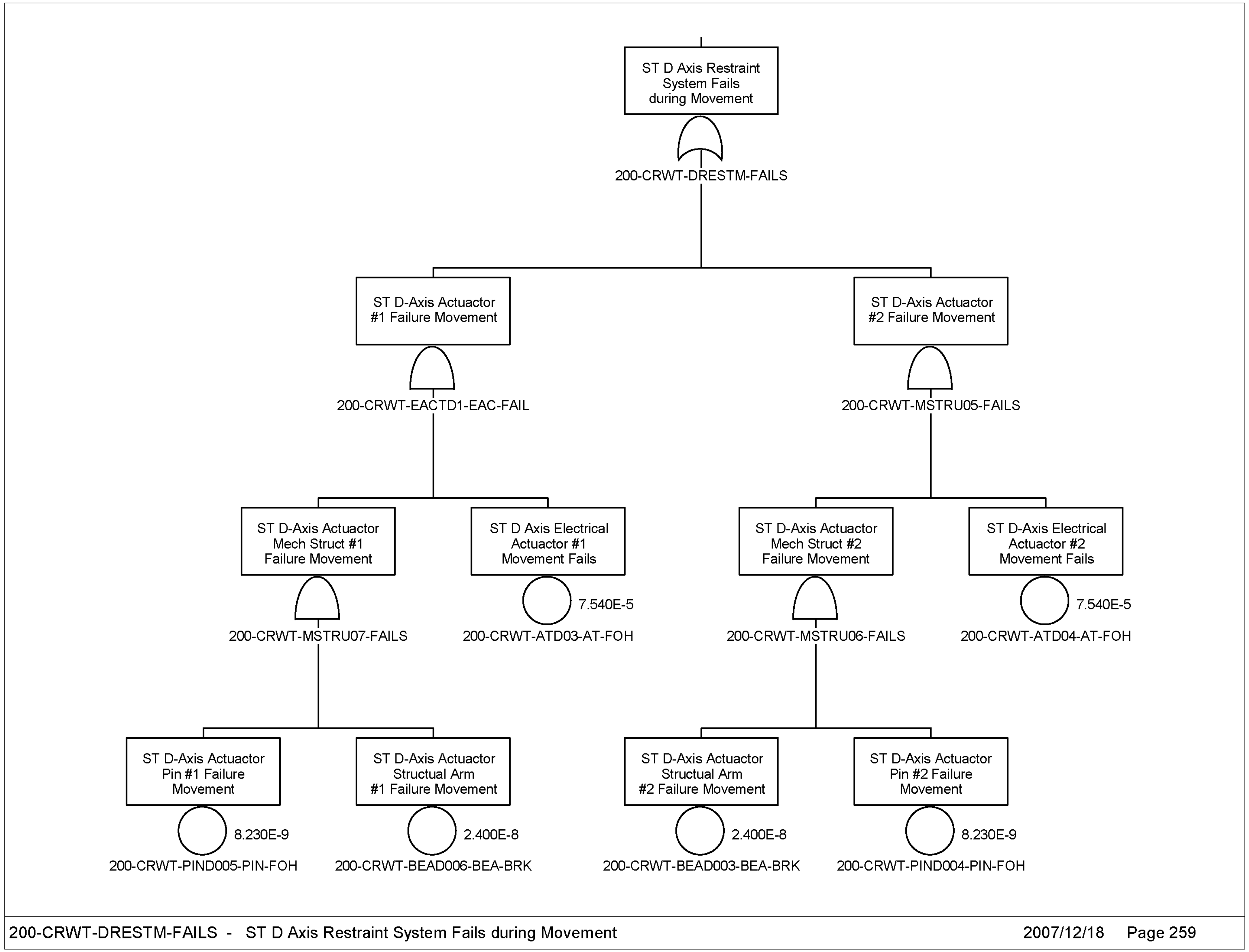
Source: Original

Figure B6.4-17. Site Transporter R- and D-Axis Restraint Failure During Movement of Cask



Source: Original

Figure B6.4-18. Site Transporter R-Axis Actuator Electrical/Mechanical Failure Movement



Source: Original

Figure B6.4-19. Site Transporter D-Axis Restraint System Fails during Movement

B6.4.3 Site Transporter Rollover (Tipover) (ESD-08)

B6.4.3.1 Description

Although the site transporter has been designed to have a low center of gravity and a wide footprint, there is a possibility of a rollover caused by a track failure with a subsequent operator failure to stop the site transporter upon loss of a track. The track would have to fail in a manner such that it binds (i.e., rolls up), the site transporter drives over the failed track, and the site transporter tilts to an angle that results in a tipover condition.

B6.4.3.2 Success Criteria

The design of the site transporter prevents the majority of scenarios that could potentially cause a site transporter rollover. The site transporter is designed to negotiate a 5% grade and a 2% cross-slope. In addition, the aging overpack is physically prevented from being lifted more than 12 in. The combination of the low lift of the aging overpack, the low center of gravity, and wide footprint of the site transporter results in a stable platform during movements.

During movement, a site transporter track failure could result in a potential tipover situation. There is no design constraint for this failure mode; preventing this situation relies on an operator awareness and response to this situation to initiate an emergency stop command. The operator has several seconds to respond to the track failure; however, since this is a recovery action, no credit is taken for the operator response.

B6.4.3.3 Design Requirements and Features

Requirements

Operators have the capability of stopping the site transporter in sufficient time to keep the site transporter from running off the end of a broken track.

Design Feature

The center of gravity of a loaded site transporter with aging overpack ensures stability.

The site transporter operator has the capability to stop the operation of the site transporter during abnormal conditions.

B6.4.3.4 Fault Tree Model

Human error is conservatively postulated to result in a rollover/tipover if the operator does not stop the site transporter in sufficient time to prevent the site transporter from running off the broken track.

B6.4.3.5 Basic Event Data

Table B6.4-5 lists the basic events used in the site transporter drop load during lift/movement fault tree. Uncertainty and cut set results are provided in Figures B6.4-20 and B6.4-21 respectively.

Table B6.4-5. Basic Event Probability for the site transporter Rollover

Basic Events Probability Report					
Project: Yucca-Mountain		Case: Current			
ST Rollover		Units: Per Hour			
Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CRWT-TRD0001-TRD-FOH	3	5.890E-007	1.000E+000	5.890E-007	1.000E+000
200-CRWT-TRD0002-TRD-FOH	3	5.890E-007	1.000E+000	5.890E-007	1.000E+000
200-CRWT-TRD0003-TRD-FOH	3	5.890E-007	1.000E+000	5.890E-007	1.000E+000
200-CRWT-TRD0004-TRD-FOH	3	5.890E-007	1.000E+000	5.890E-007	1.000E+000
200-CRWT-TRK0001-TRD-FOH	3	5.890E-007	1.000E+000	5.890E-007	1.000E+000
200-OP-FAILSTOP-HFI-NOD	1	1.000E+000	1.000E+000	0.000E+000	0.000E+000

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation, Fail. = failure; Miss. = mission; Prob. = probability; ST = site transporter.

Source Original

B6.4.3.5.1 Human Failure Events

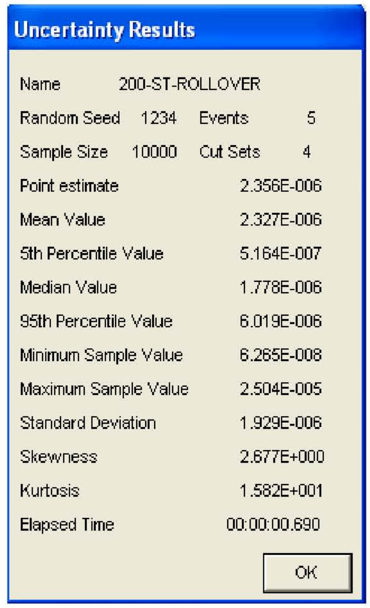
There is one human error failure event included in this model. It is conservatively set to a value of 1E+0 because unsafe actions that require an equipment failure to cause an initiating event are generically assigned a screening HEP of 1.0 (Table E6.4-1).

B6.4.3.5.2 Common-Cause Failures

There are no common-cause failures identified for this fault tree in that the failure of one track could potentially result in a rollover (tipover).

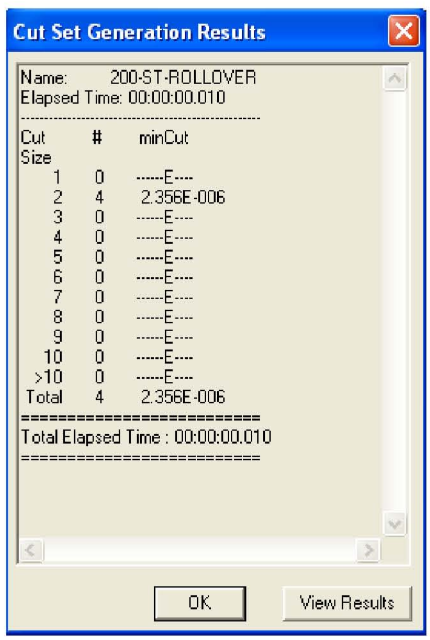
B6.4.3.6 Uncertainty and Cut set Generation

Figures B6.4-20 and B6.4-21 contain the uncertainty and the cut set generation results for “Site Transporter Rollover (Tipover)” fault tree using a cutoff probability of 1E-15. The fault tree can be found on Figure B6.4-22.



Source: Original

Figure B6.4-20. Uncertainty Results for the Site Transporter Rollover Fault Tree



Source: Original

Figure B6.4-21. Cut Set Generation Results for the Site Transporter Rollover Fault Tree

B6.4.3.7 Cut sets

Table B6.4-6 contains the cut sets for the "Site Transporter Rollover" fault tree.

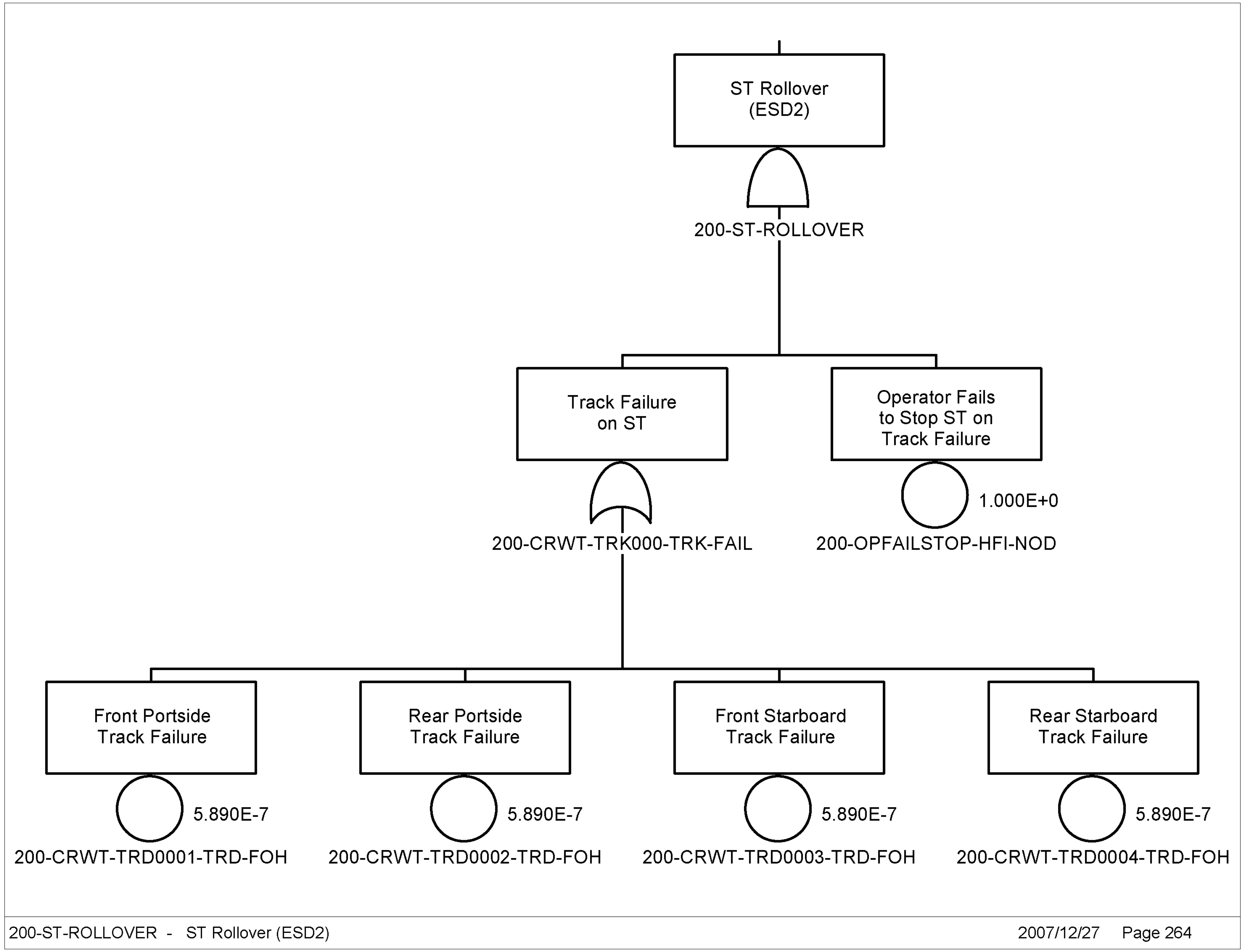
Table B6.4-6. Cut Sets for the Site Transporter Rollover (Tipover)

Fault Tree	Cut set %	Prob./Freq.	Basic Event	Description	Probability
200-ST-ROLLOVER	25.00	5.890E-007	200-CRWT-TRD0001-TRD-FOH	Front portside track failure	5.9E-007
			200-OPFAILSTOP-HFI-NOD	Operator fails to stop ST on track failure	1.0E+000
	25.00	5.890E-007	200-CRWT-TRD0002-TRD-FOH	Rear portside track failure	5.9E-007
			200-OPFAILSTOP-HFI-NOD	Operator fails to stop ST on track failure	1.0E+000
	25.00	5.890E-007	200-CRWT-TRD0003-TRD-FOH	Front starboard track failure	5.9E-007
			200-OPFAILSTOP-HFI-NOD	Operator fails to stop ST on track failure	1.0E+000
	25.00	5.890E-007	200-CRWT-TRD0004-TRD-FOH	Rear starboard track failure	5.9E-007
			200-OPFAILSTOP-HFI-NOD	Operator fails to stop ST on track failure	1.0E+000
		2.356E-006	= Total		

NOTE: Freq. = frequency; Prob. = probability; ST = site transporter.

Source: Original

B6.4.3.8 Fault Tree



Source: Original

Figure B6.4-22. Operator causes Site Transporter Tipover

B6.4.4 Site Transporter Spurious Movement (ESD-06)

B6.4.4.1 Description

The fault tree for "Site Transporter Spurious Movement" in this event sequence addresses activities associated with site transporter transfers of aging overpack to or from staging in the Loading Room.

B6.4.4.2 Success Criteria

Spurious movement of the site transporter is prevented by the inherent design constraints of the site transporter. There is only sufficient electrical power to perform one type of operation at a time. For example, it is not possible to command a lift/lower of the aging overpack when the site transporter is moving. Spurious signals can not be generated when primary power is removed from the site transporter (i.e., diesel engine shut down and/or facility electrical power cord disconnected). There are no batteries or capacitors in the site transporter that can store electrical energy.

B6.4.4.3 Design Requirements and Features

Requirements

Site transporter power and the remote control pendant is removed from the site transporter when it has been positioned within the Loading Room.

It shall be required to remove facility power and the control pendant from the site transporter when it has been properly position within the Loading Room. On removal of facility AC power, the site transporter immediately enters the "lock mode" safe state. The "lock mode" safe state is not be reversible without specific operator action.

Features

There are no electrical storage devices in the design of the site transporter. When the facility AC power cable is removed, the site transporter is incapable of movement.

A shield door interlock ensures that facility power has been removed from the site transporter.

B6.4.4.4 Fault Tree Model

The fault tree model for "Site Transporter Spurious Movement" in the Loading Room accounts for failure to remove facility power and the possibility of the site transporter receiving a spurious movement command for the remote control device.

B6.4.4.5 Basic Event Data

Table B6.4-7 lists the basic events used in the "Site Transporter Spurious Movement" fault tree.

Table B6.4-7. Basic Event Probability for Site Transporter Spurious Movement

Basic Events Probability Report					
Project: Yucca-Mountain	Case: Current				
ST Spurious Movement		Units: Per Hour			
Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-CR---IEL001--IEL-FOH	3	3.43E-05	0.00E+00	3.43E-05	1.00E+00
200-CR---IEL002--IEL-FOH	3	3.43E-05	0.00E+00	3.43E-05	1.00E+00
200-CR---IELCCF--IEL-CCF	3	1.60E-04	1.00E+00	1.60E-04	1.00E+00
200-OPNOUNPLUGST-HFI-NOD	1	1.00E-03	1.00E-03	0.00E+00	1.00E+00
200-ST---HC000--HC--SPO	1	1.74E-03	1.74E-03	0.00E+00	1.00E+00
200-ST---SC002--SC--FOH	3	1.28E-04	0.00E+00	1.28E-04	1.00E+00
200-ST---SC021---SC---SPO	3	3.20E-05	0.00E+00	3.20E-05	1.00E+00

NOTE: [a]For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; Fail. = failure; Miss. = mission; Prob. = probability; ST = site transporter.

Source: Original

B6.4.4.5.1 Human Failure Events

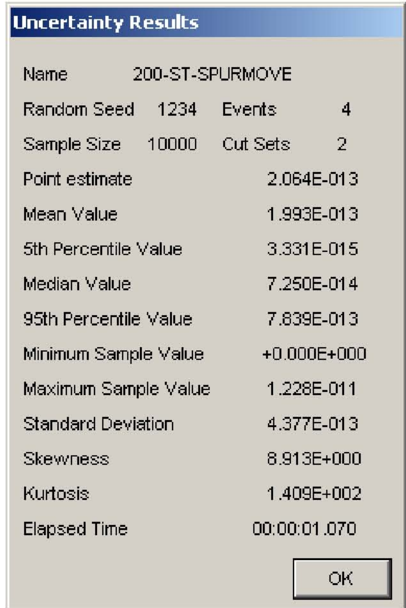
There is one human error associated with this fault tree that addresses an operator failure to unplug the site transporter power cable after it has been parked in the Unloading Room.

B6.4.4.5.2 Common-Cause Failures

There is one common-cause failure associated with two interlock failures on the slide gates. An alpha factor of 0.047 was used to determine the common-cause value using two of two as the failure criteria (Table C3-1, CCCF = 2).

B6.4.4.6 Uncertainty and Cut Set Generation

Figures B6.4-23 and B6.4-24 contain the uncertainty and the cut set generation results for "Site Transporter Spurious Movement" fault tree using a cutoff probability of 1E-15. The fault tree is shown in Figure B6.4-25.



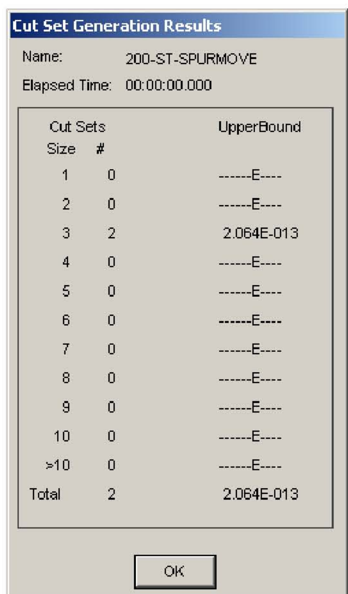
Uncertainty Results

Name	200-ST-SPURMOVE
Random Seed 1234	Events 4
Sample Size 10000	Cut Sets 2
Point estimate	2.064E-013
Mean Value	1.993E-013
5th Percentile Value	3.331E-015
Median Value	7.250E-014
95th Percentile Value	7.839E-013
Minimum Sample Value	+0.000E+000
Maximum Sample Value	1.228E-011
Standard Deviation	4.377E-013
Skewness	8.913E+000
Kurtosis	1.409E+002
Elapsed Time	00:00:01.070

OK

Source: Original

Figure B6.4-23. Uncertainty Results for the Site Transporter Spurious Movement Fault Tree



Cut Set Generation Results

Name: 200-ST-SPURMOVE

Elapsed Time: 00:00:00.000

Cut Sets Size	#	UpperBound
1	0	------E----
2	0	------E----
3	2	2.064E-013
4	0	------E----
5	0	------E----
6	0	------E----
7	0	------E----
8	0	------E----
9	0	------E----
10	0	------E----
>10	0	------E----
Total	2	2.064E-013

OK

Source: Original

Figure B6.4-24. Cut Set Generation Results for the Site Transporter Spurious Movement Fault Tree

B6.4.4.7 Cut Sets

Table B6.4-8 contains the cut sets for the Site Transporter Spurious Movement fault tree.

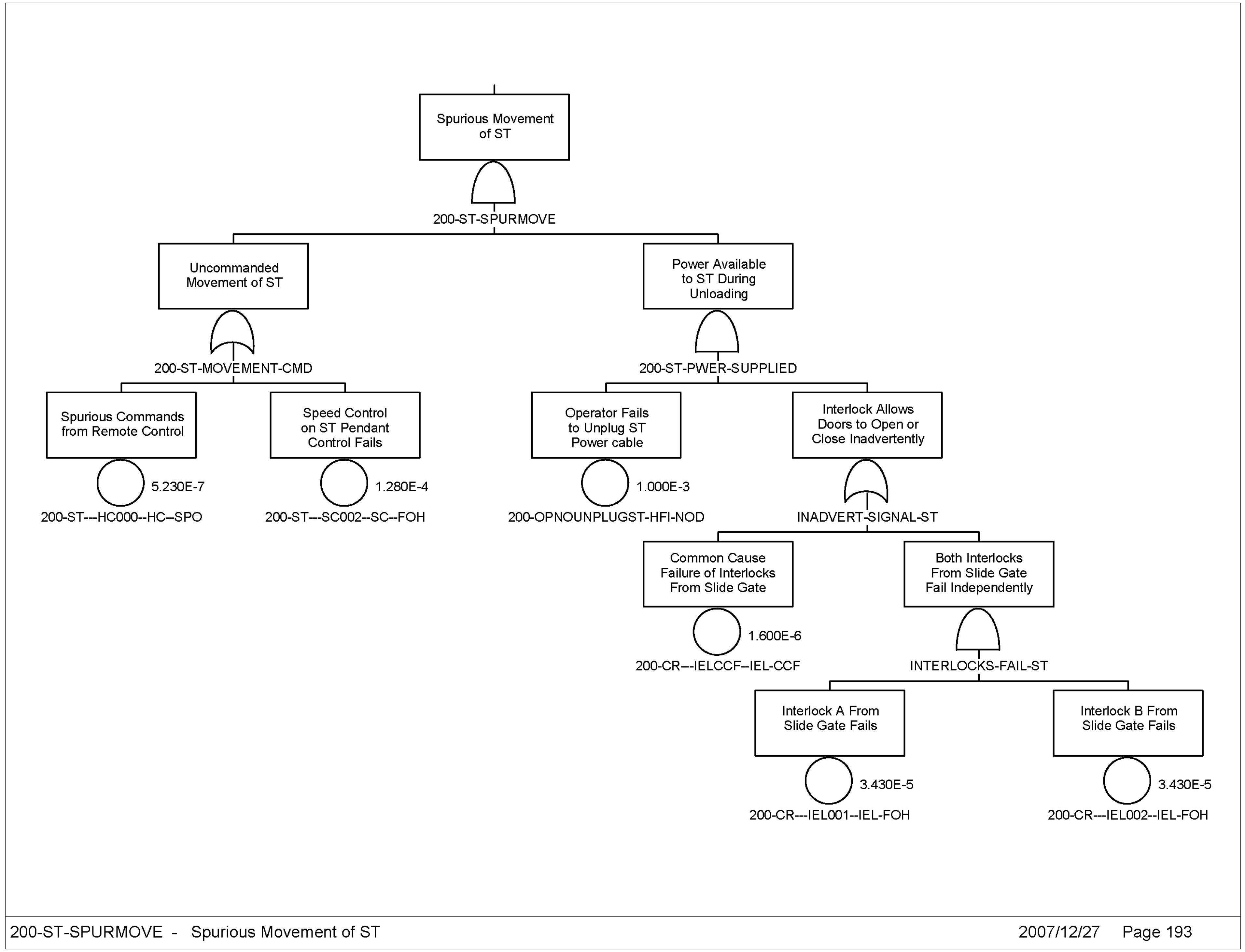
Table B6.4-8. Cut Sets for the Site Transporter Spurious Movement

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
200-ST-SPURMOVE	80.00	1.651E-013	200-CR---IELCCF—IEL-CCF	Common-cause failure of interlocks from slide gate	1.3E-006
			200-OPNOUNPLUGST-HFI-NOD	Operator fails to unplug ST power cable	1.0E-003
			200-ST---SC002--SC--FOH	Speed control on ST pendant control fails	1.3E-004
	20.00	4.128F-014	200-CR---IELCCF—IEL-CCF	Common-cause failure of interlocks from slide gate	1.3E-006
			200-OPNOUNPLUGST-HFI-NOD	Operator fails to unplug ST power cable	1.0E-003
			200-ST---SC021---SC--SPO	On-Board Controller Initiates Spurious Signal	3.2E-005
		2.048E-013	= Total		

NOTE: Freq. frequency; Prob. = probability; ST = site transporter.

Source: Original

B6.4.4.8 Fault Tree



Source: Original

Figure B6.4-25. Spurious Movement of Site Transporter

B7 HEATING VENTILATION AND AIR CONDITIONING FAULT TREE ANALYSIS

B7.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

B7.1.1 BSC (Bechtel SAIC Company) 2007. *Project Design Criteria Document.* 000-3DR-MGR0-00100-000-007. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071016.0005.

B7.1.2 BSC 2007. *Receipt Facility Composite Vent Flow Diagram Tertiary Confinement Non-ITS HVAC Supply Sys & ITS Exhaust.* 200-M50-VCT0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071002.0021.

B7.1.3 BSC 2007. *Receipt Facility ITS Confinement Areas HEPA Exhaust System - Train A Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071204.0017.

B7.1.4 BSC 2007. *Receipt Facility ITS Confinement Areas HEPA Exhaust System - Train B Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00102-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071204.0018.

B7.1.5 BSC 2007. *RF Air Pressure Drop Calculation (ITS),* 200-M8C-VCT0-00600-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070525.0007.

B7.1.6 BSC 2007. *RF Equipment Sizing and Selection Calculation (ITS).* 200-M8C-VCT0-00500-000-00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071220.0033.

Design Constraints

B7.1.7 NRC (Nuclear Regulatory Commission) 2007. *Preclosure Safety Analysis - Dose Performance Objectives and Radiation Protection Program.* HLWRS-ISG-03. Washington, D.C.: Nuclear Regulatory Commission. ACC: MOL.20070918.0096.

B7.2 IMPORTANT TO SAFETY HVAC DESCRIPTION

B7.2.1 Overview

The ITS heating, ventilation, and air-conditioning (HVAC) is a two train system of identical components. One train is always operational and one train is in standby mode. This system is not configured to run both trains at the same time without bypassing control circuitry. This off-normal situation is not addressed in this analysis.

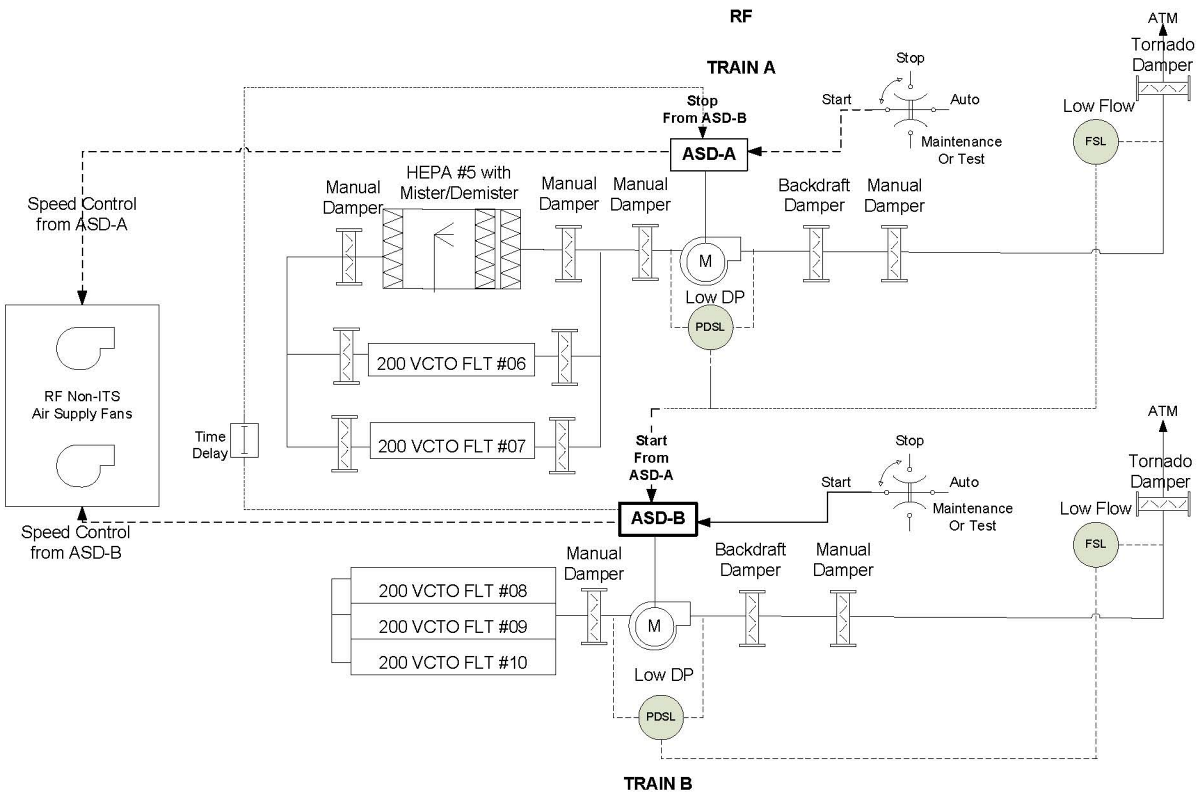
Figure B7.2-1 shows the locations of the various pieces of ITS HVAC equipment described in the following sections. Sizing of the ITS HVAC in the RF (Ref. B7.1.6, Section 6.1) was performed to ensure desired air distribution, ventilation rates, and transport velocities were attainable to maintain the required delta pressure within the tertiary confinement (C2) zones in this facility.

In the RF each HVAC train exhausts air through separate discharge ducts to the atmosphere. Although these trains are interconnected through interior duct work, the trains are independent. A backdraft damper is used on each train to ensure there is no airflow from the atmosphere back through the standby train.

This HVAC system is composed of four subsystems:

- A series of dampers are used to control pressure, flow, and flow direction.
- Three high-efficiency particulate air (HEPA) filters, each consisting of one medium efficiency roughing filter (60-90% efficiency), two high efficiency filters for particulate removal (99.97% efficiency) (Ref. B7.1.1, Section 4.9.2.2.6; and Ref. B7.1.3), and a mister/demister for maintaining proper humidity levels[2].
- One exhaust fan with a rated capacity of 40,500 cfm and an exhaust fan motor rated at 200 hp (Ref. B7.1.6, Sections 6.1.1 and 3.1.5).
- Control circuitry with logic contained in an erasable programmable read-only memory (EPROM) located in the ASD controller used for controlling the speed of the operating fan and on fault detection (Ref. B7.1.6, Section 3.2.3) for off-nominal conditions, shutting down the operating train and transmitting signals to the standby system to start[3].

[2] There is a water deluge system in each HEPA filter which is used in fire scenarios. Refer to the facility fire analysis for information regarding these pieces of equipment.
[3] The ASD also controls non-ITS supply fans that are adjusted to maintain airflow in the facility.



NOTE: The diagram has been simplified with respect to the HEPA filter equipment shown for Trains A and B. The equipment configuration for HEPA Filters identified as 200 VCTO FLT 06, 07, 08, 09 and 10 are identical to the HEPA FLT 05. In addition, Train B has the same manual input/output dampers shown for Train A. ASD = adjustable speed drive; ATM = atmosphere; DP = delta pressure; FLT = filter; FSL = flow sensor low; ITS = important to safety; HEPA = high-efficiency particulate air (filter); M = motor for exhaust fan; PDSL = pressure differential sensor low; RF = Receipt Facility.

Source: Original

Figure B7.2-1. Block Diagram of the RF ITS HVAC System

B7.2.2 Damper Subsystem Description

The ITS HVAC system utilizes manual, backdraft, and tornado dampers to control the delta pressure inside the containment area or to isolate the standby system from the outside atmosphere.

Manual dampers are located on the input and output sides of the HEPA filter. These filters are used to isolate the HEPA filter, if required, during maintenance. There is a manual damper on the input side of the exhaust fan that is used to isolate the entire HEPA filter subsystem for maintenance on the HEPA filters or the exhaust fan. One additional manual damper is located between the backdraft and the tornado damper which can be used to isolate the entire train.

A backdraft damper is located on the exhaust side of the fan. This damper is normally open for the operating train and closed on the standby train. This damper prevents a reverse airflow through the standby system as a result of the negative delta pressure in the containment C2 areas.

A tornado damper is used to control airflow automatically to prevent the transmission of tornado pressure surges from outside the facility.

B7.2.3 HEPA Filters

The three HEPA filter units are identical, consisting of a 3 by 3 array of medium (nine filters) and two banks of high-efficiency HEPAs (18 filters). A bag-in/bag-out procedure is used to replace the HEPA filters. Each filter is sized for a maximum flow of 1,500 cfm (Ref. B7.1.6, Section 3.2.2). The failure analysis includes the HEPA filter bank for plugs and leaks, mister/demister for humidity control, and the medium roughing filter.

The HEPA subsystem also contains the following components that are not modeled in the analysis: Inlet test section, combination test section, the outlet test section, and the deluge system during fire scenarios.

B7.2.4 Direct Drive Exhaust Fan and Motor

The exhaust fan and motors are sized to provide a maximum airflow rate of 40,500 cfm. To meet delta pressure requirements for the RF, the exhaust system must provide an airflow rate of 33,700 cfm (Ref. B7.1.6, Appendix A, Table A-1). At this airflow rate, the exhaust system provides for a total of 15.1 inches of water column required to maintain delta pressure in the facility (Ref. B7.1.6, Section 3.1.4).

The exhaust fan motor is rated at 1,800 rpm (Ref. B7.1.6, Section 3.1.3) but the actual speed is controlled by the ASD. The ASD adjusts the speed to maintain delta pressure when facility doors are opened, HEPA filters loose efficiency, or for changing outside wind speeds.

B7.2.5 Control Circuitry

The ITS HVAC system is controlled by EPROMs[4]. This control logic is contained in the ASD which is used to monitor the delta pressure across the exhaust fan and airflow rate exhausting to the atmosphere. Changes in air pressure cause the ASD to change the speed of the exhaust fan motor. The ASD also controls the speed of the non-ITS supply fans ((Ref. B7.1.3), (Ref. B7.1.4), and (Ref. B7.1.2))[5].

At any time the ASD can not return the delta pressure to normal operating conditions, the ASD shuts down the operating train and sends a signal to the standby train to start up. When the standby ASD receives this signal, it starts the standby system and sends a signal to the operational train to shut down. There is an interlock to preclude the operation of both trains at

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[4] Although there are programmable logic controls in various locations throughout the RF, none of these are ITS.

[5] The supply fans are used to stabilize the airflow within the RF. These fans are non-ITS so they are not accounted for in this analysis except in a degraded mode of operation.

the same time. Time delays are built-in to the ASD processing system to preclude spurious signals received from the sensors triggering a false transfer.

B7.2.6 ITS HVAC Normal Operations

In normal operations, Train A is operational and Train B is in standby. EPROMs within the ASD monitor the pressure differential across the exhaust fan and the flow rate of the exhaust to the atmosphere. There are no PLCs used in the ITS HVAC control system and all interlocks are hardwired for ITS operations. The delta pressure sensor and low flow sensor are ITS equipment with defined set points for the RF. ASD-A response to the various deviations from these set points are shown in Table B7.2-1.

Table B7.2-1. ASD Response to Variations in Delta Pressure

DP Pressure Sensor	Low Flow Sensor	ASD Response
High DP (Plugged HEPA)	Low Flow	Switch trains
High DP	High Flow	Decrease RPM of exhaust fan
High DP	Nominal Flow	Increase RPM of supply fans
Low DP (HEPA Leak)	High Flow	Switch trains
Low DP	Nominal Flow	Decrease RPM of supply fans
Low DP	Low Flow	Increase RPM of exhaust fan

NOTE: ASD = adjustable speed drive; DP = delta pressure; HEPA = high-efficiency particulate air (filter); RPM = revolutions per minute.

Source: Original

If the responses can not return the delta pressure and flow rates to nominal states, the ASD issues the command to the ASD-B to start up Train B. ASD-B commands the startup of Train B exhaust fan and send a signal back to ASD-A to shut down. An interlock prevents both trains from operating at the same time.

Under normal operations with non-ITS supply fans working, all three HEPA filter assemblies in the train must be working to achieve the exhaust flow rate of 33,700 cfm (Ref. B7.1.6, Section 6.1.1 (item 1)). Each HEPA filter array can filter 13,500 cfm at maximum efficiency (Ref. B7.1.6, Section 6.1.1 (item 2)). The design has some reserve capacity but not enough to maintain the required delta pressure if one of the HEPA filters fail. Under normal operations, the only redundancy in the design is the second train.

Misters/demisters are included as part of the HEPA filters to control the temperature and relative humidity of the air passing through the filters. The water deluge system is not considered to be normal operations and is handled in the fire suppression analyses.

During receipt of a transportation cask containing DPCs or TAD canisters, or during the export of an aging overpack, delta pressure is lost for a period of time not to exceed 7 minutes per event.[6] This occurs as a direct consequence of opening vestibule doors to allow for entry or exit of the site transporter, the site prime mover, or the horizontal cask transfer trailer.

B7.2.7 ITS HVAC Degraded Operations

The ITS HVAC system maintains proper delta pressure throughout Class C2 designated containment areas. Exhausted air from the RF is made-up from opening/closing doors to the outside, leaks in the structure, and from one of two supply fans which are controlled by the ASD on the operating train. One of these fans, in conjunction with other air makeup sources, can provide sufficient airflow through the C2 containment areas for the HVAC to maintain delta pressure. These supply fans are not ITS and therefore, are not connected to the ITS power system for the RF. Should there be a loss of non-ITS site power or for a mechanical reason, these supply fans shut down; the HVAC system can be operated in a degraded mode. Since there is less air to exhaust, Train A no longer has to exhaust 33,700 cfm. It then becomes possible to maintain delta pressure with two of three HEPA filters. This special case has been added to the fault trees for the failure to maintain delta pressure in the RF. In this case, there is redundancy within the train and a common-cause failure mode has been added to the fault tree.

B7.2.8 ITS HVAC Testing and Maintenance

Under normal operations Train A continues to operate until a failure is detected or the train is shut down for maintenance. Normal maintenance renders Train B unavailable for service 40 hours per year[7]. During maintenance, the Train B start/stop/auto/maintenance switch is placed in the maintenance position. When maintenance is completed, the standby system (Train B) is started and operational system (Train A) is shut down and considered to be the standby train (Train B). Maintenance may be scheduled consecutively for this train or at some future date. Under normal operations, maintenance does not result in the loss of/or the inability of the operating train to perform its intended function.

Testing is considered part of routine maintenance. When the maintenance has been completed, maintenance personnel turn the standby train on and check for normal operations including delta pressure, flow rate, and that all failure indicators are reset/off. Maintenance personnel also observe the forced shutdown of the operating system as the standby train is turned on.

Flow rates are monitored as part of testing to ensure that the manual dampers for the active train are in the proper position to achieve a balanced airflow across the three HEPA filters. Once the dampers have been adjusted, they do not require further adjustment unless a damper or combination of dampers must be closed to isolate a component in the train or the entire train.

[6] This is a conservative estimate of the time it will take for the HVAC system to return the vestibule to a negative pressure.

[7] The majority of operational-level maintenance can be performed on the operational train and, therefore, does not affect the overall availability of the standby train.

B7.3 DEPENDENCIES AND INTERACTIONS

Dependencies are broken down into five categories with respect to their interactions with systems, structures, and components. The five areas considered are addressed in Table B7.3-1 with the following dependencies:

1. Functional dependence.
2. Environmental dependence.
3. Spatial dependence.
4. Human dependence.
5. Failures based on external events.

Table B7.3-1. Dependencies and Interactions Analysis

Systems, Structures, Components	Dependencies and Interactions				
	Functional	Environmental	Spatial	Human	External Events
ASD	Flow and pressure sensors	—	—	—	—
	Speed control for fan/motor	—	—	—	—
DP Exhaust Fans	—	Wind speed	—	—	—
Stop/Start/Auto Switch Position	—	—	—	Wrong position	—
Dampers	—	—	—	Wrong position	—
ITS Power	HVAC shuts down	—	—	—	—
Non-ITS Power	—	—	—	—	Supply fans stop
HEPA	—	—	—	Failure to notice leak	—
HVAC Maintenance	—	—	—	Trains can not switch	—
Vestibule Doors	Open only one door at a time	—	—	—	—

NOTE: ASD = adjustable speed drive; DP = delta pressure; HEPA = high-efficiency particulate air (filter); HVAC = heating, ventilation, and air-conditioning; ITS = important to safety.

Source: Original

B7.4 HVAC RELATED FAILURE SCENARIO

B7.4.1 Failure to Maintain Delta Pressure

B7.4.1.1 Description

There is a single failure scenario used in this analysis. The components of the HVAC system used inside buildings to maintain C2 in areas that are normally clean and where airborne contamination is not expected during normal facility operations. The ITS HVAC equipment maintains a positive airflow from outer confinement areas through the HEPA filters to the atmosphere (Ref. B7.1.2).

Within the RF the areas designated as C2 are the following: Cask Preparation Room, Cask Unloading Room, Loading Room, and the Canister Transfer Room on the second floor.

B7.4.1.2 Success Criteria

Success criteria for maintaining delta pressure in the RF requires that one of two HVAC trains is operational. The sizing of the exhaust motor and fan assembly maintain the delta pressure in sustained winds of 40 mph with less than three second gusts up to 90 mph. In addition, delta pressure is lost for a period of time, not to exceed seven minutes, in the RF if, and only if, one of the vestibule doors is open. These doors are interlocked to ensure only one door is open at a time during normal operations.

Switching between the active and standby trains is controlled by ASD-A (active train) which continually monitors the pressure across the exhaust fan and the air flow rate exhausting from the RF. These sensors are in a one-of-two configuration. This means that the ASD initiates the transfer of operations from the “active train” to the “standby train” when either one of these sensors can not be returned to a normal operating range by the ASD, by controlling, in some combination, the speed of the supply and exhaust fans.

ASD-A must be able to recognize an uncorrectable airflow rate in Train A and transmit a signal to ASD-B to start. Having received the start command, ASD-B must send a signal back to ASD-A commanding a stop.

The delta pressure is maintained during/after the switchover by having the “start/stop/maintenance or test/auto” switch in the auto position, the Train B exhaust fan and motor started, and the airflow across the HEPA filters adjusted by ASD-B.

With the exception of the tornado and backdraft dampers, all control dampers in the ITS HVAC system are manual dampers. These dampers are typically set once for air balancing. These dampers may be adjusted or closed when maintenance is required on the standby train. Should the damper setting be changed, it would require the maintenance personnel to return the damper to its proper position to ensure balanced airflow.

B7.4.1.3 Design Requirements and Features

Requirements

There is only one HVAC train in operation at any time. The second train is in standby (exception-when Train B is off-line for maintenance).

Alarms are on a panel in the continuously manned central control station and responded to by operators. Alarm conditions are: ASD trouble, fan failure, motor running/stop, and flow rate problem. Operators are not required to respond to the alarm (ITS-HVAC trains are switched automatically); however, operators are expected to notify maintenance that a switch has occurred and maintenance is required to determine and correct the cause of the failure.

Design Features

ITS HVAC system is in normal operations with three HEPA filter units. Each HEPA filter unit consists of one 3 × 3 medium filter array and two 3 × 3 HEPA high-efficiency filter arrays.

The only difference between the ITS HVAC in the RF, CRCF, and WHF facilities are the number of non-ITS fans operating in the facility.

TESTING AND MAINTENANCE

Requirements

HVAC maintenance personnel are notified when an alarm condition exists. Repairs are performed as soon as possible to return train to a standby operational system.

While an HVAC train is undergoing maintenance, the train is not available for service.

Testing that requires the exhaust fan to run is performed on the active HVAC system.

Features

Normal maintenance is performed in accordance with manufacturer's recommendations; however, the majority of preventative maintenance does not require shutting down the active system.

B7.4.1.4 Fault Tree Model

The top event in this fault tree is "Delta Pressure not Maintained in RF Facility." This is defined as the inability of the ITS HVAC system to maintain proper delta pressure within the facility. The ITS HVAC system is a two train system. The configuration of the ITS HVAC systems in these facilities is essentially identical. The only variations are the number of non-ITS supply fans used to stabilize the airflow within these buildings.

- The fault tree model for the loss of delta pressure in the facility includes those components that have been designated as ITS. There is only one exception and that is the inclusion of two non-ITS supply fans. The fans were added to stabilize air pressure differentials in the facility during normal operations and provide a capability for operating in a degraded mode.

- There are two interlocks in the ITS HVAC system. The first addresses the potential for opening two or more of the entrance/exit vestibule doors. (Note: There is no physical connection between this door interlock and the HVAC system.) The second interlock prevents two HVAC trains from operating at the same time.

- The mission time for the ITS HVAC system is currently set to 720 hours (Ref. B7.1.7). To take into account the differences in failure rates for active and standby systems, all basic events in the standby train are set to half that of the active system. For ease of implementation in SAPHIRE, the rate data is maintained constant and the mission time is set to 1/2 the mission time or 360 hours.

B7.4.1.5 Basic Event Data

Table B7.4-1 contains a list of basic events used in the loss of delta pressure in the RF. The model contains undeveloped transfers to ITS power systems. These failures are addressed in Section B8. Reliability data for basic events is detailed in Attachment C with the following exceptions:

- Three are associated with human error. HFE detailed analysis is in Section 6.4 and Attachment E:
  - Opening two or more vestibule doors (200-VCTO-DR00001-HFI-NOD).
  - Failure to properly restore system after maintenance (200-VCTO-HEPALK-HFI-NOD).
  - Failure to notice HEPA filter leak (200-VCTO-HFIA000-HFI-NOM).
- Unavailability of the standby train due to scheduled maintenance which is based on a conservative estimate (40 hours per year).
- Loss of delta pressure as a direct result of opening a vestibule door and the time it takes for the HVAC exhaust fan to re-establish delta pressure (7 minutes).
- Common-cause failure of the HEPA filters in the degraded mode.

Table B7.4-1. Basic Event Probability for the Failure to Maintain Delta Pressure in the RF Fault Tree

Basic Events Probability Report					
Project: Yucca-Mountain		Case: Current			
Loss of Delta P in RF		Units: Per Hour			
Name	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-EXCESSIVE-WIND-SPEED	1	4.700E-003	4.700E-003	1.000E-005	0.000E+000
200-VCOO-NITS-PWR-FAILS	1	2.990E-003	2.990E-003	0.000E+000	0.000E+000
200-VCOO-SFAN001-FAN-FTR	3	5.059E-002	0.000E+000	7.210E-005	7.200E+002
200-VCOO-SFAN002-FAN-FTR	3	5.059E-002	0.000E+000	7.210E-005	7.200E+002
200-VCTO--B---FAN-FTS	1	2.020E-003	2.020E-003	0.000E+000	3.600E+002
200-VCTO-DMP000A-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMP000B-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMP001A-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMP001B-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMPA05I-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMPA05O-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMPA06I-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002

Table B7.4-1. Basic Event Probability for the Failure to Maintain Delta Pressure in the RF Fault Tree (Continued)

Basic Events Probability Report					
Project: Yucca-Mountain		**Case: Current**			
Loss of Delta P in RF		**Units: Per Hour**			
Name	**Calc. Type[a]**	**Calc. Prob.**	**Fail. Prob.**	**Lambda**	**Miss. Time[a]**
200-VCTO-DMPA06O-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMPA07I-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMPA07O-DMP-FRO	3	6.033E-005	0.000E+000	8.380E-008	7.200E+002
200-VCTO-DMPB08I-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMPB08O-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMPB09I-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMPB09O-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMPB10I-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DMPB10O-DMP-FRO	3	3.017E-005	0.000E+000	8.380E-008	3.600E+002
200-VCTO-DR00001-HFI-NOD	1	1.000E-002	1.000E-002	0.000E+000	0.000E+000
200-VCTO-DRS0000-DRS-OPN	1	1.600E-004	1.600E-004	0.000E+000	0.000E+000
200-VCTO-DTC0A-DTC-RUP	3	2.675E-003	0.000E+000	3.720E-006	7.200E+002
200-VCTO-DTC0B-DTC-RUP	3	1.338E-003	0.000E+000	3.720E-006	3.600E+002
200-VCTO-FAN00A-FAN-FTR	3	5.059E-002	0.000E+000	7.210E-005	7.200E+002
200-VCTO-FAN00B-FAN-FTR	3	2.562E-002	0.000E+000	7.210E-005	3.600E+002
200-VCTO-FAN00B-FAN-FTS	1	2.020E-003	2.020E-003	0.000E+000	0.000E+000
200-VCTO-FANA-PRM-FOH	3	5.380E-007	0.000E+000	5.380E-007	0.000E+000
200-VCTO-FANB-PRM-FOH	3	1.937E-004	0.000E+000	5.380E-007	3.600E+002
200-VCTO-FSLAB0-SRF-FOH	3	7.701E-004	0.000E+000	1.070E-006	7.200E+002
200-VCTO-HEPA05-DMS-FOH	3	6.545E-003	0.000E+000	9.120E-006	7.200E+002
200-VCTO-HEPA06-DMS-FOH	3	6.545E-003	0.000E+000	9.120E-006	7.200E+002
200-VCTO-HEPA07-DMS-FOH	3	6.545E-003	0.000E+000	9.120E-006	7.200E+002
200-VCTO-HEPA0A5-HEP-LEK	3	2.158E-003	0.000E+000	3.000E-006	7.200E+002
200-VCTO-HEPAA05-HEP-LEK	3	3.000E-006	0.000E+000	3.000E-006	0.000E+000
200-VCTO-HEPAA05-HEP-PLG	3	3.070E-003	0.000E+000	4.270E-006	7.200E+002
200-VCTO-HEPAA06-DMS-FOH	3	6.545E-003	0.000E+000	9.120E-006	7.200E+002
200-VCTO-HEPAA06-HEP-LEK	3	2.158E-003	0.000E+000	3.000E-006	7.200E+002
200-VCTO-HEPAA06-HEP-PLG	3	3.070E-003	0.000E+000	4.270E-006	7.200E+002
200-VCTO-HEPAA07-HEP-LEK	3	2.158E-003	0.000E+000	3.000E-006	7.200E+002
200-VCTO-HEPAA07-HEP-PLG	3	3.070E-003	0.000E+000	4.270E-006	7.200E+002
200-VCTO-HEPAB08-DMS-FOH	3	3.278E-003	0.000E+000	9.120E-006	3.600E+002
200-VCTO-HEPAB08-HEP-LEK	3	1.079E-003	0.000E+000	3.000E-006	3.600E+002
200-VCTO-HEPAB08-HEP-PLG	3	1.536E-003	0.000E+000	4.270E-006	3.600E+002
200-VCTO-HEPAB09-DMS-FOH	3	3.278E-003	0.000E+000	9.120E-006	3.600E+002
200-VCTO-HEPAB09-HEP-LEK	3	1.079E-003	0.000E+000	3.000E-006	3.600E+002
200-VCTO-HEPAB09-HEP-PLG	3	1.536E-003	0.000E+000	4.270E-006	3.600E+002
200-VCTO-HEPAB10-DMS-FOH	3	3.278E-003	0.000E+000	9.120E-006	3.600E+002

Table B7.4-1. Basic Event Probability for the Failure to Maintain Delta Pressure in the RF Fault Tree (Continued)

Basic Events Probability Report					
Project: Yucca-Mountain		**Case: Current**			
Loss of Delta P in RF		**Units: Per Hour**			
Name	**Calc. Type[a]**	**Calc. Prob.**	**Fail. Prob.**	**Lambda**	**Miss. Time[a]**
200-VCTO-HEPAB10-HEP-LEK	3	1.079E-003	0.000E+000	3.000E-006	3.600E+002
200-VCTO-HEPAB10-HEP-PLG	3	1.536E-003	0.000E+000	4.270E-006	3.600E+002
200-VCTO-HEPAB-CCF	3	3.852E-005	0.000E+000	1.070E-007	3.600E+002
200-VCTO-HEPA-CCF	3	7.704E-005	0.000E+000	1.070E-007	7.200E+002
200-VCTO-HEPALK-HFI-NOD	1	1.000E+000	1.000E+000	0.000E+000	0.000E+000
200-VCTO-HFIA000-HFI-NOM	1	1.000E-001	1.000E-001	0.000E+000	0.000E+000
200-VCTO-IEL0001-IEL-FOD	1	2.750E-005	2.750E-005	0.000E+000	0.000E+000
200-VCTO-PDSLA0B-SRP-FOD	1	3.990E-003	3.990E-003	0.000E+000	7.200E+002
200-VCTO-TDMP00A-DTM-FOH	3	1.614E-002	0.000E+000	2.260E-005	7.200E+002
200-VCTO-TDMP00B-DTM-FOD	1	8.710E-004	8.710E-004	0.000E+000	0.000E+000
200-VCTO-TDMP00B-DTM-FOH	3	8.103E-003	0.000E+000	2.260E-005	3.600E+002
200-VCTO-TRAINB-MAINT	1	2.740E-003	2.740E-003	0.000E+000	0.000E+000
200-VCTO-UDMP000-UDM-FOH	3	8.103E-003	0.000E+000	2.260E-005	3.600E+002

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
Calc. = calculation; DP = delta pressure; Fail. = failure; Miss. = mission; P = pressure; Prob. = probability; RF = Receipt Facility.

Source: Original

B7.4.1.5.1 Human Failure Events

There are three basic HFE associated with human error listed in Table B7.4-2. They are for inadvertently opening two or more vestibule doors at the same time, failure to notice that there is a HEPA leak and leaving the start/stop/auto/maintenance switch on the standby train in the wrong position.

Table B7.4-2. Human Failure Events

Basic Event Name	Basic Event Description
200-VCTO-DR00001-HFI-NOD	Operators open 2 or more vestibule doors in RF
200-VCTO-HEPALK-HFI-NOD	Operator fails to notice HEPA filter leak in train A (or train B)
200-VCTO-HFIA000-HFI-NOM	Human error exhaust fan switch wrong position

NOTE: HEPA = high-efficiency particulate air; RF = Receipt Facility.

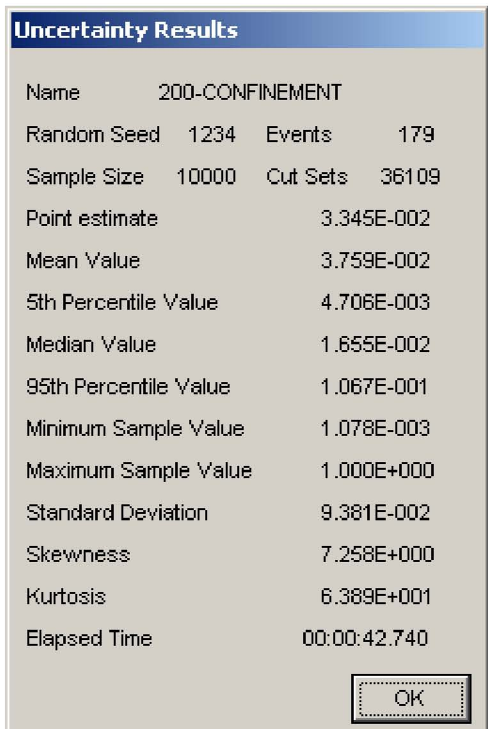
Source: Original

B7.4.1.5.2 Common-Cause Failures

There are two CCF identified in the HVAC model associated with the potential of a HEPA filter failure in the degraded mode where there is a two of three success situation. A 0.025 alpha factor, from Attachment C, Table C3-1, multiplied by the failure rate of a plugged HEPA filter is used to determine the failure rate of the CCF event.

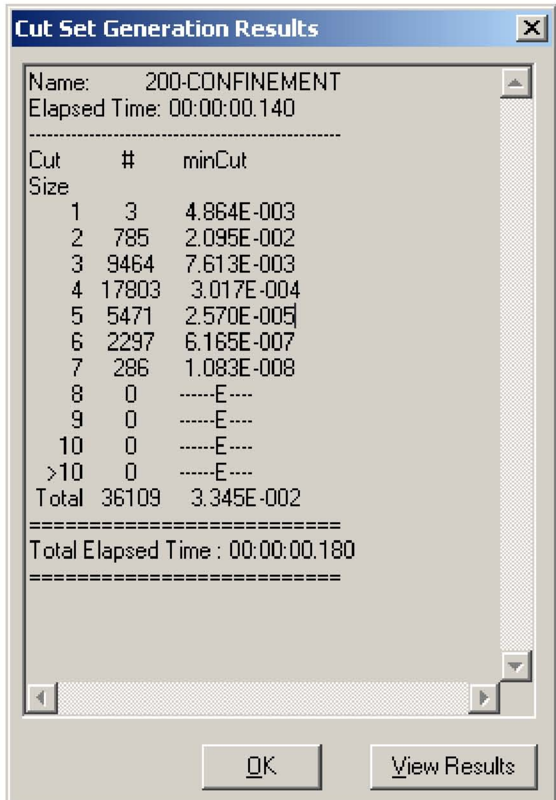
B7.4.1.6 Uncertainty and Cut Set Generation

Figure B7.4-1 contains the uncertainty results obtaining from running the fault trees for "Failure to Maintain Delta Pressure." Figure B7.4-2 provides the cut set generation results for the "Failure to Maintain Delta Pressure" fault tree. These results are for the HVAC system coupled with loss of electrical power, which is discussed separately in Section B8.



Source: Original

Figure B7.4-1. Uncertainty Results of the Failure to Maintain Delta Pressure Fault Tree



Source: Original

Figure B7.4-2. Cut Set Generation Results for the Failure to Maintain Delta Pressure Fault Tree

B7.4.1.7 Cut Sets

Table B7.4-3 contains the top 35 cut sets for the "Failure to Maintain Delta Pressure" in the RF fault tree.

Table B7.4-3. Dominant Cut Sets for the Failure to Maintain Delta Pressure in the RF Fault Tree

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
200-CONFINEMENT	15.12	5.059E-003	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	14.05	4.700E-003	200-EXCESSIVE-WIND-SPEED	Sustained Wind Exceeds 40 MPH & Gust to 90 MPH	4.7E-003
	5.30	1.772E-003	26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.7E-001
			26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.7E-001
			LOSP	Loss of offsite power	3.0E-003
	4.83	1.614E-003	200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001

Table B7.4-3. Dominant Cut Sets for the Failure to Maintain Delta Pressure in the RF Fault Tree (Continued)

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
			200-VCTO-TDMP00A-DTM-FOH	Damper (Tornado) Failure	1.6E-002
	3.88	1.296E-003	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-FAN00B-FAN-FTR	Exhaust Fan in Train B Fails	2.6E-002
	1.96	6.545E-004	200-VCTO-HEPA05-DMS-FOH	Moisture Separator/Demiste r HEPA 05 Fails	6.5E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	1.96	6.545E-004	200-VCTO-HEPA06-DMS-FOH	Moisture Separator/Demiste r HEPA 06 Fails	6.5E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	1.96	6.545E-004	200-VCTO-HEPA07-DMS-FOH	Moisture Separator/Demiste r HEPA 07 Fails	6.5E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	1.61	5.378E-004	200-#EEE-MCC0001-MCC-FOH	RF ITS MCC 00001 Fails	5.4E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	1.24	4.136E-004	200-VCTO-FAN00B-FAN-FTR	Exhaust Fan in Train B Fails	2.6E-002
			200-VCTO-TDMP00A-DTM-FOH	Damper (Tornado) Failure	1.6E-002
	1.23	4.099E-004	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-TDMP00B-DTM-FOH	Tornado damper Train B Fails	8.1E-003
	1.23	4.099E-004	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-UDMP000-UDM-FOH	Backdraft Damper for Train B exhaust Fails	8.1E-003
	1.14	3.816E-004	200-#EEE-LDCNTRA-C52-SPO	Load Center A Feed Circuit Breaker Spurious Operation	3.8E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	1.14	3.816E-004	200-#EEE-MCC0001-C52-SPO	RF ITS MCC 0001 Feed Breaker Spurious Operation	3.8E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.92	3.070E-004	200-VCTO-HEPAA05-HEP-PLG	HEPA #A05 Train A Plugged	3.1E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.92	3.070E-004	200-VCTO-HEPAA06-HEP-PLG	HEPA #A10 Train A Plugged	3.1E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001

Table B7.4-3. Dominant Cut Sets for the Failure to Maintain Delta Pressure in the RF Fault Tree (Continued)

Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
	0.92	3.070E-004	200-VCTO-HEPAA07-HEP-PLG	HEPA #A07 Train A Plugged	3.1E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.88	2.938E-004	200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
			26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.7E-001
			26D-#EEY-OB-SWGA-C52-SPO	13.8kV ITS SWGR A feed Breaker Spurious Operation	3.8E-003
	0.88	2.938E-004	200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
			26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.7E-001
			27A-#EEE-BUS2DGA-C52-SPO	13.8kV Open Bus 2 ITS Load Breaker Spurious Operation	3.8E-003
	0.81	2.721E-004	200-#EEE-MCC0002-MCC-FOH	RF ITS MCC00002 Failure	5.4E-003
			200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
	0.80	2.675E-004	200-VCTO-DTC0A-DTC-RUP	Duct Fails between HEPA and Exhaust Fan (10 feet)	2.7E-003
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.79	2.646E-004	200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
			26D-#EEESWGRDGA-AHU-FTR	13.8kV ITS Switchgear room Air Handling Unit Fails	2.6E-003
	0.77	2.559E-004	200-VCT0-EXH-009-FAN-FTR	RF ITS Elec Exhaust Fan 00005 Fails to Run	5.1E-002
			200-VCT0-EXH-010-FAN-FTR	RF ITS Elec Exh. Fan 0010 Fails to Run	5.1E-002
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.69	2.302E-004	200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
			26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.7E-001
			LOSP	Loss of offsite power	3.0E-003
	0.65	2.158E-004	200-VCTO-HEPAA06-HEP-LEK	HEPA #06 Train A Leaks	2.2E-003
			200-VCTO-HEPALK-HFI-NOD	Operator Fails to Notice HEPA Filter Leak in Train B	1.0E+000
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.65	2.158E-004	200-VCTO-HEPAA07-HEP-LEK	HEPA #07 Train A Leaks	2.2E-003
			200-VCTO-HEPALK-HFI-NOD	Operator Fails to Notice HEPA Filter Leak in Train B	1.0E+000

Table B7.4-3. Dominant Cut Sets for the Failure to Maintain Delta Pressure in the RF Fault Tree (Continued)

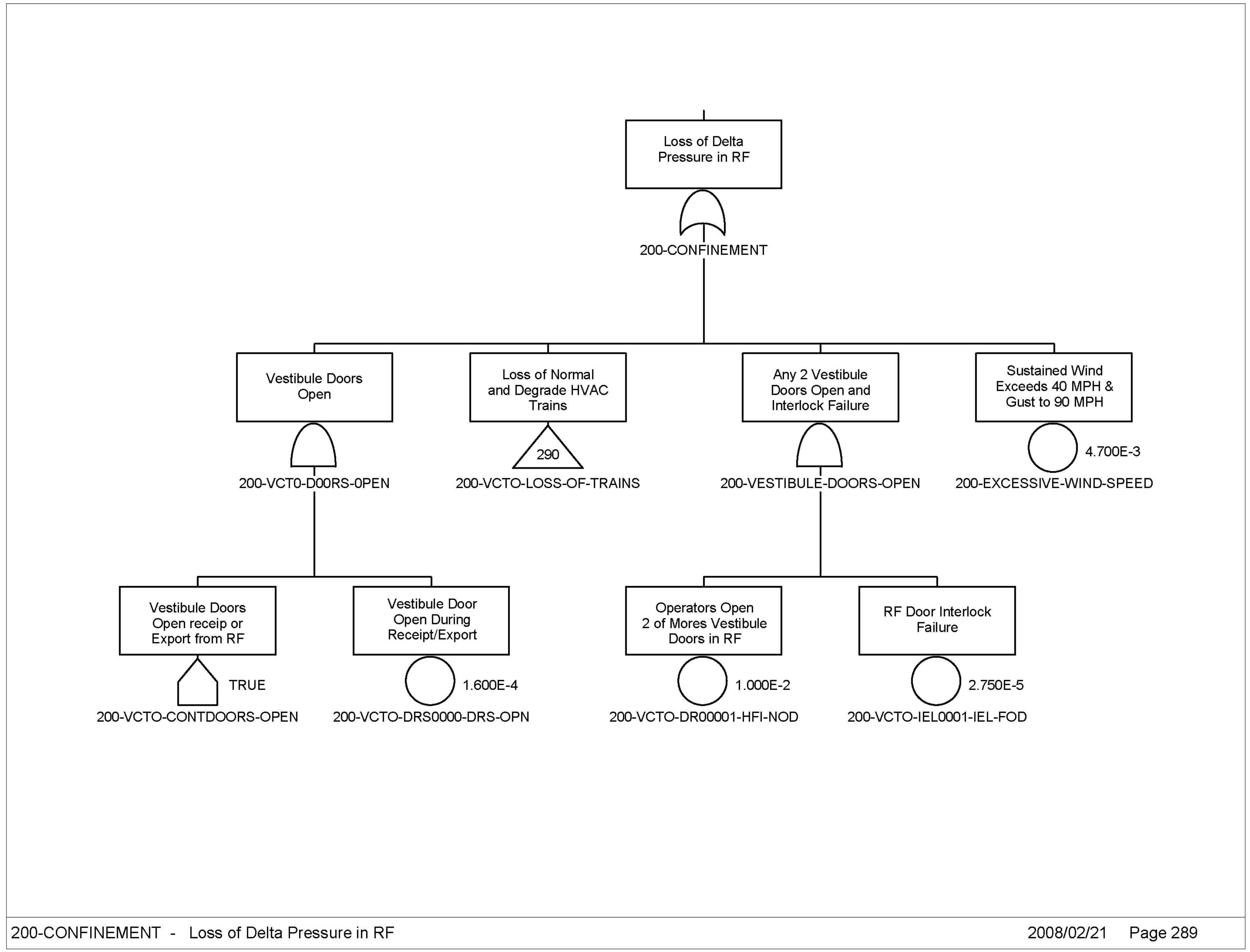
Fault Tree	Cut Set %	Prob./Freq.	Basic Event	Description	Probability
			200-VCTO-HFIA000-HFI-NOM	Human Error Exhaust Fan Switch Wrong Position	1.0E-001
	0.58	1.930E-004	200-#EEE-LDCNTRB-C52-SPO	RF Load Center Circuit Breaker (AC) Spur Op	3.8E-003
			200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
	0.58	1.930E-004	200-#EEE-MCC0002-C52-SPO	RF MCC-00002 Feed Breaker Spurious Operation	3.8E-003
			200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
	0.50	1.677E-004	200-VCTO-FAN00B-FAN-FTR	Exhaust Fan in Train B Fails	2.6E-002
			200-VCTO-HEPA05-DMS-FOH	Moisture Separator/Demister HEPA 05 Fails	6.5E-003
	0.50	1.677E-004	200-VCTO-FAN00B-FAN-FTR	Exhaust Fan in Train B Fails	2.6E-002
			200-VCTO-HEPA06-DMS-FOH	Moisture Separator/Demister HEPA 06 Fails	6.5E-003
	0.50	1.677E-004	200-VCTO-FAN00B-FAN-FTR	Exhaust Fan in Train B Fails	2.6E-002
			200-VCTO-HEPA07-DMS-FOH	Moisture Separator/Demister HEPA 07 Fails	6.5E-003
	0.50	1.658E-004	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-HEPAB08-DMS-FOH	Moisture Separator/Demister HEPA 08 Fails	3.3E-003
	0.50	1.658E-004	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-HEPAB09-DMS-FOH	Moisture Separator/Demister HEPA 09 Fails	3.3E-003
	0.50	1.658E-004	200-VCTO-FAN00A-FAN-FTR	Exhaust Fan in Train A Fails	5.1E-002
			200-VCTO-HEPAB10-DMS-FOH	Moisture Separator/Demister HEPA 10 Fails	3.3E-003
	0.48	1.600E-004	200-VCTO-DRS0000-DRS-OPN	Vestibule Door Open During Receipt/Export	1.6E-004
		3.345E-002	= Total		

NOTE: Elec = electrical; Exh = exhaust; Freq. = frequency; HEPA = high-efficiency particulate air (filter); HVAC = heating, ventilation, and air-conditioning; Prob. Probability.

Source: Original

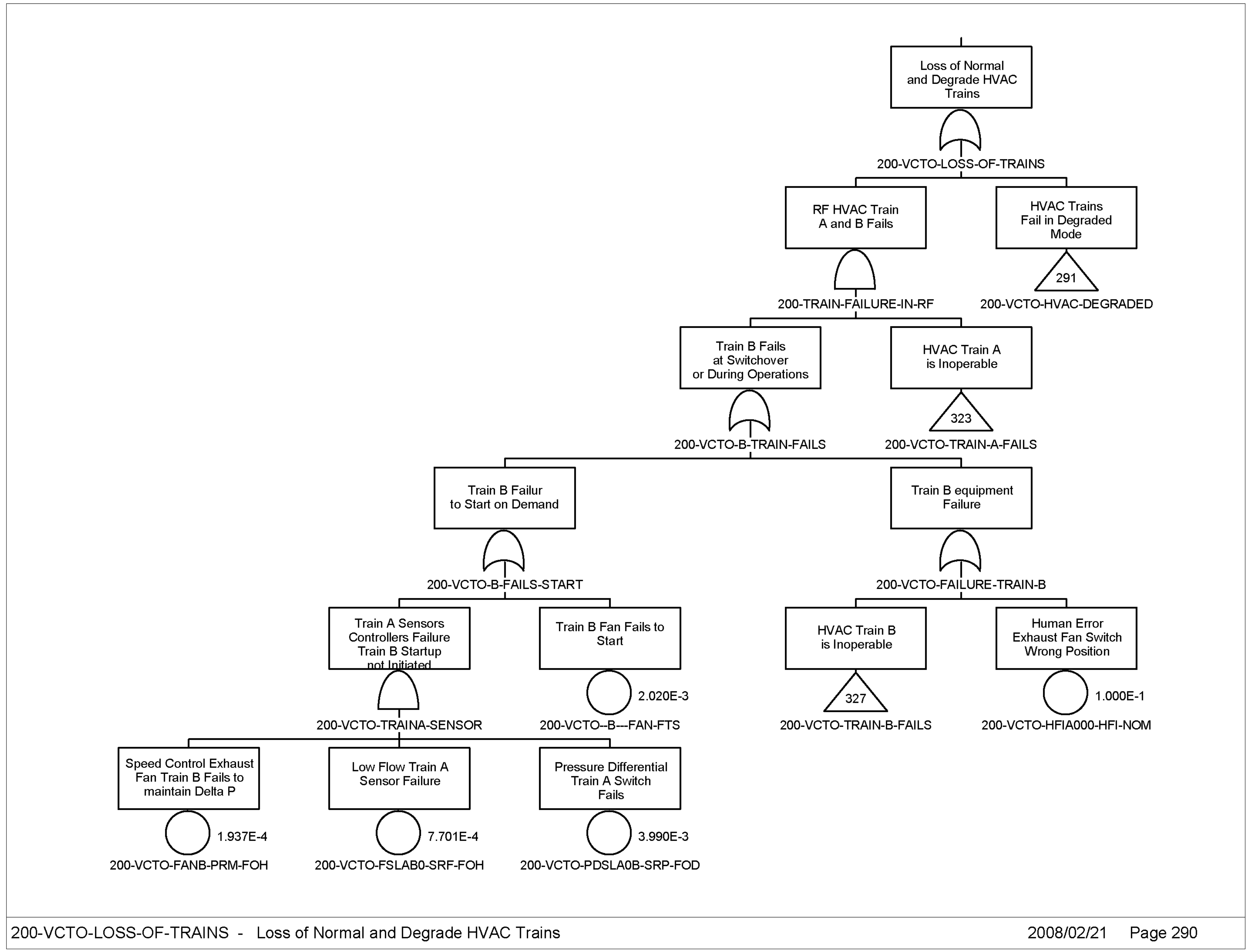
B7.4.1.8 HVAC Fault Trees

For purposes of this report, the transfers to the ITS electrical system for the HVAC equipment is ignored. For specifics on the electrical system, refer to the “AC Power System Fault Tree Analysis” in Section B8. The HVAC fault tree developed for the “Loss of Delta Pressure in RF” is shown in Figures B7.4-3 through B7.4-23.



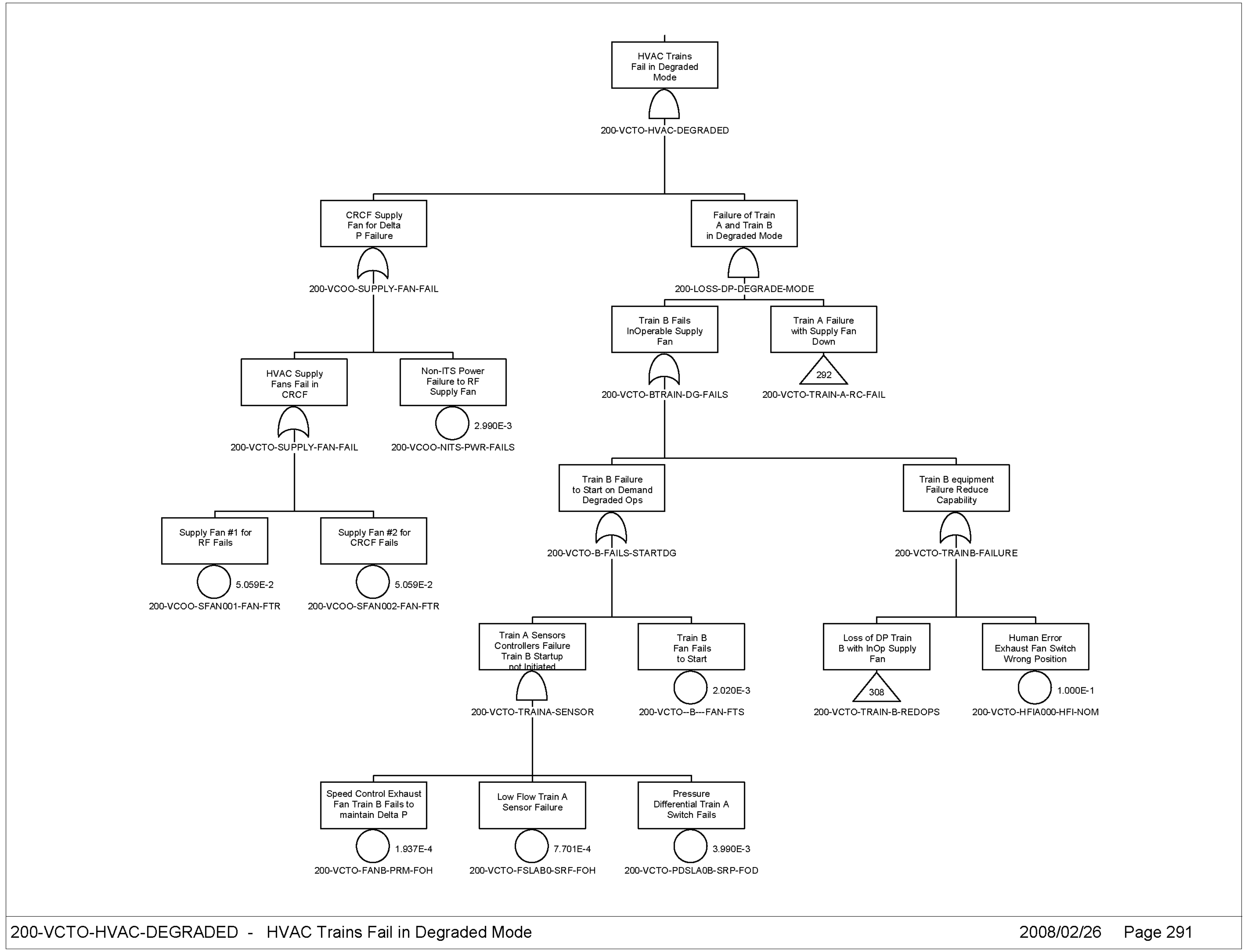
Source: Original

Figure B7.4-3. Delta Pressure not Maintained in RF



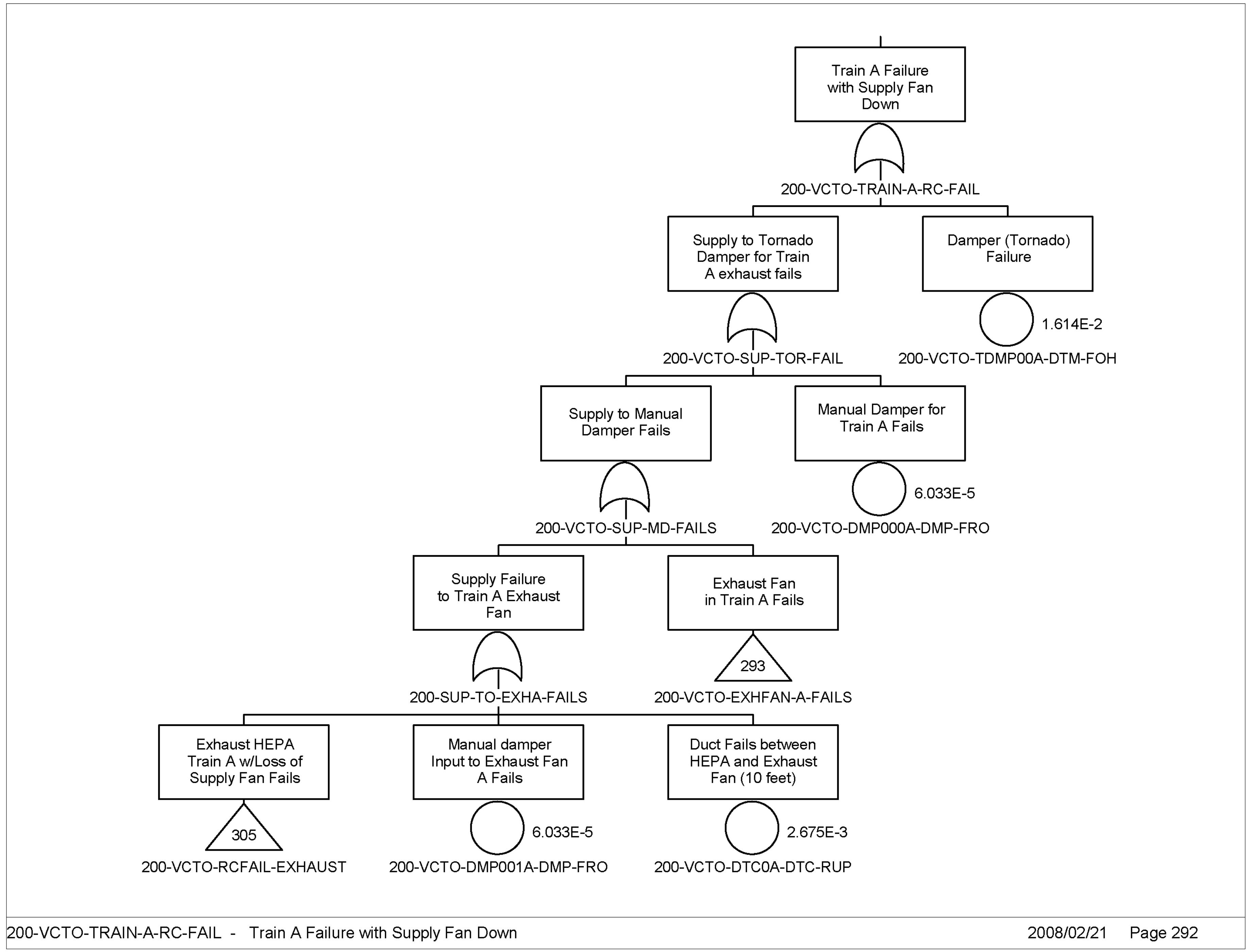
Source: Original

Figure B7.4-4. Loss of Normal and Degraded HVAC Trains



Source: Original

Figure B7.4-5. HVAC Trains Fail in Degraded Mode



Source: Original

Figure B7.4-6. Train A Failure with Supply Fan Down

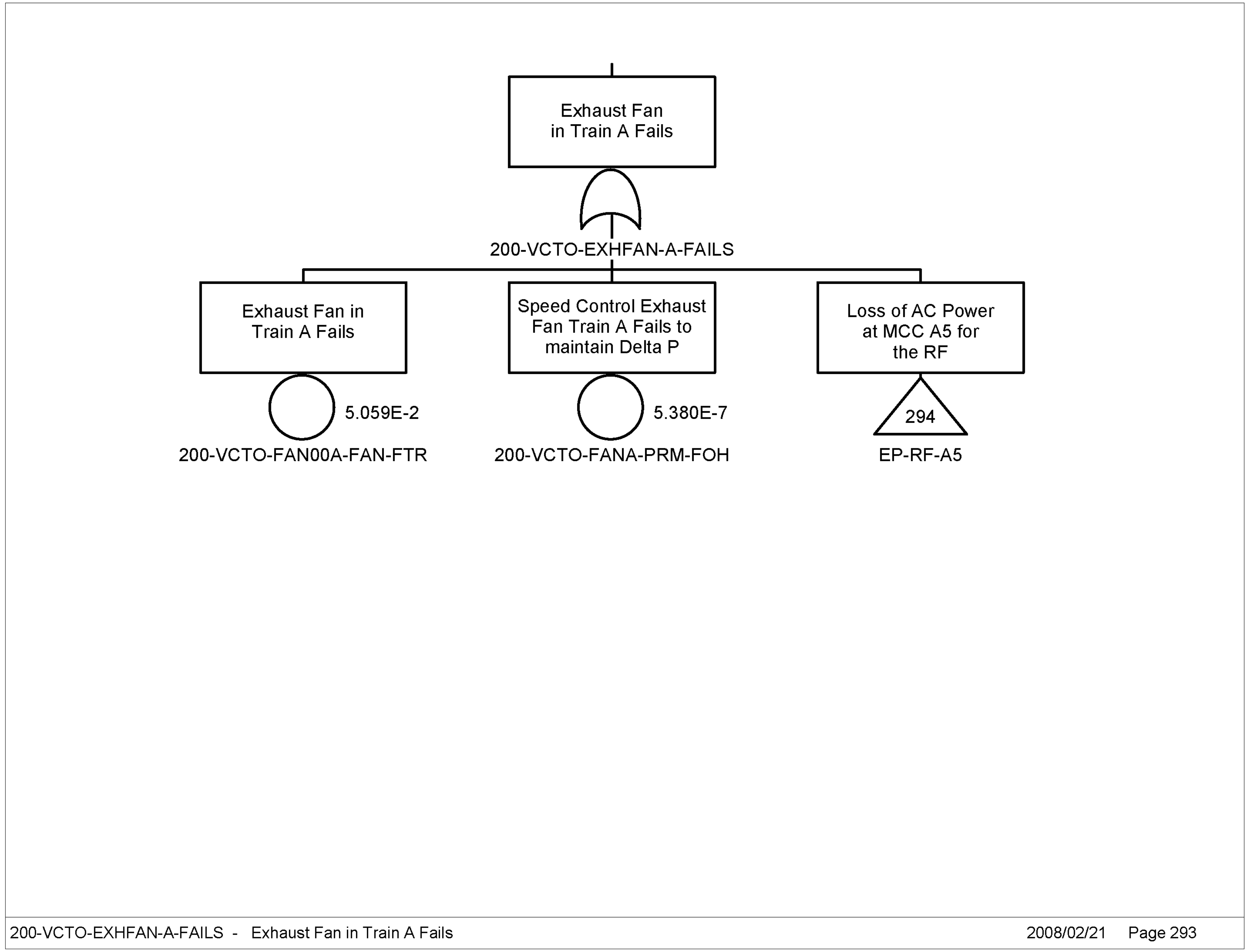


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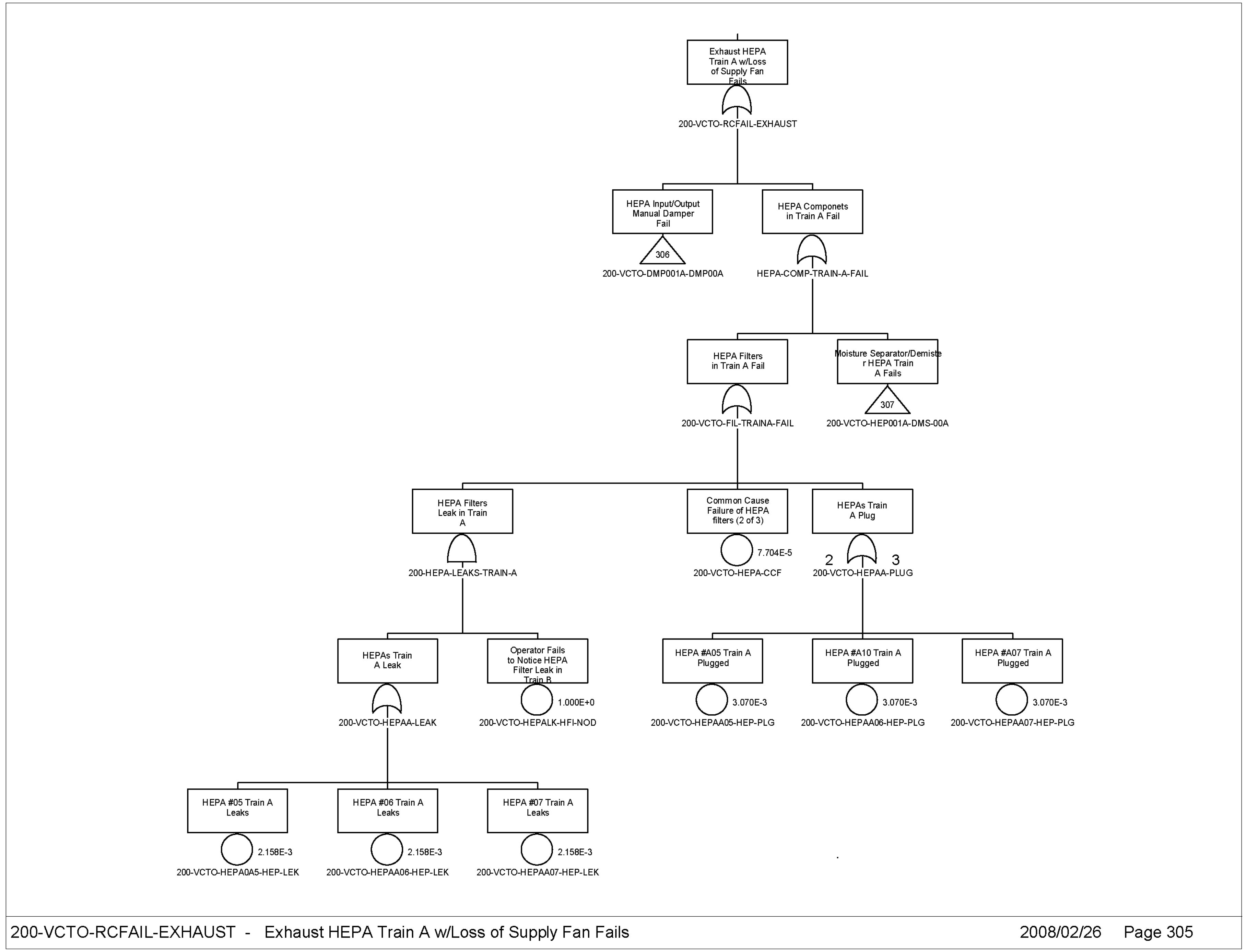
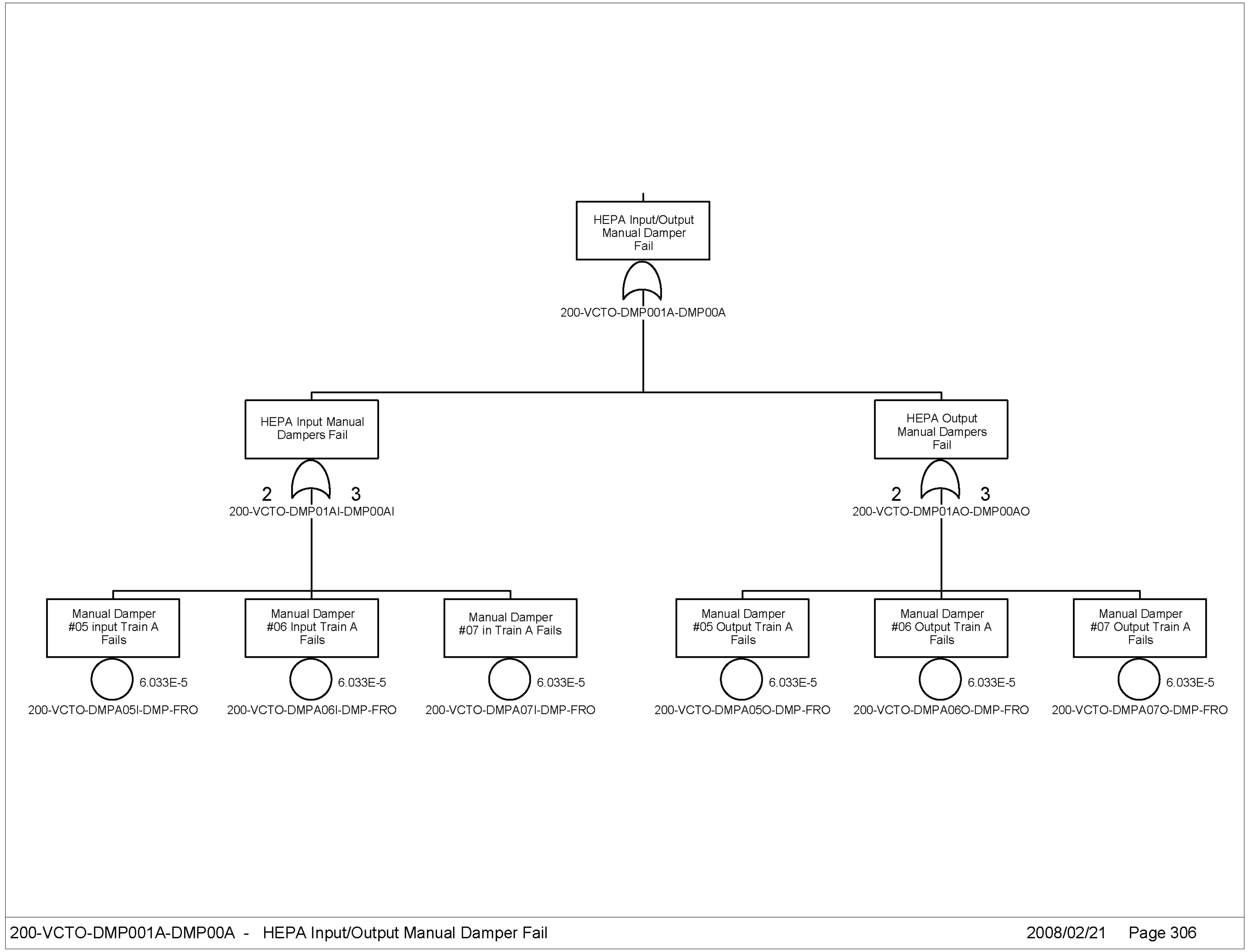
Figure B7.4-7. Exhaust Fan in Train A Fails



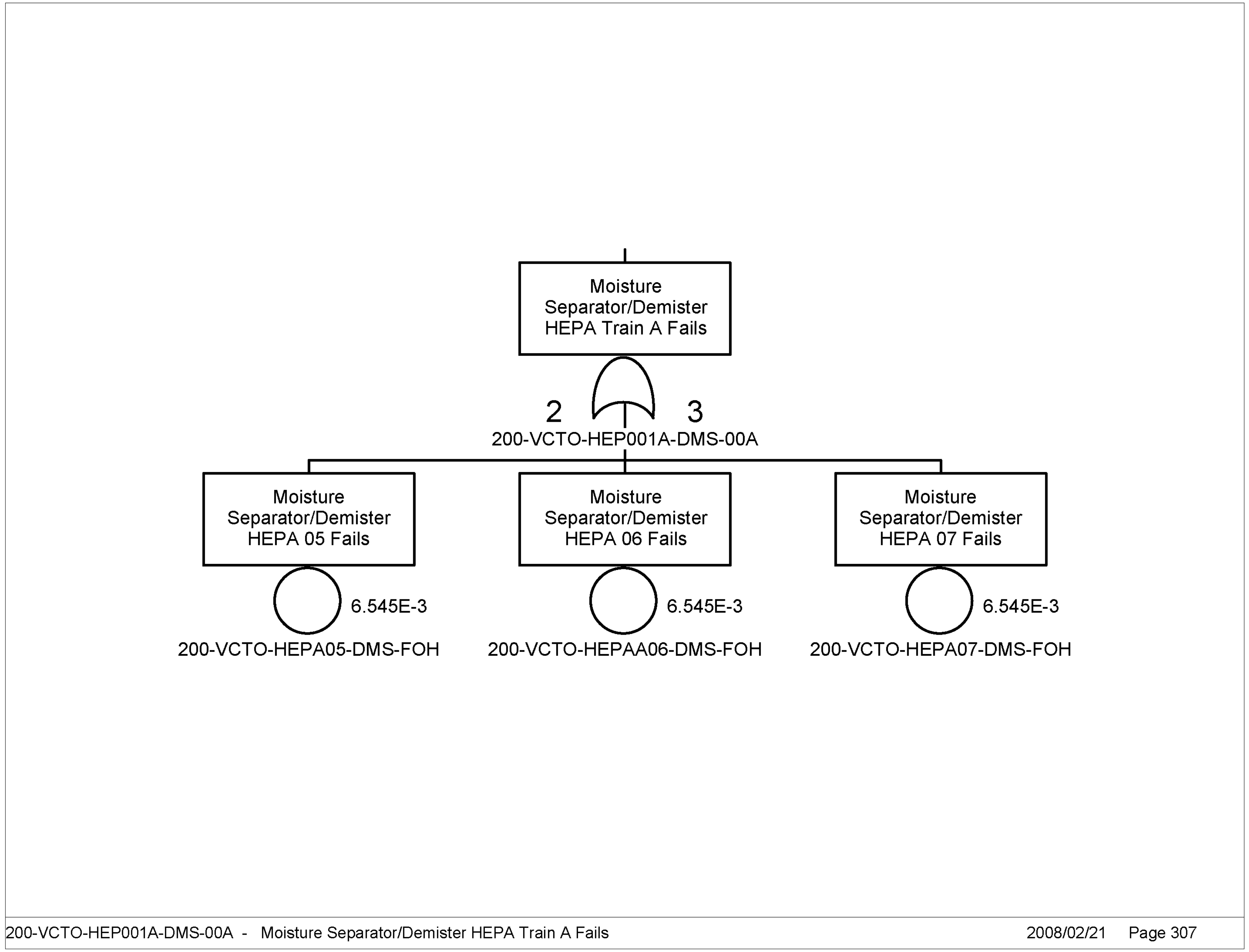
Source: Original

Figure B7.4-8. Exhaust HEPA Train A with Loss of Supply Fan



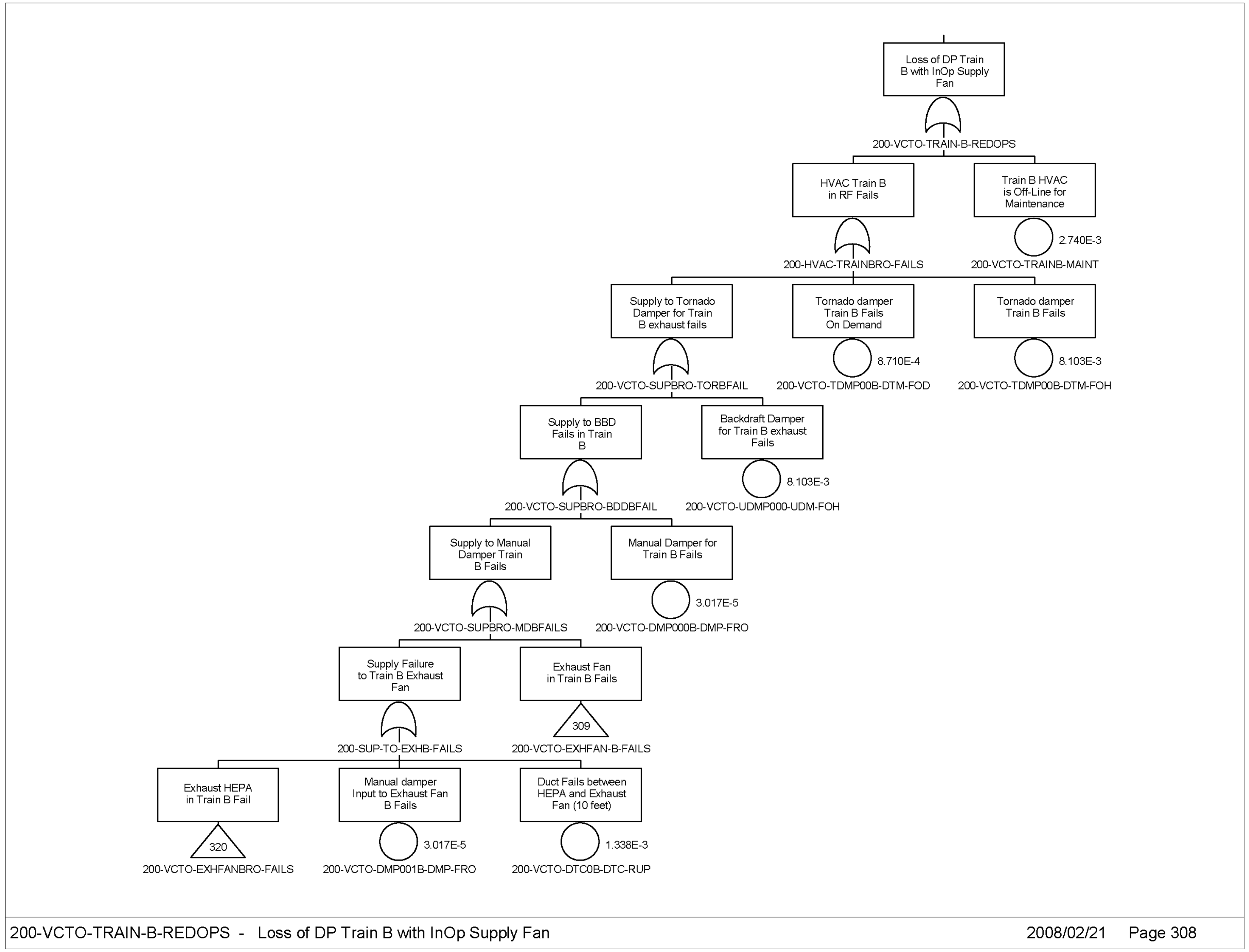
Source: Original

Figure B7.4-9. HEPA Input/Output Manual Damper Fail



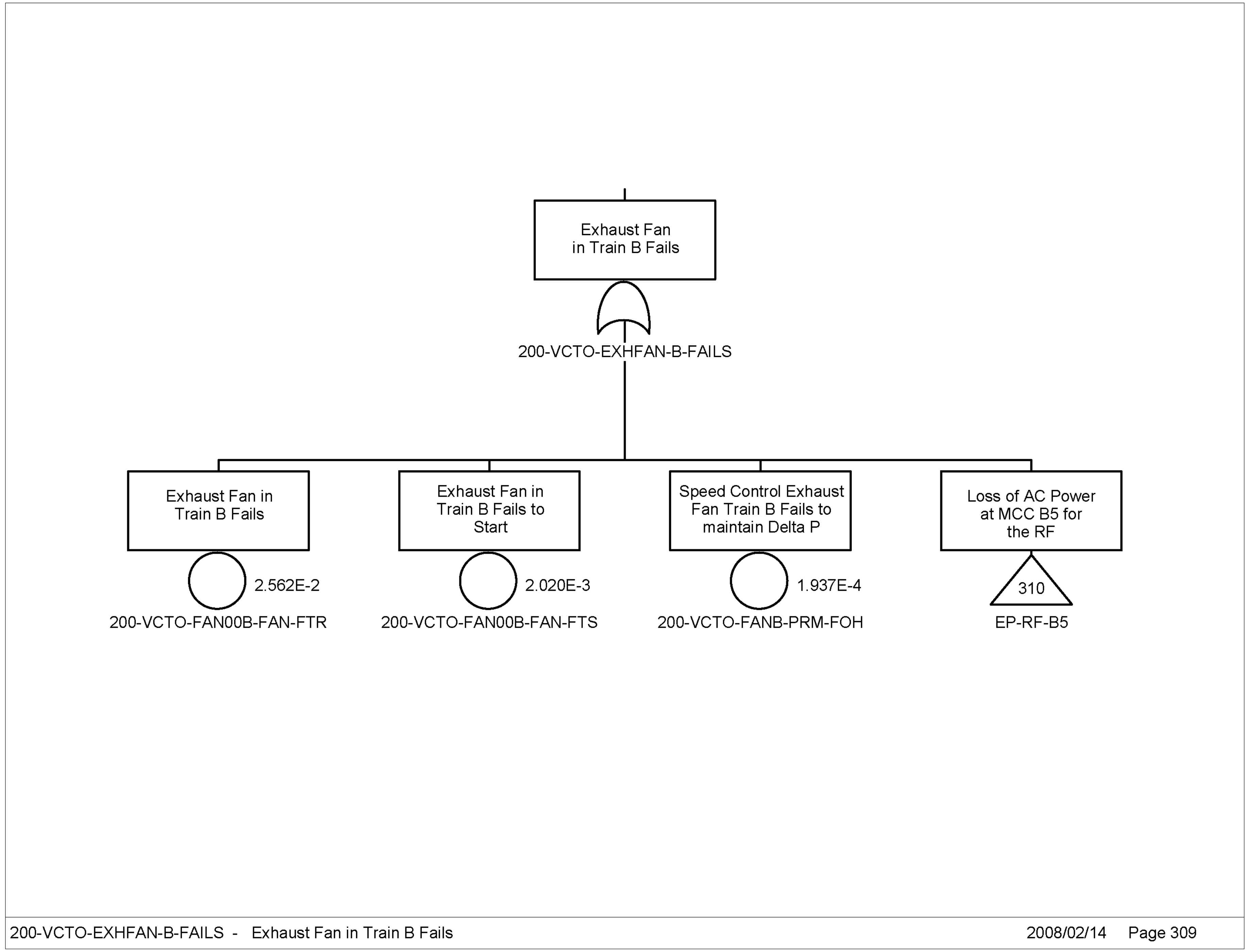
Source: Original

Figure B7.4-10. Moisture Separator/Demister HEPA Train A Fails



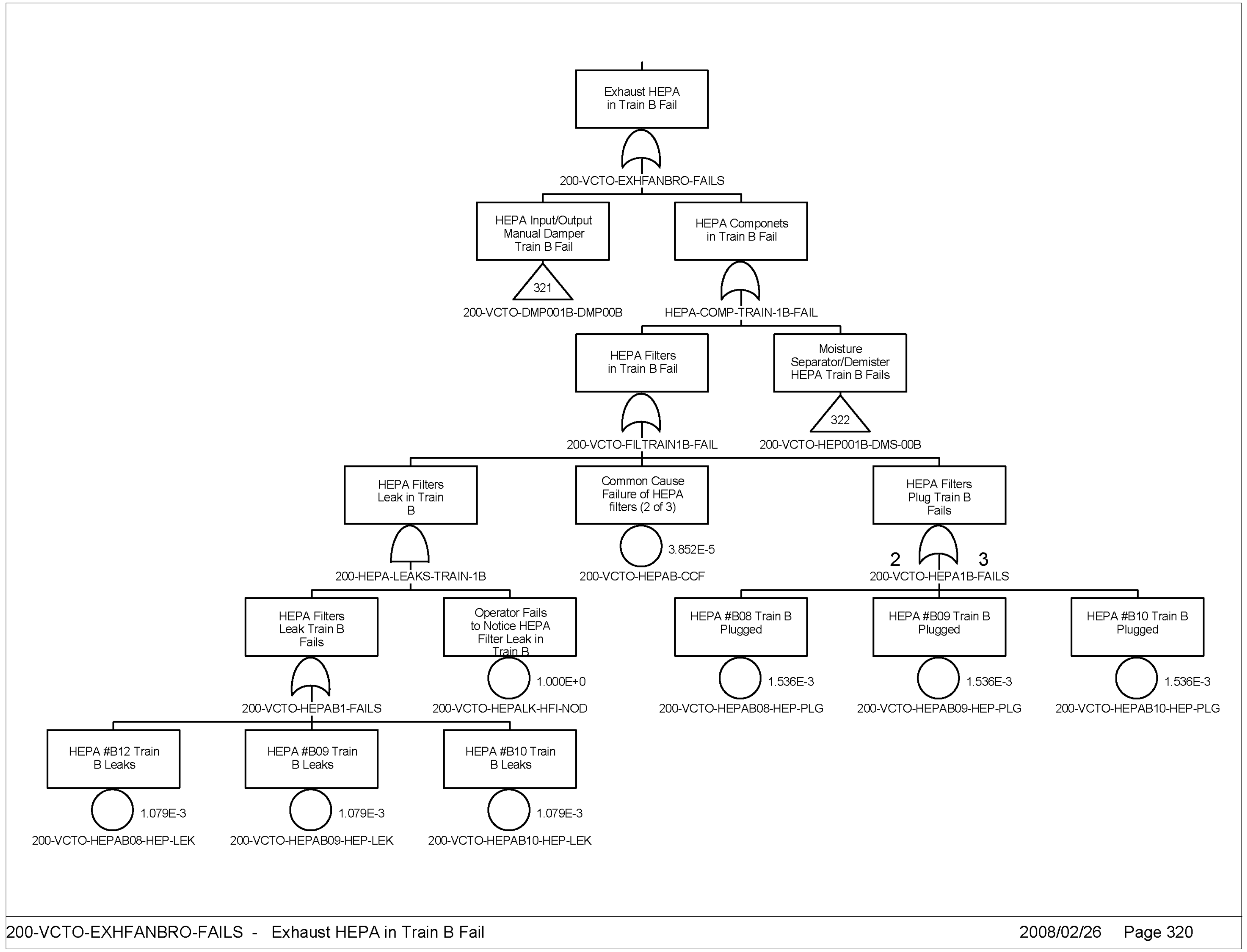
Source: Original

Figure B7.4-11. Loss of DP Train B with Inoperative Supply Fan



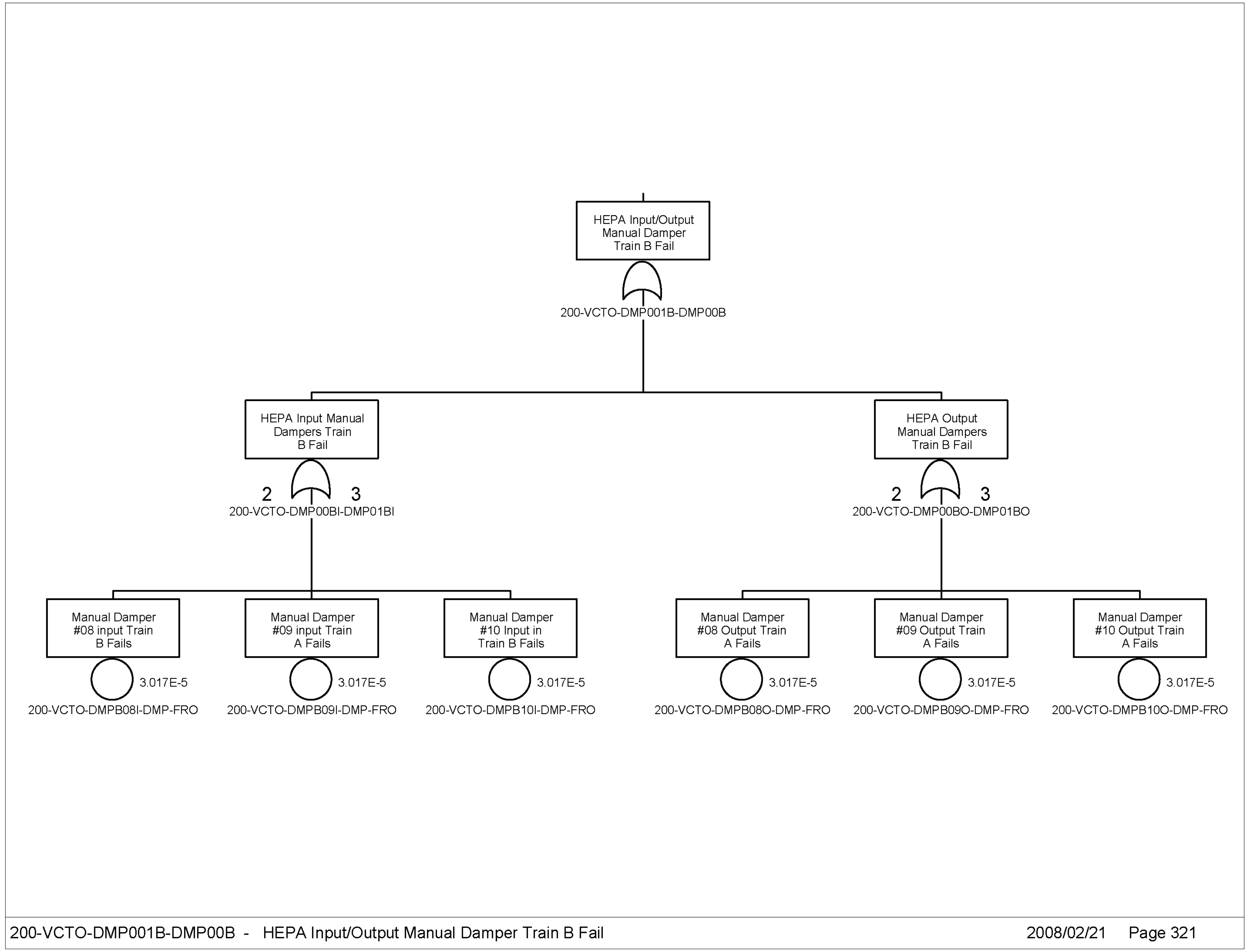
Source: Original

Figure B7.4-12. Exhaust Fan in Train B Fails



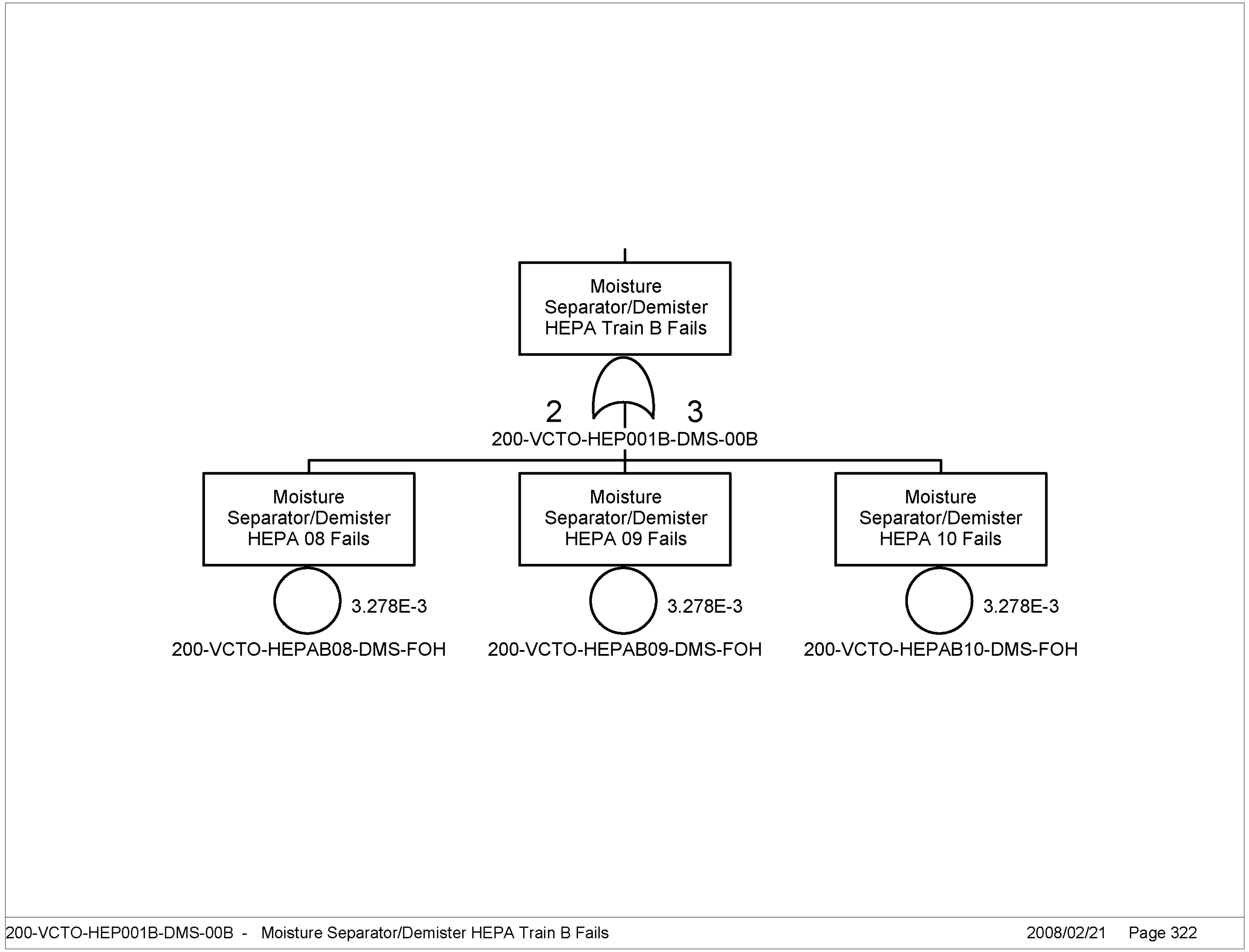
Source: Original

Figure B7.4-13. Exhaust HEPA in Train B Fail



Source: Original

Figure B7.4-14. HEPA Input/Output Manual Damper Train B Fail



Source: Original

Figure B7.4-15. Moisture Separator/Demister HEPA Train B Fails

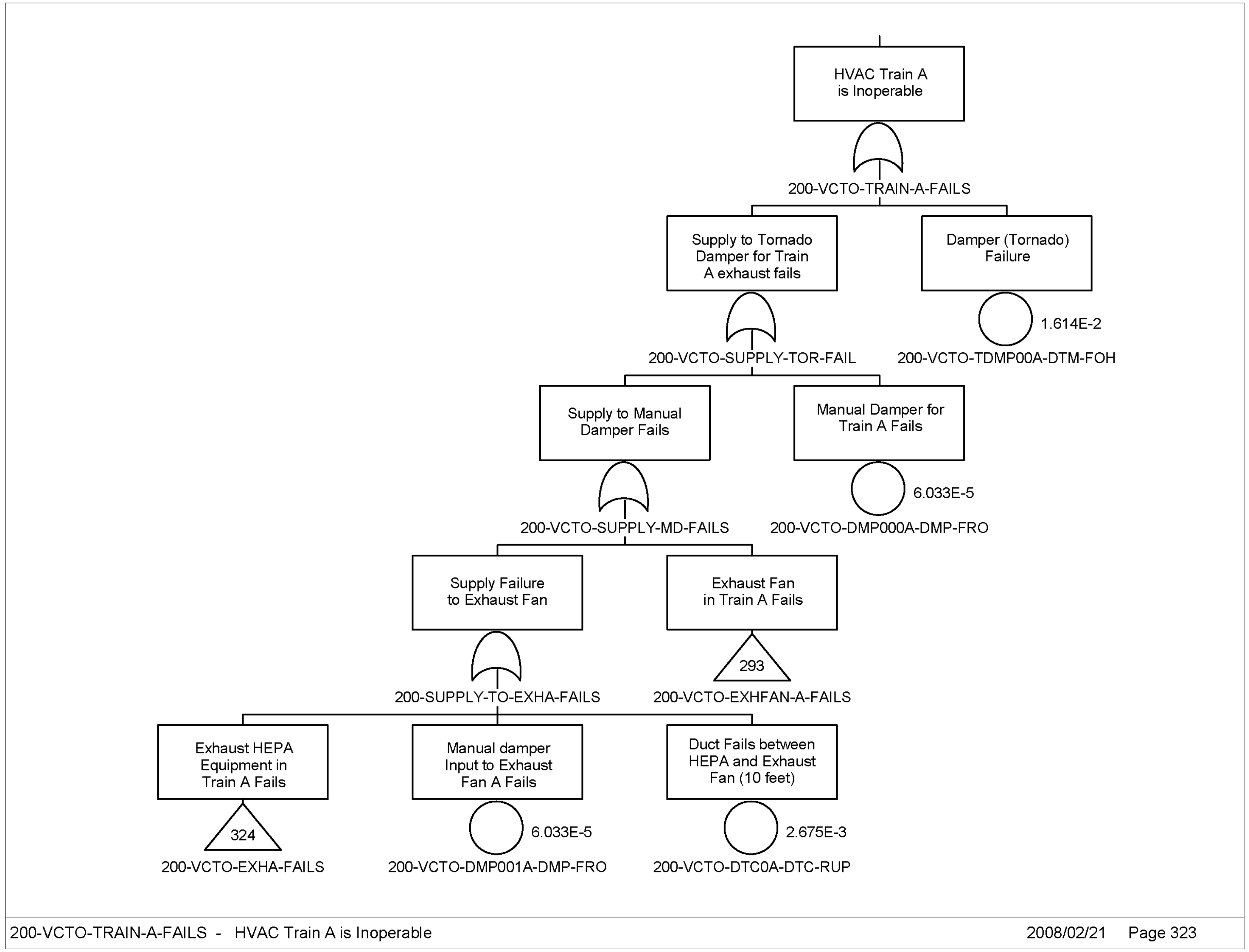


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Figure B7.4-16. HVAC Train A is Inoperable

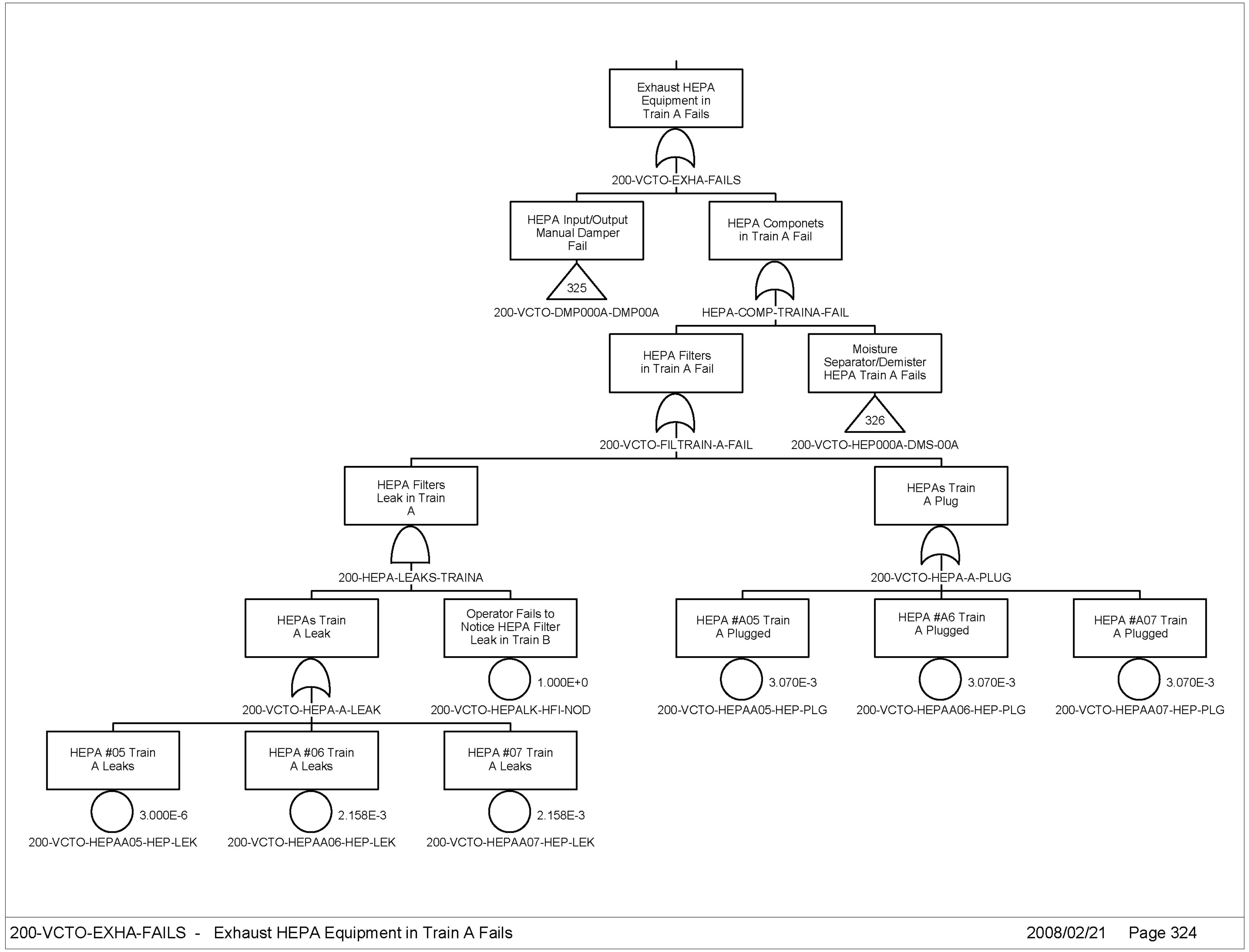

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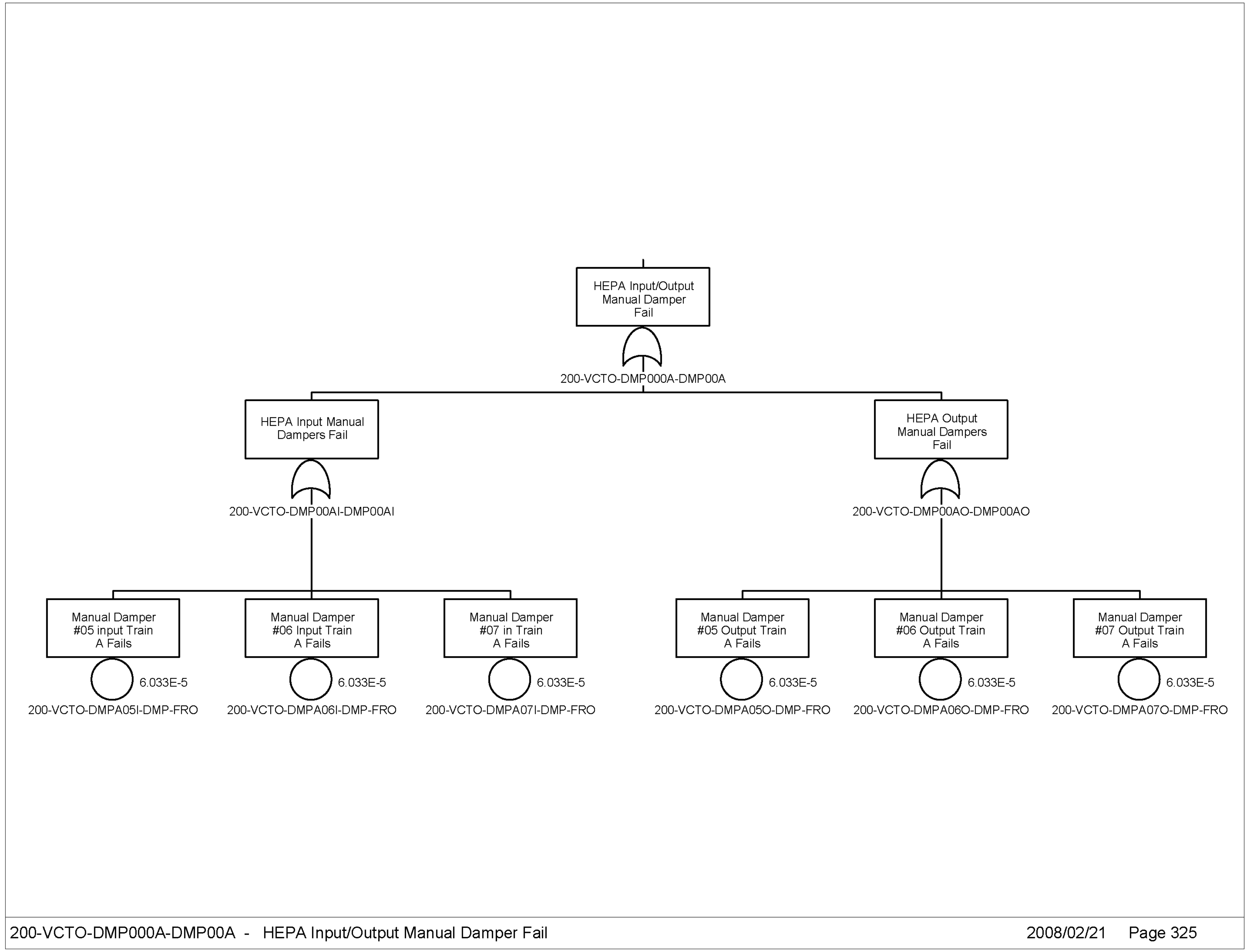
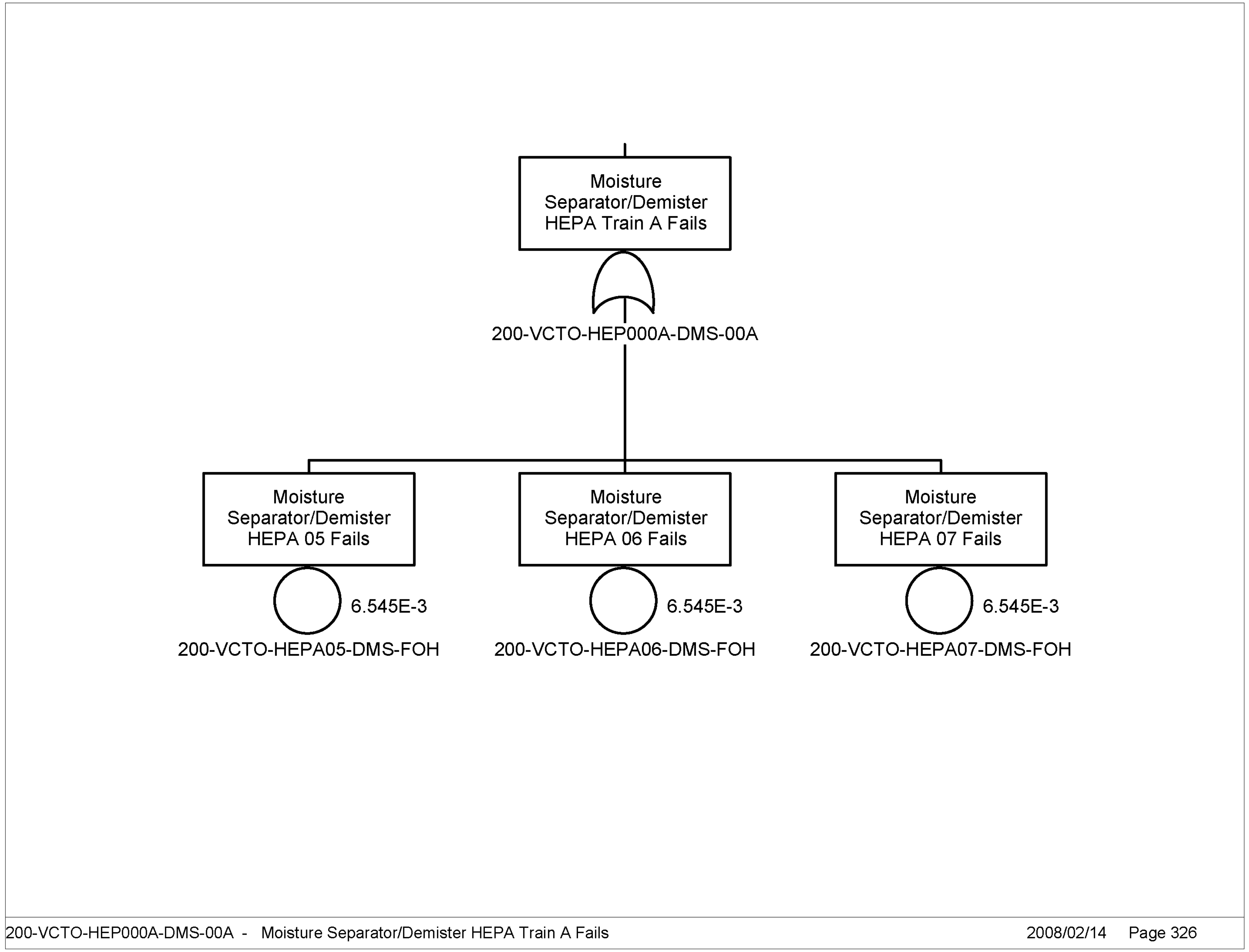
Figure B7.4-17. Exhaust HEPA Equipment in Train A Fails



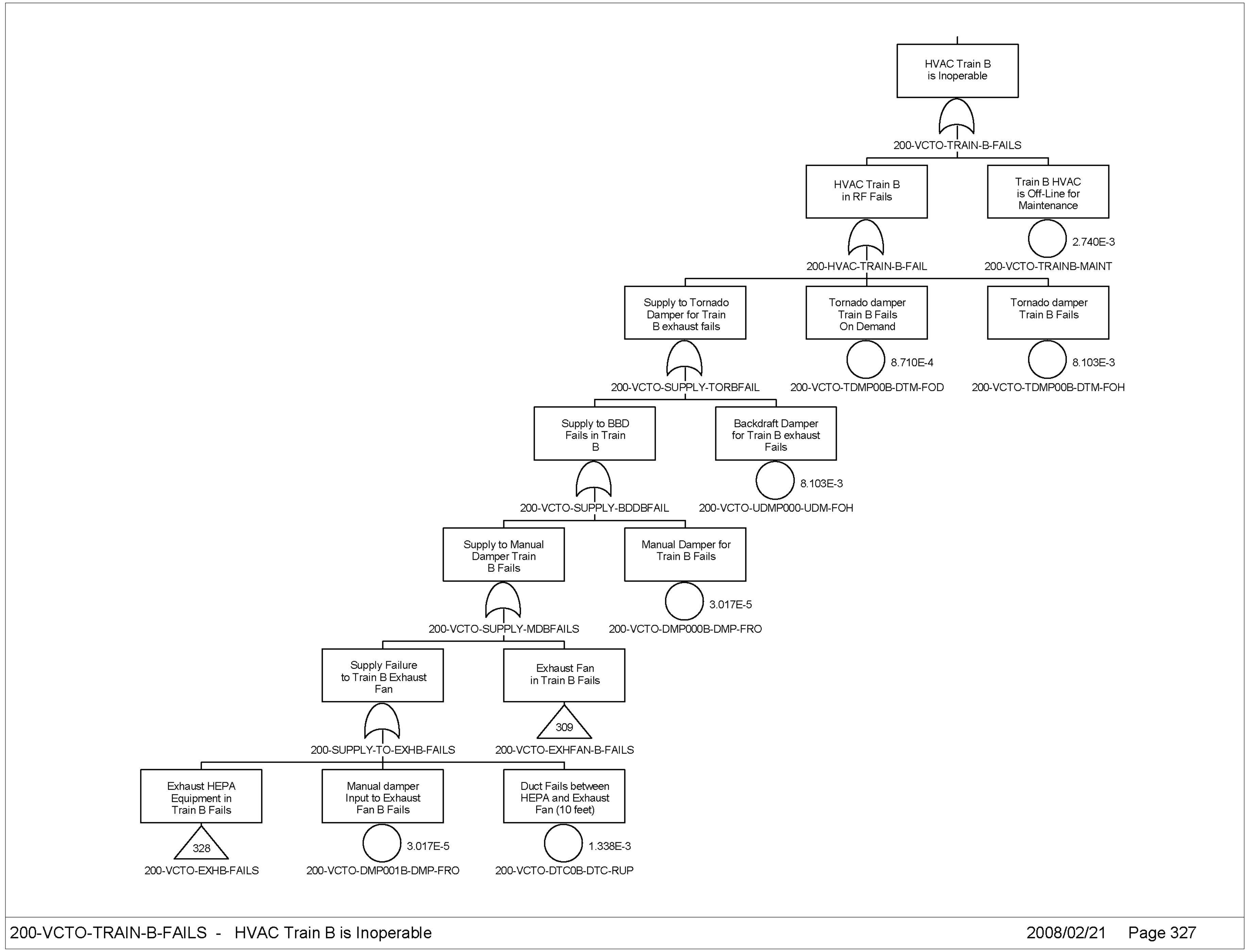
Source: Original

Figure B7.4-18. HEPA Input/Output Manual Damper Fail



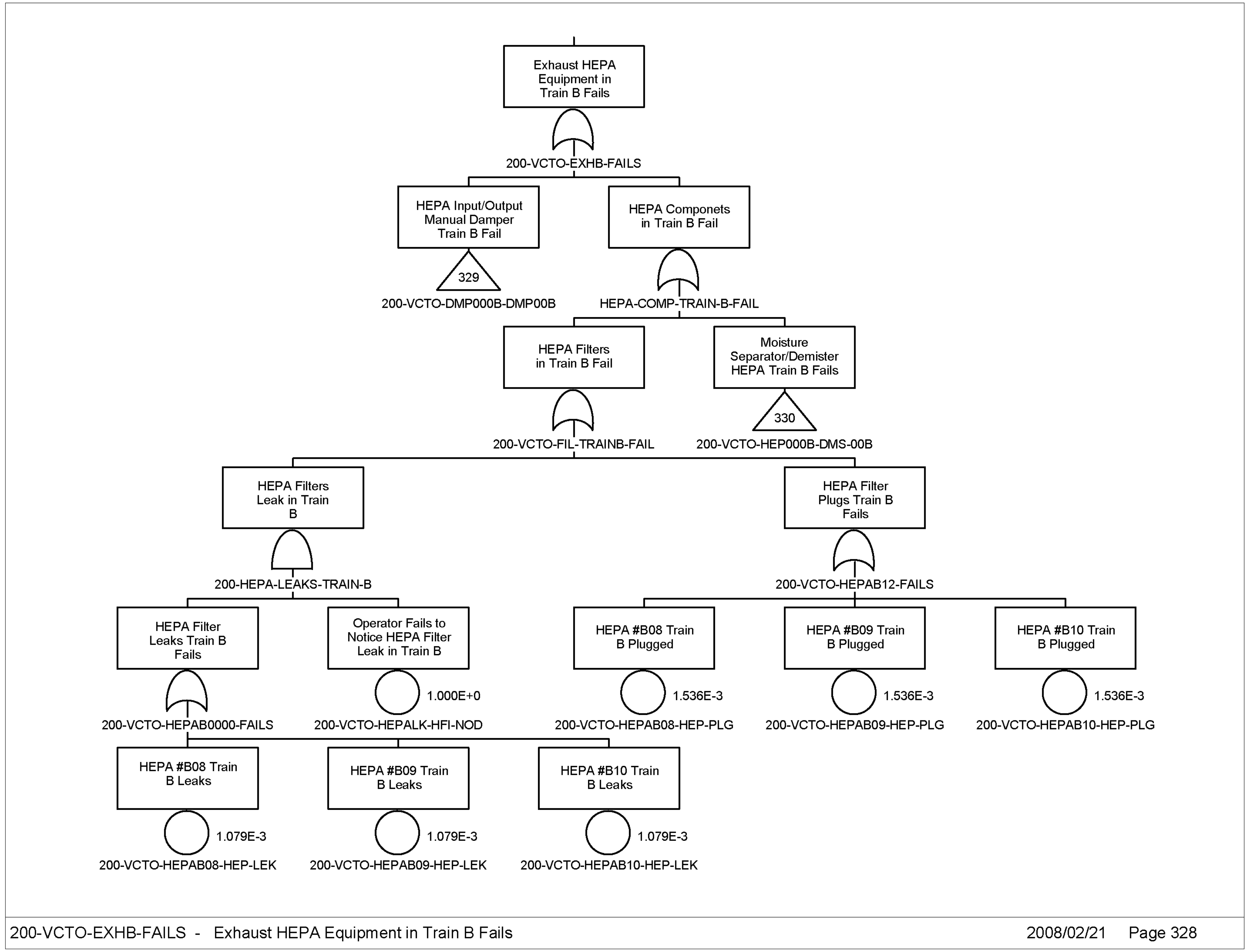
Source: Original

Figure B7.4-19. Moisture Separator/Demister HEPA Train A Fails



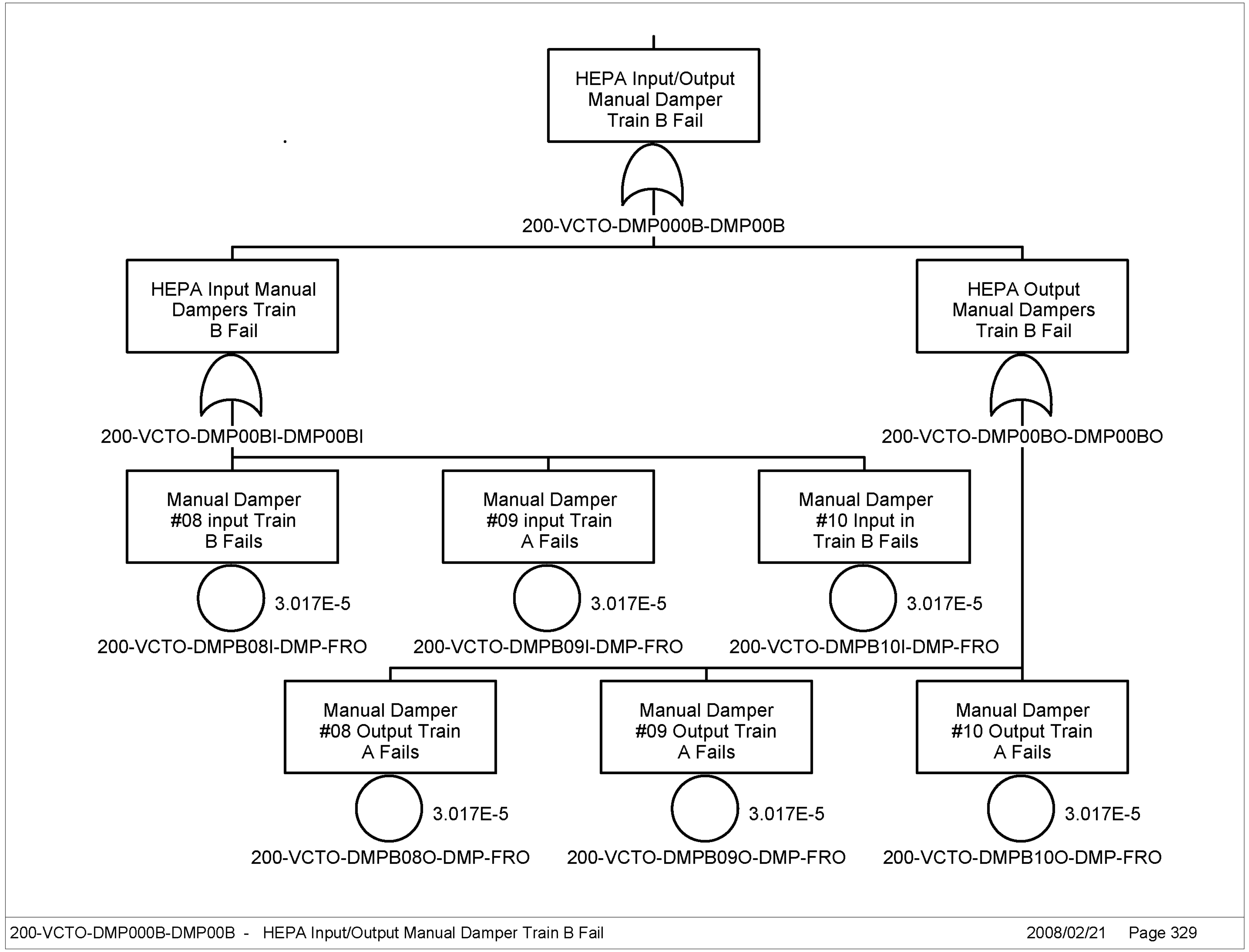
Source: Original

Figure B7.4-20. HVAC Train B is Inoperable



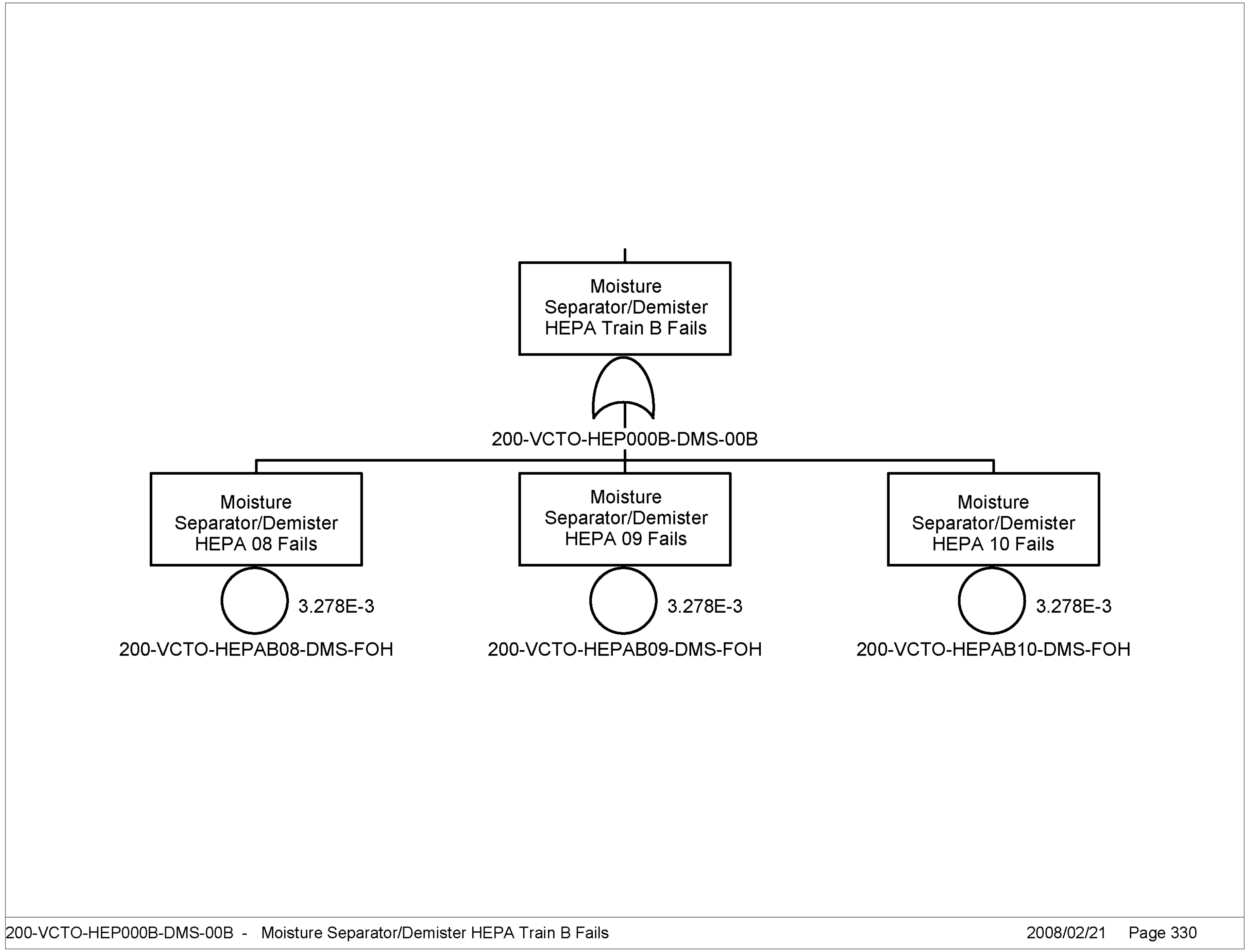
Source: Original

Figure B7.4-21. Exhaust HEPA Equipment in Train B Fails



Source: Original

Figure B7.4-22. HEPA Input/Output Manual Damper Train B Fail



Source: Original

Figure B7.4-23. Moisture Separator/Demister HEPA Train B Fails

B8 IMPORTANT TO SAFETY AC POWER FAULT TREE ANALYSIS

B8.1 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

The inputs in this Section noted with an asterisk (*) indicate that they fall into one of the designated categories described in Section 4.1, relative to suitability for intended use.

B8.1.1 BSC (Bechtel SAIC Company) 2007. *Emergency Diesel Generator Facility – 480V ITS MCC 26D-EEE0-MCC-00001 Single Line Diagram (Train A).* 26D-E10-EEE0-00301-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071130.0026.

B8.1.2 BSC 2007. *Emergency Diesel Generator Facility – 480V ITS MCC 26D-EEE0-MCC-00002 Single Line Diagram (Train B).* 26D-E10-EEE0-00401-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071130.0027.

B8.1.3 BSC 2007. *Emergency Diesel Generator Facility – Fuel Oil System Calculation.* 26D-M6C-EG00-00200-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071025.0001.

B8.1.4 BSC 2007. *Emergency Diesel Generator Facility – Generator Room Ventilation System Calculation.* 26D-M5C-VNI0-00100-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071015.0018.

B8.1.5 BSC 2007. *Emergency Diesel Generator Facility – ITS 125V DC System Single Line Diagram (Train* A*).* 26D-E10-EED0-00101-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071026.0015.

B8.1.6 BSC 2007. *Emergency Diesel Generator Facility – ITS 125V DC System Single Line Diagram (Train B).* 26D-E10-EED0-00201-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071026.0016.

B8.1.7 BSC 2007. *Emergency Diesel Generator Facility - Switchgear and Battery Rooms Ventilation System Calculation.* 26D-M5C-VNI0-00200-000-00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071022.0001.

B8.1.8 BSC 2007. *Normal Power System 13.8 kV Site Distribution Overall Single Line Diagram.* 000-E10-EEN0-00202-000 REV 00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080206.0078.

B8.1.9 BSC 2007. *Receipt Facility 480V-ITS Load Center Train A 200-EEE0-LC-00001 Single Line Diagram*. 200-E10-EEE0-00301-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0018.

B8.1.10 BSC 2007. *Receipt Facility 480V ITS Load Center Train B 200-EEE0-LC-00002 Single Line Diagram*. 200-E10-EEE0-00401-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0019.

B8.1.11 BSC 2007. *Receipt Facility 480V ITS MCC Train A 200-EEE0-MCC-00001 Single Line Diagram*. 200-E10-EEE0-00101-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0016.

B8.1.12 BSC 2007. *Receipt Facility 480V ITS MCC Train B 200-EEE0-MCC-00002 Single Line Diagram*. 200-E10-EEE0-00201-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0017.

B8.1.13 BSC 2007. *Receipt Facility Confinement ITS Battery Room Exhaust System - Train A Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00302-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071201.0004.

B8.1.14 BSC 2007. *Receipt Facility Confinement ITS Battery Room Exhaust System - Train B Ventilation & Instrumentation Diagram*. 200-M80-VCT0-00304-000-00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC. ENG. 20071201.0005.

B8.1.15 BSC 2007. *Receipt Facility Confinement ITS Electrical Room HVAC System - Train A Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00301-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC. ENG.20071002.0027.

B8.1.16 BSC 2007. *Receipt Facility Confinement ITS Electrical Room HVAC System - Train B Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00303-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC. ENG.20071002.0029.

B8.1.17 BSC 2008. *Emergency Diesel Generator Facility-13.8 kV ITS Switchgear 26D-EEE0-SWGR-00001 Single Line Diagram (Train A)*. 26D-E10-EEE0-00101-000-00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080204.0001.

B8.1.18 BSC 2008. *Emergency Diesel Generator Facility-13.8 kV ITS Switchgear 26D-EEE0-SWGR-00002 Single Line Diagram (Train B)*. 26D-E10-EEE0-00201-000-00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20080204.0002.

B8.1.19 *Eide, S.A.; Gentillon, C.D.; Wierman, T.E.; and Rasmuson, D.M. 2005. *Analysis of Loss of Offsite Power Events: 1986-2004.* Volume 1 of *Reevaluation of Station Blackout Risk at Nuclear Power Plants.* NUREG/CR-6890. Washington, D.C.: U.S. Nuclear Regulatory Commission. ACC: MOL.20071114.0164.

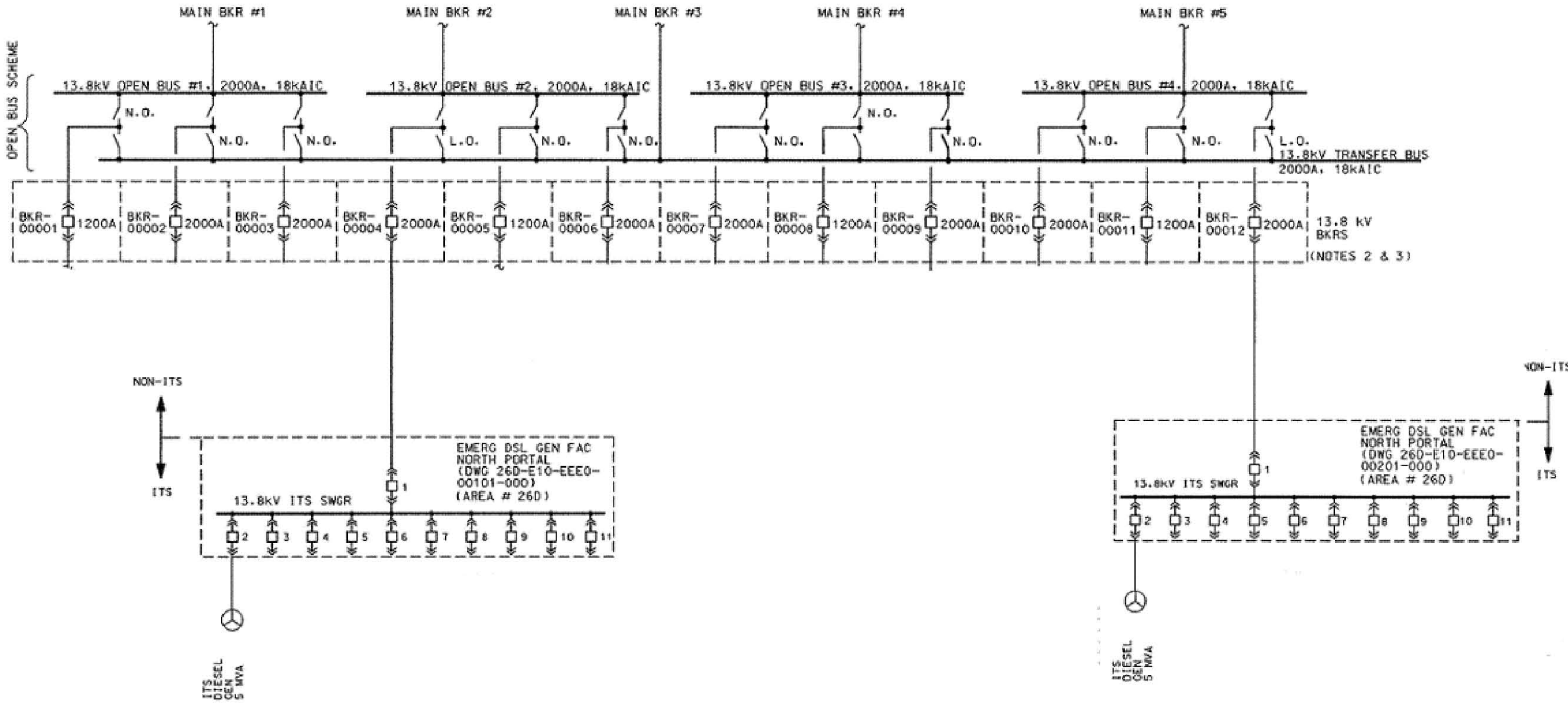
B8.2 IMPORTANT TO SAFETY AC POWER DESCRIPTION

The ITS AC power system supplies power to the ITS systems (the HVAC systems in the three CRCFs, the WHF, and the RF). The ITS power system makes use of two elements: the onsite ITS power supply and ITS equipment needed to supply power from the onsite ITS power supply to the ITS loads in each of the site facilities. During normal operations AC power is supplied from two offsite 138kV power lines through the 138kV – 13.8kV switchyard and then through the plant AC power distribution system to the various facilities throughout the site. Off-normal conditions for the distribution of AC power occur during a loss of offsite power (LOSP). A LOSP may be the result of problems on the power grid, or may be the result of failures within the plant AC power systems (most likely within the 138kV – 13.8kV switchyard). Under these conditions, the AC power source for the RF ITS equipment is two onsite ITS diesel generators. There are several diesel generators located onsite. However there are only two generators designated as ITS; the two that support each division of ITS equipment in the three CRCFs, the WHF, and the RF. Power is supplied to ITS loads via the same onsite AC power distribution system that is used during normal operation. Each ITS diesel generator supplies power to one train (A or B) of ITS systems. Each ITS diesel generator, its associate support systems, and the power distribution system is independent, electrically isolated, of the other diesel generator, its support systems, and power distribution system.

B8.2.1 Normal AC Power Distribution

Normal AC power to the RF ITS equipment is provided via two 13.8kV ITS switchgears (A and B), one supplying RF Train A ITS loads and the second supplying power to RF Train B ITS loads. These two 13.8kV ITS switchgears (Figures B8.2-1 through B8.2-3) are normally aligned to receive power from the site 138kV - 13.8kV switchyard through open buses 2 and 4.

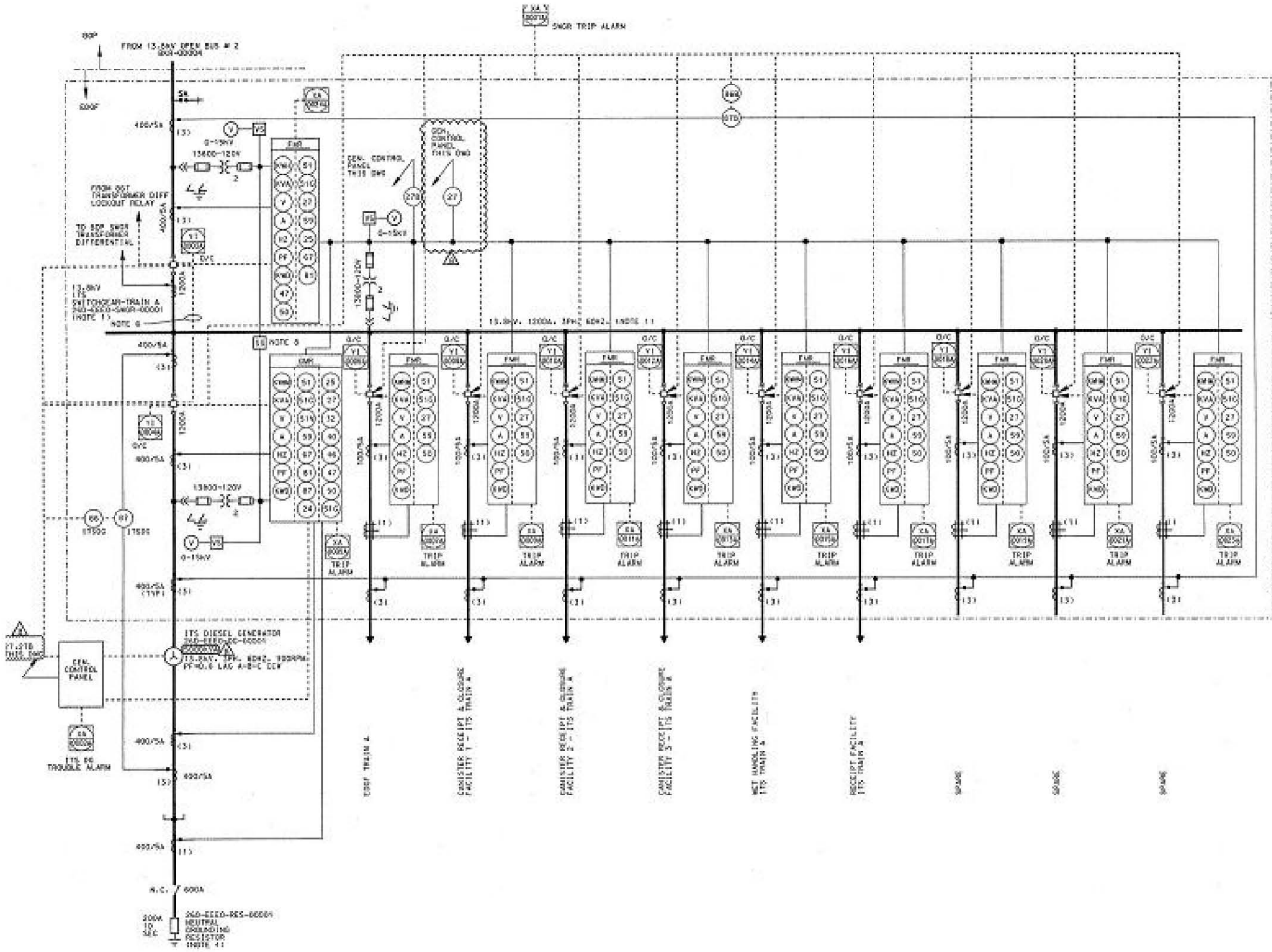
In addition to supplying power to the ITS loads in the RF, the 13.8kV ITS switchgear supplies power to equipment in the Emergency Diesel Generator Facility (EDGF) required to support ITS diesel generator operation. These loads include the diesel generator room fans, 13.8kV ITS switchgear room and battery room air handling unit, the ITS diesel generator fuel oil pumps, and DC power (via a battery charger) to operate the ITS switchgear circuit breakers (Figures B8.2-4 and B8.2-5 for ITS diesel generator train A and Figures B8.2-6 and B8.2-7 for ITS diesel generator train B)



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.8.

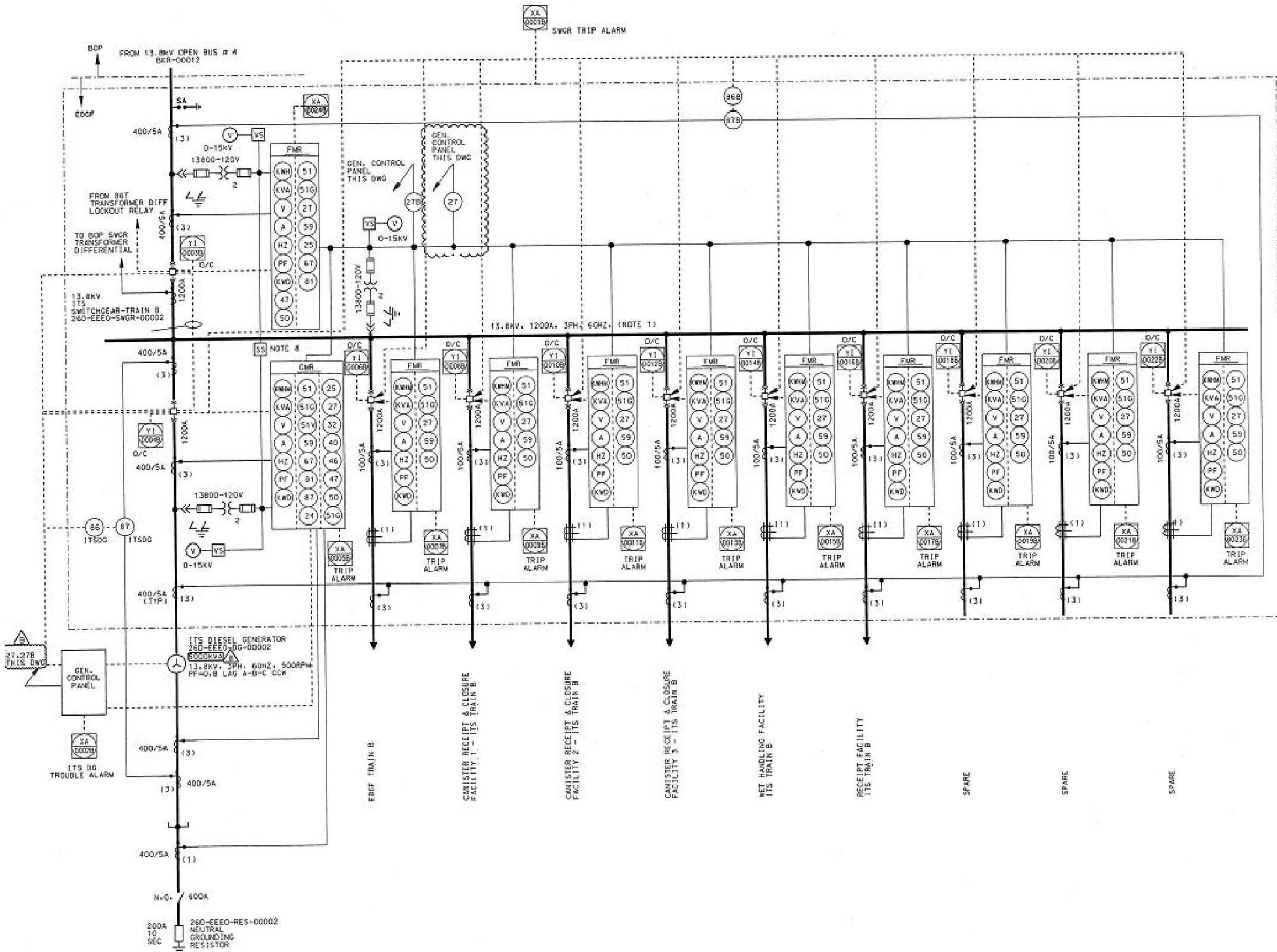
Figure B8.2-1. AC Power – Main Electrical Distribution



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.17.

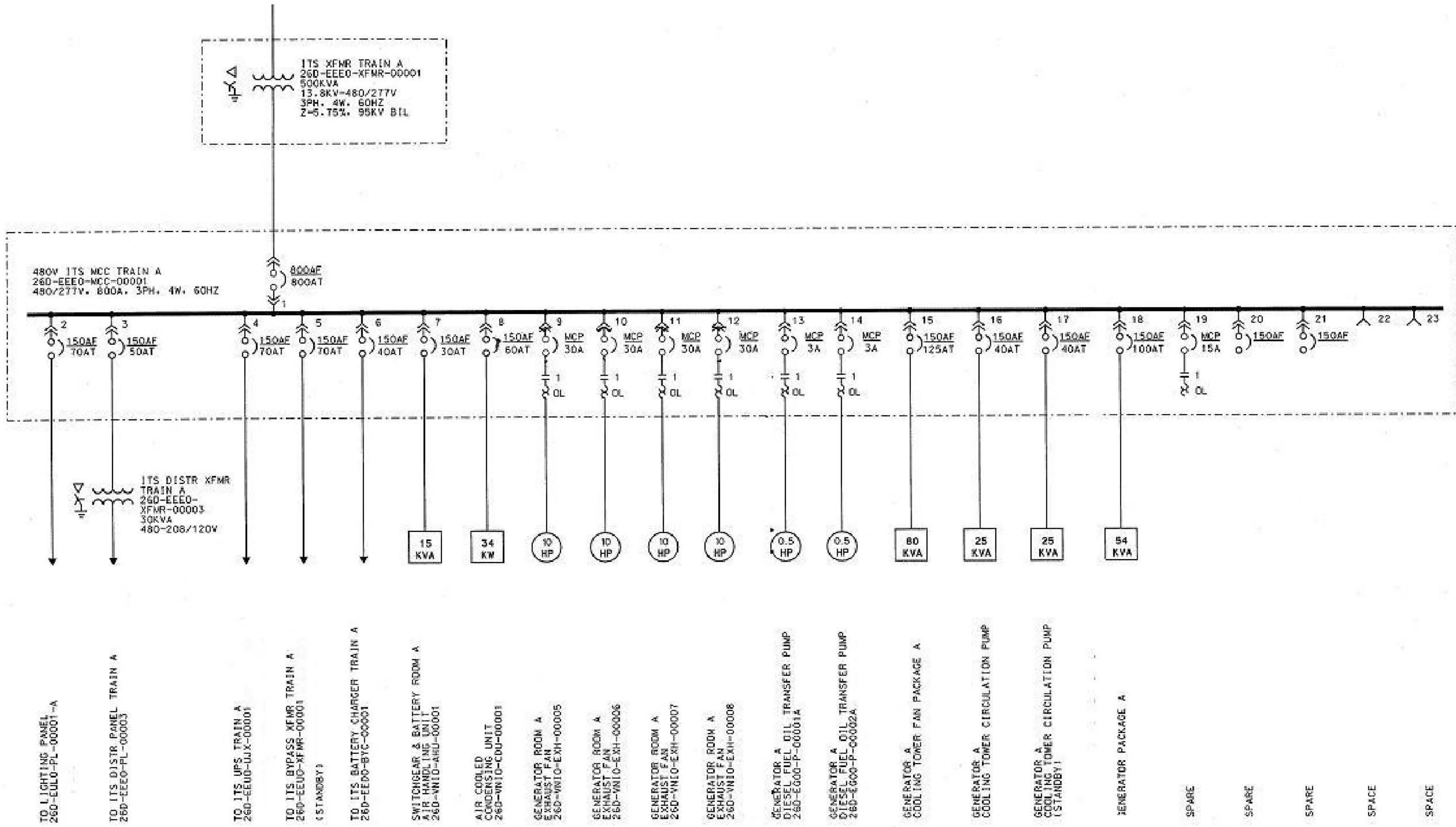
Figure B8.2-2. AC Power – 13.8kV ITS Switchgear Train A



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.8.

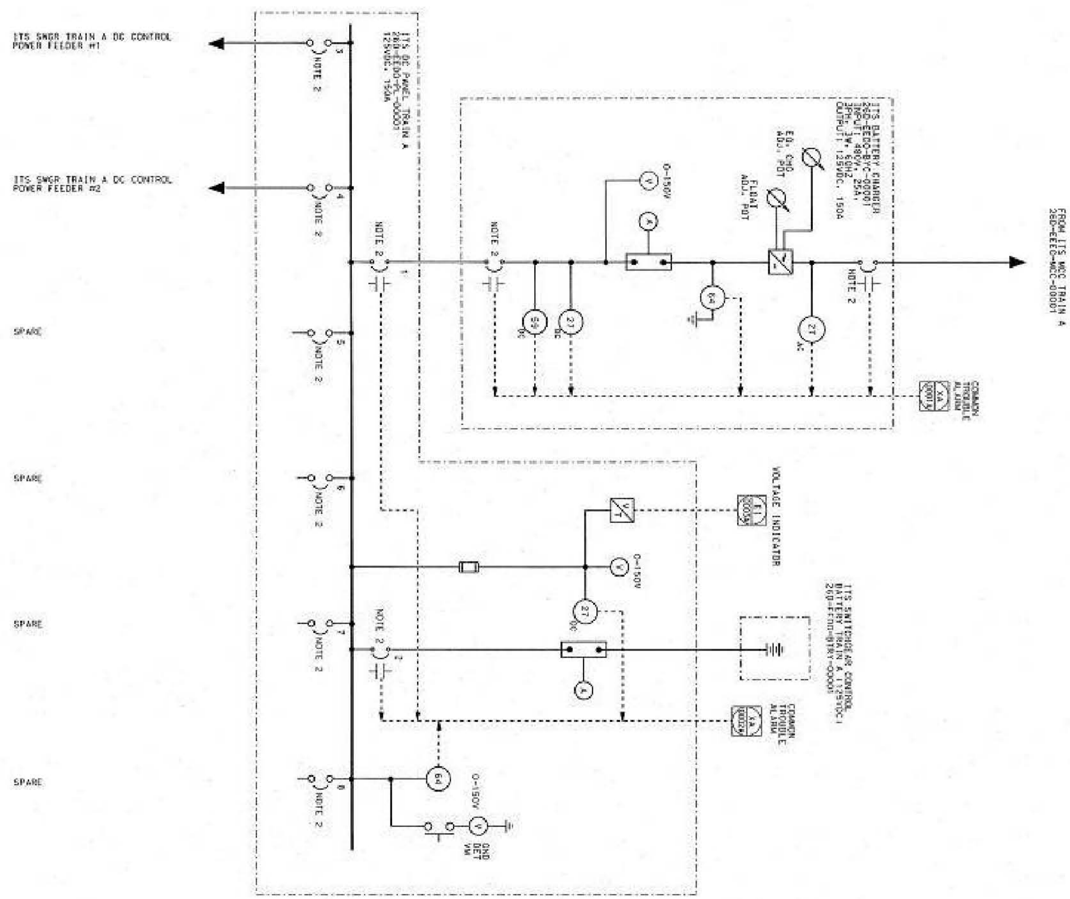
Figure B8.2-3. AC Power – 13.8kV ITS Switchgear Train B



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.1.

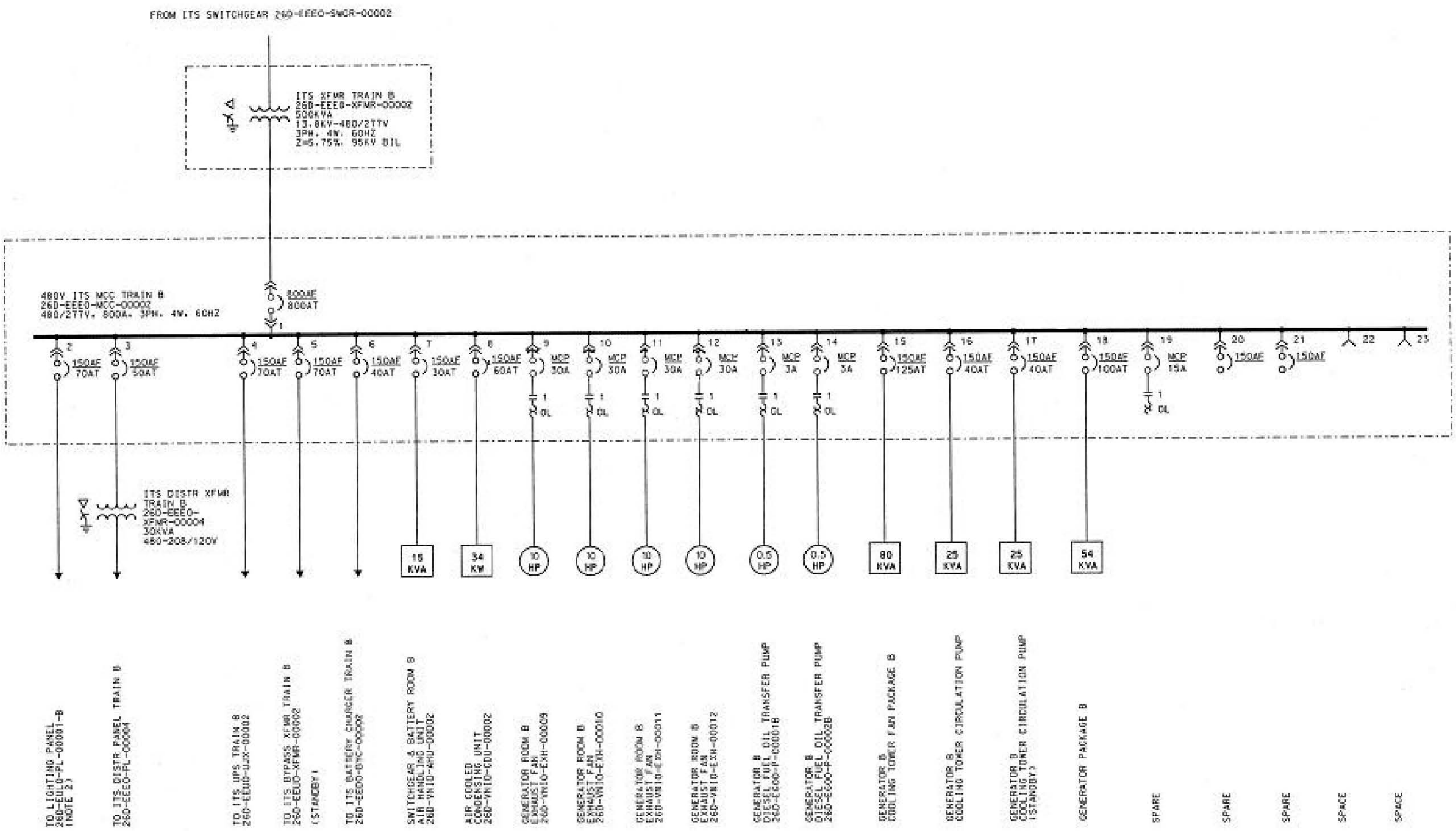
Figure B8.2-4. Emergency Diesel Generator Facility – 480V ITS MCC Train A



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.5.

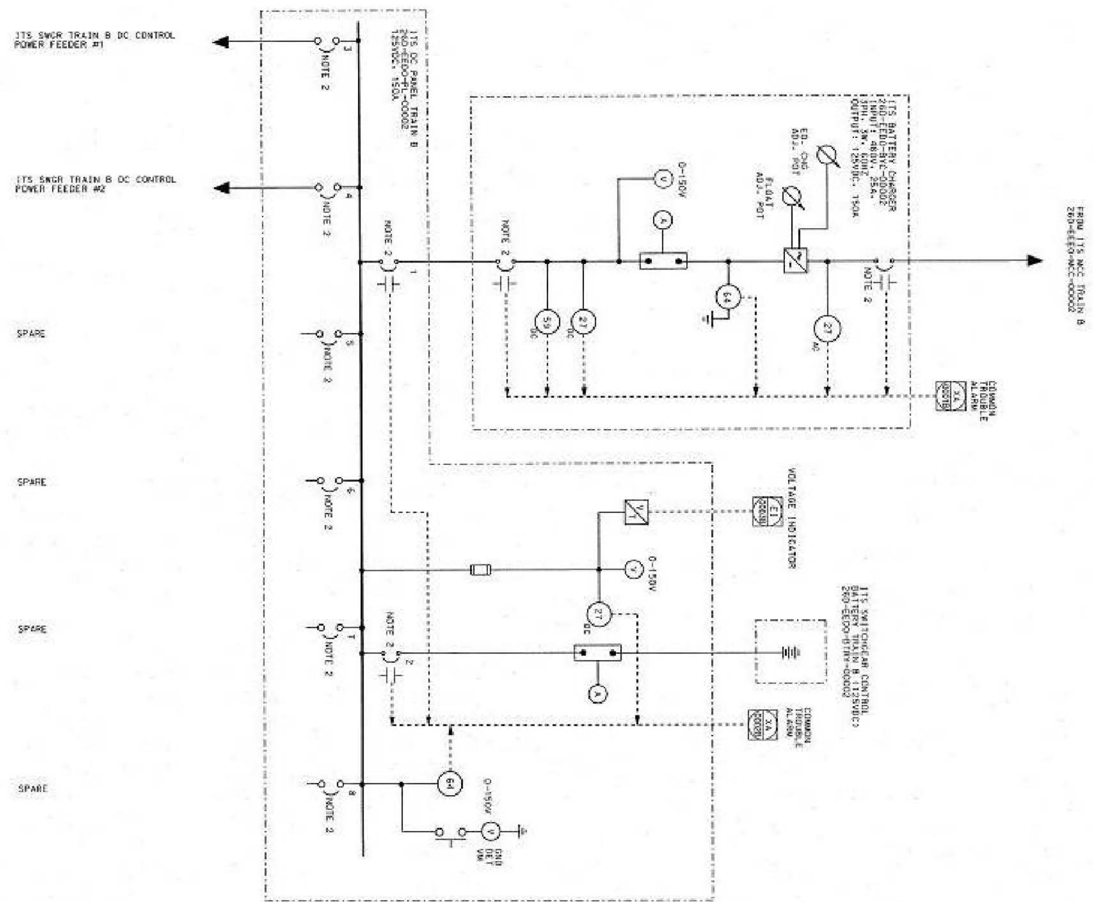
Figure B8.2-5. ITS 125 V DC System Train A



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Ref. B8.1.2.

Figure B8.2-6. Emergency Diesel Generator Facility – 480 V ITS MCC Train B



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.6.

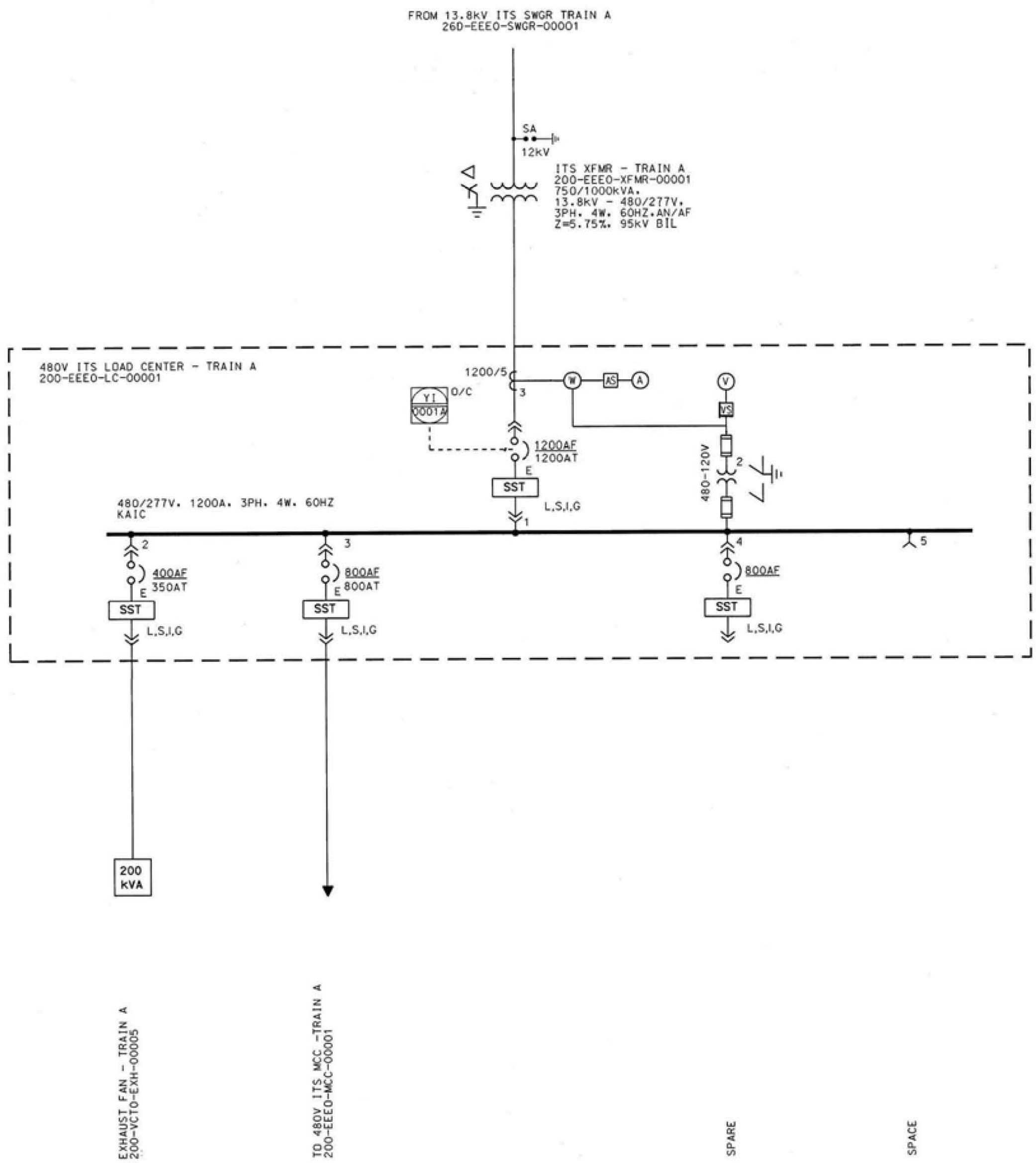
Figure B8.2-7. ITS 125V DC System Train B

The ITS loads within the RF are powered via two ITS 480/277V load centers and ITS 480/277V motor control centers (MCC) located within separate areas in the RF. ITS 480/277V load center Train A (Figure B8.2-8) and ITS 480/277V MCC Train A (Figure B8.2-10) support Train A of the RF ITS HVAC.

For the remainder of this attachment these are referred to as ITS load center Train A and ITS MCC Train A.

The ITS 480/277V load center Train B (Figure B8.2-9) and ITS 480/277V MCC Train B (Figure B8.2-11) support Train B of the RF ITS HVAC.

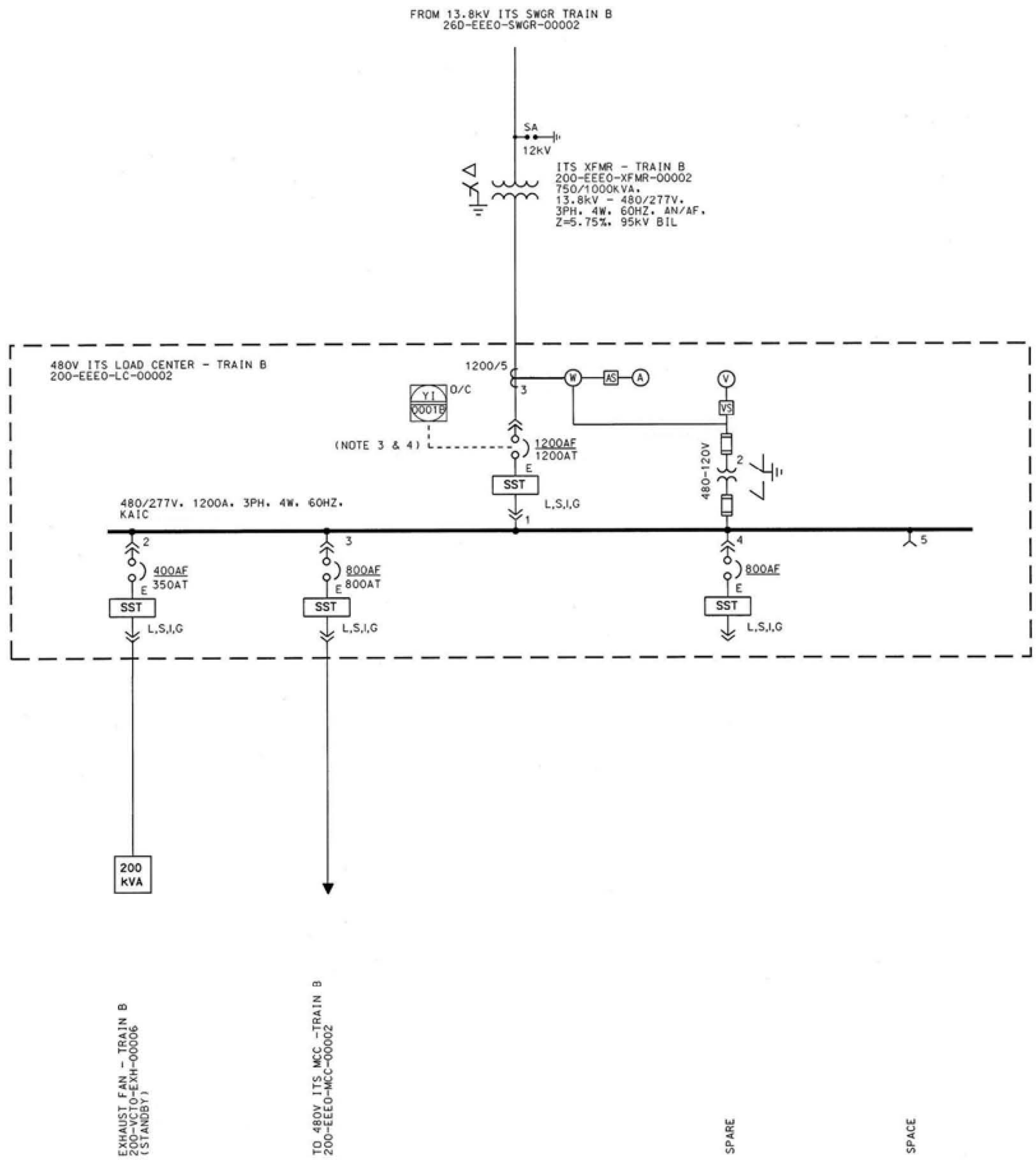
For the remainder of this attachment these are referred to as ITS load center Train B and ITS MCC Train B. Each division of the AC power supply from the 13.8kV ITS switchgears to the RF passes through a 13.8kV to 480V transformer (Figures B8.2-8 through B8.2-11).



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.9.

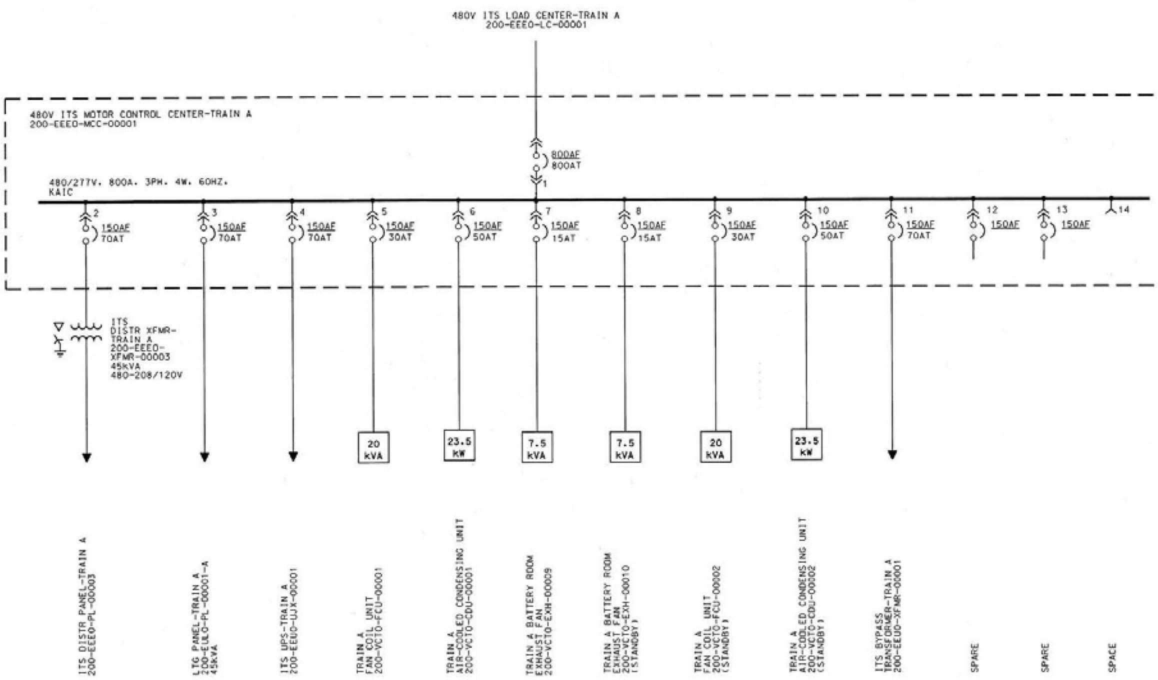
Figure B8.2-8. RF 480V ITS Load Center Train A



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.10.

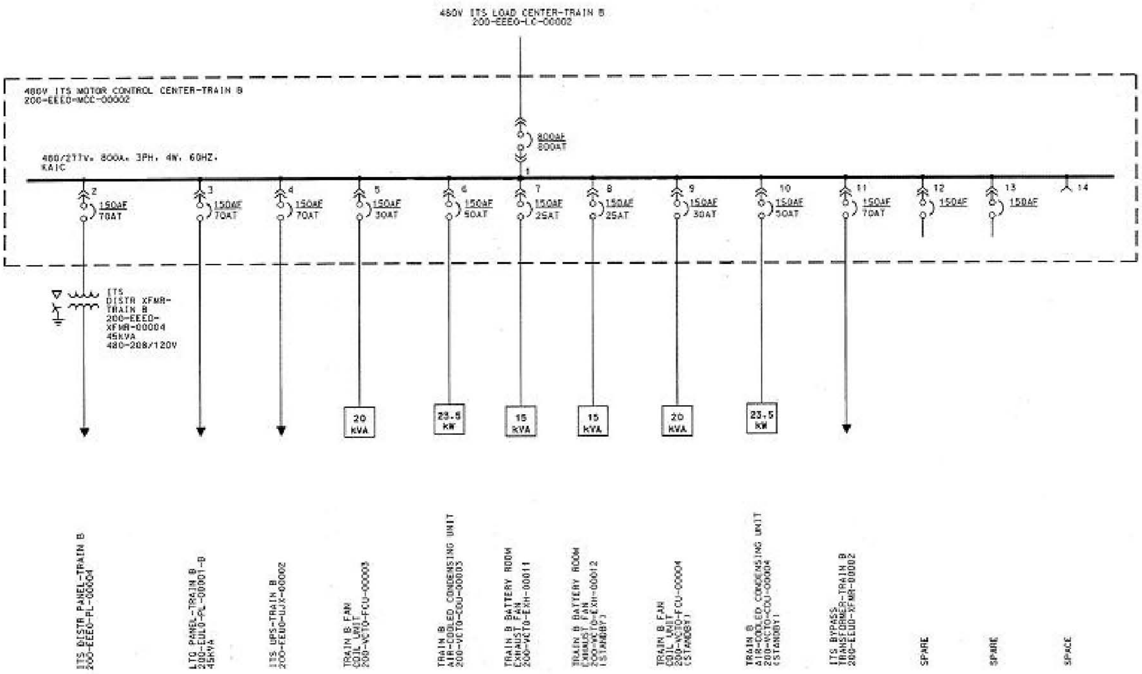
Figure B8.2-9. RF 480V ITS Load Center Train B



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.11.

Figure B8.2-10. RF 480V ITS MCC Train A



NOTE: Legibility of figure does not affect technical content of the document. Details are found in the source document.

Source: Adapted from Ref. B8.1.12.

Figure B8.2-11. RF 480V ITS MCC Train B

B8.2.2 ITS Onsite AC Power

The ITS power supply system is intended to provide back-up power to selected buildings and operations in the event of LOSP. A LOSP could result from a loss of power on the offsite power grid or a failure within the site 138kV to 13.8kV switchyard. This portion of the ITS power supply system consists of two identical divisions of diesel generator supplied AC power. The primary components in each division include a diesel generator, support systems for the diesel generator, and a load sequencer.

Both ITS diesel generators are located in the EDGF. Each is sized to provide sufficient 13.8kV power to support all of the ITS loads in one ITS switchgear (A or B) in six facilities (three CRCFs, the WHF, the RF, and the EDGF). The ITS diesel generator starts upon detection of an under voltage condition via an under voltage relay of the 13.8kV ITS switchgear. (The switchyard to switchgear feeder breaker also trips open upon detection of this under voltage condition.) Each ITS diesel generator is equipped with a complete set of support systems including HVAC systems, uninterruptible power system (UPS) and DC power systems, a fuel oil system, diesel generator start subsystem, diesel generator cooling subsystem, and lube oil subsystem that are separate and independent from the support system for the other ITS diesel generator.

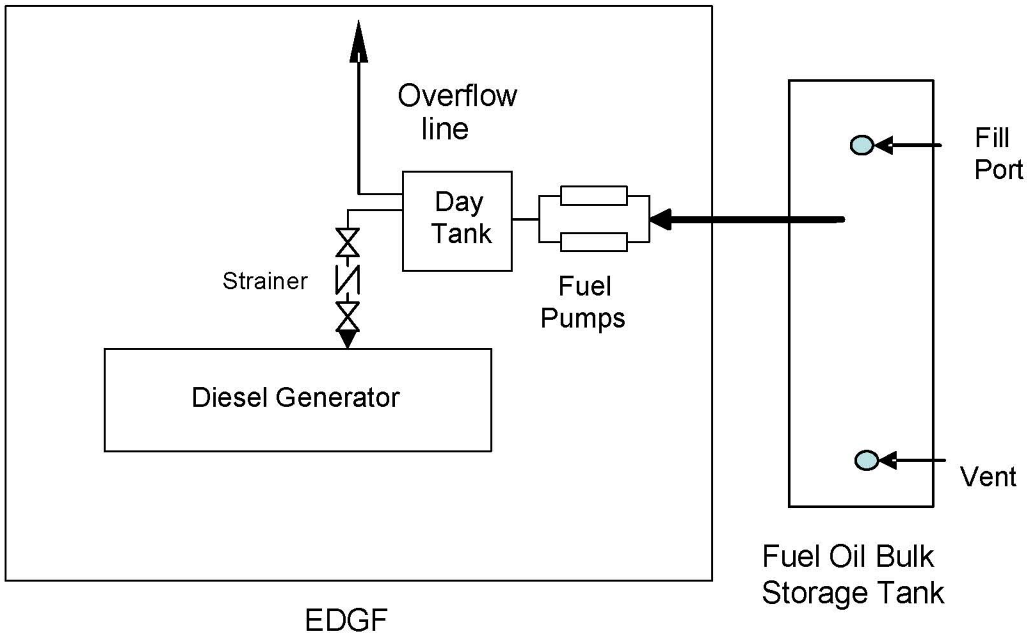
The EDGF is divided into several areas/rooms supporting the two trains of ITS AC power. Separate HVAC systems are provided for each room. The 125V DC power system (one for each ITS division) provides the necessary power to operate (open/close) the medium voltage circuit breakers on the ITS switchgears. The UPS supports the ITS diesel generator control systems. The UPS is not included in the ITS AC power model. A UPS is generally very reliable and inclusion of this support system would not noticeably impact the ITS AC power system failure probability. The HVAC for the 13.8kV ITS Switchgear Room and Battery Room for each train of the ITS power system includes an air handling unit and two exhaust fans for each battery room for both air flow and temperature control (Ref. B8.1.7). The system for each of the ITS diesel generator rooms consists of four fans, as maintaining air flow is sufficient to maintain room temperature within the ITS diesel generator operational limits. All four fans must operate to maintain an acceptable temperature within the ITS Diesel Generator Room (Ref. B8.1.4).

The 125V DC power system (one for each ITS diesel generator) provides essential power needed to start and load the diesel generator upon a LOSP. DC power for each division of the ITS power supply in the EDGF is supplied by a single battery. The battery is continuously charged through a single battery charger powered (through a transformer and the 480V ITS MCC (Ref. B8.1.1)) from the 13.8kV ITS switchgear (Figures B8.2-5 and B8.2-7).

Each ITS diesel generator fuel oil system consists primarily of a bulk storage tank, two fuel pumps, and a day tank (Figure B8.2-12). The bulk storage tank, located outside of the EDGF, has a capacity sufficient to operate the ITS diesel generator for two weeks. Each fuel pump is sized to be capable of providing sufficient makeup flow to the day tank once the level in the day tank has dropped to a one hour supply for the ITS diesel generator, and to refill the tank while the ITS diesel generator is running. The day tank, located within the EDGF, has a capacity to support four hours of ITS diesel generator operation (Ref. B8.1.3).

The lube oil subsystem, the diesel generator cooling subsystem, and the starting subsystem are considered as part of the diesel generator and their failures are not modeled as separate events in the fault trees.

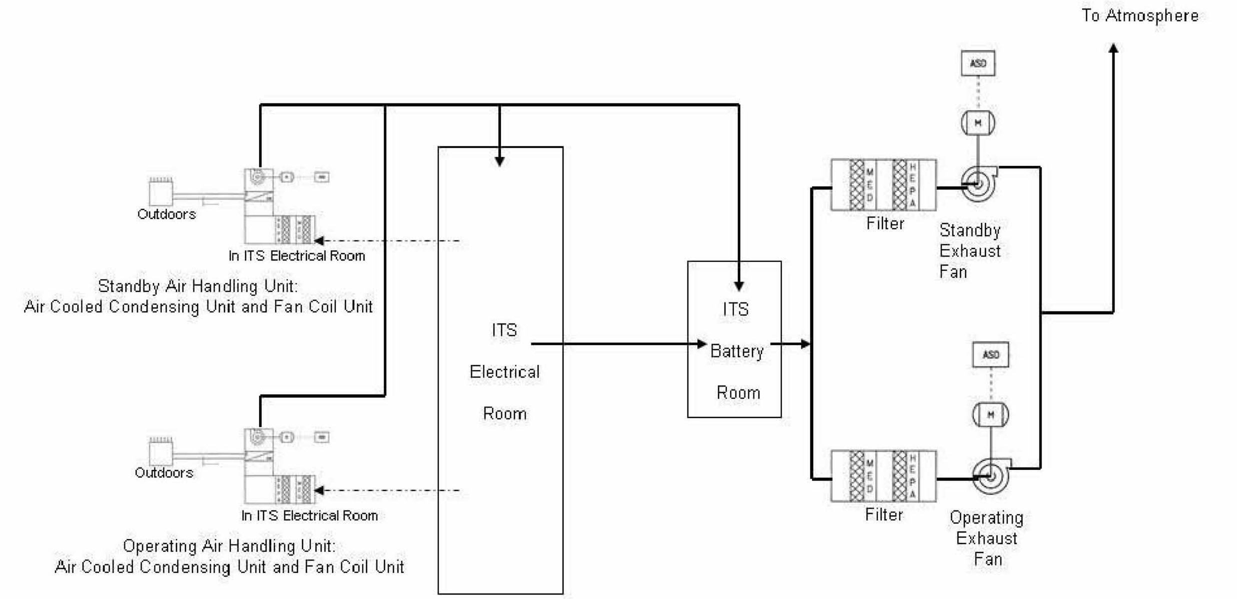
The load sequencer controls the sequence of events that occur after a LOSP and the diesel generator starts. Upon a LOSP, and after the diesel generator starts and reaches its rated capacity, the load sequencer connects the diesel generator to the 13.8kV ITS switchgear and then reconnects all division ITS loads, including the RF ITS loads.



Source: Modified from Ref. B8.1.3.

Figure B8.2-12. ITS Diesel Generator Fuel Oil System

Within the RF, ventilation and cooling for the ITS Electrical Rooms and ITS Battery Rooms is provided by a dedicated ventilation system. A separate ventilation train is provided for each train of ITS Electrical/Battery Rooms. Each train consists of two air handling units (each consisting of an air cooled condensing unit and a fan coil unit), two exhaust fans and associated ducting and instrumentation (Fig B8.2-13). Each air handling unit and exhaust fan is rated at 100% capacity. Two air handling units, one in each train (air cooled condensing units 200-VCT0-CDU-00001 and 200-VCT0-CDU-00003, and fan coil units 200-VCT0-FCU-00001 and 200-VCT0-FCU-00003) are normally operating while the second one in each train (air cooled condensing units 200-VCT0-CDU-00002 and 200-VCT0-CDU-00004, and fan coil units 200-VCT0-FCU-00002 and 200-VCT0-FCU-00004) is normally in standby. Similarly, two exhaust fans, one in each train, (exhaust fan 200-VCT0-EXH-00009 and 200-VCT0-EXH-00011) are normally operating while the second one in each train (exhaust fan 200-VCT0-EXH-00010 and 200-VCT0-EXH-00012) is normally in standby ((Ref. B8.1.15), (Ref. B8.1.13), (Ref. B8.1.16), and (Ref. B8.1.14)).



Source: Ref. B8.1.15, Ref. B8.1.13, Ref. B8.1.16, and Ref. B8.1.14.

Figure B8.2-13. Simplified Diagram of Representative Train of RF ITS Electrical and ITS Battery Rooms Ventilation System

B8.2.3 ITS AC Power Normal Operations

Under normal operating conditions, AC power is supplied from two 138kV offsite power lines. Power is passed through the 138kV – 13.8kV switchyard to the two independent 13.8kV ITS switchgears. From here, power is transmitted to two 13.8kV - 480V transformers, one supporting Train A and one supporting Train B of the RF. Power to individual ITS equipment within each facility is provided via the ITS load centers and ITS MCCs (one of each for Train A and Train B).

The AC power system is normally operating, but one division at a time may be taken out of service for maintenance. With one division out of service, only one division of the supported ITS systems can be considered to be operable.

B8.2.4 ITS AC Power Off-Normal Operations

The off-normal condition of interest for the ITS AC power system is a LOSP. During a LOSP, both ITS diesel generators are required to start and accept loads in a timely manner. Upon a

LOSP, the onsite power distribution system supporting ITS loads is disconnected from the switchyard; a circuit breaker between the 13.8kV ITS switchgear and the switchyard in each division automatically opens. Both diesel generators start automatically and are connected to the 13.8kV ITS switchgear when the connecting breaker is closed by the load sequencer. The load sequencer then reconnects the RF loads to the 13.8kV ITS switchgear. Both diesel generators continue to supply AC power until normal power is restored.

B8.2.5 ITS AC Power Testing and Maintenance

The normal AC power system is operated continuously. Maintenance is performed on an as needed basis. The diesel generators and supporting subsystems are normally in a standby mode. Routine tests are performed to ensure that the ITS diesel generator can start and load, in the event of a loss of normal power, including during a LOSP event.

Requirements

The ITS diesel generators and their associated support components (start systems, lube oil, HVAC) are tested monthly on a staggered basis.

Features

Normal maintenance is performed in accordance with manufacturer's recommendations.

Maintenance outages that remove a division of ITS AC power from operation is limited to one week.

B8.2.5.1 Fault Trees

Requirements:

The fault tree model for the ITS AC power system includes: (1) those components that have been declared as ITS, and (2) those AC power distribution system components whose failure requires the ITS AC power system to perform. The ITS power system includes components that are normally in standby (e.g., the diesel generator) and components that are normally in operation. The portions of the normal AC power distribution system modeled include the AC power distribution system from the 13.8kV ITS switchgear to the facility ITS load centers.

The mission time for the ITS AC power system is set to 720 hours. This is based on the mission time requirement for the RF HVAC system following the potential breach of a waste canister.

Features

Common-cause failures have been included for fourteen events. Six are associated with ITS diesel generator operation: two for the ITS diesel generators (failure to start or run) themselves and four for the pair of fuel pumps (failure to start and run for each pair) that support each ITS diesel generator. Three more are associated with the failure to open/close of the breakers that disconnect the 13.8kV ITS Switchgear from the normal offsite power supply, the ITS load center feed breakers, and the breakers that connect the ITS diesel generators to the 13.8kV ITS switchgear. Four are associated with the RF Confinement ITS Electrical and Battery Rooms Ventilation System: one for the failure to start and run of the system standby exhaust fans, one for the failure to run of the operating exhaust fans, one for the failure to start and run of the standby air handling units, and one for the failure to run of the operating air handling units. The final CCF event modeled is associated with the RF 13.8kV - 480V ITS transformers. Additional detail about the treatment of CCF failures can be found in Attachment C.

Four human error conditions are incorporated into the model (details are provided in Section B.8.4 of this attachment). All four address the failure to properly restore portions of the system to operable status following maintenance.

The ITS diesel generator lube oil, cooling systems, and start subsystems are considered to be part of the diesel generator and are not modeled as separate systems.

B8.3 DEPENDENCIES AND INTERACTIONS

Dependencies are broken down into five categories with respect to their interactions with structures, systems, and components. The five areas considered are addressed in Table B8.3-1 with the following dependencies:

1. Functional dependence.
2. Environmental dependence.
3. Spatial dependence.
4. Human dependence.
5. Failures based on external events.

Table B8.3-1. Dependencies and Interactions Analysis

Structures, Systems, and Components	Dependencies & Interactions				
	Functional	Environ-mental	Spatial	Human	External Events
ITS diesel generators	Start systems, load sequencer	EDGF Diesel Generator Room HVAC	—	Test and maintenance	—
13.8kV ITS Switchgear	ITS Diesel generator, RF 13.8kV – 480V ITS transformers	EDGF Switchgear Room HVAC	—	Test and maintenance	Offsite power
ITS Load Centers and MCCs	ITS Diesel generator, 13.8kV ITS switchgear	RF ITS AC Power Room Ventilation	—	Test and maintenance	Offsite power
AC load breakers	EDGF DC power system	—	—	Test and maintenance	
RF 13.8 kV to 480V ITS transformers	ITS Diesel generator, 13.8kV ITS switchgear	—	—	Test and maintenance	Offsite power

Table B8.3-1. Dependencies and Interactions Analysis (Continued)

Structures, Systems, and Components	Dependencies & Interactions				
	Functional	Environ-mental	Spatial	Human	External Events
RF ITS AC Power Room Ventilation	RF ITS MCCs	—	—	Test and maintenance	—

NOTE: AC = alternating current; EDGF = Emergency Diesel Generator Facility; HVAC = heating, ventilation, and air conditioning (filter); ITS = important to safety; kV = kilovolt; MCC = motor control centers; RF = Receipt Facility; V = volt.

Source: Original

B8.4 ITS AC POWER FAILURE SCENARIOS

For the RF the ITS AC power system has two credible failure scenarios:

1. Loss of AC power to RF ITS load center Train A. Failure to provide power to the RF ITS HVAC system Train A powered by ITS load center Train A.

2. Loss of AC power to RF ITS load center Train B. Failure to provide power to the RF ITS HVAC system Train B powered by ITS load center Train B.

B8.4.1 Loss of AC Power to RF ITS Load Center Train A

B8.4.1.1 Description

RF confinement following the potential breach of a waste canister is provided, in part, by the RF ITS HVAC system. The ITS AC power system provides the power needed to operate the ITS HVAC system equipment. This fault tree models the components that are required to provide AC power from either the normal offsite power supplies or from ITS diesel generator A to ITS load center Train A.

B8.4.1.2 Success Criteria

Success criteria for this train of the ITS AC power system is providing AC power from either the normal power system, or from the ITS diesel generator (diesel generator A) to the ITS HVAC division powered through RF ITS load center Train A. The AC power system must operate in support of the ITS HVAC system for as long as necessary to successfully provide confinement after the potential release of material from a breached canister. Therefore, the mission time (the period for which ITS AC power must be supplied to the ITS HVAC system) is the same for the ITS AC power system as it is for the ITS HVAC system, 720 hours.

B8.4.1.3 Design Requirements and Features

Requirements

Each ITS diesel generator has support systems that are independent from the support system for the other diesel generator. Independent support systems include:

- Fuel oil systems
- HVAC systems to include the ITS Diesel Generator Room and 13.8kV ITS switchgear room systems
- Lube oil system
- ITS diesel generator cooling systems
- Diesel generator start system.

Design Features

The 13.8kV ITS switchgear is isolated from the main switchyard upon a loss of power in the switchyard, either due to a LOSP or from failures within the switchyard.

The RF load is shed from the 13.8kV ITS switchgear upon a loss of power indication.

A load sequencer controls the loading of the diesel generator onto the 13.8kV ITS switchgear upon the ITS diesel generator reaching rated output. The same load sequencer controls reloading the RF loads onto the ITS AC power system.

Environmental systems are provided to maintain the temperature in the various EDGF rooms within acceptable levels. This includes a fan system for the diesel generator room and an air handling unit for the 13.8kV ITS switchgear and battery room.

B8.4.1.4 Fault Tree Model

The top event in this fault tree is "Loss of AC Power to RF ITS Load Center Train A." This is defined as a failure of normal and ITS on-site power to ITS load center train A. Faults considered in the evaluation of this top event include: failure of components in the normal AC power system, failure of the ITS diesel generator, human events that can contribute to onsite system failures resulting in a power loss at the RF and a LOSP. In this fault tree offsite power is not modeled as an initiating event, but as a system failure. The value used for this event represents the probability that offsite power is lost in the 720 hours following a possible radioactive release from a damaged canister.

B8.4.1.5 Basic Event Data

Table B8.4-1 contains a list of basic events used in the "Loss of AC Power to RF ITS Load Center Train A" fault tree. Included are component failures, maintenance errors and the human and common-cause events identified in the previous two sections. The data, for both random and common cause failures used to develop the failure probabilities associated with these basic events comes from the component reliability data analysis (Attachment C). Human reliability analyses (Attachment E) provide the probabilities for the human events.

Mission times for the various components are based on the following:

- Fault exposure time (168 hours) for events limited to one week maintenance outages (train out of service (OOS) for maintenance)

- Mission time (360 hours) for operation of standby equipment that operates after a LOSP (distribution of the occurrence of an LOSP is evenly distributed over the 720 hours after a potential radiological release, average mission time is therefore 360 hours), and average fault exposure time for standby components tested monthly.

- Mission time (720 hours) for operating components

While some of the components are normally in operation, it is possible for any of the components to be OOS for maintenance. With Train A of AC power OOS (resulting in Train A of the facility ITS HVAC being OOS), Train B provides support to an operable ITS HVAC Train B. The intent of the maintenance events modeled is for the events to address maintenance on any component in that AC power train. This is true for the components normally in operation and the standby components. The maintenance unavailability represented by the ITS load center maintenance events model the unavailability of any component from the 13.8kV ITS switchgear through the ITS load center. The maintenance unavailability represented by the ITS diesel generator maintenance events represent the unavailability of any of the components or systems that prevent the ITS diesel generator from starting and loading onto the 13.8kV ITS switchgear. As noted earlier all of the human events are associated with the failure to restore a component to operable or standby status after maintenance. The operator-related events shown in the following table are combinations events: they include the probability that the component has been taken OOS for maintenance and that site personnel have not restored the component to operable or standby status. A screening value of 0.1 has been used for the human error probability (HEP) in all cases.

Table B8.4-1. Basic Event Probability for the Loss of AC Power to RF ITS Load Center Train A Fault Tree

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-#EEE-LDCNTRA-BUA-FOH	RF ITS Load Center A Fails	3	4.391E-04	0.000E+00	6.100E-07	7.200E+02
200-#EEE-LDCNTRA-BUA-MTN	ITS Load Center Train A OOS for Maintenance	3	1.025E-04	0.000E+00	6.100E-07	1.680E+02
200-#EEE-LDCNTRA-BUA-ROE	Failure to Restore ITS Load Center Train A post maintenance	1	1.025E-05	1.025E-05	7.910E-07	1.680E+01
200-#EEE-LDCNTRA-C52-FOD	ITS Load Center A feed breaker Fails to Reclose	1	2.240E-03	2.240E-03	0.000E+00	0.000E+00
200-#EEE-LDCNTRA-C52-SPO	Load Center A Feed Circuit Breaker Spurious Operation	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02
200-#EEE-LDCNTRB-BUA-MTN	ITS Load Center Train B OOS for Maintenance	3	1.025E-04	0.000E+00	6.100E-07	1.680E+02
200-#EEE-LDCNTRB-BUA-ROE	Failure to Restore ITS Load Center Train B post maintenance	1	1.025E-05	1.025E-05	7.910E-07	1.680E+01
200-#EEE-LDCNTRS-C52-CCF	Common cause failure of the ITS Load Center feed breakers to reclose	1	1.050E-04	1.050E-04	0.000E+00	0.000E+00
200-#EEE-RFITS-A-XMR-CCF	RF ITS Transformer train A CCF	1	4.920E-06	4.920E-06	2.910E-07	3.380E+01
200-#EEE-RFITS-A-XMR-FOH	RF ITS Transformer Train A Failure	3	2.095E-04	0.000E+00	2.910E-07	7.200E+02
200-#EEE-MCC0001-C52-SPO	RF ITS MCC 0001 Feed Breaker Spurious Operation	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02
200-#EEE-MCC0001-MCC-FOH	RF ITS MCC 00001 Fails	3	5.378E-03	0.000E+00	7.490E-06	7.200E+02
200-VCT0-AHU0001-AHU-FTR	RF ITS Elec AHU 00001 Fails to run	3	2.646E-03	0.000E+00	3.680E-06	7.200E+02
200-VCT0-AHU0001-CTL-FOD	RF ITS Elec AHU 00001 Controller Fails	1	2.030E-03	2.030E-03	0.000E+00	0.000E+00
200-VCT0-AHU0002-AHU-FTR	RF ITS ELec AHU 00002 Fails to Run	3	2.646E-03	0.000E+00	3.680E-06	7.200E+02
200-VCT0-AHU0002-CTL-FOD	RF ITS Elec AHU 00002 Controller Fails	1	2.030E-03	2.030E-03	0.000E+00	0.000E+00
200-VCT0-AHU0002-FAN-FTS	RF ITS Elec AHU 00002 Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
200-VCT0-AHU0103-AHU-CCR	CCF of the running RF ITS Elec AHUs to continue to run	1	6.200E-05	6.200E-05	0.000E+00	0.000E+00
200-VCT0-AHU0202-AHU-CCR	CCF of standby RF ITS Elec AHUs to start/run	1	1.600E-04	1.600E-04	0.000E+00	0.000E+00
200-VCT0-EXH-009-CTL-FOD	RF ITS Elec Exh fan 00009 Controller Fails	1	2.030E-03	2.030E-03	0.000E+00	0.000E+00

Table B8.4-1. Basic Event Probability for the Loss of AC Power to RF ITS Load Center A Fault Tree (Continued)

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-VCT0-EXH-009-FAN-FTR	RF ITS Elec Exhaust Fan 00009 Fails to Run	3	5.059E-02	0.000E+00	7.210E-05	7.200E+02
200-VCT0-EXH-010-CTL-FOD	RF ITS Elec Exh Fan 0010 Controller Fails	1	2.030E-03	2.030E-03	0.000E+00	0.000E+00
200-VCT0-EXH-010-FAN-FTR	RF ITS Elec Exh. Fan 0010 Fails to Run	3	5.059E-02	0.000E+00	7.210E-05	7.200E+02
200-VCT0-EXH-010-FAN-FTS	RF ITS Elec Exh fan 00010 Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
200-VCT0-EXH0911-FAN-CCR	CCF of running Exh fans for RF ITS Elec.	1	1.200E-03	1.200E-03	0.000E+00	0.000E+00
200-VCT0-EXH1012-FAN-CCF	CCF to start/run: standby Exh fans for the RF ITS Elec	1	1.300E-03	1.300E-03	0.000E+00	0.000E+00
26D-##EG-DAYTNKA-TKF-FOH	ITS DG A Day Tank (00002A) Fails	3	1.584E-04	0.000E+00	4.400E-07	3.600E+02
26D-##EG-FLITLKA-IEL-FOD	ITS DG A fuel transfer pumps Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
26D-##EG-FTP1DGA-PMD-FTR	ITS DG A Fuel Transfer Pump Fails to Run	3	1.234E-02	0.000E+00	3.450E-05	3.600E+02
26D-##EG-FTP1DGA-PMD-FTS	ITS DG A Fuel Pump 1A Fails to Start	1	2.500E-03	2.500E-03	0.000E+00	0.000E+00
26D-##EG-FTP2DGA-PMD-FTR	ITS DG A Fuel Transfer Pump 2A Fails to Run	3	1.234E-02	0.000E+00	3.450E-05	3.600E+02
26D-##EG-FTP2DGA-PMD-FTS	ITS DG A Fuel Transfer pump 2A Fails to Start	1	2.500E-03	2.500E-03	0.000E+00	0.000E+00
26D-##EG-FULPMPA-PMD-CCR	Common cause failure of ITS DG A fuel pumps to run	1	2.900E-04	2.900E-04	0.000E+00	0.000E+00
26D-##EG-FULPMPA-PMD-CCS	Common cause failure of ITS DG A fuel pumps to start	1	1.200E-04	1.200E-04	0.000E+00	0.000E+00
26D-##EG-STRTDGA-C72-SPO	ITS Switchgear A Battery Circuit Breaker (DC) Spur Op	3	3.851E-04	0.000E+00	1.070E-06	3.600E+02[d]
26D-##EG-WKTNK_A-TKF-FOH	ITS DG A Bulk Fuel Tank (00001A) Fails	3	1.584E-04	0.000E+00	4.400E-07	3.600E+02
26D-##EGBATCHRGA-BYC-FOH	ITS Switchgear A Battery: Battery Charger failure	3	1.276E-03	0.000E+00	7.600E-06	1.680E+02[c]
26D-#EEE-SWGRDGA-BUA-FOH	13.8 kV ITS Switchgear A Failure	3	4.391E-04	0.000E+00	6.100E-07	7.200E+02
26D-#EEESWGRDGA-AHU-FTR	13.8 kV ITS Switchgear room Air Handling Unit Fails	3	2.646E-03	0.000E+00	3.680E-06	7.200E+02
26D-#EEG-HVACFA1-FAN-FTR	ITS DG A room Fan 1 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02

Table B8.4-1. Basic Event Probability for the Loss of AC Power to RF ITS Load Center A Fault Tree (Continued)

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
26D-#EEG-HVACFA1-FAN-FTS	ITS DG A room Fan 1 (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-#EEG-HVACFA2-FAN-FTR	ITS DG A room Fan 2 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-#EEG-HVACFA2-FAN-FTS	ITS DG A room Fan 2 (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-#EEG-HVACFA3-FAN-FTR	ITS DG A room Fan 3 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-#EEG-HVACFA3-FAN-FTS	ITS DG A room Fan 3 (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-#EEG-HVACFA4-FAN-FTR	ITS DG A room Fan 4 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-#EEG-HVACFA4-FAN-FTS	ITS DG A room Fan 4 (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-#EEU-208_DGA-BUD-FOH	ITS DC Panel A DC Bus Failure	3	8.640E-05	0.000E+00	2.400E-07	3.600E+02[d]
26D-#EEY-DGALOAD-C52-FOD	ITS DG A Load Breaker (AC) Fails to Close	1	2.240E-03	2.240E-03	0.000E+00	0.000E+00
26D-#EEY-DGLOADS-C52-CCF	Common cause failure of ITS DG Load Breakers to close	1	1.050E-04	1.050E-04	0.000E+00	0.000E+00
26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	3	7.698E-01	0.000E+00	4.080E-03	3.600E+02
26D-#EEY-ITSDG-A-#DG-FTS	Diesel Generator Fails to Start	1	8.380E-03	8.380E-03	0.000E+00	0.000E+00
26D-#EEY-ITSDG-A-#DG-MTN	ITS DG A OOS Maintenance	1	1.950E-03	1.950E-03	0.000E+00	0.000E+00
26D-#EEY-ITSDG-A-#DG-RSS	Failure to properly return ITS DG A to service	1	1.950E-04	1.950E-04	0.000E+00	0.000E+00
26D-#EEY-ITSDG-B-#DG-MTN	ITS DG B OOS Maintenance	1	1.950E-03	1.950E-03	0.000E+00	0.000E+00
26D-#EEY-ITSDG-B-#DG-RSS	Failure to properly restore ITS DG-B to service	1	1.950E-04	1.950E-04	0.000E+00	0.000E+00
26D-#EEY-ITSDGAB-#DG-CCR	CCF ITS DG A & B Fail to Run	1	1.800E-02	1.800E-02	0.000E+00	0.000E+00
26D-#EEY-ITSDGAB-#DG-CCS	CCF DG A and B to Start	1	3.900E-04	3.900E-04	0.000E+00	0.000E+00
26D-#EEY-OB-SWGA-C52-FOD	13.8 kV ITS SWGR feed breaker (AC) Fails to open	1	2.240E-03	2.240E-03	0.000E+00	0.000E+00
26D-#EEY-OB-SWGA-C52-SPO	13.8 kV ITS SWGR A feed Breaker Spurious Operation	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02

Table B8.4-1. Basic Event Probability for the Loss of AC Power to RF ITS Load Center A Fault Tree (Continued)

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
26D-#EEY-OB-SWGS-C52-CCF	Common cause failure of 13.8kV ITS SWGR feed breakers to open	1	1.040E-04	1.040E-04	0.000E+00	0.000E+00
26D-#EG-LCKOUTRL-RLY-FTP	13.8 kV ITS Switchgear Feed breaker lock out relay fails to Open CB	3	3.152E-03	0.000E+00	8.770E-06	3.600E+02
26D-#EGLDSQNCRA-SEQ-FOD	DG A Load Sequencer Fails	1	2.670E-03	2.670E-03	0.000E+00	0.000E+00
26D-EG-BATTERYA-BTR-FOD	ITS Switchgear A Battery No Output Given Challenge	1	8.200E-03	8.200E-03	0.000E+00	0.000E+00
27A-#EEE-BUS2DGA-C52-SPO	13.8 kV Open Bus 2 ITS Load Breaker Spurious Operation	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02
27A-#EEN-OPENBS2-BUA-FOH	13.8 kV Open Bus 2 Bus Failure	3	4.391E-04	0.000E+00	6.100E-07	7.200E+02
27A-#EEN-OPNBS1A-SWP-SPO	13.8 kV Open Bus 2 to ITS Div A Electric Power Switch Spur. Xfer	3	1.116E-04	0.000E+00	1.550E-07	7.200E+02
LOSP*	Loss of offsite power	1	2.990E-03	2.990E-03	0.000E+00	0.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
[b] The designation of a circuit breaker as AC or DC refers to the system designation for the circuit breaker, it is not representative of the motive power for the circuit breaker.
[c] The failure of the battery charger would result in eventual depletion of the battery and a low power indication on both the battery and the DC bus. The 168 hr mission time was selected as a conservative estimation for the detection time of this failure.
[d] The mission times for the DC bus related failure rates do not take credit for any monitoring of bus status, which would provide nearly instantaneous indication of a bus failure or loss of power to the bus. The standby component mission time was used conservatively.
LOSP* represents the probability of losing offsite power during the 720 hours HVAC is required after any breach of a container releases radioactive material. It is based on a Loss of offsite power frequency of 3.59E-02/year from NUREG/CR6890 (Ref. B8.1.19).
AC = alternating current; AHU = air handling unit; Calc. = calculation; CCF = common-cause failure; DC = direct current; DG = diesel generator; Div = division; elec = electrical EXH = exhaust; ITS = important to safety; kV = kilovolt; Miss. = mission; OOS = out of service; op = operation; Prob. = probability; Spur. = spurious; SWGR = switchgear; Xfer = transfer.

Source: Original

B8.4.1.5.1 Human Failure Events

Four basic HFEs (Table B8.4-2) are associated with human error. All of the HFEs are associated with the failure to properly restore components to operable status following maintenance. The first two shown in Table B8.4-2 are associated with the failure to restore the normal power supply to the RF ITS Load Centers after maintenance. The last two are representative of the failure to restore the ITS diesel generators (and any other components that prevent the ITS diesel generator from starting or loading) to service after maintenance. These events are combination events consisting of the probability that a component was removed for maintenance and the failure of plant operators (assigned a screening value of 0.1) to restore the component after maintenance.

Table B8.4-2. Human Failure Events

Name	Description
200-#EEE-LDCNTRA-BUA-ROE	Failure to restore ITS load center train A post maintenance
200-#EEE-LDCNTRB-BUA-ROE	Failure to restore ITS load center train B post maintenance
26D-#EEY-ITSDG-A-#DG-RSS	Failure to properly return ITS DG A to service
26D-#EEY-ITSDG-B-#DG-RSS	Failure to properly return ITS DG-B to service

NOTE: DG = diesel generator; ITS = Important to Safety.

Source: Original

B8.4.1.5.2 Common-Cause Failures

Twelve of the fourteen CCFs identified earlier (Section B8.2.5.1.2) have been included in the analysis of the loss of ITS AC power to the ITS load center Train A. Ten of the CCF events affect both trains of ITS AC Power. Two affect only this train of the system. The remaining two affect only the other train of the system. Two are associated with the ITS diesel generators: CCF of the ITS diesel generators to start and the ITS diesel generators to run. The CCF of the ITS diesel generator fuel oil system incorporates two CCFs: CCF of the two fuel oil pumps to start and the CCF of the pumps to run. Three circuit breaker CCF events were considered. These are the CCF of the (1) 13.8kV ITS switchgear feed breakers (from 13.8kV open buses) to open on loss of offsite power, (2) ITS diesel generator load breakers to close when commanded by the load sequencer and (3) ITS load center feed breakers to close when commanded by the load sequencer. Four CCFs are associated with the RF ITS Electrical and Battery Rooms' ventilation system, two for the CCF of exhaust fans to start and run, and two for the CCF of the air handling units to start and run. The last CCF event considered is the CCF of the 13.8kV – 480V ITS transformers.

Table B8.4-3. Common-Cause Basic Events

Name	Description	Alpha-factor
200-#EEE-RFITS-A-XMR-CCF	RF ITS Transformers CCF	0.0235
200-#EEE-LDCNTRS-C52-CCF	CCF of the ITS Load Center feed breakers to reclose	0.047
26D-##EG-FULPMPA-PMD-CCR	CCF of ITS DG A fuel pumps to run	0.0235
26D-##EG-FULPMPA-PMD-CCS	CCF of ITS DG A fuel pumps to start	0.047
26D-#EEY-DGLOADS-C52-CCF	CCF of ITS DG Load Breakers to close	0.047
26D-#EEY-ITSDGAB-#DG-CCR	CCF ITS DG A & B Fail to Run	0.0235
26D-#EEY-ITSDGAB-#DG-CCS	CCF DG A and B to Start	0.047
26D-#EEY-OB-SWGS-C52-CCF	CCF of 13.8kV ITS SWGR feed breakers to open	0.047
200-VCT0-AHU0103-AHU-CCR	CCF of the running RF ITS Elec AHUs to continue to run	0.0235
200-VCT0-AHU0202-AHU-CCR	CCF of standby RF ITS Elec AHUs to start/run	0.047 start 0.0235 run
200-VCT0-EXH0911-FAN-CCR	CCF of running Exh fans for RF ITS Elec.	0.0235
200-VCT0-EXH1012-FAN-CCF	CCF to start/run: standby Exh fans for the RF ITS Elec	0.047 start 0.0235 run

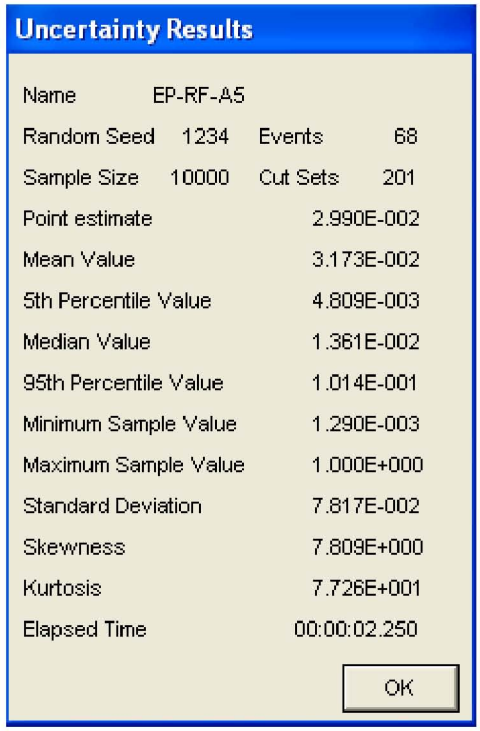
NOTE: AHU = air handling unit; CCF = common-cause failure, CRCF = Canister Receipt and Closure Facility; DG = diesel generator; elec = electrical; Exh = exhaust; ITS = important to safety; RF = Receipt Facility; SWGR = switch gear.

Source: Original

All of the common cause failures modeled are used on pairs of components with one of two success criteria (i.e., two of two failure criteria). Alpha-factors used to determine the common cause failure probability are 0.047 for demand failures and 0.0235 for time dependent failures (Table C3-1, CCCG=2, and the associated text). Two common cause failures in Table B8.4-3 are used to represent the common cause failure associated with the failure to start and failure to run for components. For these two common cause failures, the appropriate alpha-factors were applied to the start and run portions of the random failure probability to develop a single common cause failure probability for the components.

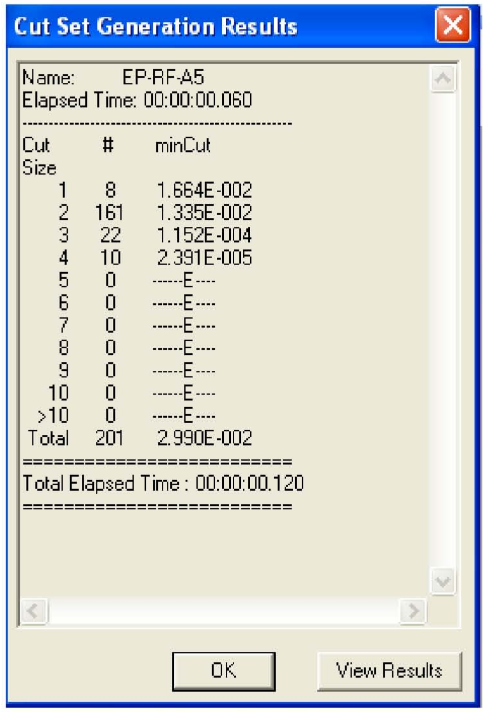
B8.4.1.6 Uncertainty and Cut Set Generation

Figure B8.4-1 contains the uncertainty results obtained from running the fault trees for the "Loss of AC Power to RF ITS Load Center Train A". Figure B8.4-2 provides the cut set generation results for the "Loss of AC Power to RF ITS Load Center Train A" fault tree.



Source: Original

Figure B8.4-1. Uncertainty Results of the Loss of AC Power to RF ITS Load Center Train A Fault Tree



Source: Original

Figure B8.4-2. Cut Set Generation Results for the Loss of AC Power to RF Load Center Train A Fault Tree

B8.4.1.7 Cut Sets

Table B8.4-4 contains the top 25 cut sets accounting for 97% of the system failure probability for the "Loss of AC Power to RF ITS Load Center Train A" fault tree.

Table B8.4-4. Dominant Cut Sets for the Loss of AC Power to RF ITS Load Center Train A

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
17.99	17.99	5.378E-03	200-#EEE-MCC0001-MCC-FOH	RF ITS MCC 00001 Fails	5.378E-03
30.75	12.76	3.816E-03	200-#EEE-LDCNTRA-C52-SPO	Load Center A Feed Circuit Breaker Spurious Operation	3.816E-03
43.51	12.76	3.816E-03	200-#EEE-MCC0001-C52-SPO	RF ITS MCC 0001 Feed Breaker Spurious Operation	3.816E-03
53.33	9.82	2.937E-03	26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.698E-01
			26D-#EEY-OB-SWGA-C52-SPO	13.8 kV ITS SWGR A feed Breaker Spurious Operation	3.816E-03
63.15	9.82	2.937E-03	26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.698E-01
			27A-#EEE-BUS2DGA-C52-SPO	13.8 kV Open Bus 2 ITS Load Breaker Spurious Operation	3.816E-03

Table B8.4-4. Dominant Cut Sets for The Loss of AC Power to RF ITS Load Center Train A (Continued)

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
72.00	8.85	2.646E-03	26D-#EEESWGRDGA-AHU-FTR	13.8 kV ITS Switchgear room Air Handling Unit Fails	2.646E-03
80.56	8.56	2.559E-03	200-VCT0-EXH-009-FAN-FTR	RF ITS Elec Exhaust Fan 00005 Fails to Run	5.059E-02
			200-VCT0-EXH-010-FAN-FTR	RF ITS Elec Exh. Fan 0010 Fails to Run	5.059E-02
88.26	7.70	2.302E-03	26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.698E-01
			LOSP	Loss of offsite power	2.990E-03
89.73	1.47	4.391E-04	200-#EEE-LDCNTRA-BUA-FOH	RF ITS Load Center A Fails	4.391E-04
91.20	1.47	4.391E-04	26D-#EEE-SWGRDGA-BUA-FOH	13.8 kV ITS Switchgear A Failure	4.391E-04
92.33	1.13	3.380E-04	26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.698E-01
			27A-#EEN-OPENBS2-BUA-FOH	13.8 kV Open Bus 2 Bus Failure	4.391E-04
93.03	0.70	2.095E-04	200-#EEE-RFITS-A-XMR-FOH	RF ITS Transformer Train B Failure	2.095E-04
93.37	0.34	1.027E-04	200-VCT0-EXH-009-CTL-FOD	RF ITS Elec Exh fan 00009 Controller Fails	2.030E-03
			200-VCT0-EXH-010-FAN-FTR	RF ITS Elec Exh. Fan 0010 Fails to Run	5.059E-02
93.71	0.34	1.027E-04	200-VCT0-EXH-009-FAN-FTR	RF ITS Elec Exhaust Fan 00009 Fails to Run	5.059E-02
			200-VCT0-EXH-010-CTL-FOD	RF ITS Elec Exh Fan 0006 Controller Fails	2.030E-03
94.05	0.34	1.025E-04	200-#EEE-LDCNTRA-BUA-MTN	ITS Load Center Train A OOS for Maintenance	1.025E-04
			/200-#EEE-LDCNTRB-BUA-MTN	ITS Load Center Train B OOS for Maintenance	9.999E-01
			/200-#EEE-LDCNTRB-BUA-ROE	Failure to Restore ITS Load Center Train B post maintenance	1.000E+000
94.39	0.34	1.022E-04	200-VCT0-EXH-009-FAN-FTR	RF ITS Elec Exhaust Fan 00005 Fails to Run	5.059E-02
			200-VCT0-EXH-010-FAN-FTS	RF ITS Elec Exh fan 00010 Fails to Start	2.020E-03
94.72	0.33	9.777E-05	26D-#EEG-HVACFA1-FAN-FTR	ITS DG A room Fan 1 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGA-C52-SPO	13.8 kV ITS SWGR A feed Breaker Spurious Operation	3.816E-03
95.05	0.33	9.777E-05	26D-#EEG-HVACFA2-FAN-FTR	ITS DG A room Fan 2 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGA-C52-SPO	13.8 kV ITS SWGR A feed Breaker Spurious Operation	3.816E-03
95.38	0.33	9.777E-05	26D-#EEG-HVACFA3-FAN-FTR	ITS DG A room Fan 3 (Motor-Driven) Fails to Run	2.562E-02

Table B8.4-4. Dominant Cut Sets for The Loss of AC Power to RF ITS Load Center Train A (Continued)

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
			26D-#EEY-OB-SWGA-C52-SPO	13.8 kV ITS SWGR A feed Breaker Spurious Operation	3.816E-03
95.71	0.33	9.777E-05	26D-#EEG-HVACFA4-FAN-FTR	ITS DG A room Fan 4 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGA-C52-SPO	13.8 kV ITS SWGR A feed Breaker Spurious Operation	3.816E-03
96.04	0.33	9.777E-05	26D-#EEG-HVACFA1-FAN-FTR	ITS DG A room Fan 1 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS2DGA-C52-SPO	13.8 kV Open Bus 2 ITS Load Breaker Spurious Operation	3.816E-03
96.37	0.33	9.777E-05	26D-#EEG-HVACFA2-FAN-FTR	ITS DG A room Fan 2 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS2DGA-C52-SPO	13.8 kV Open Bus 2 ITS Load Breaker Spurious Operation	3.816E-03
96.70	0.33	9.777E-05	26D-#EEG-HVACFA3-FAN-FTR	ITS DG A room Fan 3 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS2DGA-C52-SPO	13.8 kV Open Bus 2 ITS Load Breaker Spurious Operation	3.816E-03
97.03	0.33	9.777E-05	26D-#EEG-HVACFA4-FAN-FTR	ITS DG A room Fan 4 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS2DGA-C52-SPO	13.8 kV Open Bus 2 ITS Load Breaker Spurious Operation	3.816E-03
97.32	0.29	8.590E-05	26D-#EEY-ITSDG-A-#DG-FTR	ITS Diesel Generator A Fails to Run	7.698E-01
			27A-#EEN-OPNBS1A-SWP-SPO	13.8 kV Open Bus 2 to ITS Div A Electric Power Switch Spur. Xfer	1.116E-04

NOTE: AHU = air handling unit; CCF = common-cause failure, CRCF = Canister Receipt and Closure Facility; DG = diesel generator; elec = electrical; Exh = exhaust; ITS = important to safety; kV = kilo volt; MCC = motor control center; RF = Receipt Facility; SWGR = switch gear.

Source: Original

B8.4.2 Loss of AC Power to RF ITS Load Center Train B

B8.4.2.1 Description

RF confinement following the potential breach of a waste canister is provided, in part, by the RF ITS HVAC system. The ITS AC power system provides the AC power needed to operate the ITS HVAC system equipment. This fault tree models the components that are required to provide AC power from either the normal offsite power supplies or from ITS diesel generator B to ITS load center Train B.

B8.4.2.2 Success Criteria

The success criteria for this train of the ITS AC power system is to provide AC power from either the normal power system or from the ITS diesel generator (diesel generator B) to the ITS HVAC division powered through RF load center Train B. The AC power system must operate in support of the ITS HVAC system for as long as necessary to successfully provide confinement after the potential release of material from a breached canister. Therefore, the mission time (the period for which AC power must be supplied to the ITS HVAC system) is the same for the ITS AC power system as it is for the ITS HVAC system, 720 hours.

B8.4.2.3 Design Requirements and Features

Requirements

Each ITS diesel generator has support systems that are independent from the support system for the other diesel generator. Independent support systems include:

- Fuel oil systems
- HVAC systems to include the ITS diesel generator room and 13.8kV ITS switchgear room systems
- Lube oil system
- ITS diesel generator cooling systems
- Diesel generator start system.

Features

The 13.8kV ITS switchgear is isolated from the main switchyard upon a loss of power in the switchyard, either due to a LOSP or from failures within the switchyard.

The RF load is shed from the 13.8kV Switchgear upon a loss of power indication.

A load sequencer controls the loading of the diesel generator onto the 13.8kV ITS switchgear upon the ITS diesel generator reaching rated output. The same load sequencer controls reloading the RF loads onto the ITS AC power system.

Environmental systems are provided to maintain the temperature in the various EDGF rooms within acceptable levels. This includes a fan system for the diesel generator room and air handling units for the 13.8kV ITS switchgear and battery room.

B8.4.2.4 Fault Tree Model

The top event in this fault tree is “Loss of AC Power to RF ITS Load Center Train B.” This is defined as a failure of the normal and ITS onsite power supplies to provide power to ITS load center B. Faults considered in the evaluation of this top event include: failure of components in the normal AC power system, failure of the ITS diesel generator subsystem, human events that can contribute to onsite system failures resulting in a power loss at the RF and a LOSP. In this fault tree offsite power is not modeled as an initiating event, but as a system failure. The value used for this event represents the probability that offsite power is lost in the 720 hours following a possible radioactive release from a damaged canister.

B8.4.2.5 Basic Event Data

Table B8.4-5 contains a list of basic events used in the "Loss of AC Power to RF ITS Load Center Train B" fault tree. Included are component failures, maintenance errors and the human events and the common-cause events identified in the previous two sections. The data, for both random and CCFs used to develop the failure probabilities associated with these basic events comes from the component reliability data analysis (Attachment C). Human reliability analyses (Attachment E) provide the probabilities for the human events.

Mission times for the various components are based on the following:

- Fault exposure time (168 hours) for events limited to one week maintenance outages (train OOS for maintenance)

- Mission time (360 hours) for operation of standby equipment that would operate after a LOSP. Distribution of the occurrence of an LOSP is evenly distributed over the 720 hours after a potential radiological release; average mission time is therefore 360 hours. Average fault exposure time for standby components tested monthly.

- Mission time (720 hours) for operating components

While some of the components are normally in operation, it is possible for any of the components to be OOS for maintenance. With train A of AC power OOS (resulting in Train B of the facility ITS HVAC being OOS) Train A provides support to an operable ITS HVAC Train A. The intent of the maintenance events modeled is for the events to address maintenance on any component in that AC power division. This is true for the components normally in operation and the standby components. The maintenance unavailability represented by the ITS load center maintenance events model the unavailability of any component from the 13.8kV ITS Switchgear through the ITS load center. The maintenance unavailability represented by the ITS diesel generator maintenance events represent the unavailability of any of the components or systems that prevents the ITS diesel generator from starting and loading onto the 13.8kV ITS switchgear. As noted earlier, all of the human events are associated with the failure to restore a component to operable or standby status after maintenance. The operator-related events shown in the following table are combination events: they include the probability that the component has been taken OOS for maintenance and that site personnel have not restored the component to operable or standby status. A screening value of 0.1 has been used for the HEP in all cases.

Table B8.4-5. Basic Event Probability for the Loss of AC Power to RF ITS Load Center Train B Fault Trees

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-#EEE-LDCNTRA-BUA-MTN	ITS Load Center Train A OOS for Maintenance	3	1.025E-04	0.000E+00	6.100E-07	1.680E+02
200-#EEE-LDCNTRA-BUA-ROE	Failure to Restore ITS Load Center Train A post maintenance	1	1.025E-05	1.025E-05	7.910E-07	1.680E+01
200-#EEE-LDCNTRB-BUA-FOH	RF ITS Load Center B Fails	3	4.391E-04	0.000E+00	6.100E-07	7.200E+02
200-#EEE-LDCNTRB-BUA-MTN	ITS Load Center Train B OOS for Maintenance	3	1.025E-04	0.000E+00	6.100E-07	1.680E+02
200-#EEE-LDCNTRB-BUA-ROE	Failure to Restore ITS Load Center Train B post maintenance	1	1.025E-05	1.025E-05	7.910E-07	1.680E+01
200-#EEE-LDCNTRB-C52-FOD	13.8 ITS SWGR to RF LC B Circuit Breaker Fails on Demand	1	2.240E-03	2.240E-03	0.000E+00	0.000E+00
200-#EEE-LDCNTRB-C52-SPO	RF Load Center Circuit Breaker (AC) Spur Op	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02
200-#EEE-LDCNTRS-C52-CCF	Common cause failure of the ITS Load Center feed breakers to reclose	1	1.050E-04	1.050E-04	0.000E+00	0.000E+00
200-#EEE-RFITS-A-XMR-CCF	RF ITS Transformer trains CCF	1	4.920E-06	4.920E-06	2.910E-07	3.380E+01
200-#EEE-RFITS-B-XMR-FOH	RF ITS Transformer Train B Failure	3	2.095E-04	0.000E+00	2.910E-07	7.200E+02
200-#EEE-MCC0002-C52-SPO	RF MCC-00002 Feed Breaker Spurious Operation	3	3.816E003	0.000E+00	5.310E006	7.200E+02
200-#EEE-MCC0002-MCC-FOH	RF ITS MCC00002 Failure	3	5.378E003	0.000E+00	7.490E006	7.200E+02
200-VCT0-AHU0003-AHU-FTR	RF ITS Elec AHU 00003 Fails to run	3	2.646E003	0.000E+00	3.680E006	7.200E+02
200-VCT0-AHU0003-CTL-FOD	RF ITS Elec AHU 00003 Controller Fails	1	2.030E003	2.030E003	0.000E+00	0.000E+00
200-VCT0-AHU0004-AHU-FTR	RF ITS ELec AHU 00004 Fails to Run	3	2.646E003	0.000E+00	3.680E006	7.200E+02
200-VCT0-AHU0004-CTL-FOD	RF ITS Elec AHU 00004 Controller Fails	1	2.030E003	2.030E003	0.000E+00	0.000E+00
200-VCT0-AHU0004-FAN-FTS	RF ITS Elec AHU 00004 Fails to Start	1	2.020E003	2.020E003	0.000E+00	0.000E+00
200-VCT0-AHU0103-AHU-CCR	CCF of the running RF ITS Elec AHUs to continue to run	1	6.200E005	6.200E005	0.000E+00	0.000E+00
200-VCT0-AHU0202-AHU-CCR	CCF of standby RF ITS Elec AHUs to start/run	1	1.600E004	1.600E004	0.000E+00	0.000E+00
200-VCT0-EXH-011-CTL-FOD	RF ITS Elec Exh fan 00011 Controller Fails	1	2.030E003	2.030E003	0.000E+00	0.000E+00
200-VCT0-EXH-011-FAN-FTR	RF ITS Elec Exhaust Fan 00011 Fails to Run	3	5.059E002	0.000E+00	7.210E005	7.200E+02
200-VCT0-EXH-012-CTL-FOD	RF ITS Elec Exh Fan 0012 Controller Fails	1	2.030E003	2.030E003	0.000E+00	0.000E+00
200-VCT0-EXH-012-FAN-FTR	RF ITS Elec. Exh Fan 00012 Fails to Run	3	5.059E002	0.000E+00	7.210E005	7.200E+02

Table B8.4-5. Basic Event Probability for The Loss of AC Power to RF ITS Load Center Train B Fault Trees (Continued)

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
200-VCT0-EXH-012-FAN-FTS	RF ITS Elec Exh fan 00012 Fails to Start	1	2.020E003	2.020E003	0.000E+00	7.200E+02
200-VCT0-EXH0911-FAN-CCR	CCF of running Exh fans for RF ITS Elec.	1	1.200E003	1.200E003	0.000E+00	0.000E+00
200-VCT0-EXH1012-FAN-CCF	CCF to start/run: standby Exh fans for the RF ITS Elec	1	1.300E003	1.300E003	0.000E+00	0.000E+00
26D-##EG-DAYTNKB-TKF-FOH	ITS DG B Day fuel tank fails	3	1.584E-04	0.000E+00	4.400E-07	3.600E+02
26D-##EG-FLITLKB-IEL-FOD	ITS DG B fuel transfer pumps Interlock Failure	1	2.750E-05	2.750E-05	0.000E+00	0.000E+00
26D-##EG-FTP1DGB-PMD-FTR	ITS DG B Fuel Transfer Pump 1 (Motor Driven) Fails to Run	3	1.234E-02	0.000E+00	3.450E-05	3.600E+02
26D-##EG-FTP1DGB-PMD-FTS	ITS DG B Fuel Transfer Pump 1 (Motor Driven) Fails to Start	1	2.500E-03	2.500E-03	0.000E+00	0.000E+00
26D-##EG-FTP2DGB-PMD-FTR	ITS DG B Fuel Transfer Pump 2 (Motor Driven) Fails to Run	3	1.234E-02	0.000E+00	3.450E-05	3.600E+02
26D-##EG-FTP2DGB-PMD-FTS	ITS DG B Fuel Transfer Pump 2 (Motor Driven) Fails to Start on Demand	1	2.500E-03	2.500E-03	0.000E+00	0.000E+00
26D-##EG-FULPMPB-PMD-CCR	Common cause failure of ITS DG B fuel pumps to run	1	2.900E-04	2.900E-04	0.000E+00	0.000E+00
26D-##EG-FULPMPB-PMD-CCS	Common cause failure of ITS DG B fuel pumps to start	1	1.200E-04	1.200E-04	0.000E+00	0.000E+00
26D-##EG-HVACFN1-FAN-FTR	ITS DG B room Fan 1 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-##EG-HVACFN1-FAN-FTS	ITS DG B room Fan (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-##EG-HVACFN2-FAN-FTR	ITS DG B room Fan 2 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-##EG-HVACFN2-FAN-FTS	ITS DG B Room Fan (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-##EG-HVACFN3-FAN-FTR	ITS DG B room Fan 3 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-##EG-HVACFN3-FAN-FTS	ITS DG B Room Fan 3 (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-##EG-HVACFN4-FAN-FTR	ITS DG B Fan 4 (Motor-Driven) Fails to Run	3	2.562E-02	0.000E+00	7.210E-05	3.600E+02
26D-##EG-HVACFN4-FAN-FTS	ITS DG B Room Fan 4 (Motor-Driven) Fails to Start	1	2.020E-03	2.020E-03	0.000E+00	0.000E+00
26D-##EG-STRTDGB-C72-SPO	13.8 kV ITS SWGR Battery B Circuit Breaker (DC) Spur Op	3	3.851E-04	0.000E+00	1.070E-06	3.600E+02[d]

Table B8.4-5. Basic Event Probability for The Loss of AC Power to RF ITS Load Center Train B Fault Trees (Continued)

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
26D-##EG-WKTNK_B-TKF-FOH	ITS DG B Bulk Fuel Tank Fails	3	1.584E-04	0.000E+00	4.400E-07	3.600E+02
26D-##EGBATCHRGB-BYC-FOH	ITS DG B Battery Charger failure	3	1.276E-03	0.000E+00	7.600E-06	1.680E+02[c]
26D-#EEE-SWGRDGB-AHU-FTR	EDGF Switchgear Room Air Handling Unit Failure to Run	3	2.646E-03	0.000E+00	3.680E-06	7.200E+02
26D-#EEE-SWGRDGB-BUA-FOH	13.8 kV ITS Switchgear B Bus Failure	3	4.391E-04	0.000E+00	6.100E-07	7.200E+02
26D-#EEU-208_DGB-BUD-FOH	DC Bus Failure	3	8.640E-05	0.000E+00	2.400E-07	3.600E+02[d]
26D-#EEY-DGBLOAD-C52-FOD	ITS DG B Load Breaker Fails to Close	1	2.240E-03	2.240E-03	0.000E+00	0.000E+00
26D-#EEY-DGLOADS-C52-CCF	Common cause failure of ITS DG Load Breakers to close	1	1.050E-04	1.050E-04	0.000E+00	0.000E+00
26D-#EEY-ITS-DGB-#DG-FTS	Diesel Generator Fails to Start	1	8.380E-03	8.380E-03	0.000E+00	0.000E+00
26D-#EEY-ITSDG-A-#DG-MTN	ITS DG A OOS Maintenance	1	1.950E-03	1.950E-03	0.000E+00	0.000E+00
26D-#EEY-ITSDG-A-#DG-RSS	Failure to properly return ITS DG A to service	1	1.950E-04	1.950E-04	0.000E+00	0.000E+00
26D-#EEY-ITSDG-B-#DG-MTN	ITS DG B OOS Maintenance	1	1.950E-03	1.950E-03	0.000E+00	0.000E+00
26D-#EEY-ITSDG-B-#DG-RSS	Failure to properly restore ITS DG-B to service	1	1.950E-04	1.950E-04	0.000E+00	0.000E+00
26D-#EEY-ITSDGAB-#DG-CCR	CCF ITS DG A & B Fail to Run	1	1.800E-02	1.800E-02	0.000E+00	0.000E+00
26D-#EEY-ITSDGAB-#DG-CCS	CCF DG A and B to Start	1	3.900E-04	3.900E-04	0.000E+00	0.000E+00
26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	3	7.698E-01	0.000E+00	4.080E-03	3.600E+02
26D-#EEY-OB-SWGB-C52-FOD	Circuit Breaker (AC) Fails to open	1	2.240E-03	2.240E-03	0.000E+00	0.000E+00
26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02
26D-#EEY-OB-SWGS-C52-CCF	Common cause failure of 13.8kV ITS SWGR feed breakers to open	1	1.040E-04	1.040E-04	0.000E+00	0.000E+00
26D-#EG-BATTERYB-BTR-FOD	ITS SWGR Control Battery B No Output	1	8.200E-03	8.200E-03	0.000E+00	0.000E+00
26D-#EG-LDSQNCRB-SEQ-FOD	ITS DG B load sequencer fails	1	2.670E-03	2.670E-03	2.670E-03	0.000E+00
26D-#EG-LOCKOUTB-RLY-FTP	13.8 ITS SWGR Lockout Relay (Power) Fails to Open CB	3	3.152E-03	0.000E+00	8.770E-06	3.600E+02
27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3	3.816E-03	0.000E+00	5.310E-06	7.200E+02

Table B8.4-5. Basic Event Probability for The Loss of AC Power to RF ITS Load Center Train B Fault Trees (Continued)

Name	Description[b]	Calc. Type[a]	Calc. Prob.	Fail. Prob.	Lambda	Miss. Time[a]
27A-#EEN-OPENBS4-BUA-FOH	13.8 kV Open Bus 4 Bus Failure	3	4.391E-04	0.000E+00	6.100E-07	7.200E+02
27A-#EEN-OPNBS3B-SWP-SPO	13.8 kV Open Bus 4 to ITS B Electric Power Switch Spur Xfer	3	1.116E-04	0.000E+00	1.550E-07	7.200E+02
LOSP*	Loss of offsite power	1	2.990E-03	2.990E-03	0.000E+00	0.000E+00

NOTE: [a] For Calc. Type 3 with a mission time of 0, SAPHIRE performs the quantification using the system mission time.
[b] The designation of a circuit breaker as AC or DC refers to the system designation for the circuit breaker, it is not representative of the motive power for the circuit breaker.
[c] The failure of the battery charger would result in eventual depletion of the battery and a low power indication on both the battery and the DC bus. The 168 hr mission time was selected as a conservative estimation for the detection time of this failure.
[d] The mission times for the DC bus related failure rates do not take credit for any monitoring of bus status, which would provide nearly instantaneous indication of a bus failure or loss of power to the bus. The standby component mission time was used conservatively.
LOSP* represents the probability of losing offsite power during the 720 hours HVAC is required after any breach of a container releases radioactive material. It is based on a Loss of offsite power frequency of 3.59E-02/year from NUREG/CR-6890 (Ref. B8.1.19).
AC = alternating current; AHU = air handling unti; Calc. = calculation; CCF = common-cause failure; DC = direct current; DG = diesel generator; Div = division; elc = electrical; exh = exhaust; ITS = important to safety; kV = kilovolt; Miss. = mission; OOS = out of service; op = operation; Prob. = probability; Spur. = spurious; SWGR = switchgear; Xfer = transfer.

Source: Original

B8.4.2.5.1 Human Failure Events

Four basic HFEs (Table B8.4-6) are associated with human error. All of the HFEs are associated with the failure to properly restore components to operable status following maintenance. The first two shown in Table B8.4-6 are associated with the failure to restore the normal power supply to the RF ITS load centers after maintenance. The last two are representative of the failure to restore the ITS diesel generators (and any other components that prevents the ITS diesel generator from starting or loading) to service after maintenance. These events are combination events consisting of the probability that a component was removed for maintenance and the failure of plant operators (assigned a screening value of 0.1) to restore the component after maintenance.

Table B8.4-6. Human Failure Events

Name	Description
200-#EEE-LDCNTRA-BUA-ROE	Failure to Restore ITS Load Center Train A post maintenance
200-#EEE-LDCNTRB-BUA-ROE	Failure to Restore ITS Load Center Train B post maintenance
26D-#EEY-ITSDG-A-#DG-RSS	Failure to properly return ITS DG A to service
26D-#EEY-ITSDG-B-#DG-RSS	Failure to properly return ITS DG B to service

NOTE: DG = diesel generator; ITS = Important to Safety.

Source: Original

B8.4.2.5.2 Common-Cause Failures

Twelve of the fourteen CCFs identified earlier (Table B8.4-7) have been included in the analysis of the loss of ITS AC power to the ITS load center Train A. Ten of the CCF events affect both trains of ITS AC power. Two affect only this train of the system. The remaining two affect only the other train of the system. Two are associated with the ITS diesel generators: CCF of the ITS diesel generators to start and common-cause failure of the ITS diesel generators to run.

The CCF of the ITS diesel generator fuel oil system incorporates two CCFs: CCF of the two fuel oil pumps to start and the CCF of the pumps to run. Three circuit breaker CCF events were considered. These are the CCF of the (1) 13.8kV ITS switchgear feed breakers (from 13.8kV open buses) to open on loss of offsite power, (2) ITS diesel generator load breakers to close when commanded by the load sequencer and (3) ITS load center feed breakers to close when commanded by the load sequencer. Four CCF are associated with the RF ITS Electrical and Battery Rooms ventilation system, two for the CCF of exhaust fans to start and run, and two for the CCF of the air handling units to start and run. The last CCF event considered is the CCF of the 13.8kV - 480V ITS transformers.

All of the CCFs modeled are used on pairs of components with one of two success criteria (i.e., two of two failure criteria). Alpha-factors used to determine the common cause failure probability are 0.047 for demand failures and 0.0235 for time dependent failures (Table C3-1, CCCG=2, and the associated text). Two CCF in Table B8.4-7 are used to represent the CCF associated with the failure to start and failure to run for components. For these two CCFs, the appropriate alpha-factors were applied to the start and run portions of the random failure probability to develop a single CCF probability for the components.

Table B8.4-7. Common-Cause Basic Events

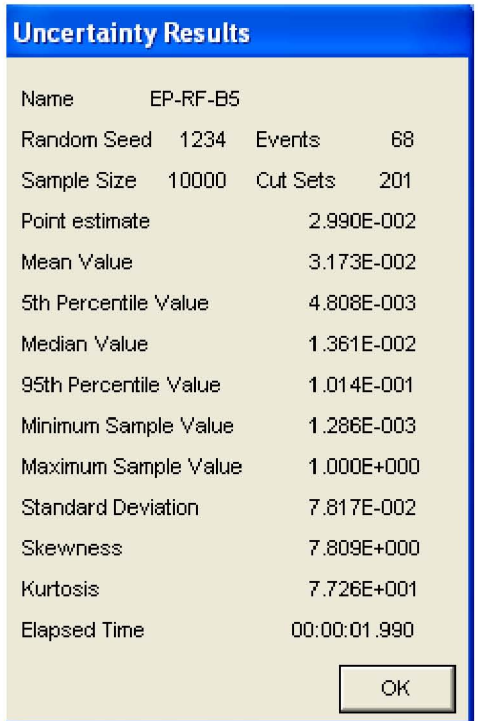
Name	Description	Alpha-factor
200-#EEE-RFITS-A-XMR-CCF	RF ITS Transformers CCF	0.0235
200-#EEE-LDCNTRS-C52-CCF	CCF of the ITS Load Center feed breakers to reclose	0.047
26D-##EG-FULPMPB-PMD-CCR	CCF of ITS DG B fuel pumps to run	0.0235
26D-##EG-FULPMPB-PMD-CCS	CCF of ITS DG B fuel pumps to start	0.047
26D-#EEY-DGLOADS-C52-CCF	CCF of ITS DG Load Breakers to close	0.047
26D-#EEY-ITSDGAB-#DG-CCR	CCF ITS DG A & B Fail to Run	0.0235
26D-#EEY-ITSDGAB-#DG-CCS	CCF DG A and B to Start	0.047
26D-#EEY-OB-SWGS-C52-CCF	CCF of 13.8 kV ITS SWGR feed breakers to open	0.047
200-VCT0-AHU0103-AHU-CCR	CCF of the running RF ITS Elec AHUs to continue to run	0.0235
200-VCT0-AHU0202-AHU-CCR	CCF of standby RF ITS Elec AHUs to start/run	0.047 start 0.0235 run
200-VCT0-EXH0911-FAN-CCR	CCF of running Exh fans for RF ITS Elec.	0.0235
200-VCT0-EXH1012-FAN-CCF	CCF to start/run: standby Exh fans for the RF ITS Elec	0.047 start 0.0235 run

NOTE: AHU = air handling unit; CCF = common-cause failure, CRCF = Canister Receipt and Closure Facility;
DG = diesel generator; elec = electrical; exh = exhaust; ITS = important to safety; RF = Receipt Facility; SWGR = switchgear.

Source: Original

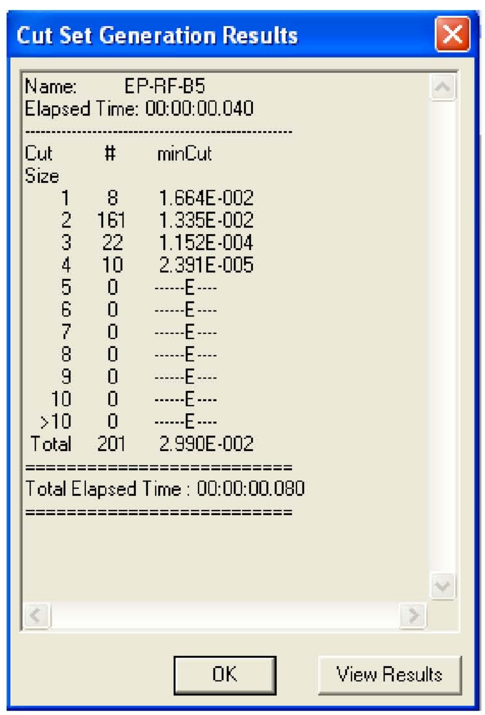
B8.4.2.6 Uncertainty and Cut Set Generation

Figure B8.4-3 contains the uncertainty results obtained from running the fault tree for “Loss of AC Power to RF ITS Load Center Train B”. Figure B8.4-4 provides the cut set generation results for the “Loss of AC Power to RF ITS Load Center Train B”.



Source:

Figure B8.4-3. Uncertainty Results of the AC Power to RF ITS Load Center Train B Fault Tree



Source:

Figure B8.4-4. Cut Set Generation Results AC Power to RF ITS Load Center Train B Fault Tree

B8.4.2.7 Cut Sets

Table B8.4-8 contains the top 25 cut sets that contribute 97% of the total system failure probability for the "Loss of AC Power to RF ITS Load Center Train B" fault tree.

Table B8.4-8. Dominant Cut Sets for the Loss of AC Power to RF ITS Load Center Train B

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
17.99	17.99	5.378E-03	200-#EEE-MCC0002-MCC-FOH	RF ITS MCC00002 Failure	5.378E-03
30.75	12.76	3.816E-03	200-#EEE-LDCNTRB-C52-SPO	RF Load Center Circuit Breaker (AC) Spur Op	3.816E-03
43.51	12.76	3.816E-03	200-#EEE-MCC0002-C52-SPO	RF MCC-00002 Feed Breaker Spurious Operation	3.816E-03
53.33	9.82	2.937E-03	26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.698E-01
			26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
63.15	9.82	2.937E-03	26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.698E-01
			27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
72.00	8.85	2.646E-03	26D-#EEE-SWGRDGB-AHU-FTR	EDGF Switchgear Room Air Handling Unit Failure to Run	2.646E-03
80.56	8.56	2.559E-03	200-VCT0-EXH-011-FAN-FTR	RF ITS Elec Exhaust Fan 00011 Fails to Run	5.059E-02
			200-VCT0-EXH-012-FAN-FTR	RF ITS Elec. Exh Fan 00012 Fails to Run	5.059E-02
88.26	7.70	2.302E-03	26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.698E-01
			LOSP	Loss of offsite power	2.990E-03
89.73	1.47	4.391E-04	200-#EEE-LDCNTRB-BUA-FOH	RF ITS Load Center B Fails	4.391E-04
91.20	1.47	4.391E-04	26D-#EEE-SWGRDGB-BUA-FOH	13.8 kV ITS Switchgear B Bus Failure	4.391E-04
92.33	1.13	3.380E-04	26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.698E-01
			27A-#EEN-OPENBS4-BUA-FOH	13.8 kV Open Bus 4 Bus Failure	4.391E-04
93.03	0.70	2.095E-04	200-#EEE-RFITS-B-XMR-FOH	RF ITS Transformer Train B Failure	2.095E-04
93.37	0.34	1.027E-04	200-VCT0-EXH-011-FAN-FTR	RF ITS Elec Exhaust Fan 00011 Fails to Run	5.059E-02
			200-VCT0-EXH-012-CTL-FOD	RF ITS Elec Exh Fan 0012 Controller Fails	2.030E-03

Table B8.4-8. Dominant Cut Sets for The Loss of AC Power to RF ITS Load Center Train B (Continued)

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
93.71	0.34	1.027E-04	200-VCT0-EXH-011-CTL-FOD	RF ITS Elec Exh fan 00011 Controller Fails	2.030E-03
			200-VCT0-EXH-012-FAN-FTR	RF ITS Elec. Exh Fan 00012 Fails to Run	5.059E-02
94.05	0.34	1.025E-04	/200-#EEE-LDCNTRA-BUA-MTN	ITS Load Center Train A OOS for Maintenance	9.999E-01
			/200-#EEE-LDCNTRA-BUA-ROE	Failure to Restore ITS Load Center Train A post maintenance	1.000E+000
			200-#EEE-LDCNTRB-BUA-MTN	ITS Load Center Train B OOS for Maintenance	1.025E-04
94.39	0.34	1.022E-04	200-VCT0-EXH-011-FAN-FTR	RF ITS Elec Exhaust Fan 00011 Fails to Run	5.059E-02
			200-VCT0-EXH-012-FAN-FTS	RF ITS Elec Exh fan 0012 Fails to Start	2.020E-03
94.72	0.33	9.777E-05	26D-##EG-HVACFN4-FAN-FTR	ITS DG B Fan 4 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
95.05	0.33	9.777E-05	26D-##EG-HVACFN3-FAN-FTR	ITS DG B room Fan 3 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
95.38	0.33	9.777E-05	26D-##EG-HVACFN2-FAN-FTR	ITS DG B room Fan 2 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
95.71	0.33	9.777E-05	26D-##EG-HVACFN1-FAN-FTR	ITS DG B room Fan 1 (Motor-Driven) Fails to Run	2.562E-02
			26D-#EEY-OB-SWGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
96.04	0.33	9.777E-05	26D-##EG-HVACFN4-FAN-FTR	ITS DG B Fan 4 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
96.37	0.33	9.777E-05	26D-##EG-HVACFN3-FAN-FTR	ITS DG B room Fan 3 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
96.70	0.33	9.777E-05	26D-##EG-HVACFN2-FAN-FTR	ITS DG B room Fan 2 (Motor-Driven) Fails to Run	2.562E-02
			27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
97.03	0.33	9.777E-05	26D-##EG-HVACFN1-FAN-FTR	ITS DG B room Fan 1 (Motor-Driven) Fails to Run	2.562E-02

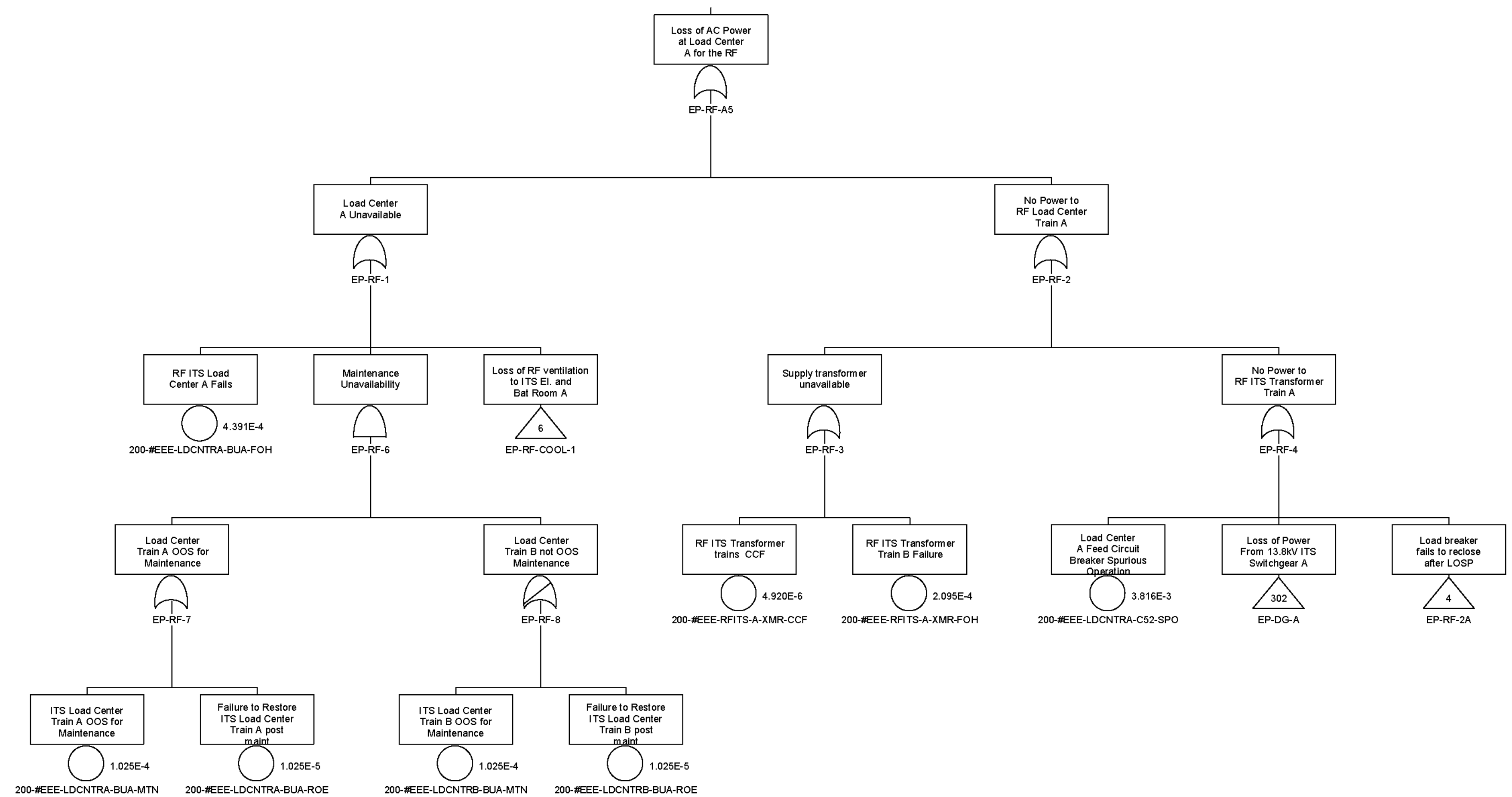
Table B8.4-8. Dominant Cut Sets for The Loss of AC Power to RF ITS Load Center Train B (Continued)

% Total	% Cut Set	Prob./ Frequency	Basic Event	Description	Event Prob.
			27A-#EEE-BUS3DGB-C52-SPO	Circuit Breaker (AC) Spurious Operation	3.816E-03
97.32	0.29	8.590E-05	26D-#EEY-ITSDGB-#DG-FTR	Diesel Generator Fails to Run	7.698E-01
			27A-#EEN-OPNBS3B-SWP-SPO	13.8 kV Open Bus 4 to ITS B Electric Power Switch Spur Xfer	1.116E-04

NOTE: AC = alternating current; AHU = air handling unit; CCF = common-cause failure, CRCF = Canister Receipt and Closure Facility; DG = diesel generator; elec = electrical; exh = exhaust; ITS = important to safety; RF = Receipt Facility; SWGR = switch gear; Xfer = transfer.

Source: Original

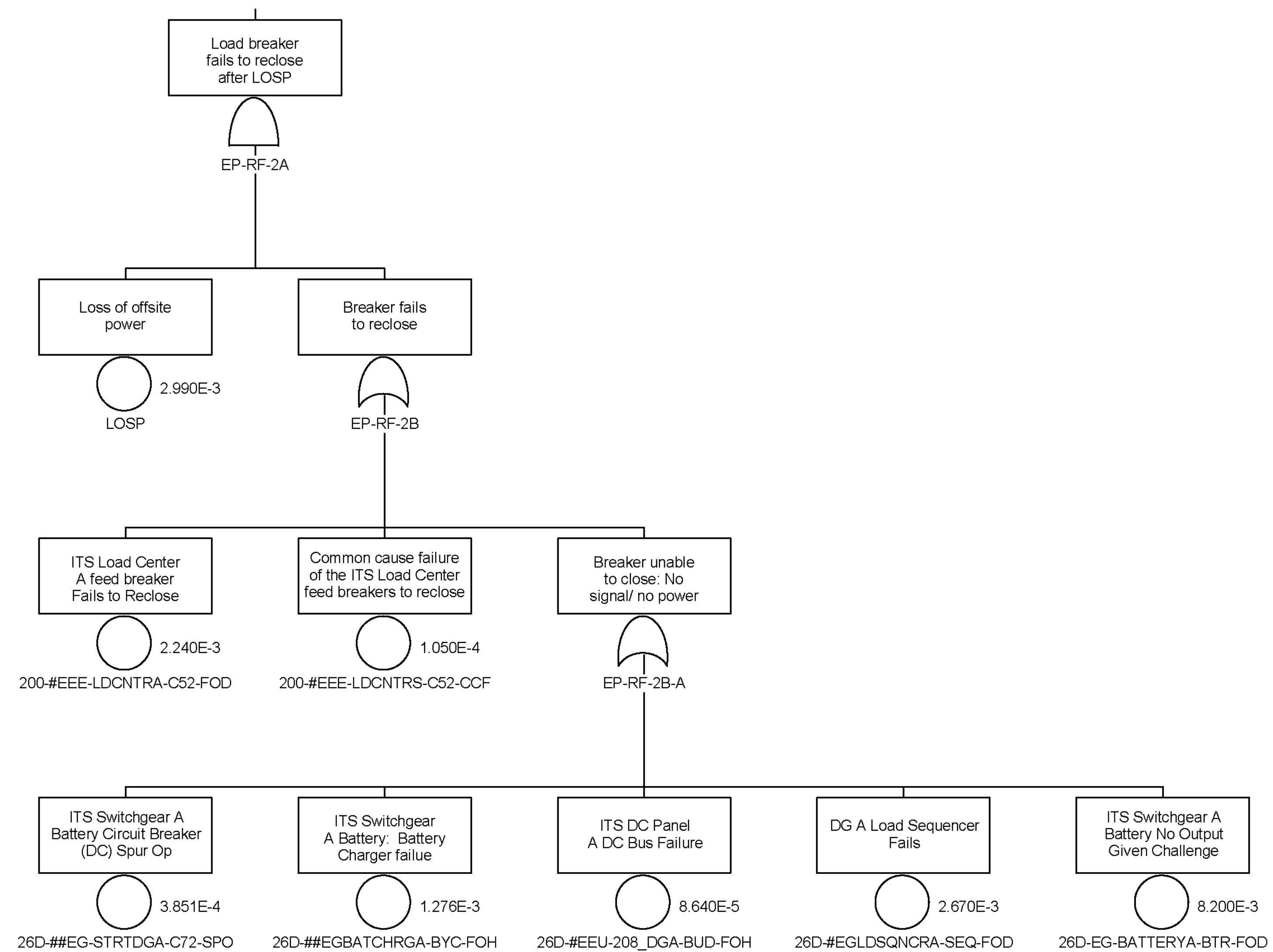
B8.4.2.8 AC Power Fault Trees



EP-RF-A5 - Loss of AC Power at Load Center A for the RF 2008/03/11 Page 3

Source: Original

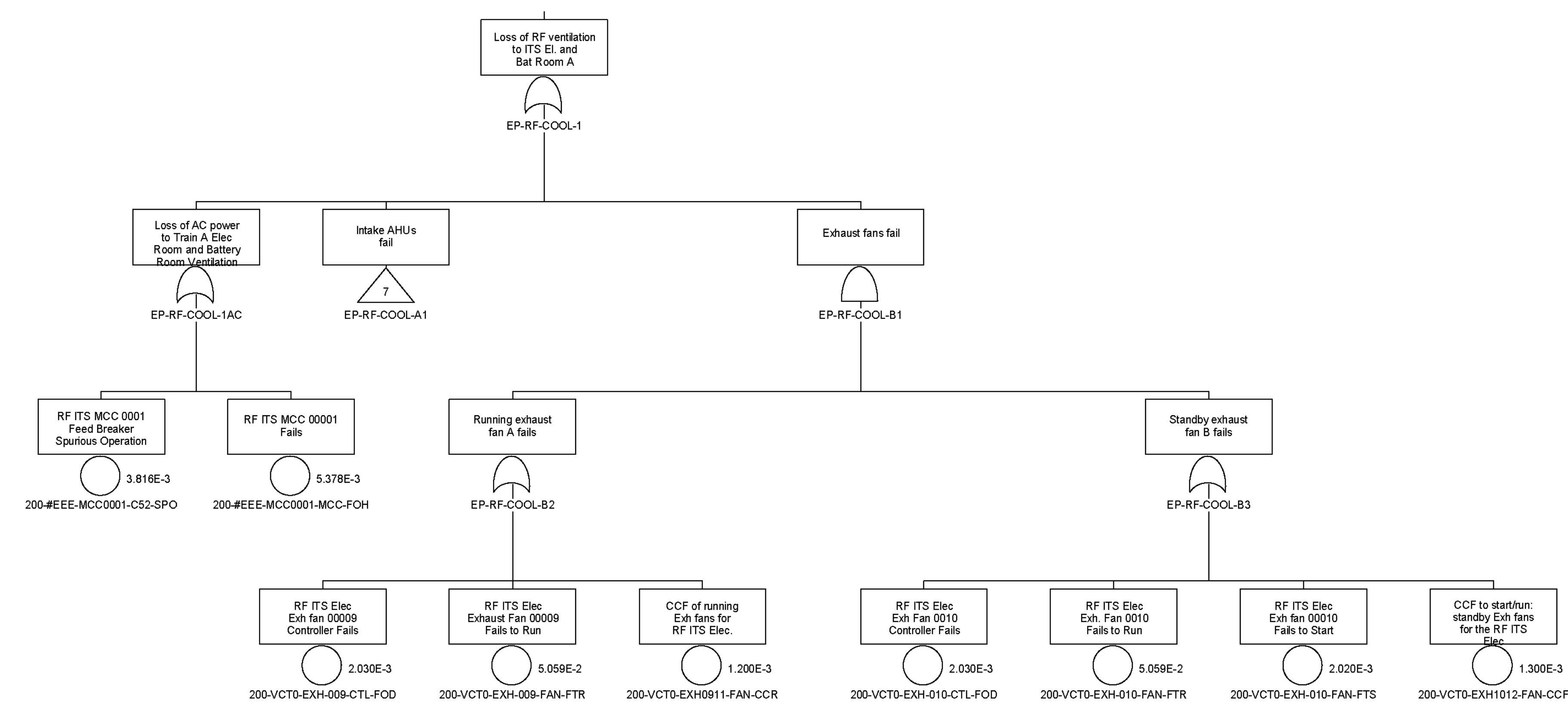
Figure B8.4-5. Loss of AC Power to RF ITS Load Center Train A (Sheet 1)



EP-RF-2A - Load breaker fails to reclose after LOSP 2008/03/10 Page 4

Source: Original

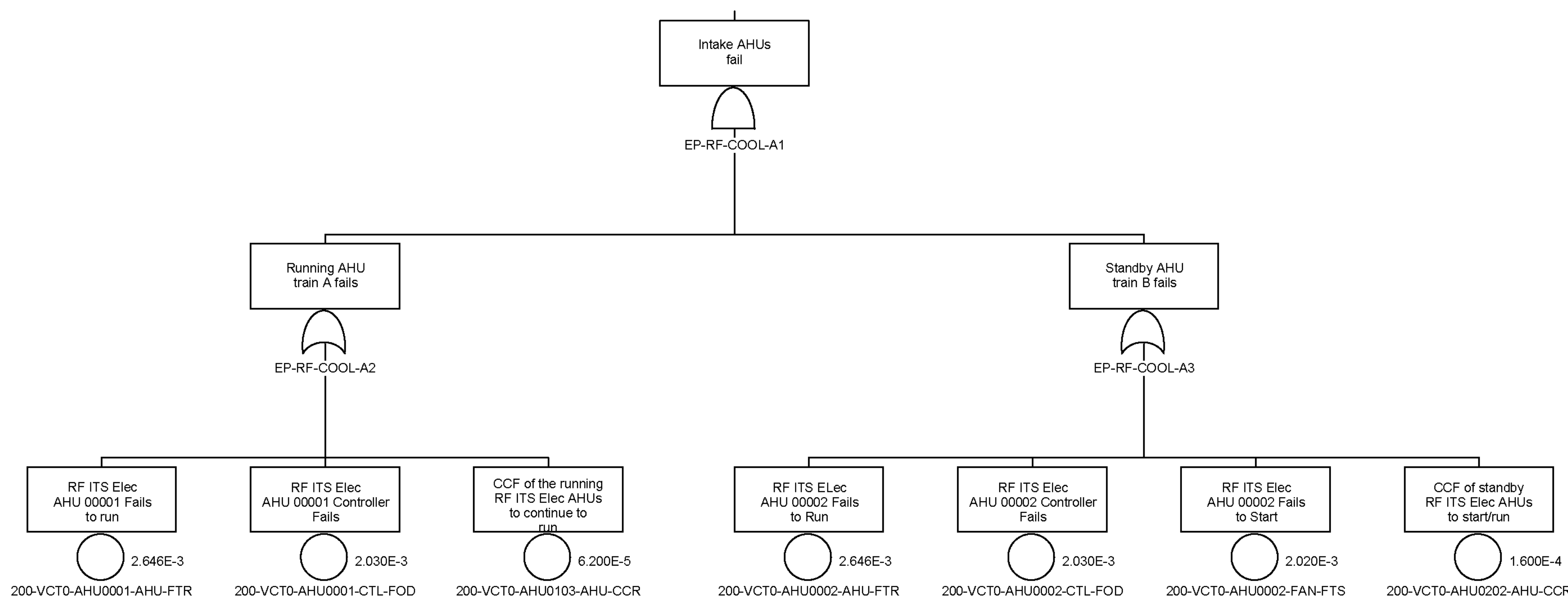
Figure B8.4-6. Loss of AC Power to RF ITS Load Center Train A (Sheet 2)



EP-RF-COOL-1 - Loss of RF ventilation to ITS El. and Bat Room A 2008/03/11 Page 6

Source: Original

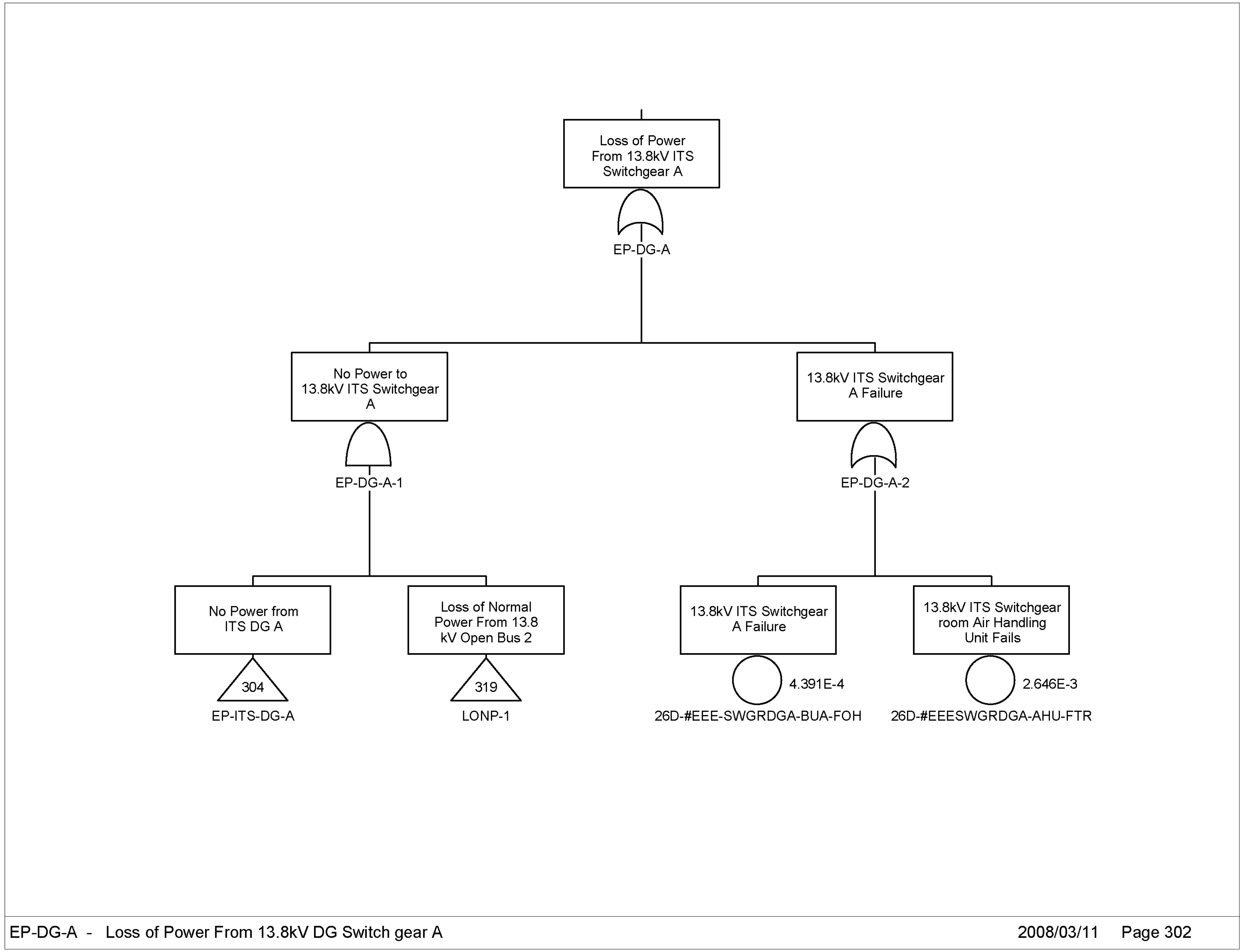
Figure B8.4-7. Loss of AC Power to RF ITS Load Center Train A (Sheet 3)



EP-RF-COOL-A1 - Intake AHUs fail 2008/03/10 Page 7

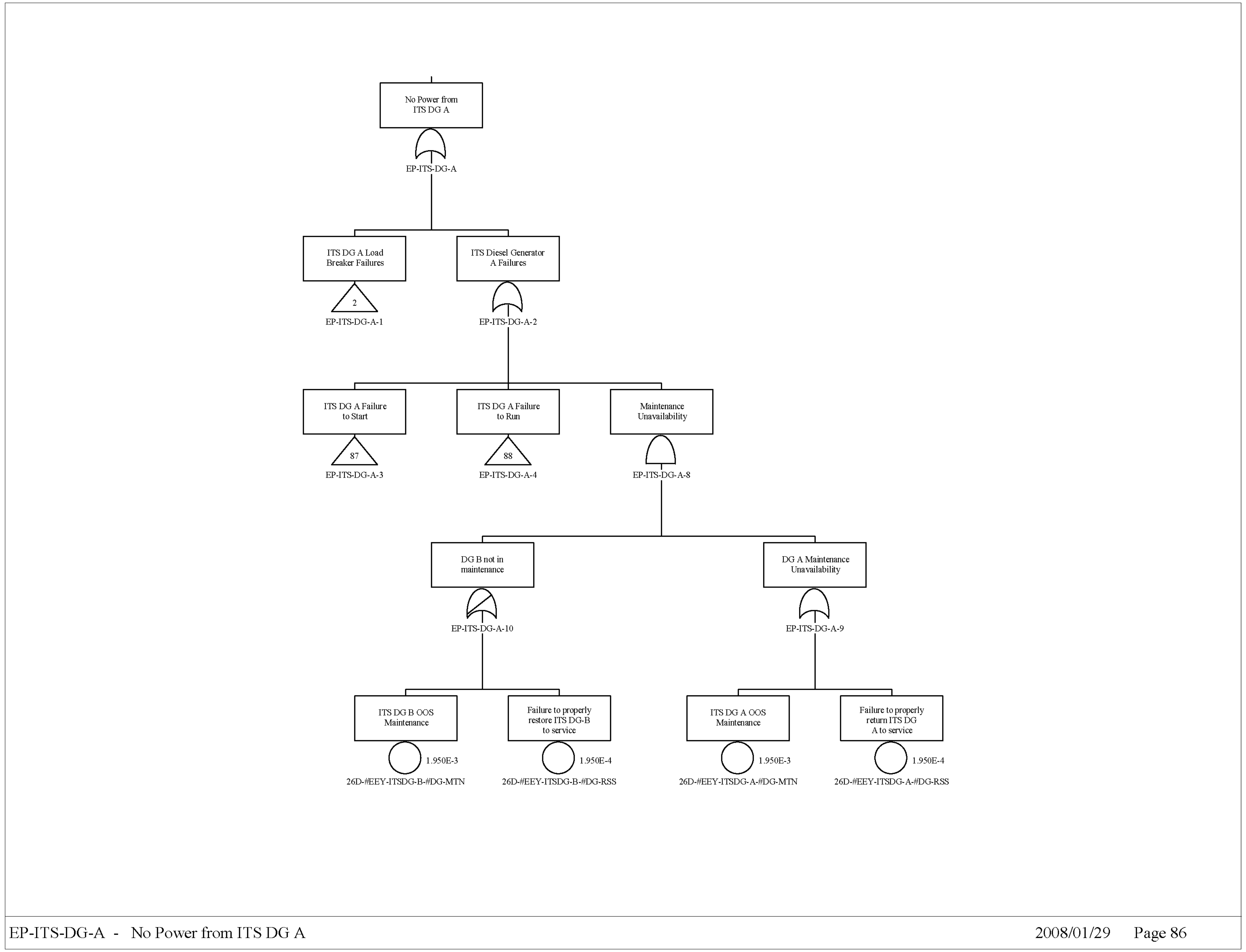
Source: Original

Figure B8.4-8. Loss of AC Power to RF ITS Load Center Train A (Sheet 4)



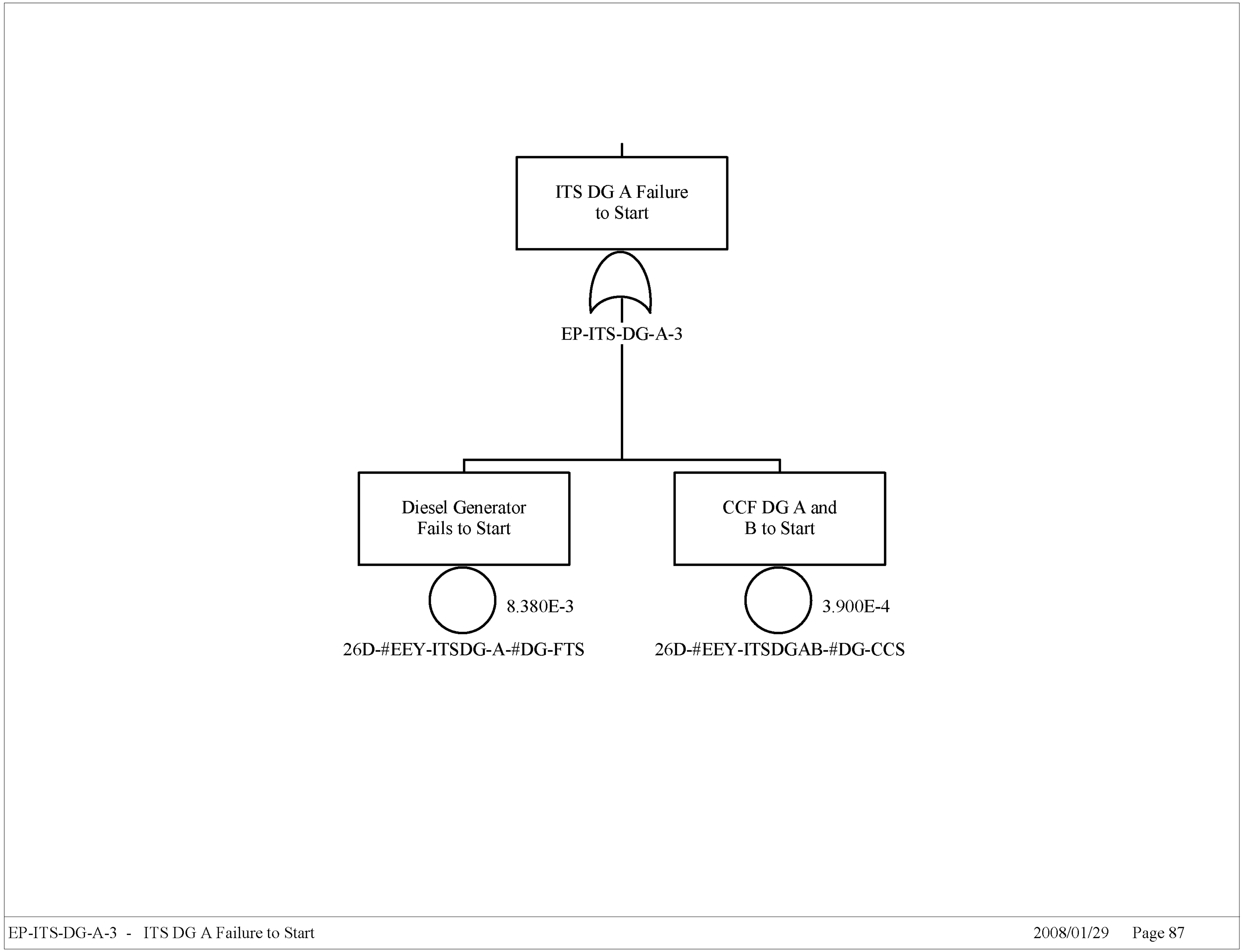
Source: Original

Figure B8.4-9. Loss of AC Power to RF ITS Load Center Train A (Sheet 5)



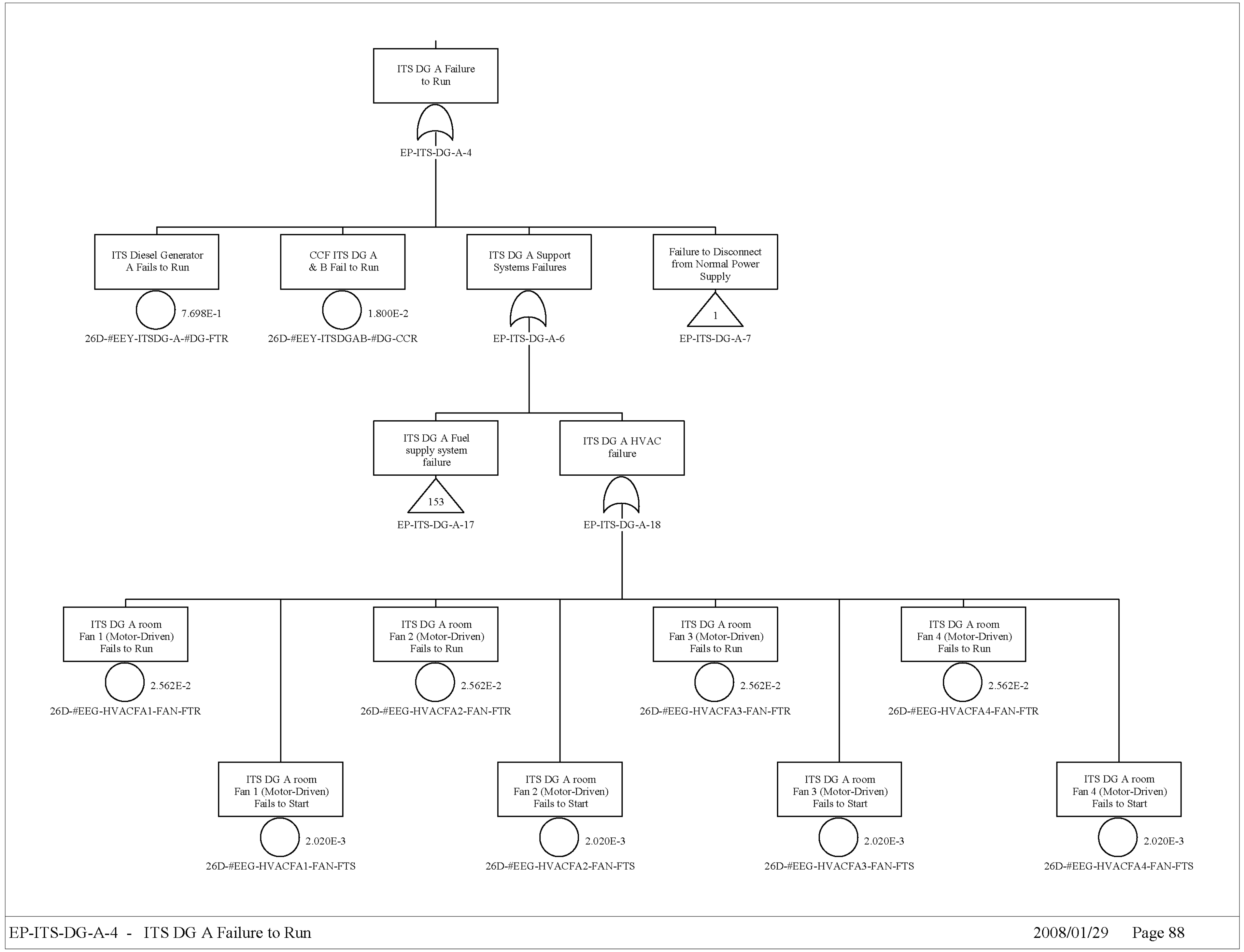
Source: Original

Figure B8.4-10. Loss of AC Power to RF ITS Load Center Train A (Sheet 6)



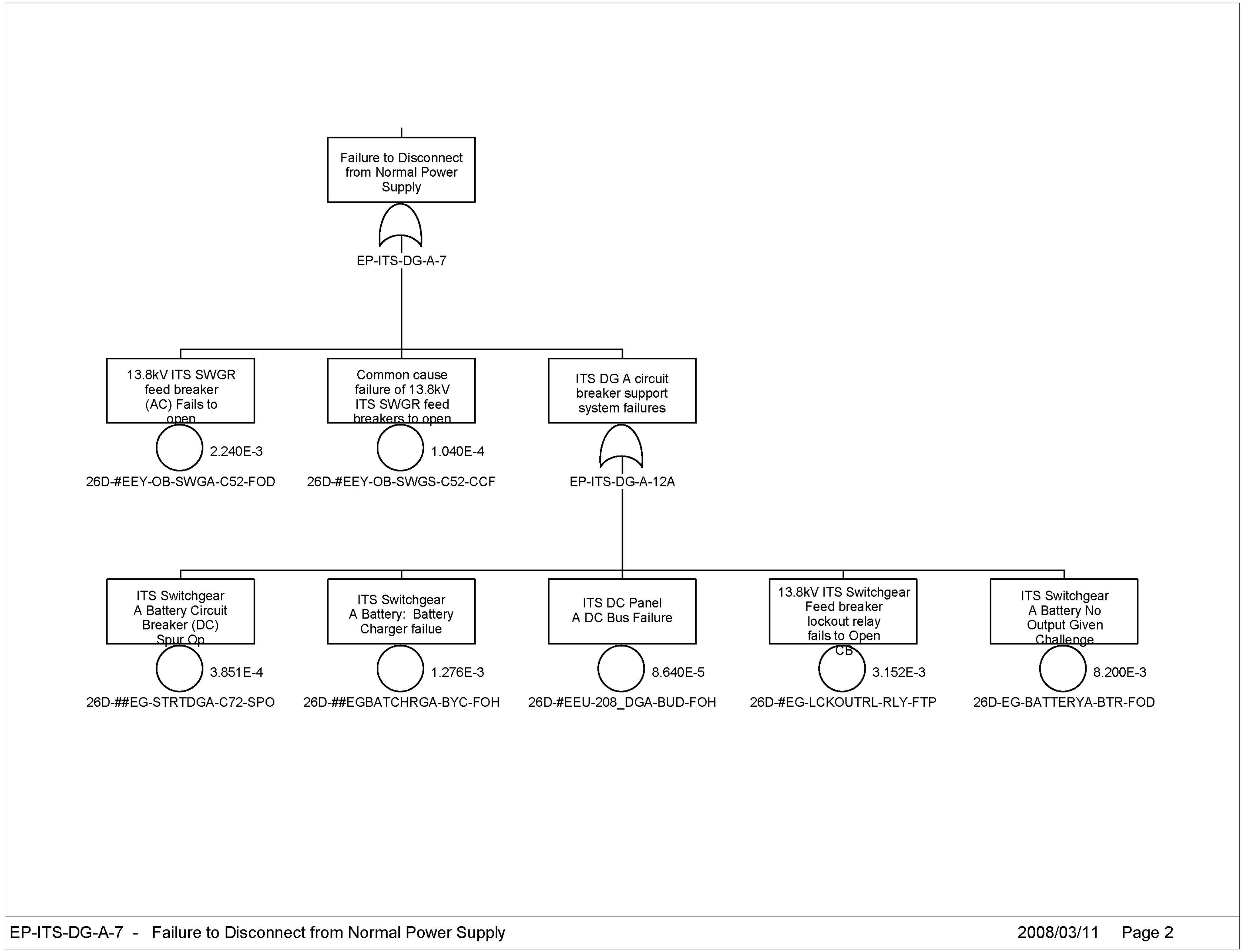
Source: Original

Figure B8.4-11. Loss of AC Power to RF ITS Load Center Train A (Sheet 7)



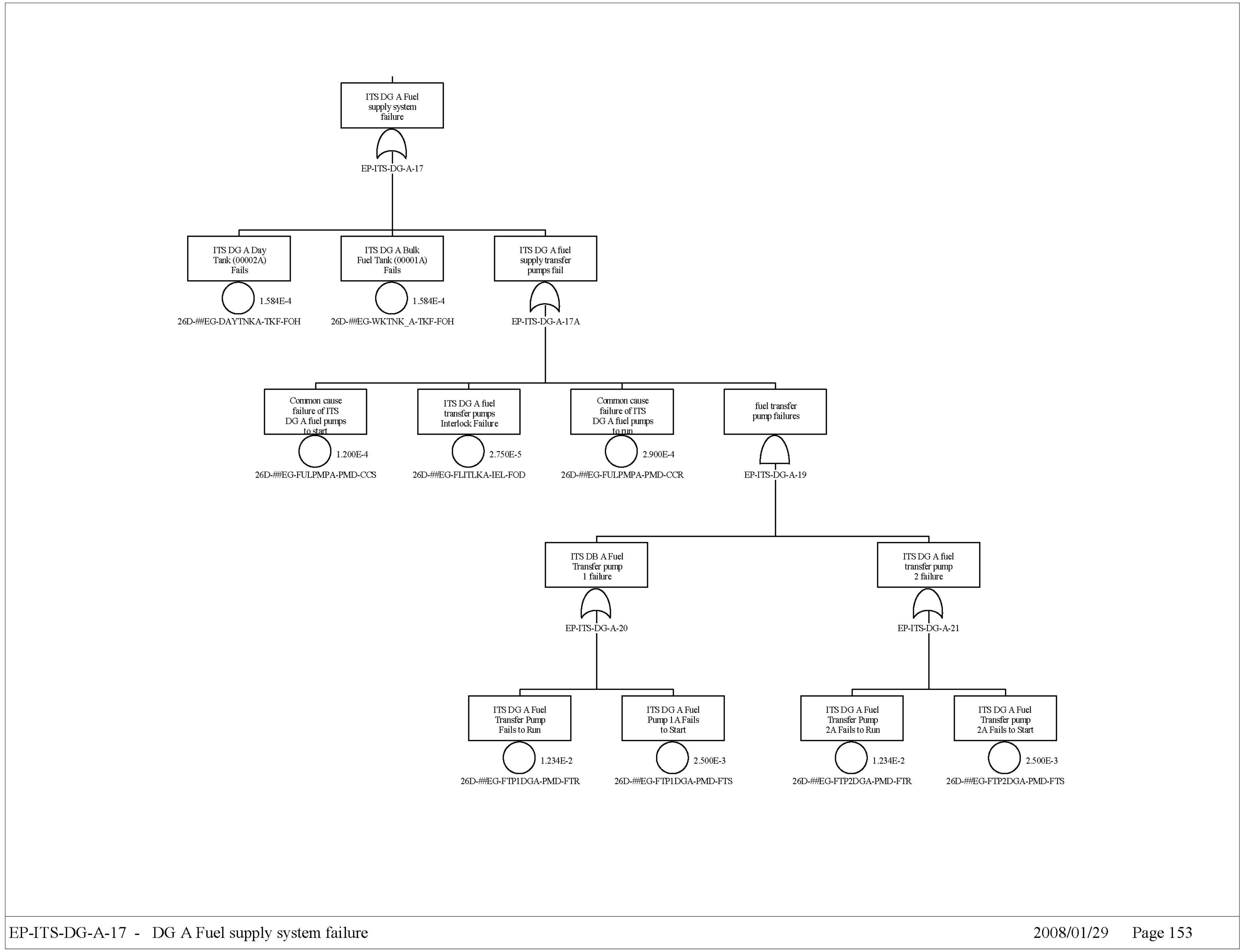
Source: Original

Figure B8.4-12. Loss of AC Power to RF ITS Load Center Train A (Sheet 8)



Source: Original

Figure B8.4-13. Loss of AC Power to RF ITS Load Center Train A (Sheet 9)



Source: Original

Figure B8.4-14. Loss of AC Power to RF ITS Load Center Train A (Sheet 10)

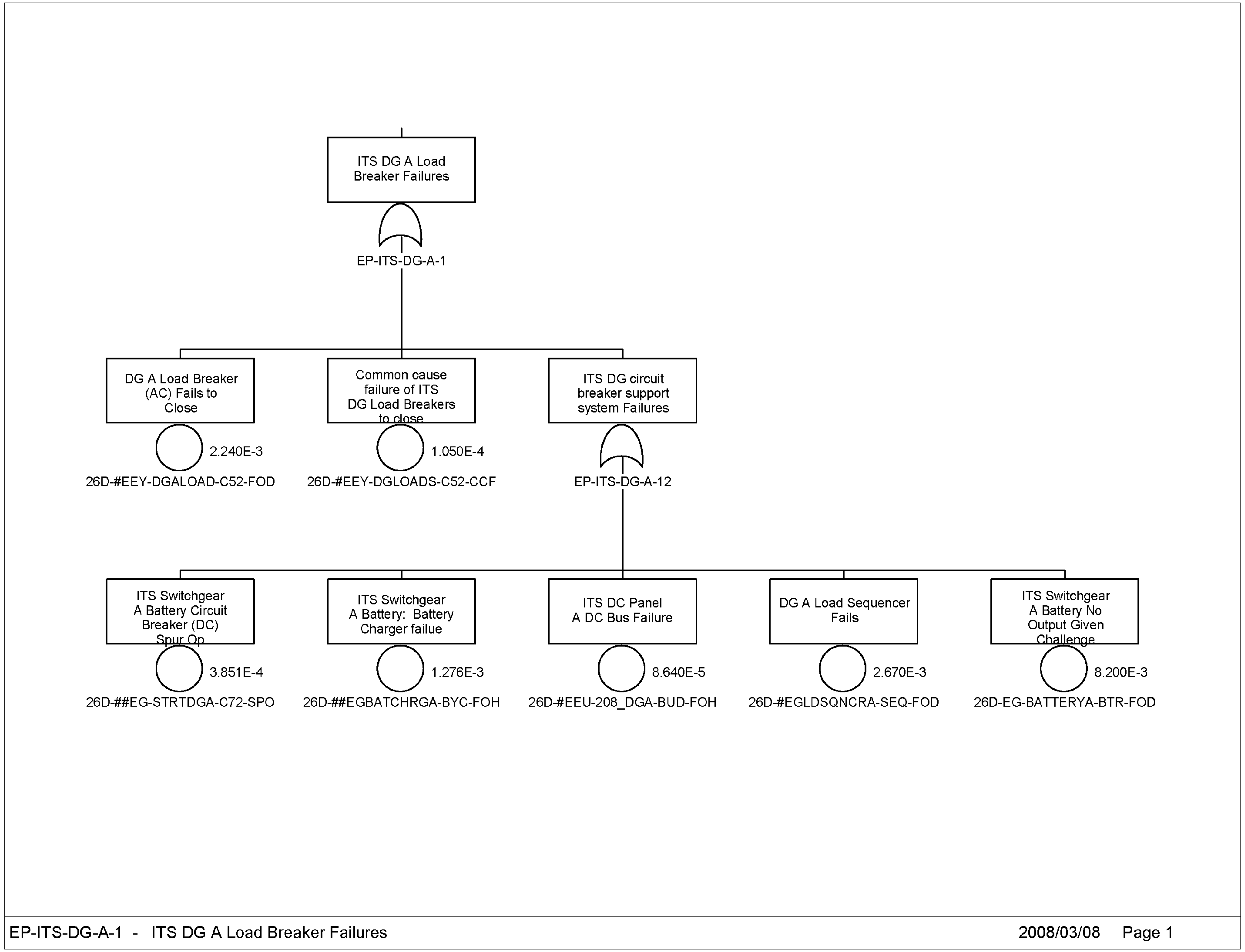


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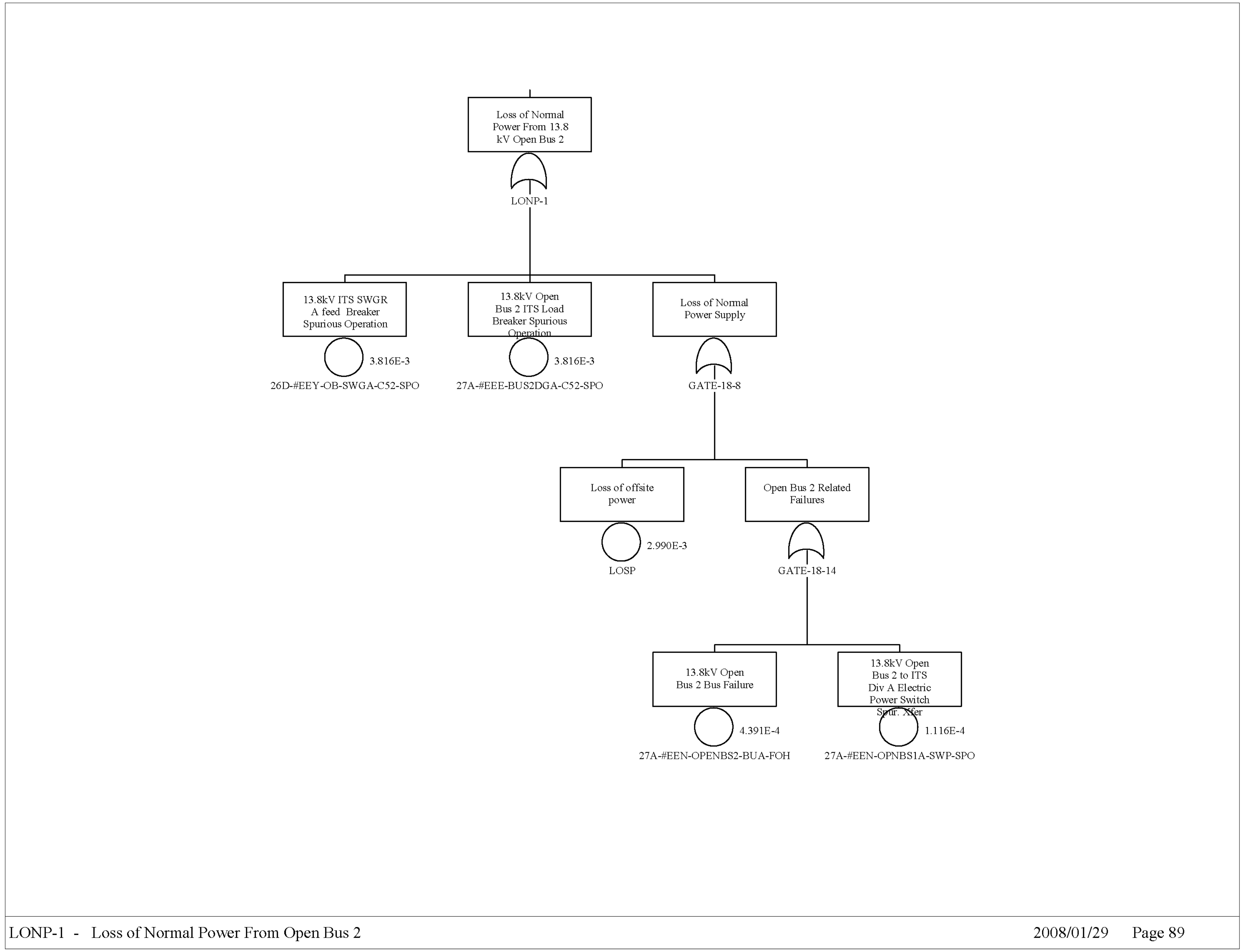
Figure B8.4-15. Loss of AC Power to RF ITS Load Center Train A (Sheet 11)



Source: Original

Figure B8.4-16. Loss of AC Power to RF ITS Load Center Train A (Sheet 12)

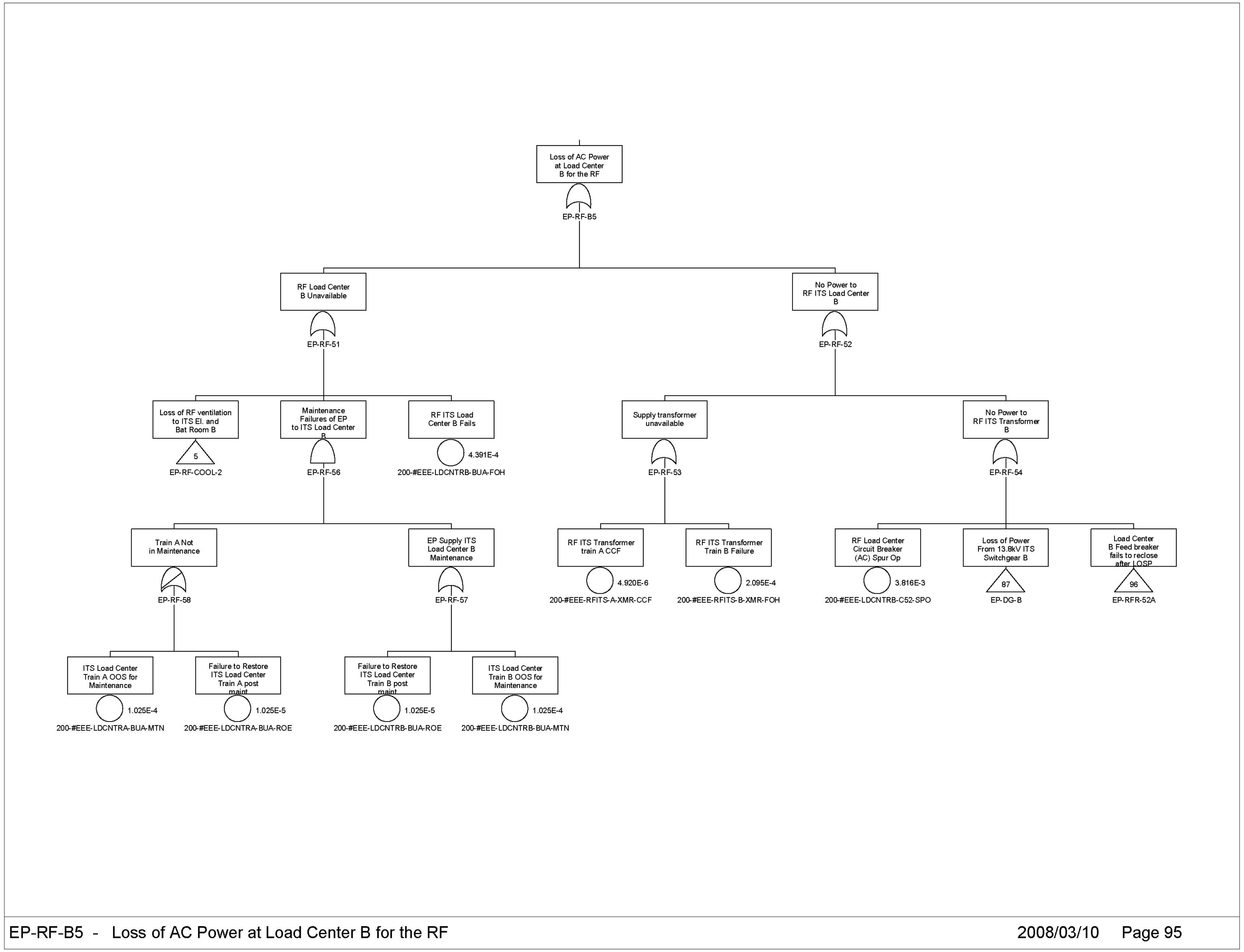


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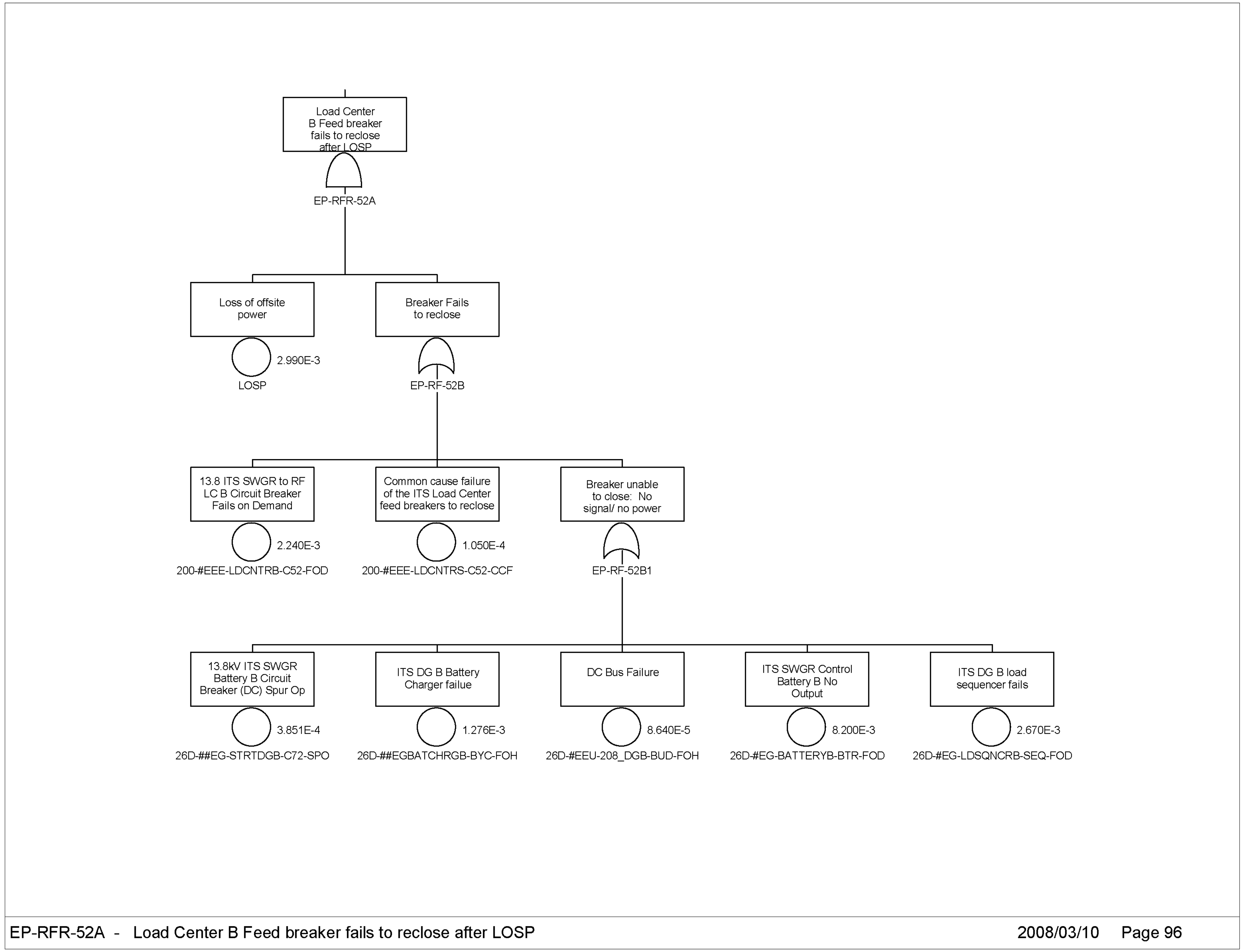
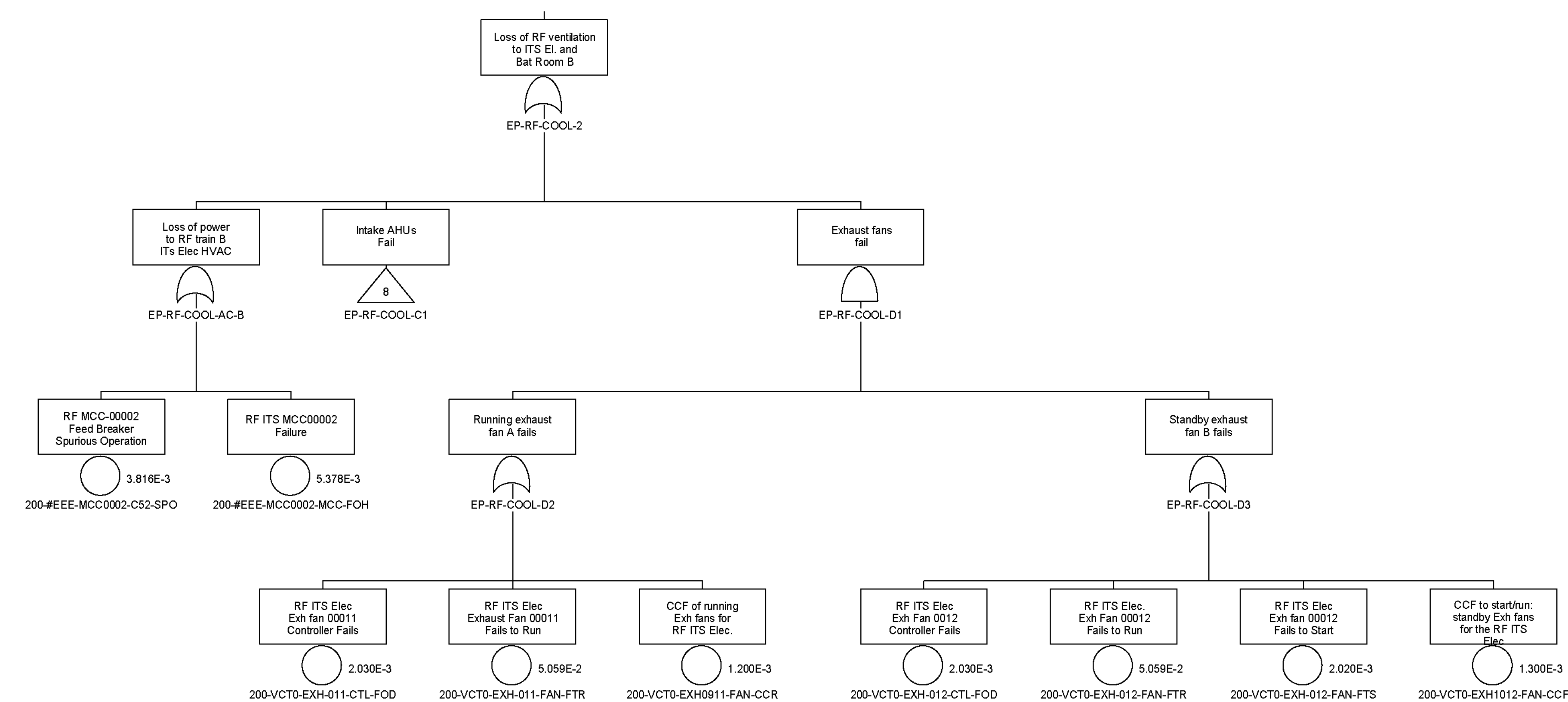
Figure B8.4-17. Loss of AC Power to RF ITS Load Center Train B (Sheet 1)



Source: Original

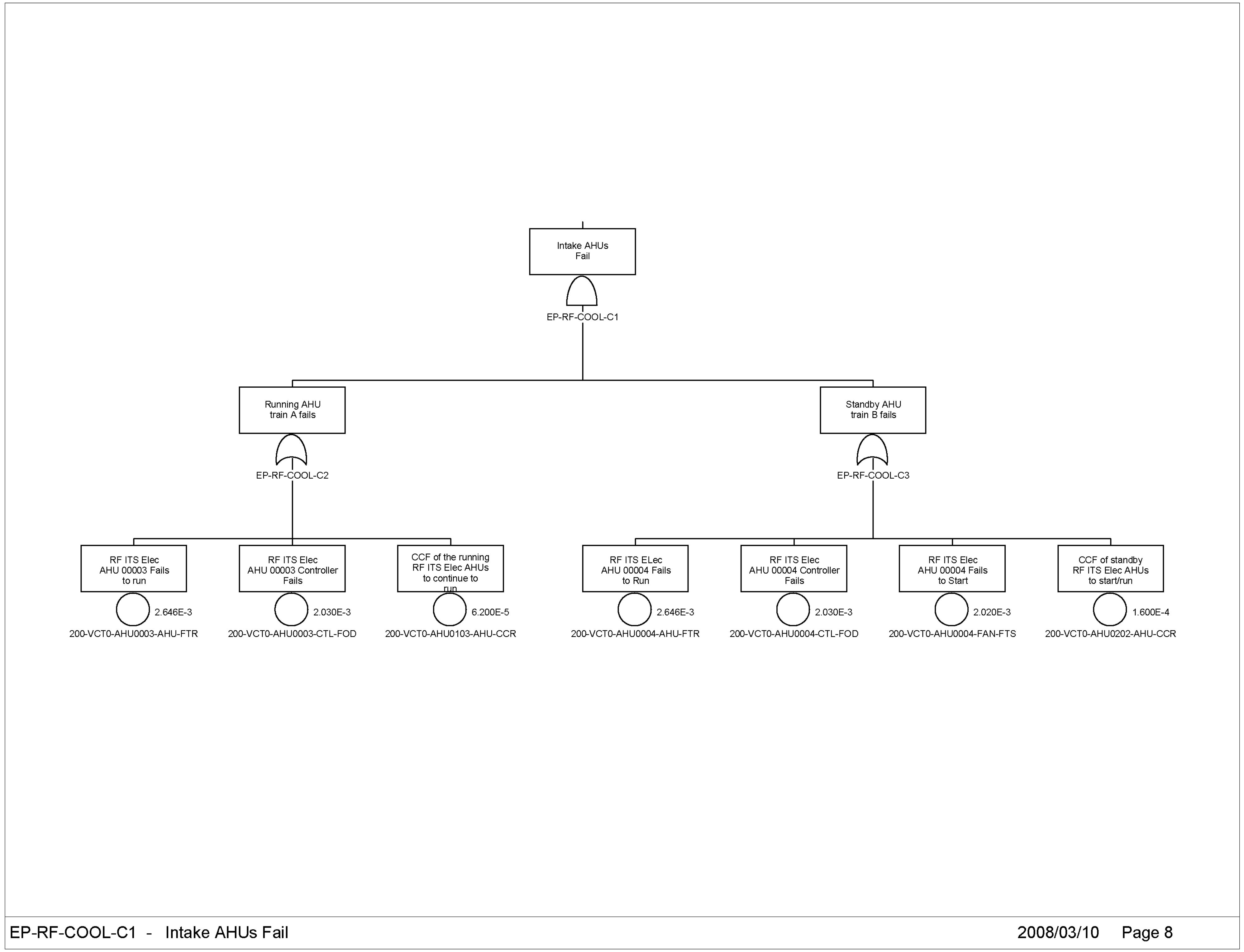
Figure B8.4-18. Loss of AC Power to RF ITS Load Center Train B (Sheet 2)



EP-RF-COOL-2 - Loss of RF ventilation to ITS El. and Bat Room B 2008/03/11 Page 5

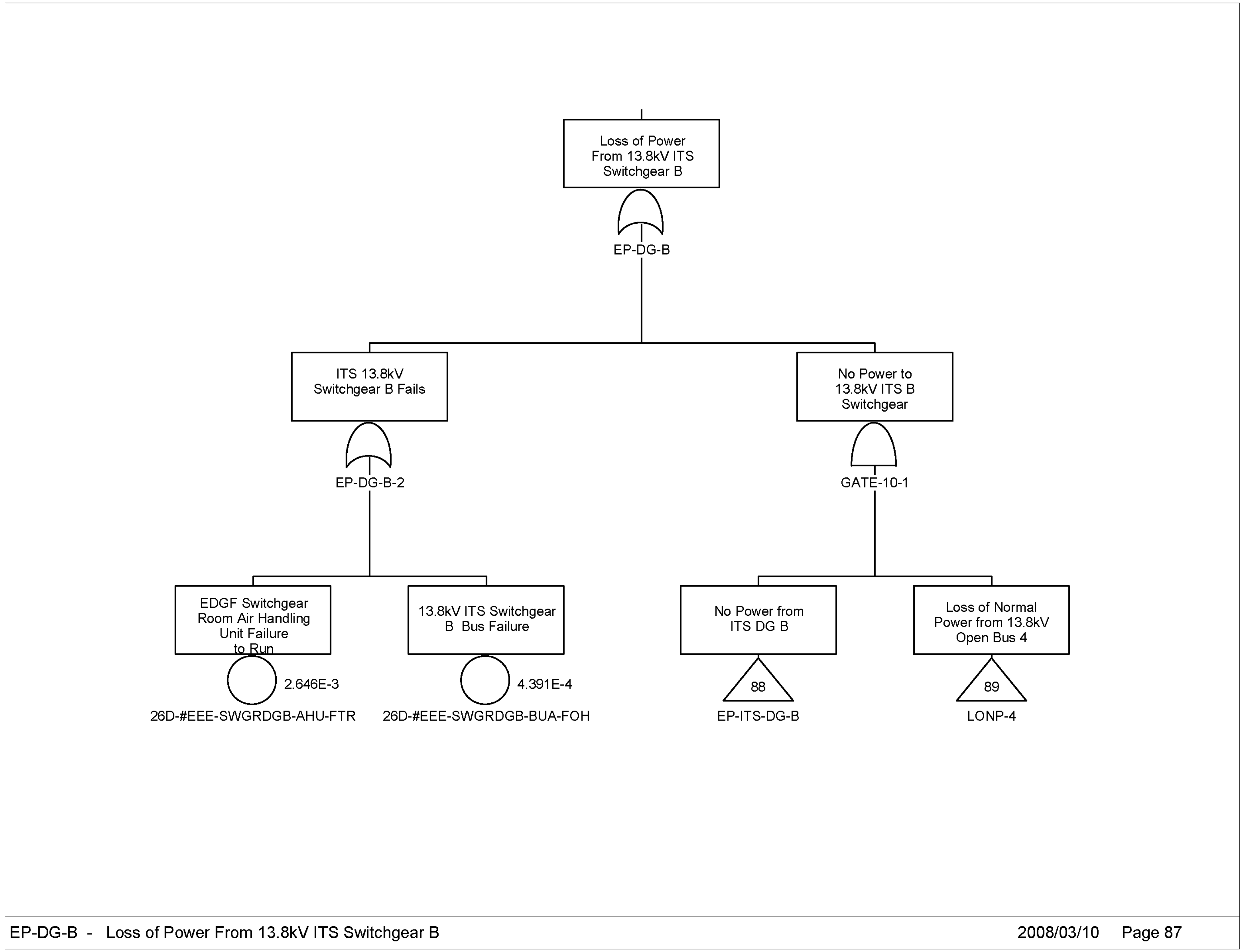
Source: Original

Figure B8.4-19. Loss of AC Power to RF ITS Load Center Train B (Sheet 3)



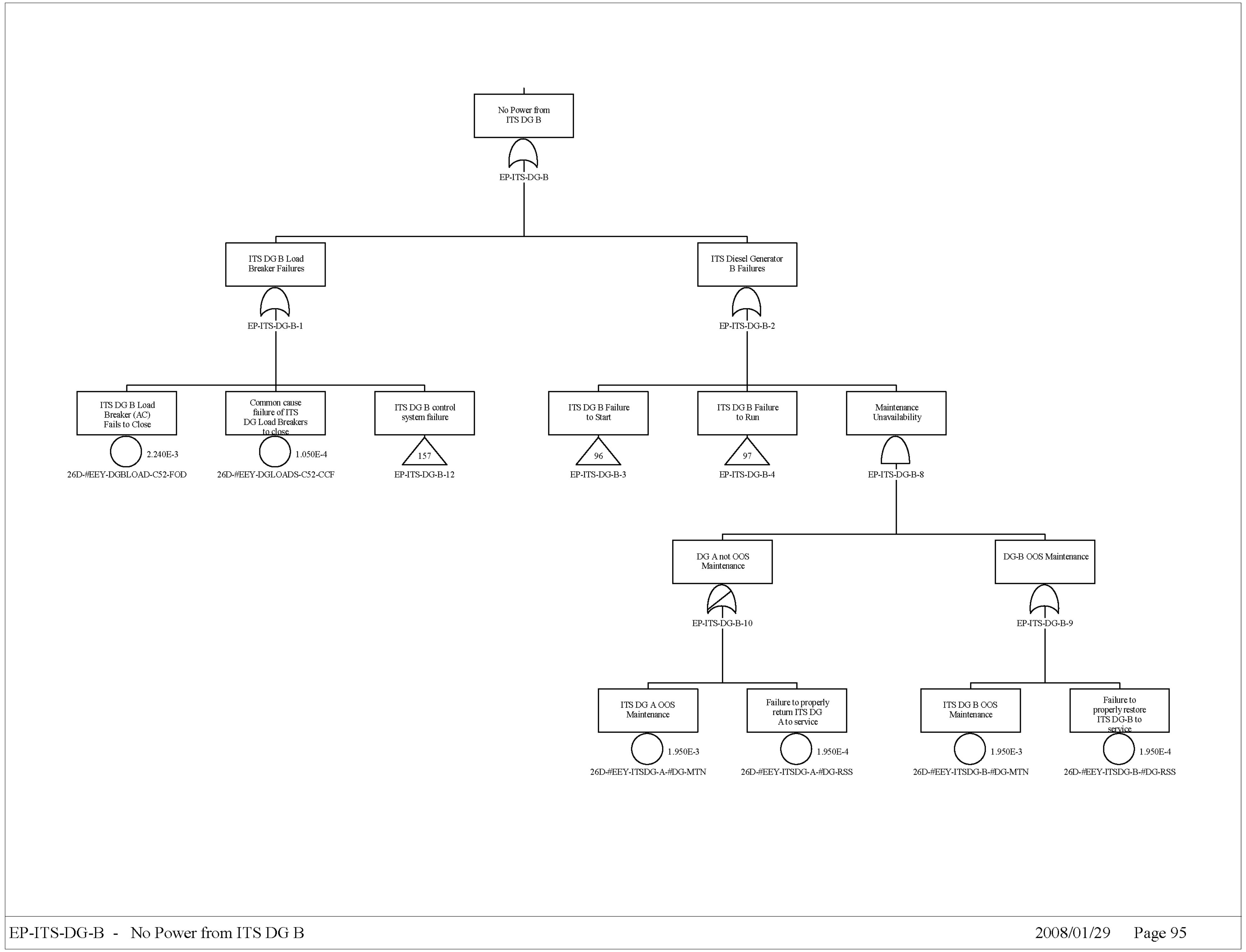
Source: Original

Figure B8.4-20. Loss of AC Power to RF ITS Load Center Train B (Sheet 4)



Source: Original

Figure B8.4-21. Loss of AC Power to RF ITS Load Center Train B (Sheet 5)



Source: Original

Figure B8.4-22. Loss of AC Power to RF ITS Load Center Train B (Sheet 6)

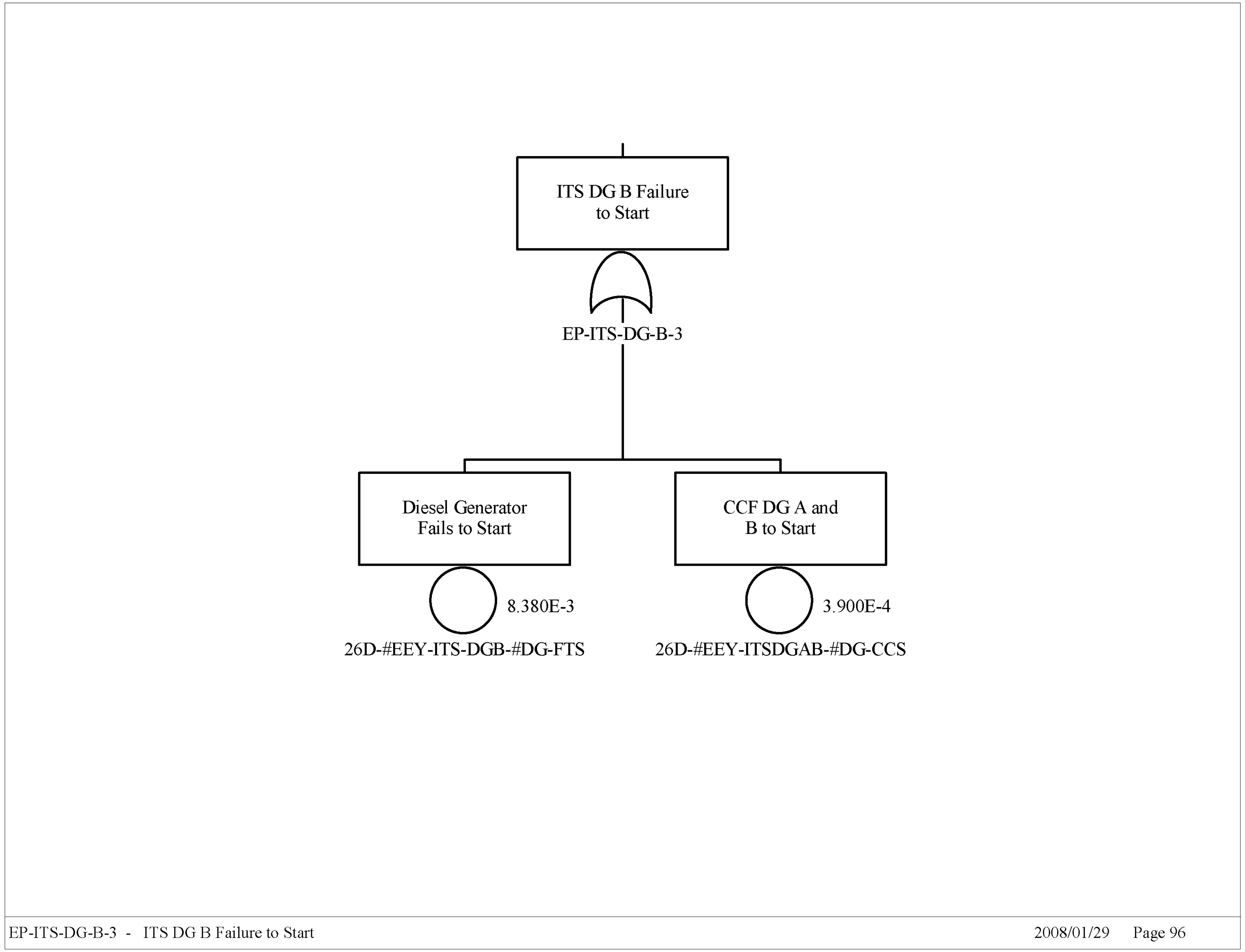


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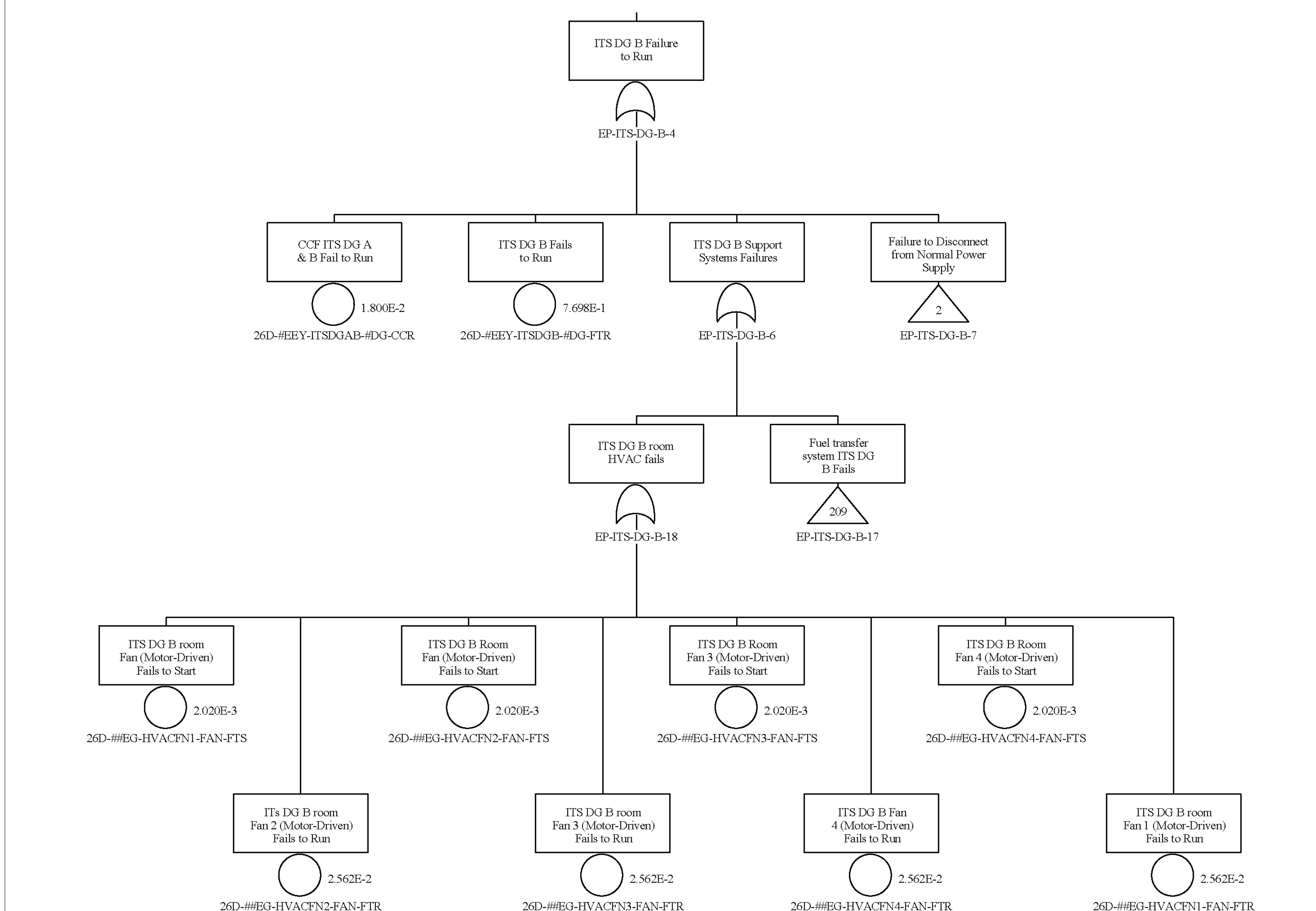
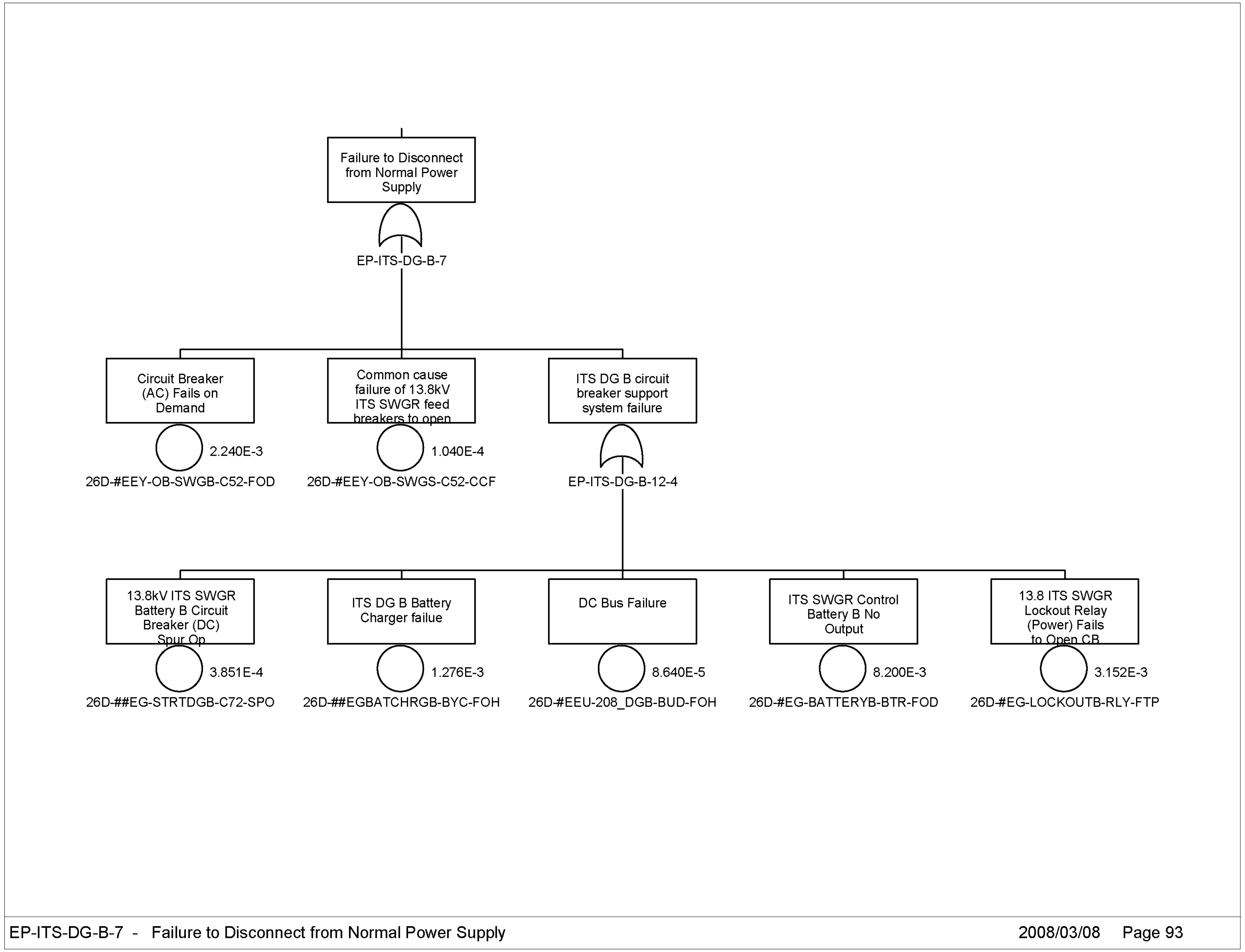
Figure B8.4-23. Loss of AC Power to RF ITS Load Center Train B (Sheet 7)



EP-ITS-DG-B-4 - ITS DG B Failure to Run 2008/01/29 Page 97

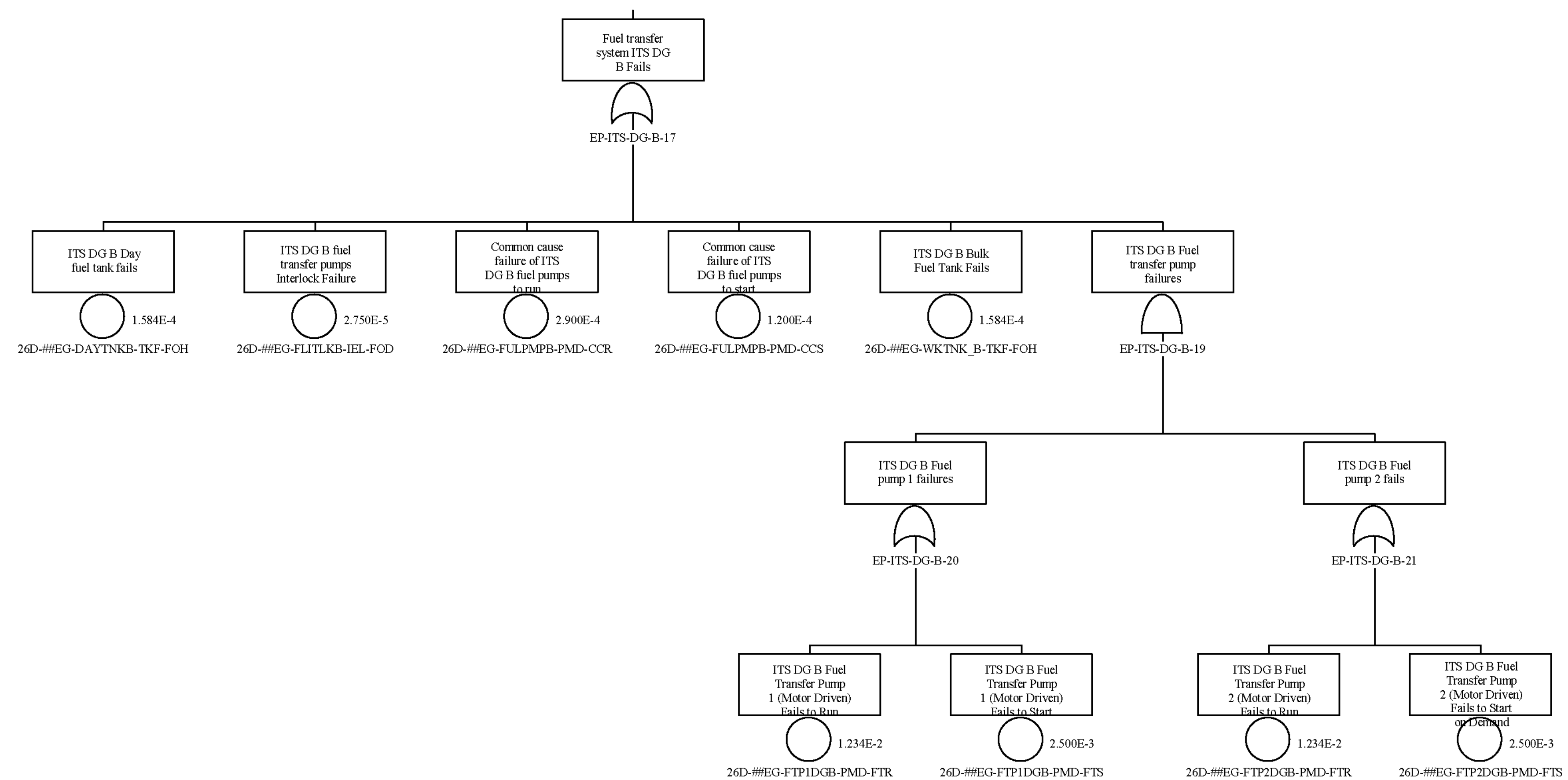
Source: Original

Figure B8.4-24. Loss of AC Power to RF ITS Load Center Train B (Sheet 8)



Source: Original

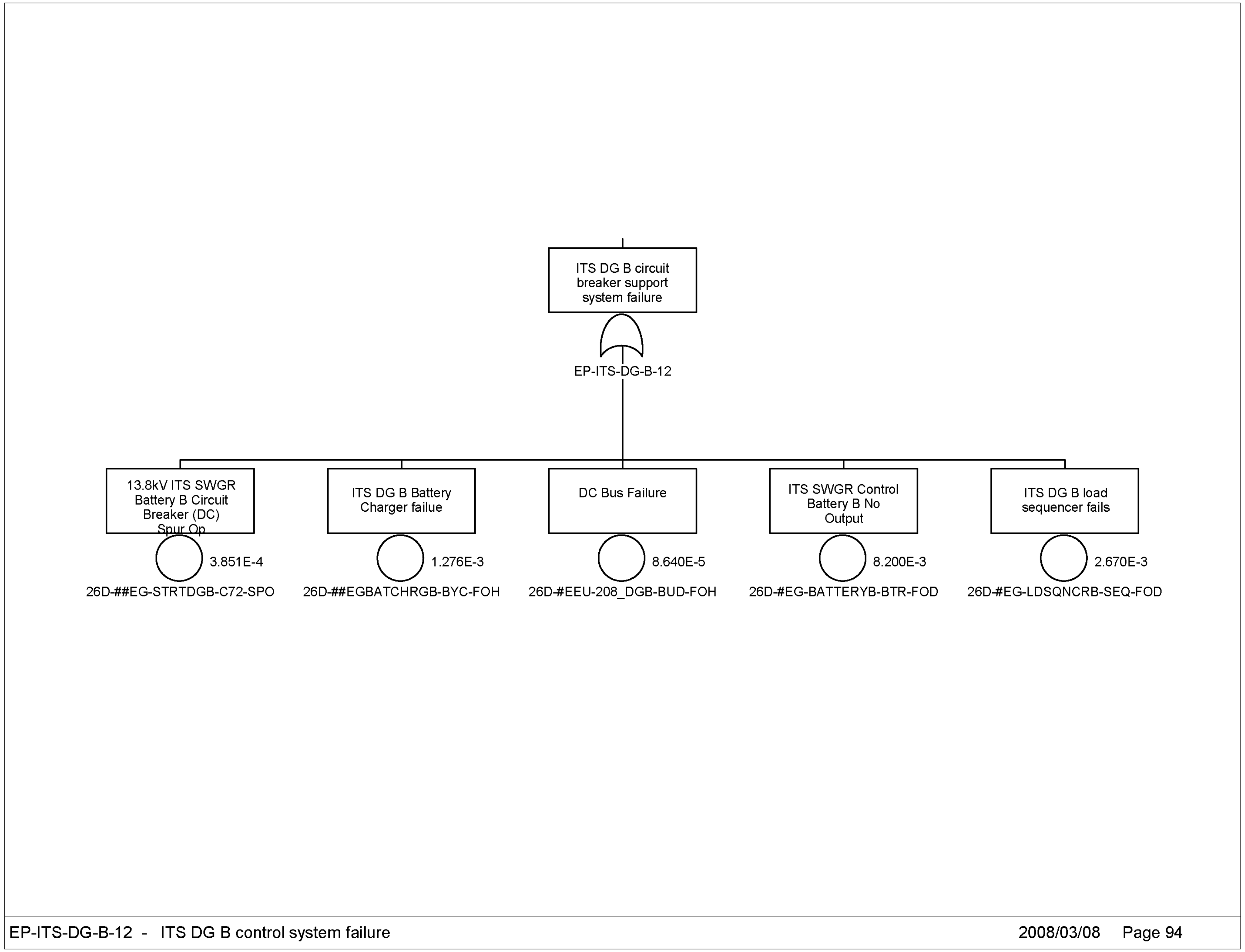
Figure B8.4-25. Loss of AC Power to RF ITS Load Center Train B (Sheet 9)



EP-ITS-DG-B-17 - Fuel transfer system DG B Fails 2008/01/29 Page 209

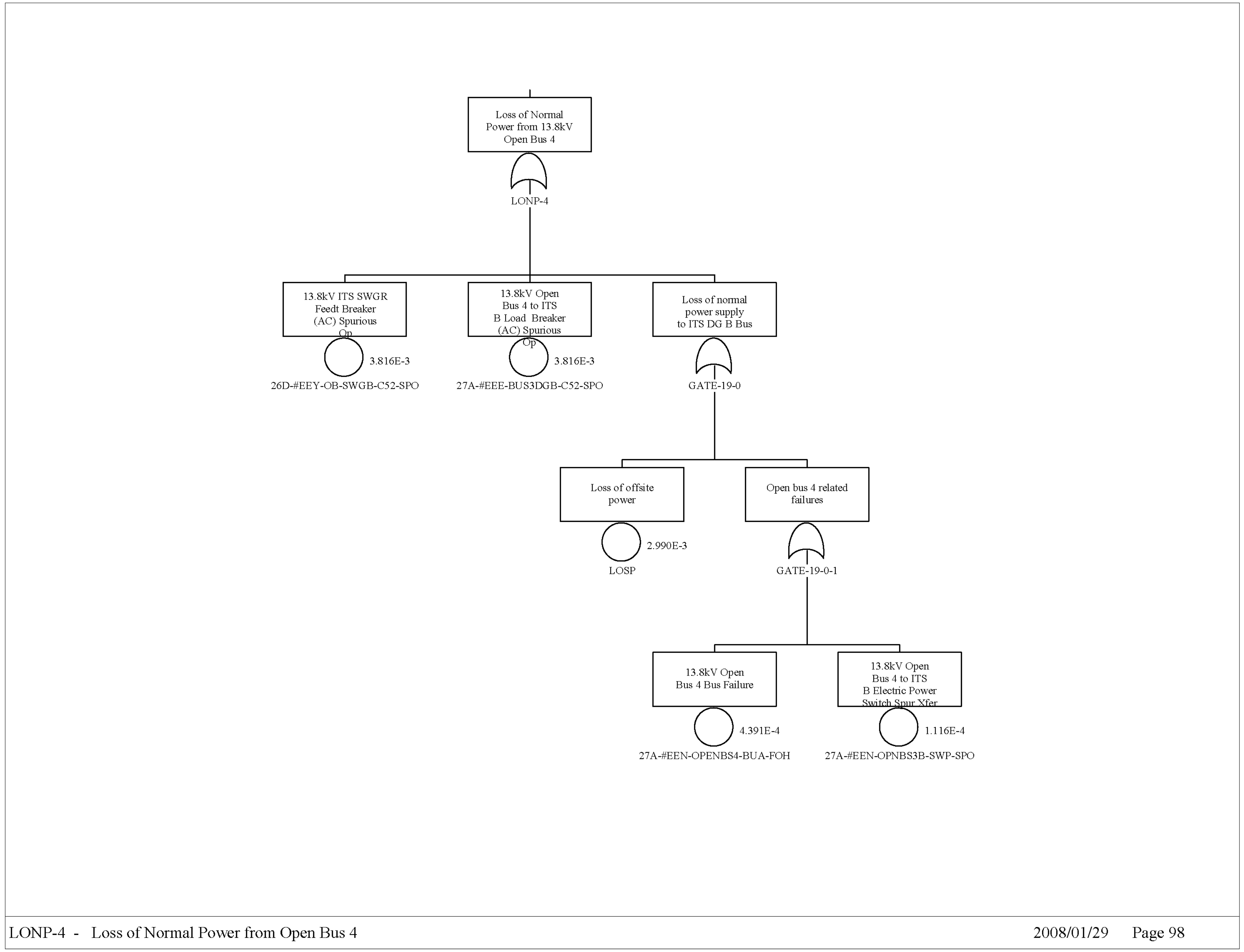
Source: Original

Figure B8.4-26. Loss of AC Power to RF ITS Load Center Train B (Sheet 10)



Source: Original

Figure B8.4-27. Loss of AC Power to RF ITS Load Center Train B (Sheet 11)



Source: Original

Figure B8.4-28. Loss of AC Power to RF ITS Load Center Train B (Sheet 12)

B9 PIVOTAL EVENT ANALYSIS

Miscellaneous linking fault trees that were not discussed in Attachment A are described in this section. Attachment A describes fault trees that provided links between the event trees and basic events, fault trees containing split fractions, and initiating event fault trees described in this attachment. This section describes the remaining types of initiating event fault trees that do not fit into these categories.

There are eight types of fault trees discussed in this section:

1. Dropping an object onto a cask or canister.
2. Impact to a cask by another vehicle or object.
3. Spurious movement of a crane causing impact to or tipping-over of a cask.
4. Loss of shielding leading to direct exposure.
5. Potential moderator sources.
6. Shield door impact with a conveyance.
7. Failure of shielding during canister transfer.
8. Failure in a large fire.

B9.1 FAULT TREES INVOLVING DROPPING AN OBJECT

These "drop on" fault trees describe dropping an object onto a cask or a canister and are listed in Table B9.1-1. A typical fault tree for drop of an object onto a transportation cask is shown in Figure B9.1-1.

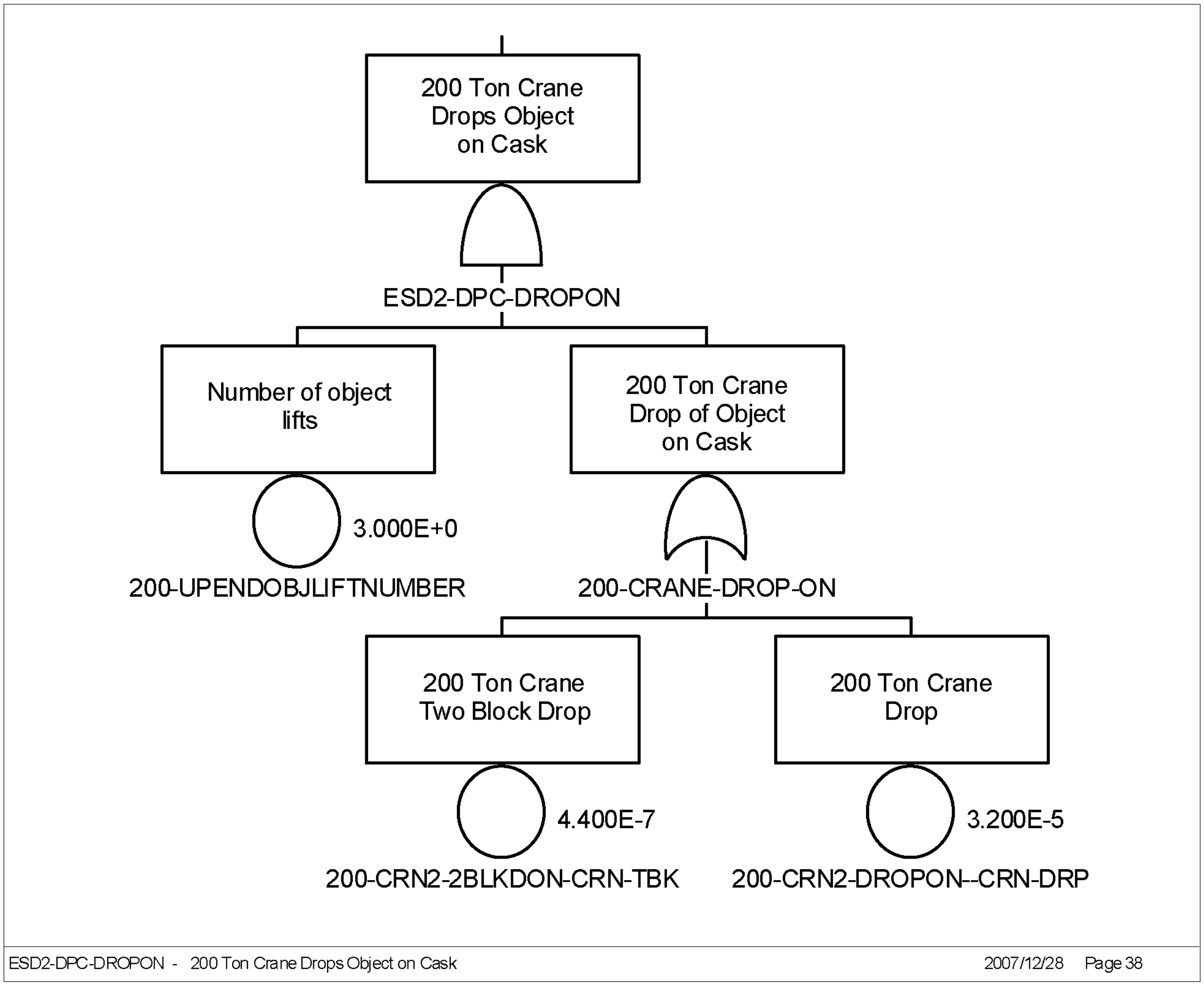
Table B9.1-1. Drop-On Fault Trees

Fault Tree Name	Applies To
ESD2-DPC-DROPON	DPC Transportation Cask
ESD2-TAD-DROPON	TAD Canister Transportation Cask
ESD3-DPC-DROPON	DPC Transportation Cask
ESD3-TAD-DROPON	TAD Canister Transportation Cask
ESD6-DPC-DROPON	DPC Transportation Cask
ESD6-TAD-DROPON	TAD Canister Transportation Cask
ESD7-DPC-DROPON	DPC in AO
ESD7-TAD-DROPON	TAD Canister in AO

NOTE: AO = aging overpack; DPC = dual-purpose canister; ESD = event sequence diagram; TAD = transportation, aging, and disposal.

Source: Original

In Figure B9.1-1, the fault tree is for a 200-ton crane drop of a lifting fixture or a lid onto a transportation cask or aging overpack from a normal height or from a much higher than normal height due to a two-blocking event. The probabilities of crane drops are based on historical data discussed in Section 6.3 and Attachment C.



Source: Original

Figure B9.1-1. Typical 200-Ton Crane Drop-On Fault Tree

ESD-06 and ESD-07 DPC/TAD "drop on" trees pertaining to the transportation cask or aging overpack are addressed in Attachment A.

B9.2 IMPACT TO A CASK BY ANOTHER VEHICLE OR OBJECT

These trees involve side impacts to the transportation cask by another vehicle or object. Table B9.2-1 lists the fault trees that describe these impacts.

Table B9.2-1. Transportation Cask Impact Fault Trees

Fault Tree Name	Applies To
ESD2-DPC-IMPACT	DPC transportation cask
ESD2-TAD-IMPACT	TAD canister transportation cask
ESD3-DPC-IMPACT	DPC transportation cask
ESD3-TAD-IMPACT	TAD canister transportation cask
ESD4-DPC-IMPACT	DPC transportation cask
ESD4-TAD-IMPACT	TAD canister transportation cask
ESD5-DPC-IMPACT	Impact of shield door into conveyance
ESD5-TAD-IMPACT	Impact of shield door into conveyance
ESD7-DPC-IMPACT	DPC aging overpack
ESD7-TAD-IMPACT	TAD canister aging overpack

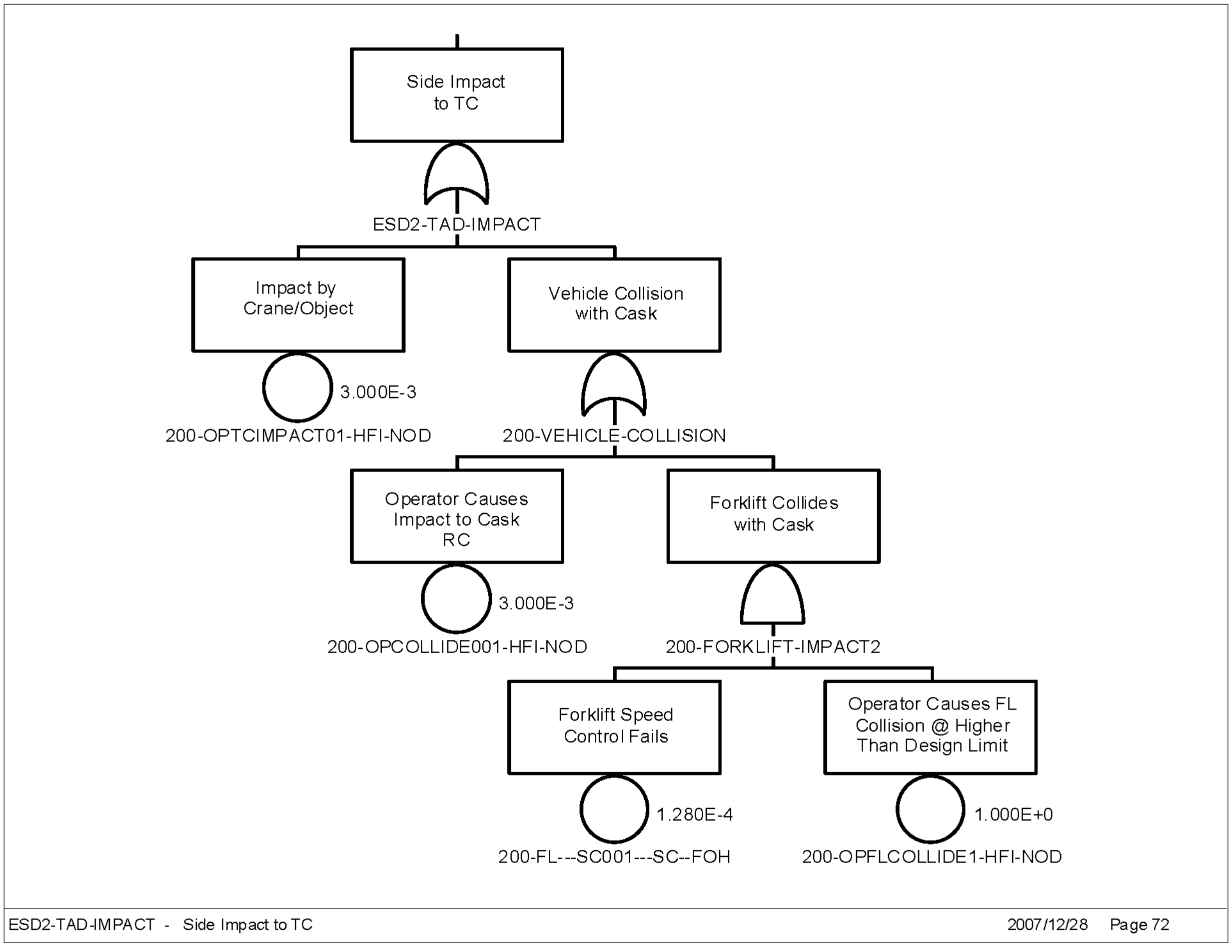
NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; TAD = transportation, aging, and disposal.

Source: Original

DPC and TAD canister impacts in ESD-04 are attributable to human error and discussed in Attachment A.

Figure B9.2-1 illustrates a side impact to a transportation cask for ESD-02 and ESD-07 due to the following operator errors:

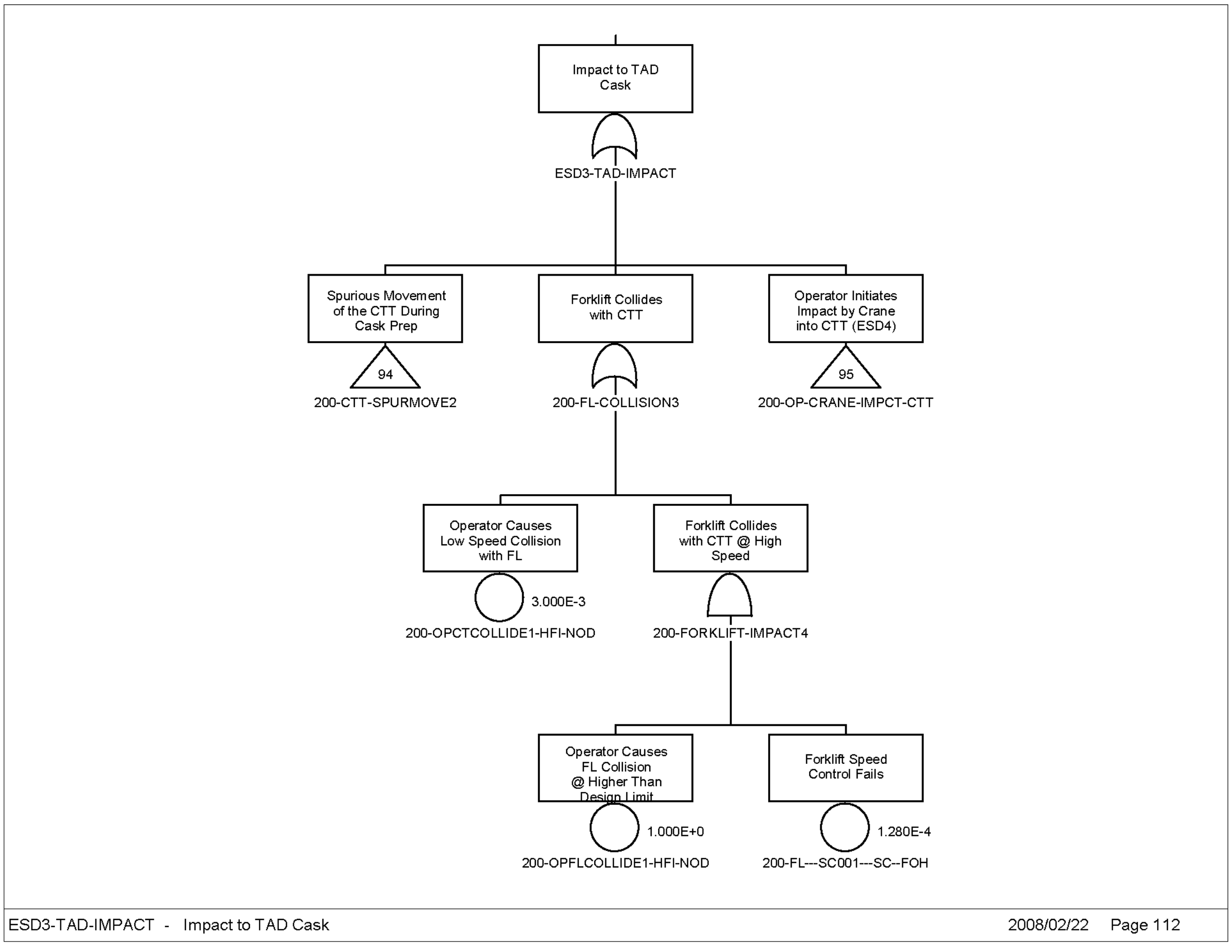
- Operator causing impact by the crane or object being carried by the crane
- Operator impacting a vehicle (such as a forklift) into the cask at the design speed
- Operator causing a forklift impact at higher than the design speed coupled with failure of the forklift speed control.



Source: Original

Figure B9.2-1. Typical Side Impact Fault Tree

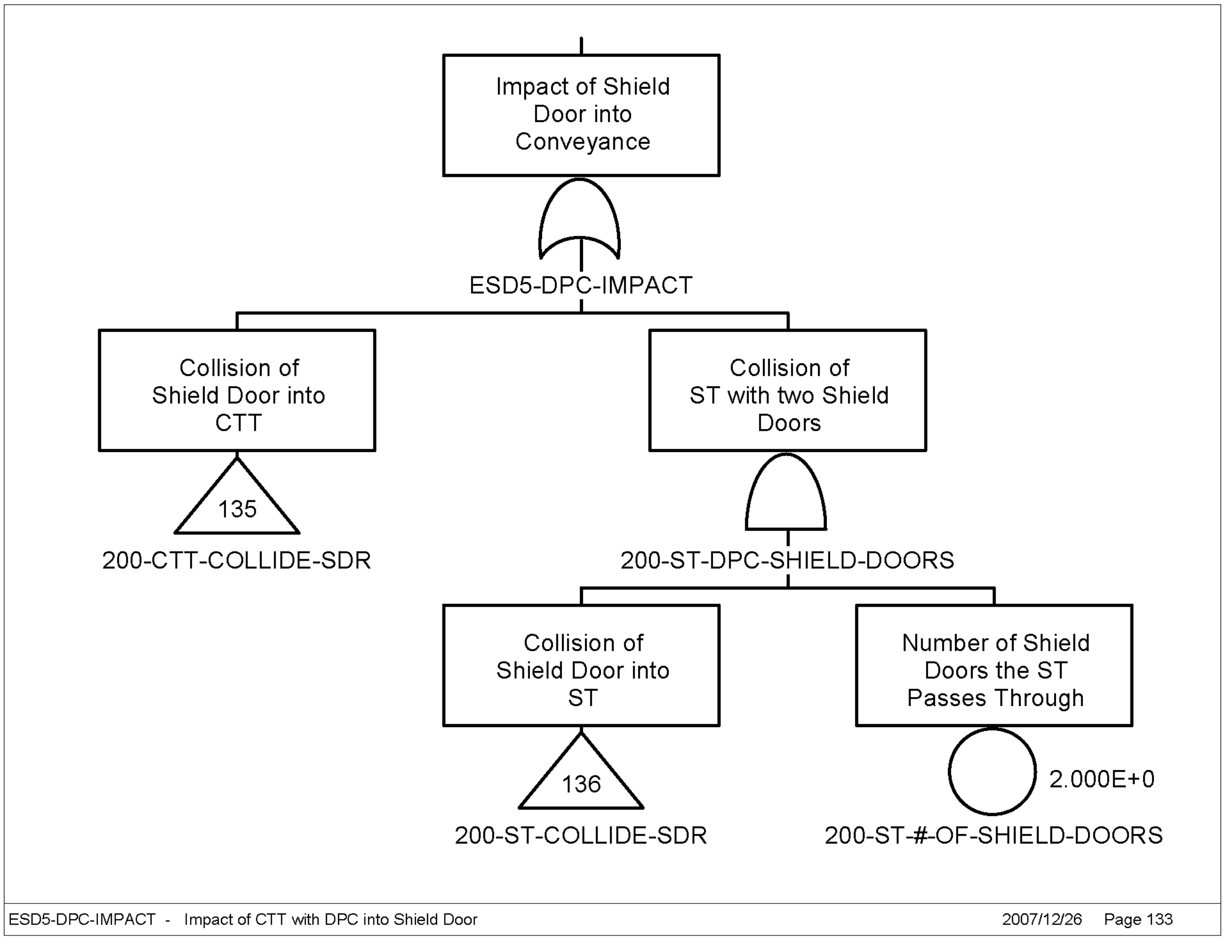
Figure B9.2-2 for ESD-03 is identical to Figure B9.2-1 with the addition of a possible side impact caused by the spurious movement of the CTT during cask loading. Details on spurious movement of the CTT during cask preparation are described in Attachment B, Section B2.



Source: Original

Figure B9.2-2. Typical Side Impact with Spurious Movement of CTT Fault Tree

Figure B9.2-3 for ESD-05 illustrates a side impact to the conveyance (either the cask transfer trolley or site transporter) by the shield door. The waste form is carried on a CTT or a site transporter where the site transporter passes through two shield doors and the CTT passes through one door. Details on the collision n of the shield door into the conveyance are described in Attachment B, Section B3.



Source: Original

Figure B9.2-3. Typical Side Impact of CTT with DPC to the Shield Door Fault Tree

B9.3 IMPACT TO A CASK DUE TO SPURIOUS MOVEMENT

These trees involve impacts to or tipover of the transportation cask due to operator error or spurious movements of the crane or CTT. Table B9.3-1 lists the fault trees that describe these impacts.

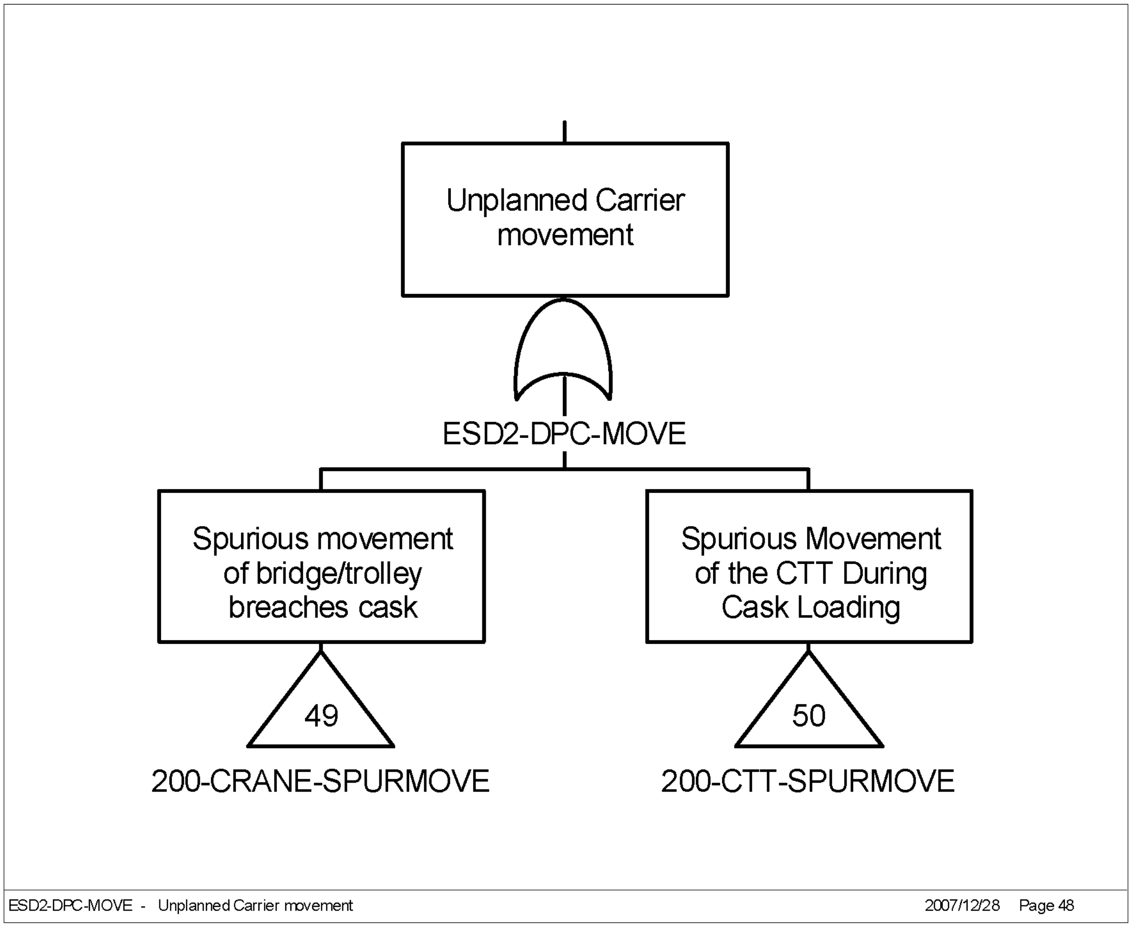
Table B9.3-1. Transportation Cask Impacts or Tip-over Fault Trees

Fault Tree Name	Applies To
ESD2-DPC-MOVE	DPC transportation cask
ESD2-TAD-MOVE	TAD canister transportation cask
ESD3-DPC-TIP	DPC transportation cask
ESD3-TAD-TIP	TAD canister transportation cask
ESD6-DPC-SPUR	DPC transportation cask
ESD6-TAD-SPUR	TAD canister transportation cask

NOTE: DPC = dual-purpose canister; ESD = event sequence diagram; TAD = transportation, aging, and disposal.

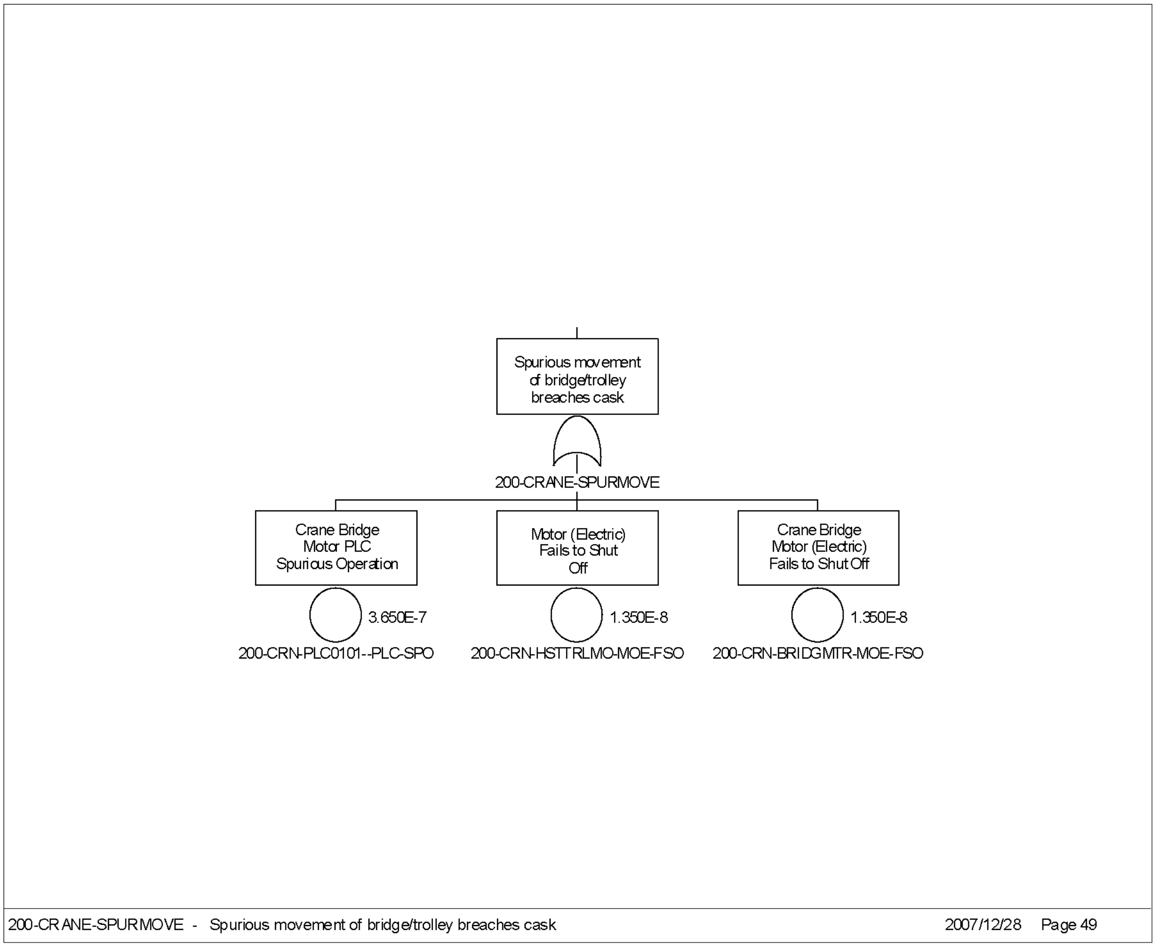
Source: Original

Figure B9.3-1 describes an impact to a cask due to spurious movement of the CTT during loading or spurious movement of the crane. The fault tree for spurious movement of the CTT (identified as transfer gate 200-CTT-SPURMOVE) is described in Attachment B, Section B2. Spurious movement of the crane occurs due to failure of either the crane bridge or hoist motor to shut off, or spurious signals from the crane bridge motor PLC which is illustrated in Figure B9.3-2.



Source: Original

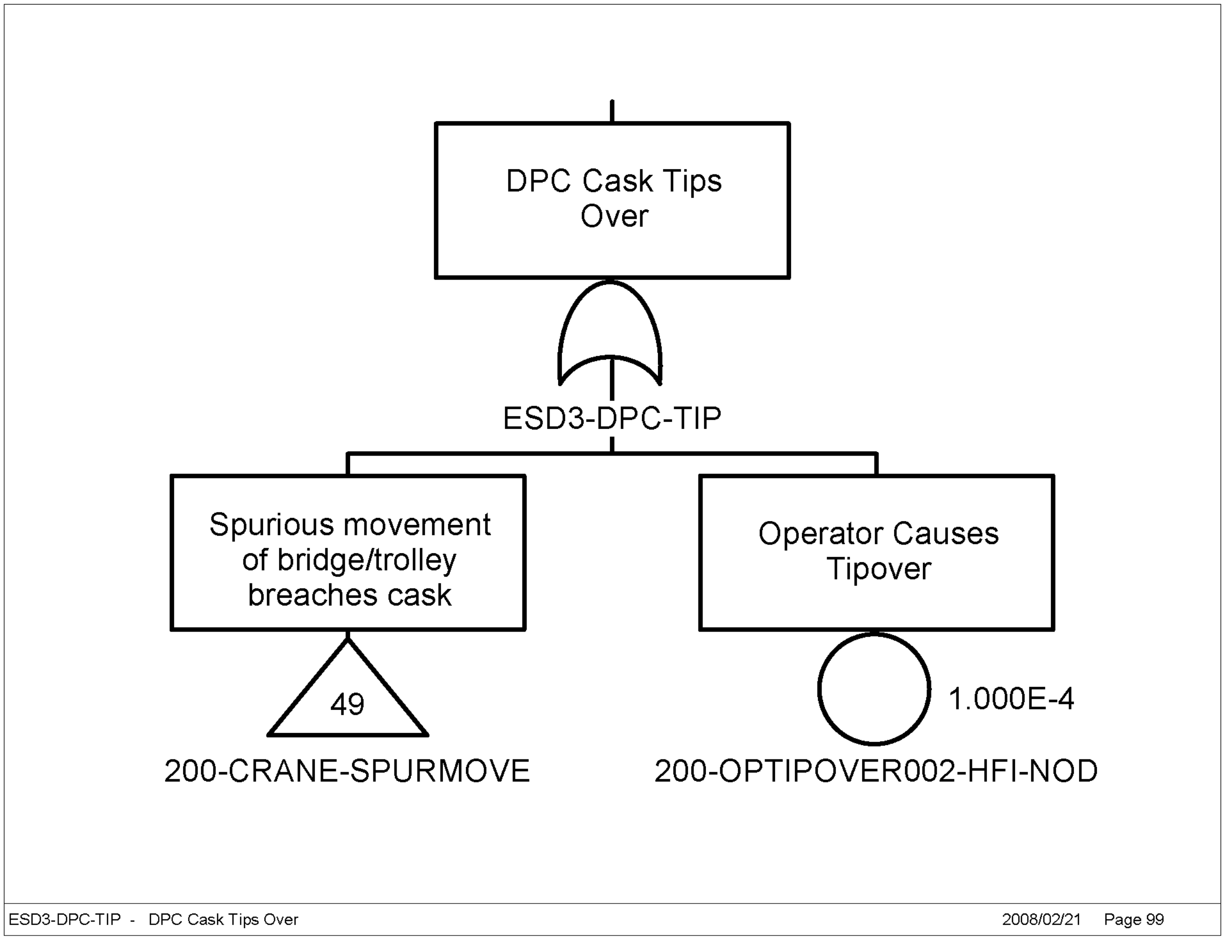
Figure B9.3-1. Spurious Movement of the Crane or CTT Fault Tree



Source: Original

Figure B9.3-2. Spurious Movement of the Crane Fault Tree

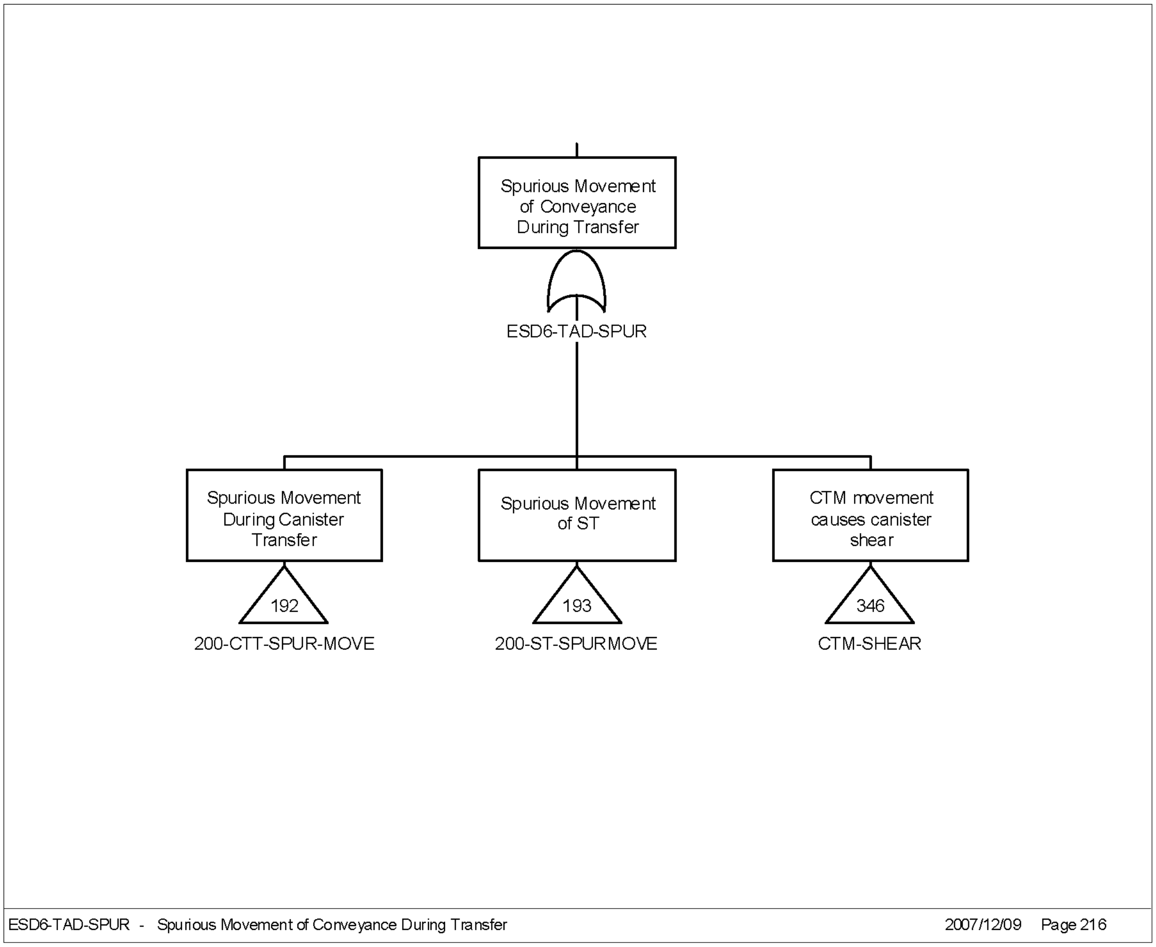
Impacts due to tip-over in ESD-03 caused by operator error or spurious movement of the crane are shown in Figure B9.3-3.



Source: Original

Figure B9.3-3. Tip-Over Fault Tree

ESD-06 spurious movement of conveyances addresses the possibility of impacts to the cask during movements on the CTT, site transporter, and CTM as shown in Figure B9.3-4. Details on 200-CTT-SPUR-MOVE are addressed in Attachment B, Section B2; 200-ST-SPURMOVE in Attachment B, Section B6; and 200-CTM-SHEAR in Attachment B, Section B4.



Source: Original

Figure B9.3-4. Spurious Conveyance Movement Fault Tree

B9.4 LOSS OF SHIELDING LEADING TO DIRECT EXPOSURE

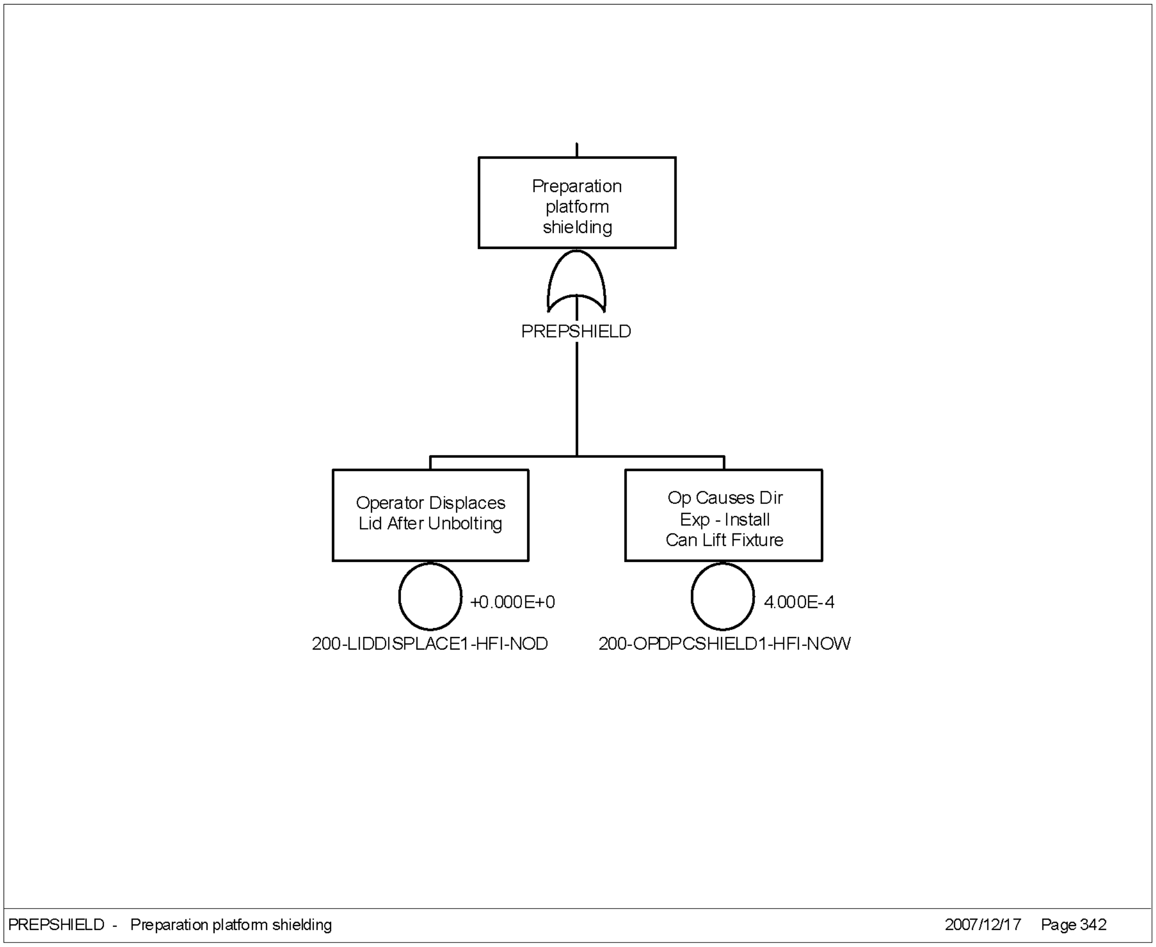
These fault trees describe direct exposure during canister transfer operations in the RF. Table B9.4-1 lists the fault trees that describe these direct exposures.

Table B9.4-1. Direct Exposure Fault Trees

Fault Tree Name	Applies To
PREPSHIELD	DPC and TAD canister
CTMSHIELD	DPC and TAD canister

NOTE: DPC = dual-purpose canister; TAD = transportation, aging, and disposal.
Source: Original

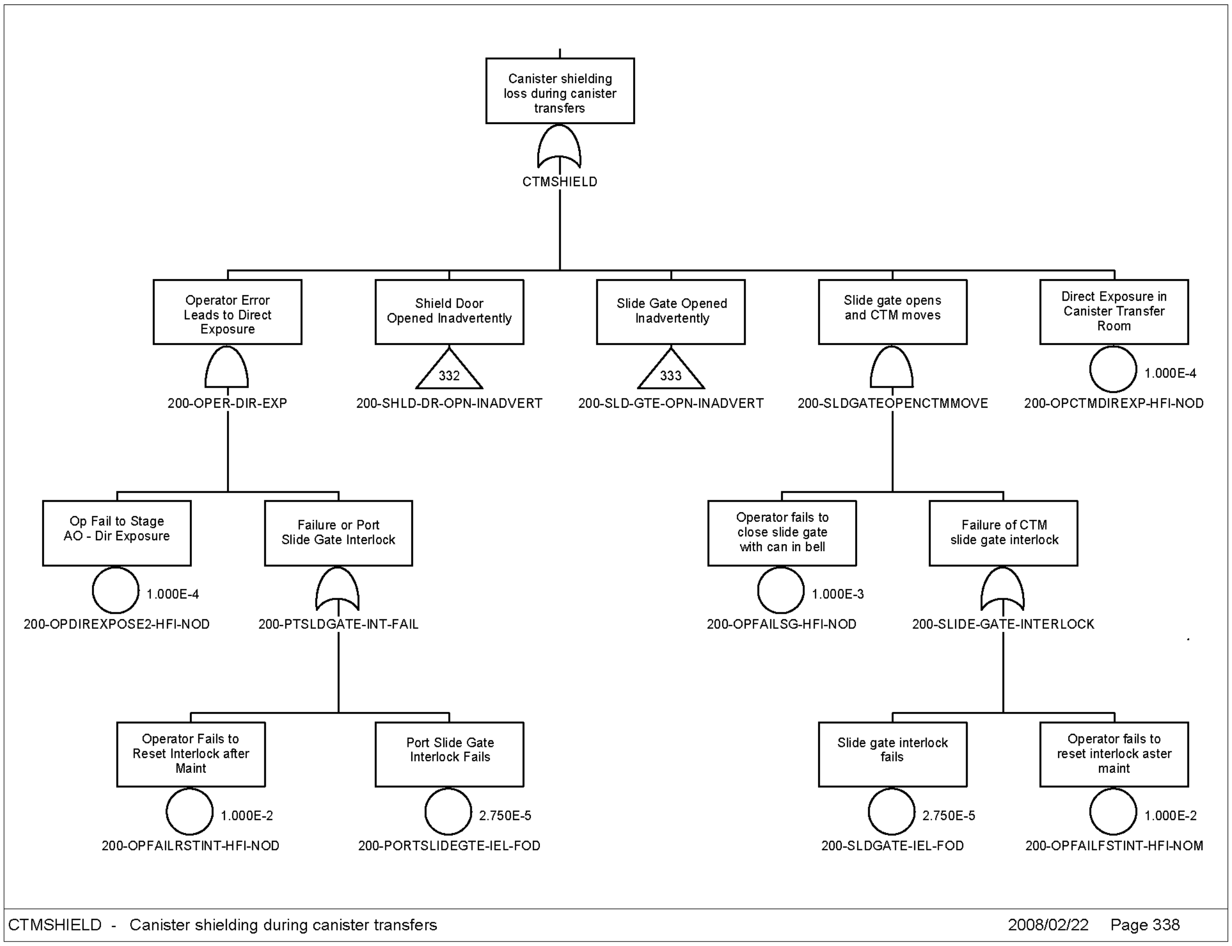
Figure B9.4-1 addresses the potential of a direct exposure resulting from human errors associated with transportation cask preparation activities for lid removal.



Source: Original

Figure B9.4-1. Human Errors Resulting in Direct Exposure during Cask Preparation Activities

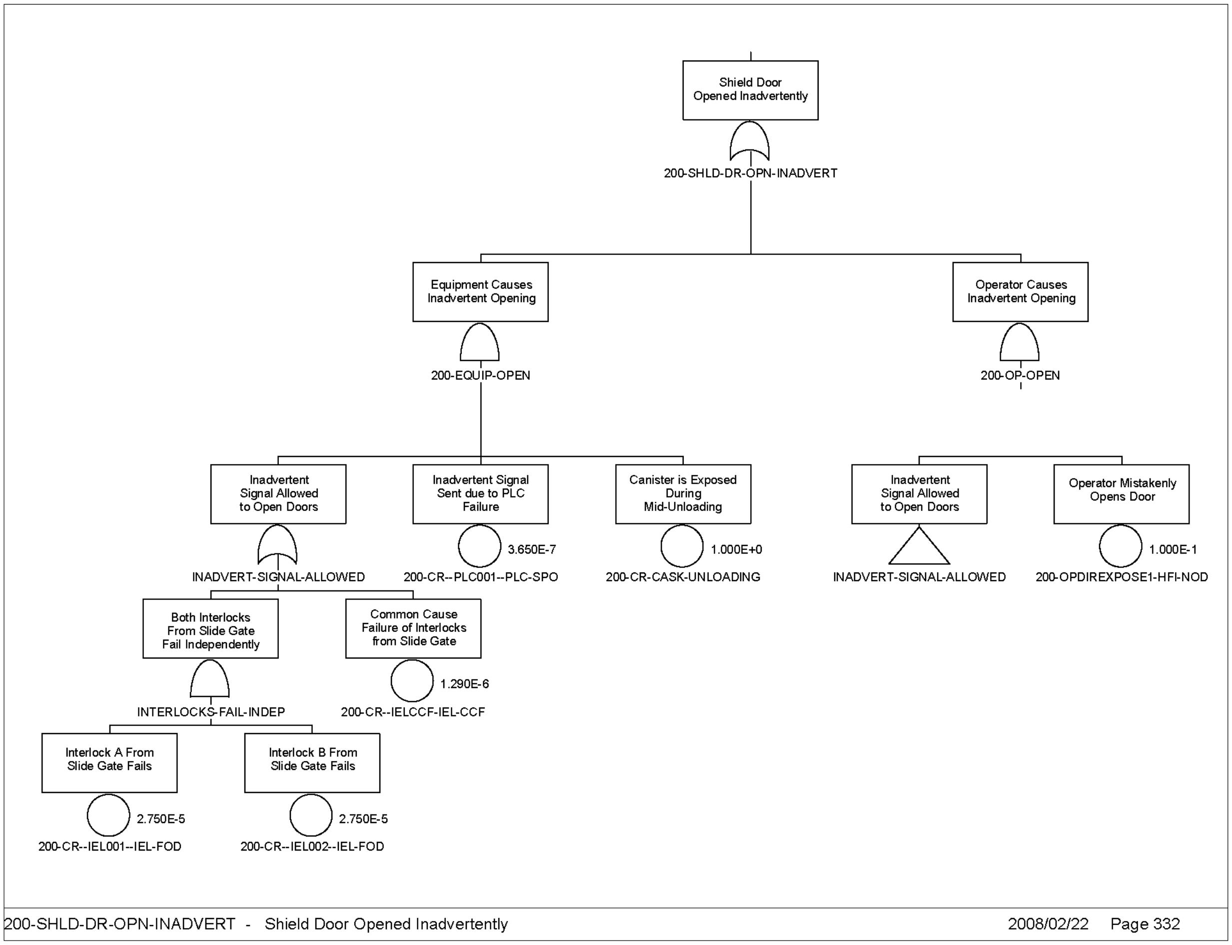
Figure B9.4-2 illustrates the potential causes of direct exposure during canister transfer. The potential causes include operator error coupled with interlock failures, and inadvertent opening of the shield door or slide gate. Fault trees for inadvertent opening of the shield door or slide gate are described in "Loading/Unloading Room Shield Door and Slide Gate Fault Tree Analysis" in Attachment B, Section B3.



Source: Original

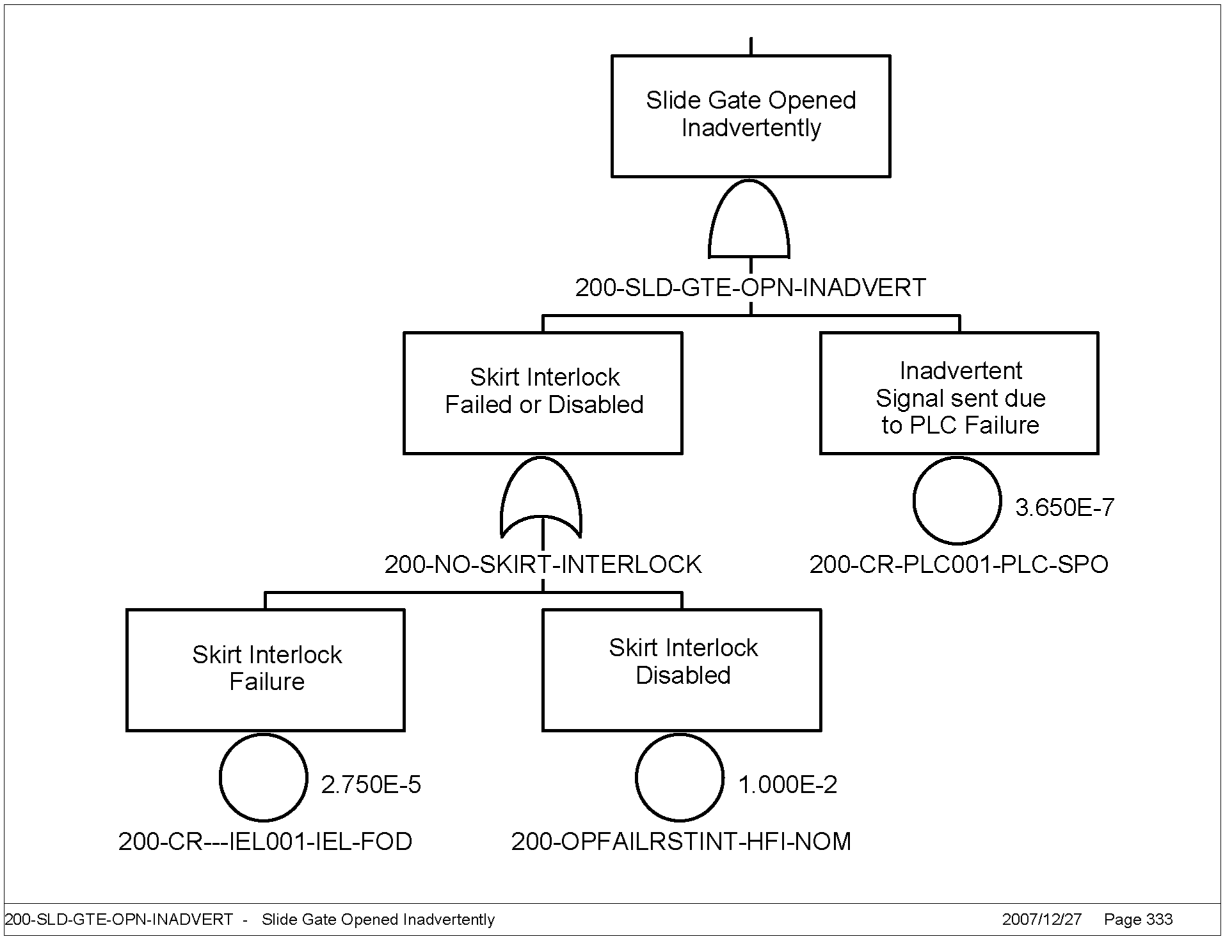
Figure B9.4-2. Typical Direct Exposure Fault Tree due to Shield Door or Slide Gate Opening

Figures B9.4-3 and B9.4-4 illustrate the failures associated with inadvertently opening of the shield door and slide gate respectively.



Source: Original

Figure B9.4-3. Shield Door Opened Inadvertently Resulting in Direct Exposure



Source: Original

Figure B9.4-4. Slide Gate Opened Inadvertently Resulting in Direct Exposure

B9.5 MODERATOR SOURCE

Internal floods are potential sources of moderator addition into a canister associated with pivotal events in the event sequences included in Section 6.1. Moderator addition into a canister can occur following a breach of the canister and a subsequent internal flooding. Table B9.5-1 lists the fault trees that describe the moderator events during RF operations.

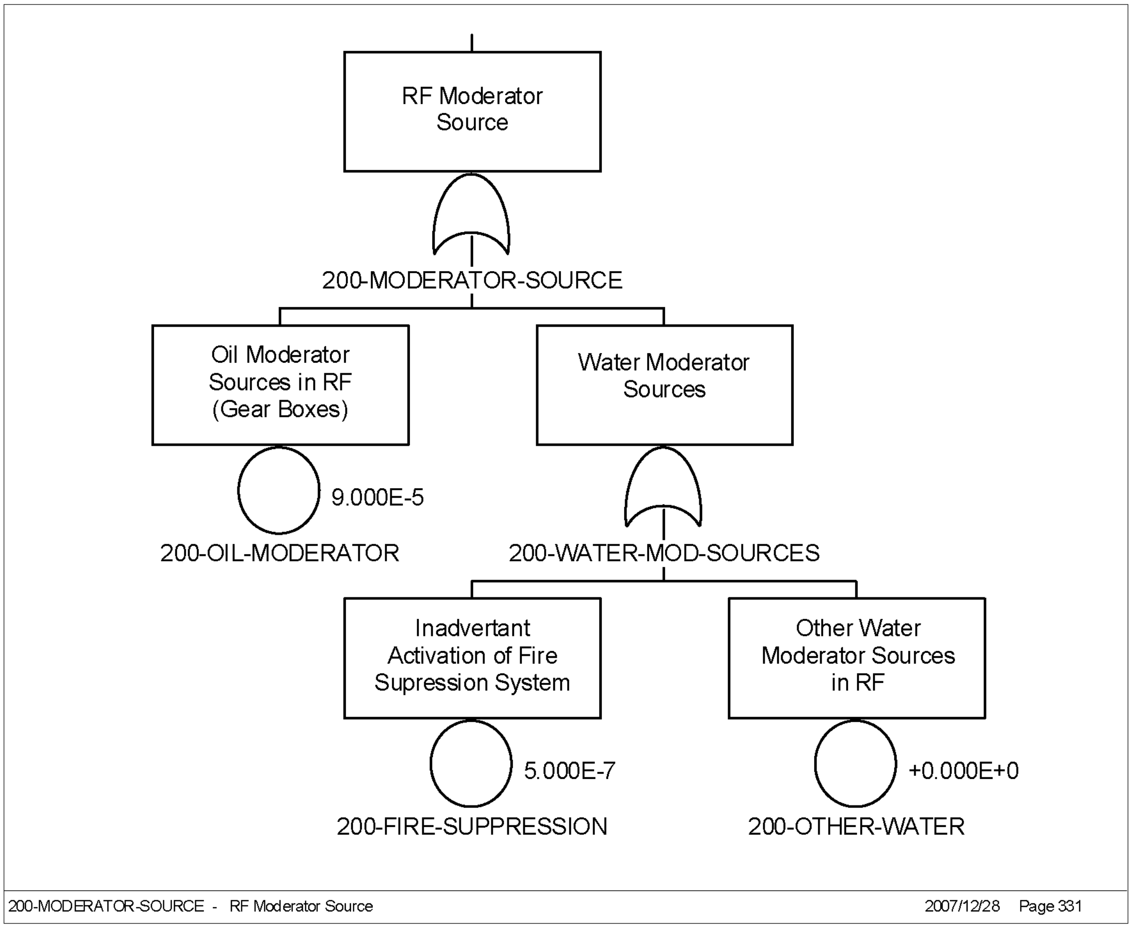
Table B9.5-1. Moderator Fault Trees

Fault Tree Name	Applies To
200-MODERATOR	DPC and TAD canister transportation cask and AO
200-MODERATOR-FIRE	DPC and TAD canister transportation cask and AO

NOTE: AO = aging overpack; DPC = dual-purpose canister; TAD = transportation, aging, and disposal.

Source: Original

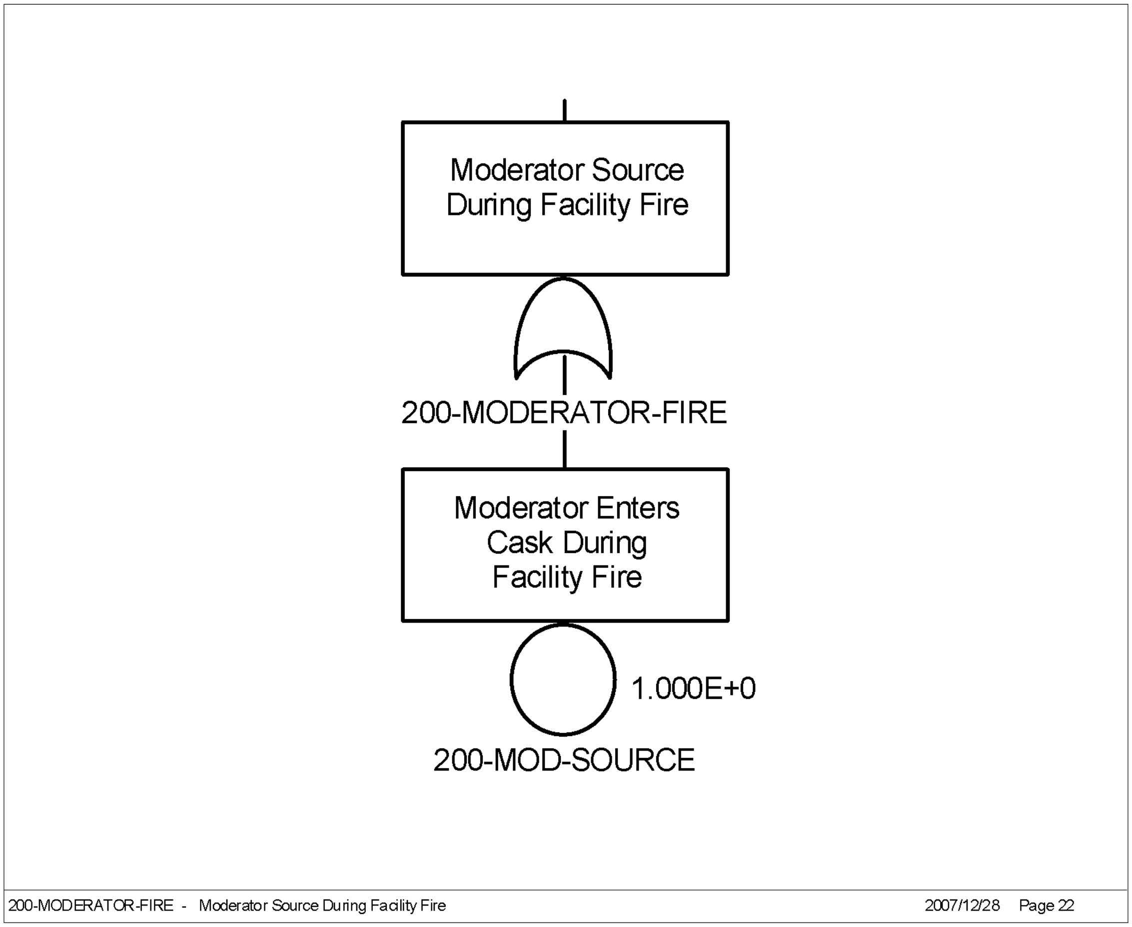
Figure B9.5-1 illustrates the possibility of a moderator source during normal operations in the RF. Potential sources are: oil from the 200-ton crane gear box, water from an inadvertent activation of the fire suppression system, and other water sources in the facility (e.g., water pipes). Details on moderator source failures are addressed in Section 6.2.2.9.



Source: Original

Figure B9.5-1. Moderator Source (no fire)

Figure B9.5-2 addresses the possibility of a moderator entering a cask during a facility fire in the RF. A conservative value of 1.000E+0 has been established for this event.



Source: Original

Figure B9.5-2. Moderator Source (Fire)

B9.6 IMPACT OF SHIELD DOOR INTO CONVEYANCE

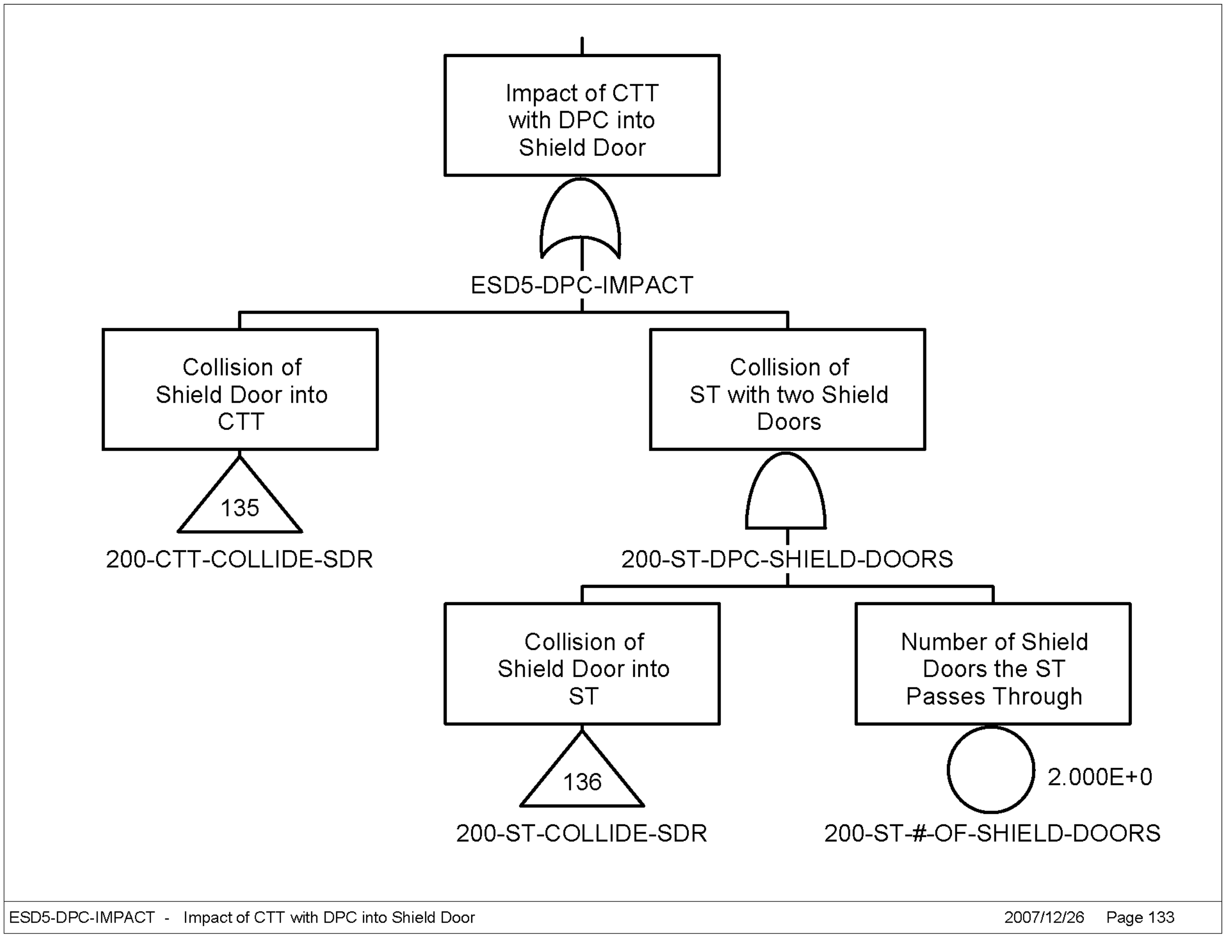
These fault trees describe collision of a moving shield door with the CTT or site transporter. Table B9.6-1 lists the fault trees that describe these impacts. The DPCs and TAD canisters are transported in the CRCF in transportation casks on a CTT or in aging overpacks on a site transporter

Table B9.6-1. Impact of Shield Door Fault Trees

Fault Tree Name	Applies To
ESD5-DPC-IMPACT	DPCs in transportation casks or AOs
ESD5-TAD-IMPACT	TADs in transportation casks or AOs

NOTE: AO = aging overpack; DPC = dual-purpose canister; TAD = transportation, aging, and disposal canister.
Source: Original

Figure B9.6-1 illustrates the fault tree for shield door impact to a conveyance carrying a DPC. The DPC are carried on a CTT or a site transporter where the site transporter passes through two shield doors and the CTT passes through one door. The same quantity of DPCs that are carried on a CTT are also carried on a site transporter. The fault trees for collision of the shield door into a CTT or site transporter are described in Attachment B3.

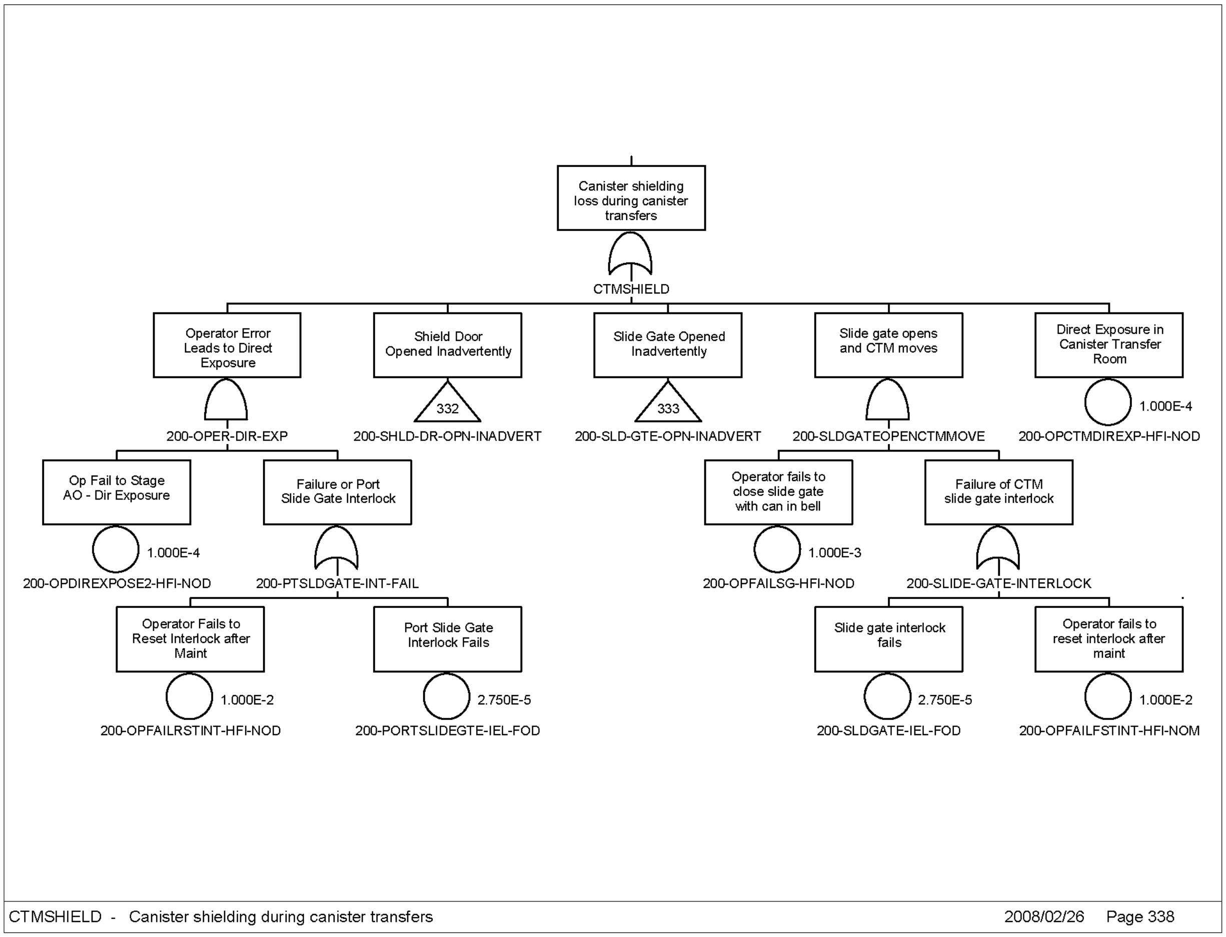


Source: Original

Figure B9.6-1. Impact of Shield Door into Conveyance with DPC

B9.7 SHIELDING FAILURE DURING CANISTER TRANSFERS

These fault trees describe the failure of shielding leading to direct exposure during canister transfers. Figure B9.7-1 describes the failure of shielding during DPC or TAD canister transfers by the CTM. Fault trees for inadvertent opening of the shield door or slide gate are described in Attachment B3.



Source: Original

Figure B9.7-1. Canister Shielding Loss during Canister Transfers

B9.8 CASK OR CANISTER FAILURE IN A FIRE

These fault trees (Figures B9.8-1 and B9.8-2) describe the probability of failure of a cask or canister in a large fire and involve split fractions associated with the probability of the canister being in a transportation cask, aging overpack, or CTM. The probability that the canister is in a particular configuration is derived from fire data in Attachment F and the derivation is shown in Section 6.5. The failure probability in a diesel versus a non-diesel fire is the same in these calculations.

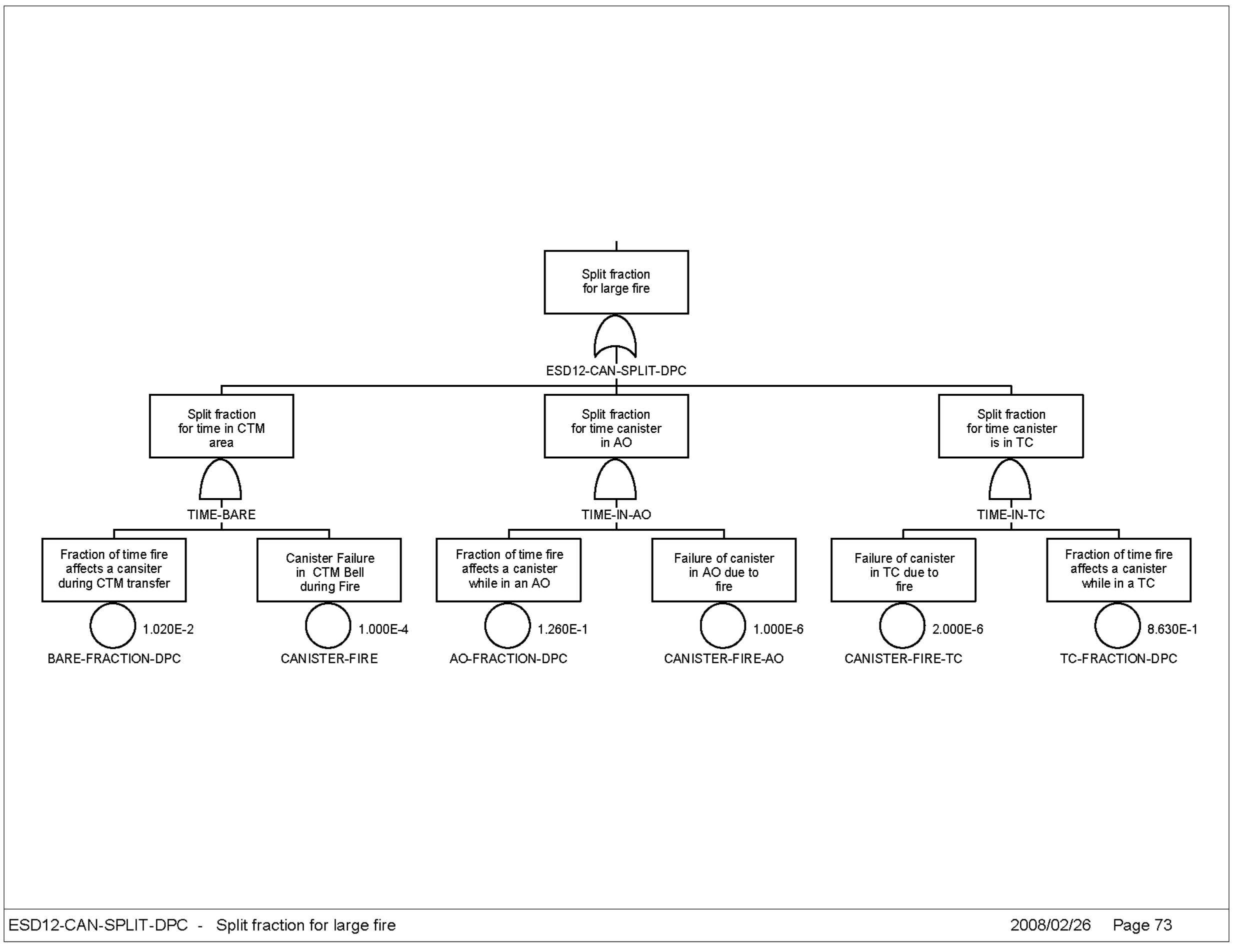
Table B9.8-1 lists the fault trees that describe these failures.

Table B9.8-1. Fault Trees for Canister Failure in a Fire

Fault Tree Name	Applies To
ESD12-CAN-SPLIT-DPC	DPC in TC threatened by large fire
ESD12-CAN-SPLIT-TAD	TAD in TC threatened by large fire

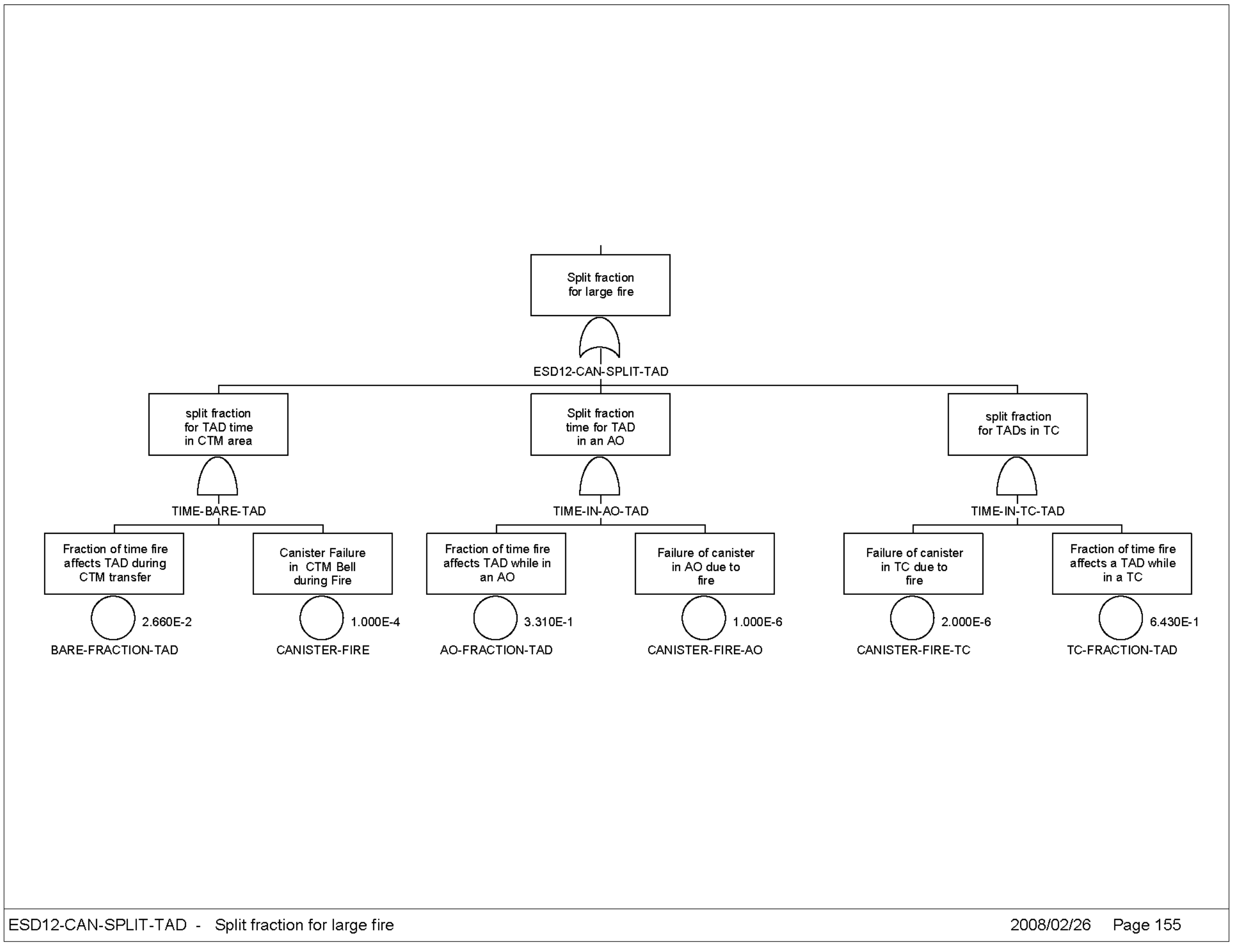
NOTE: DPC = dual-purpose canister; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original



Source: Original

Figure B9.8-1. DPC Failure in a Large Fire



Source: Original

Figure B9.8-2. TAD Canister Failure in a Large Fire

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ACTIVE COMPONENT RELIABILITY DATA ANALYSIS

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ACRONYMS AND ABBREVIATIONS

Acronyms

CCF	common-cause failure
CTM	canister transfer machine
CTT	cask transfer trolley
DOE	U.S. Department of Energy
GROA	geologic repository operations area
HEPA	high-efficiency particulate air filter
HLW	high-level radioactive waste
HVAC	heating, ventilation, and air conditioning
MCC	motor control centers
MCO	multicanister overpack
NRC	U.S. Nuclear Regulatory Commission
PCSA	preclosure safety analysis
PRA	probabilistic risk assessment
SFTM	spent fuel transfer machine
SNF	spent nuclear fuel
TEV	transport and emplacement vehicle
TYP	component type code
TYP-FM	component type and failure mode code
UPS	uninterruptible power supply
YMP	Yucca Mountain Project

Abbreviations

AC	alternating current
DC	direct current
hr	hour

ATTACHMENT C
ACTIVE COMPONENT RELIABILITY DATA ANALYSIS

The purpose of component-level reliability data analysis is to provide reliability information for logic model quantification at the appropriate level agreed upon by the systems and data analysts. In this report, the term data is taken to mean reliability data analyzed as part of the preclosure safety analysis (PCSA) from published sources. The fault tree models described in Section 4.3.2 include random failures of active mechanical equipment as basic events. In order to numerically solve these models, estimates of the likelihood of failure of these equipment basic events are needed. This attachment provides a summary of the approach for developing these active component reliability estimates by gathering and reviewing industry-wide data, and applying Bayesian combinatorial methods to develop mean values and uncertainty bounds that best represented the range of the industry-wide information. The discussion also addresses the method used for estimating the probability of common-cause failures among multiple components. Finally, a table is given showing the template data values input to the Yucca Mountain Project (YMP) PCSA SAPHIRE models (Section 4.2).

C1 INDUSTRY-WIDE COMPONENT RELIABILITY DATA

While data from the facility being studied is the preferred source of equipment failure rate information, it is common in a safety analysis for information from other facilities in the same industry to be used when facility-specific data is sparse or unavailable. Because the YMP activities are atypical of nuclear power plant activities and no operating history exists, it was necessary to develop the required data from the experience of other industries.

C1.1 COMPONENT DEFINITION

The purpose of component-level data analysis is to provide reliability information for logic model quantification at the appropriate level agreed upon by the systems and data analysts. To do this, it is necessary to clearly define component types, boundaries, and failure modes. The system analysis fault tree basic events identify the component and failure mode combinations requiring data, and the analysts' descriptions provide an understanding of the component operating environments. In response to these identified data needs, the data analysts compile data at the component failure mode level for input to the SAPHIRE models. However, this is best achieved via an iterative process between the system and data analysts to ensure that all basic events are properly quantified with appropriate failure data estimates.

1. Component Type. Corresponds to the category of equipment at the level for which data is required by the logic model and at which data will be developed by the data analyst. Examples of such component types are motor-driven pumps, cameras, diesel generators, and heat exchangers. For certain complex components, a larger component type such as the canister transfer machine (CTM) is likely to be broken down by the system analyst in the logic model into constituent component types including motors and brakes, not only to facilitate the data analysis but to evaluate the contribution of various subcomponents to the overall component failure.

2. Component Boundaries. The boundary definition task is closely connected with the tasks of defining systems boundaries and fault tree construction. Therefore this task is performed jointly with the system analysts.

3. Failure Mode. Failure mode is defined as an undesirable component state (e.g., normally closed motor operated valve doesn't open on demand because of valve mechanical damage that occurred before the demand itself).

4. Selection of Model and Parameters. Stochastic models of failures of different systems component are defined for component failure probability estimation depending on the system operational mode. A set of available models is given in SAPHIRE for Windows and includes the following:

   A. Components of stand-by systems. The main parameter of stand-by system is the unavailability upon demand. Such system unavailability can be modeled by fault tree, where basic events probabilities are equal to system components unavailabilities averaged by time. This model treats the time to failure as a random value with exponential distribution. Such component unavailability is the function of time. In case of periodic test, unavailability is a periodic function of time. For simplifying the calculation, time dependency is usually replaced by the average value over the considered interval. For periodically tested components, the interval average is the average value for the test interval.

      Three types of stand-by system components are identified:

      1) Periodically tested stand-by components. For such components it is necessary to estimate following parameters: failure rate, probability of failure per demand, average restoring time (for repair), and average outage time due to test and maintenance.

      2) Non-tested stand-by component. For such components, the exposure time is set to unit projected operation time for calculation of unavailability. But often the component is tested indirectly or replaced. For example, if the system gets a real actuation signal, the state of the non-tested component can be determined. In this case, the average time to failure for a component is set to the average interval between system actuations. In some instances, the component can be replaced along with the tested components. In this case, test interval for non-tested component is set to average time to failure of tested component.

      3) Monitored components. State of some stand-by components is tested continuously (monitoring). In this case component failure is revealed immediately.

B. Components of systems in operation. For systems in operation, the most important parameter is the probability of failure during the defined mission time. This probability may be estimated based on fault trees or another logic model, where basic event probabilities are set to unavailabilities of components over the interval mission time. Failures of operating components are modeled using an exponentially distribution with a failure rate different from the failure rate in stand-by mode.

Operating systems contain two main types of components: restorable and non-restorable.

1) Non-restorable components. Components that cannot be restored in case of failure. Exponential distribution of time between failures for such components is characterized by failure rate, λ.

2) Restorable components. Components that may be restored in case of failure. In this case restoration means restoration without outage of operation.

C. Stand-by systems following demand. Stand-by systems must fulfill a specific function during the defined time after successful start. During this time such systems are described in the same way as operating systems.

D. Constant probability per demand. The model treats component failure probability as a fixed probability for every demand. For such components, tests are excluded from consideration.

For YMP, the operational mode of failure and standby failures predominate; therefore, constant failure rates and constant probabilities per demand were constructed.

Component types and failure modes were initially identified based upon a listing of the components considered to be likely to be encountered in the analysis. This list was compiled from expertise in database development and familiarity with general component requirements in a variety of facilities. As the fault tree modeling progressed, this list was augmented and tailored to the specific active components included in the PCSA models based on the YMP design.

Correspondingly, it was necessary to develop an active component and failure mode coding scheme that would be consistent with the fault tree model basic events, the needs of the SAPHIRE models, as well as with standard repository naming conventions for YMP equipment types.

The YMP PCSA basic event naming convention was therefore developed to incorporate the following information in the 24 character basic event (BE) name (consistent with the BE field in SAPHIRE):

- Area code – physical design or construction area where a component would be installed
- System locator code – operational systems and processes

- Component function identifiers – component function
- Sequence code – numeric sequence and train assignment
- Component type code – three character identifier for general component type, such as battery, actuator, or pump
- Failure mode code – three character identifier for the way in which the component is considered in the fault tree models to have failed, (e.g., FTS for fails to start or FOD for fails on demand).

The area, system locator, and component function codes were obtained from engineering standards from the YMP repository as a whole to be consistent with overall site naming conventions. The sequence codes were taken from the component identification numbers on project drawings, if the design had progressed to that point at the time of the data development and modeling.

Active component type codes were developed to be consistent with the component function identifiers, but since the type codes were limited to three digits and the function identifiers were occasionally four-characters long, in some instances it was necessary to truncate the identifier to construct the type code.

Failure mode codes (FM) were developed using prior database conventions or abbreviations that would be as intuitively obvious as possible.

Both type (TYP) and failure mode were limited to three characters each in order to be consistent with the input constraints and conventions of the SAPHIRE template database feature, which allows the same component failure data to be applied to all items in the model.

A list of the component type and failure mode combinations is provided in Table C1.1-1.

Industry-wide data sources were then collected and reviewed to identify failure rates per hour or failure probabilities per demand that would be relevant to each of the 146 TYP-FM combinations.

Table C1.1-1. YMP PCSA Component Types (TYP) and Failure Modes (FM)

TYP-FM	Component Name & Failure Mode
AHU-FTR	Air Handling Unit Failure to Run
ALM-SPO	Alarm/Annunciator Spurious Operation
AT-FOH	Actuator (Electrical) Failure
ATH-FOH	Actuator (Hydraulic) Failure
ATP-SPO	Actuator (Pneumatic Piston) Spurious Operation
AXL-FOH	Axle Failure
B38-FOH	Bearing Failure
BEA-BRK	Lifting Beam/Boom Breaks
BLD-RUP	Air Bag Ruptures

Table C1.1-1. YMP PCSA Component Types (TYP) and Failure Modes (FM) (Continued)

TYP-FM	Component Name & Failure Mode
BLK-FOD	Block or Sheaves Failure on Demand
BRH-FOD	Brake (Hydraulic) Failure on Demand
BRK-FOD	Brake Failure on Demand
BRK-FOH	Brake (Electric) Failure
BRP-FOD	Brake (Pneumatic) Failure on Demand
BRP-FOH	Brake (Pneumatic) Failure
BTR-FOD	Battery No Output Given Challenge
BTR-FOH	Battery Failure
BUA-FOH	AC Bus Failure
BUD-FOH	DC Bus Failure
BYC-FOH	Battery Charger Failure
C52-FOD	Circuit Breaker (AC) Fails on Demand
C52-SPO	Circuit Breaker (AC) Spurious Operation
C72-SPO	Circuit Breaker (DC) Spurious Operation
CAM-FOH	Cam Lock Fails
CBP-OPC	Cables (Electrical Power) Open Circuit
CBP-SHC	Cables (Electrical Power) Short Circuit
CKV-FOD	Check Valve Fails on Demand
CKV-FTX	Check Valve Fails to Check
CON-FOH	Electrical Connector (Site Transporter) Failure
CPL-FOH	Coupling (Automatic) Failure
CPO-FOH	Control system Onboard (TEV or Trolley) Failure
CRD-FOH	Badge/Card Reader Failure
CRJ-DRP	Jib Crane Load Drop
CRN-DRP	200-Ton Crane Load Drop
CRN-TBK	200-Ton Crane Two-Blocking Load Drop
CRS-DRP	Crane using Slings Load Drop
CRW-DRP	Waste Package Crane Load Drop
CRW-TBK	Waste Package Crane Two-Blocking Load Drop
CSC-FOH	Cask Cradle Failure
CT-FOD	Controller Mechanical Jamming
CT-FOH	Controller Failure
CT-SPO	Controller Spurious Operation
CTL-FOD	Logic Controller Fails on Demand
DER-FOM	Derailment Failure per Mile
DG-FTR	Diesel Generator Fails to Run
DG-FTS	Diesel Generator Fails to Start
DGS-FTR	Diesel Generator - Seismic - Fails to Run for 29 Days
DM-FOD	Drum Failure on Demand
DM-MSP	Drum Misspooling (Hourly)
DMP-FOH	Damper (Manual) Fails to Operate
DMP-FRO	Damper (Manual) Fails to Remain Open (Transfers Closed)

Table C1.1-1. YMP PCSA Component Types (TYP) and Failure Modes (FM) (Continued)

TYP-FM	Component Name & Failure Mode
DMS-FOH	Demister (Moisture Separator) Failure
DRV-FOH	Drive (Adjustable Speed) Failure
DRV-FSO	Drive (Adjustable Speed) Failure to Stop on Demand
DTC-RUP	Duct Ruptures
DTM-FOD	Damper (Tornado) Failure on Demand
DTM-FOH	Damper (Tornado) Failure
ECP-FOH	Position Encoder Failure
ESC-FOD	Emergency Stop Button Controller Failure to Stop (on Demand)
FAN-FTR	Fan (Motor-Driven) Fails to Run
FAN-FTS	Fan (Motor-Driven) Fails to Start on Demand
FRK-PUN	Forklift Puncture
G65-FOH	Governor Failure
GPL-FOD	Grapple Failure on Demand
GRB-FOH	Gear Box Failure
GRB-SHH	Gear Box Shaft/Coupling Shears
GRB-STH	Gear Box Stripped
HC-FOD	Hand Held Radio Remote Controller Fails to Stop (on Demand)
HC-SPO	Hand Held Radio Remote Controller Spurious Operation
HEP-LEK	Filter (HEPA) Leaks [Bypassed]
HEP-PLG	Filter (HEPA) Plugs
HOS-LEK	Hose Leaking
HOS-RUP	Hose Ruptures
IEL-FOD	Interlock Failure on Demand
IEL-FOH	Interlock Failure
LC-FOD	Level Controller Failure on Demand
LRG-FOH	Lifting Rig or Hook Failure
LVR-FOH	Lever (Two Position; Up-Down) Failure
MCC-FOH	Motor Control Centers (MCCs) Failure
MOE-FOD	Motor (Electric) Fails on Demand
MOE-FSO	Motor (Electric) Fails to Shut Off
MOE-FTR	Motor (Electric) Fails to Run
MOE-FTS	Motor (Electric) Fails to Start (Hourly)
MOE-SPO	Motor (Electric) Spurious Operation
MSC-FOH	Motor Speed Control Module Failure
MST-FOH	Motor Starter Failure
NZL-FOH	Nozzle Failure
PIN-BRK	Pin (Locking or Stabilization) Breaks
PLC-FOD	Programmable Logic Controller Fails on Demand
PLC-FOH	Programmable Logic Controller Fails to Operate
PLC-SPO	Programmable Logic Controller Spurious Operation
PMD-FTR	Pump (Motor Driven) Fails to Run
PMD-FTS	Pump (Motor Driven) Fails to Start on Demand

Table C1.1-1. YMP PCSA Component Types (TYP) and Failure Modes (FM) (Continued)

TYP-FM	Component Name & Failure Mode
PPL-RUP	Piping (Lined) Catastrophic
PPM-PLG	Piping (Water) Plugs
PPM-RUP	Piping (Water) Ruptures
PR-FOH	Passive Restraint (Bumper) Failure
PRM-FOH	eProm (HVAC Speed Control) Failure
PRV-FOD	Pressure Relief Valve Fails on Demand
PV-SPO	Pneumatic Valve Spurious Operation
QDV-FOH	Quick Disconnect Valve Failure
RCV-FOH	Air Receiver Fails to Supply Air
RLY-FTP	Relay (Power) Fails to Close/Open
SC-FOH	Speed Control Failure
SC-SPO	Speed Control Spurious Operation
SEL-FOH	Speed Selector Fails
SEQ-FOD	Sequencer Fails on Demand
SFT-COL	Spent Fuel Transfer Machine Collision/Impact
SFT-DRP	Spent Fuel Transfer Machine Fuel Drop
SFT-RTH	Spent Fuel Transfer Machine Fuel Raised Too High
SJK-FOH	Screw jack (TEV) Failure
SRF-FOH	Flow Sensor Failure
SRP-FOD	Pressure Sensor Fails on Demand
SRP-FOH	Pressure Sensor Fails
SRR-FOH	Radiation Sensor Fails
SRS-FOH	Over Speed Sensor Fails
SRT-FOD	Temperature Sensor/Transmitter Fails on Demand
SRT-FOH	Temperature Sensor/Transmitter Fails
SRT-SPO	Temperature Sensor Spurious Operation
SRU-FOH	Ultrasonic Sensor Fails
SRV-FOH	Vibration Sensor (Accelerometer) Fails
SRX-FOD	Optical Position Sensor Fails on Demand
SRX-FOH	Optical Position Sensor Fails
STU-FOH	Structure (Truck or Railcar) Failure
SV-FOD	Solenoid Valve Fails on Demand
SV-FOH	Solenoid Valve Fails
SV-SPO	Solenoid Valve Spurious Operation
SWA-FOH	Switch, Auto-Stop Fails (CTT end of Hose Travel)
SWG-FOH	13.8kV Switchgear Fails
SWP-FTX	Electric Power Switch Fails to Transfer
SWP-SPO	Electric Power Switch Spurious Transfer
TD-FOH	Transducer Failure
TDA-FOH	Transducer (Air Flow) Failure
TDP-FOH	Transducer (Pressure) Fails
TDT-FOH	Transducer (Temperature) Fails

Table C1.1-1. YMP PCSA Component Types (TYP) and Failure Modes (FM) (Continued)

TYP-FM	Component Name & Failure Mode
THR-BRK	Third Rail Breaks
TKF-FOH	Fuel Tank Fails
TL-FOH	Torque Limiter Failure
TRD-FOH	Tread (Site Transporter)
UDM-FOH	Damper (Backdraft) Failure
UPS-FOH	Uninterruptible Power Supply (UPS) Failure
WNE-BRK	Wire Rope Breaks
XMR-FOH	Transformer Failure
XV-FOD	Manual Valve Failure on Demand
ZS-FOD	Limit Switch Failure on Demand
ZS-FOH	Limit Switch Fails
ZS-SPO	Limit Switch Spurious Operation

NOTE: AC = alternating current; DC = direct current; CTT = cask transfer trailer; HEPA = high efficiency particulate air (filter); HVAC = heating, ventilation, and air conditioning; MCC = motor control center; TEV = transport and emplacement vehicle; UPS = uninterruptible power supply.

Source: Original

C1.2 INDUSTRY-WIDE RELIABILITY DATA

Industry-wide data sources are documents containing industrial or military experience on component performance. Usually they are previous safety/risk analyses and reliability studies performed nationally or internationally, but they can also be standards or published handbooks. For the YMP PCSA, an industry-wide database was constructed using a library of industry-wide data sources of reliability data from nuclear power plants, equipment used by the military, chemical processing plants, and other facilities. The sources used are listed in Table C1.2-1.

Table C1.2-1. Industry-wide Data Sources Used in YMP PCSA Active Component Reliability Database

Industry-wide Data Sources Used in YMP PCSA Active Component Reliability Database
Guidelines for Process Equipment Reliability Data with Data Tables. [CCPS] (Ref. C5.1)
Savannah River Site, Generic Data Base Development (U) [SRS Reactors] (Ref. C5.5)
The In-Plant Reliability Data Base for Nuclear Plant Components: Interim Report-The Valve Component. NUREG/CR-3154 (Ref. C5.6)
Waste Form Throughputs for Preclosure Safety Analysis.[BSC 2007](Ref. C5.7)
Probabilistic Risk Assessment (PRA) of Bolted Storage Casks, Updated Quantification and Analysis Report. [EPRI PRA] (Ref. C5.8)
Component Failure and Repair Data for Coal-Fired Power Units. EPRI AP-2071 [EPRI Pipe Failure Study] (Ref. C5.10)
Mechanical Reliability: Theory, Models and Applications. [AIAA] (Ref. C5.11)

Table C1.2-1. Industry-wide Data Sources Used in YMP PCSA Active Component Reliability Database (Continued)

Industry-wide Data Sources Used in YMP PCSA Active Component Reliability Database
Military Handbook, Reliability Prediction of Electronic Equipment. MIL-HDBK-217F [MIL-HDBK-217F] (Ref. C5.12)
The In-Plant Reliability Data Base for Nuclear Power Plant Components - Pump Component. NUREG/CR-2886. (Ref. C5.13)
Some Published and Estimated Failure Rates for Use in Fault Tree Analysis [DuPont] (Ref. C5.14)
Analysis of Station Blackout Risk. Volume 2 of Reevaluation of Station Blackout Risk at Nuclear Power Plants. NUREG/CR-6890 (Ref. C5.15)
Industry-Average Performance for Components and Initiating Events at U.S. Commercial Nuclear Power Plants. NUREG/CR-6928. (Ref. C5.16)
"Train Accidents by Cause from Form FRA F 6180.54." [Federal Railroad Administration] (Ref. C5.17)
Summary, Commercial Nuclear Fuel Assembly Damage/Misload Study – 1985-1999. [McKenna] (Ref. C5.20)
Ruggedized Card Reader/Ruggedized Keypad Card Reader. [HID] (Ref. C5.21)
IEEE Recommended Practice for the Design of Reliable Industrial and Commercial Power Systems. [IEEE-493] (Ref. C5.22)
IEEE Guide to the Collection and Presentation of Electrical, Electronic, Sensing Component, and Mechanical Equipment Reliability Data for Nuclear-Power Generating Stations. [IEEE-500] (Ref. C5.23)
The In-Plant Reliability Data Base for Nuclear Plant Components: Interim Report- Diesel Generators, Batteries, Chargers and Inverters. NUREG/CR-3831 (Ref. C5.24)
Instruments and Software Solutions (for Emergency Response and Health Physics [LAURUS] (Ref. C5.25)
A Survey of Crane Operating Experience at U.S. Nuclear Power Plants from 1968 through 2002. NUREG-1774. (Ref. C5.26)
Data Summaries of Licensee Event Reports of Valves at U.S. Commercial Nuclear Power Plants from January 1, 1976 to December 31, 1980. NUREG/CR-1363 (Ref. C5.28)
The Reliability Data Handbook. [Moss] (Ref. C5.32)
Control of Heavy Loads at Nuclear Power Plants. NUREG-0612. (Ref. C5.35)
Handbook of Reliability Prediction Procedures for Mechanical Equipment [NSWC-98-LE1] (Ref. C5.37)
"Using the EDA to Gain Insight into Failure Rates" [Rand] (Ref. C5.38)
Nuclear Computerized Library for Assessing Reactor Reliability (NUCLARR), Volume 5: Data Manual, Part 3: Hardware Component Failure Data. NUREG/CR-4639, (Ref. C5.39)
Nonelectronic Parts Reliability Data 1995. NPRD-95. [NPRD -95] (Ref. C5.40)
Umatilla Chemical Agent Disposal Facility Quantitative Risk Assessment. [SAIC Umatilla] (Ref. C5.41)

Table C1.2-1. Industry-wide Data Sources Used in YMP PCSA Active Component Reliability Database (Continued)

Industry-wide Data Sources Used in YMP PCSA Active Component Reliability Database
Offshore Reliability Data Handbook. 2nd Edition [OREDA-92] (Ref. C5.42)
Offshore Reliability Data Handbook. 4th Edition. [OREDA-2002] (Ref. C5.43)
Data Summaries of Licensee Event Reports of Pumps at U.S. Commercial Nuclear Power Plants: January 1, 1972-April 30, 1980. NUREG/CR-1205. (Ref. C5.45)
N-Reactor Level 1 Probabilistic Risk Assessment: Final Report. [N-Reactor] (Ref. C5.46)

NOTE: The code in brackets [XXXX] is used to aid the reader in identifying references in Table C4-1.

Source: Original

It was necessary to analyze the industry-wide data to compare the relevancy of the component data selected from the industry-wide data sources with the equipment in the YMP PCSA models.

The data source scope had to be sufficiently broad to cover a reasonable number of the equipment types modeled, yet with enough depth to ensure that the subject matter is appropriately addressed. For example, a separate source might have been used for electronics data versus mechanical data, so long as its use was justified by the detail and the applicability of the information provided. Lastly, the quality of the data source was considered to be a measure of the source's credibility. Higher quality data sources are based on equipment failures documented by a facility's maintenance records. Lower quality sources use either abbreviated accounts of the failure event and resulting repair activity, or do not allow the user to trace back to actual failure events. Every effort was made to use the highest quality data source available for each active component type and failure mode.

Data were selected from the industry-wide data sources using the following criteria:

- The component type (TYP) and failure mode (FM) identified in the data source had to match those in the basic events specified in the fault tree. For every component modeled, a comparison was made between the modeled component and the component found in the data source to ensure its suitability for the PCSA. Also, every attempt was made to match the failure modes. Often, the source described the failure mode as "all modes," whereas the fault tree required "fails to operate." In cases such as this, sources with more general failure modes were not used unless they were the only available sources.

- The data source had to be widely available, not proprietary. This ensured traceability and accessibility.

- Mid level or low level quality data sources were used only when high level sources were not available.

- The operating environment is an important factor in the selection of data sources. The environment of a component refers not only to its physical state, but also its operational state. The operating conditions of a component include the plant's maintenance policy and testing policy. If either of these states differed from the modeled facility's state, then the data were reconsidered and usually rejected (unless no alternative existed).

A potential disadvantage of using industry-wide data is that a source may provide failure rates that are not realistic because the source environment, either physical or operational, may not correlate to the facility modeled. Part of the PCSA active component reliability analysis effort, therefore, was to evaluate the similarity between the YMP operating environment and that represented in each generic data source to ensure data appropriateness.

An example of how data were retrieved from the various data sources is described in the following example for check valves. The failure modes modeled in the PCSA for the check valve are fails per hour (FOH), fails to check (FTX), leaks (LEK), and spurious operation (SPO).

Table C1.2-2 shows a comparison between the failure rates for the check valve and its failure modes from three different industry-wide data sources.

Table C1.2-2. Data Source Comparison for Check Valve

Data Source	Equipment Description	Failure Modes	Data Values Provided	Equipment Boundary Given?	Taxonomy Given?
(Ref. C5.1)	Valve-non-operated, Check	• Fails to Check • Significant Back Leakage	Lower, Mean, Upper	Yes	Yes
(Ref. C5.23)	Driven Equipment Valves, Check	"All Modes"	Low, Recommended, High	No	Yes
(Ref. C5.5)	Check	• Fails to Open • Fails to Close • Plugs • Internal Leakage • Internal Rupture • External Leakage • External Rupture	Mean	No	No

NOTE: AIChE = American Institute of Chemical Engineers; IEEE = Institute of Electrical and Electronics Engineers.

Source: Original

Table C1.2-3 shows actual numbers extracted from industry-wide data sources for five failure modes for check valves.

Table C1.2-3. Failure Rates Extracted from Various Data Sources for Check Valve

Failure Mode Description	Failure Mode Code	Data Source	Lower	Median	Upper	EF
Fails to Close (Hourly)	FOH	(Ref. C5.5)	$1.27 \times 10\text{-}7$	$7.74 \times 10\text{-}7$	$4.70 \times 10\text{-}6$	6.1
Leaks	LEK	(Ref. C5.5)	$6.98 \times 10\text{-}7$	$3.49 \times 10\text{-}6$	$1.75 \times 10\text{-}5$	5.0
Fails to Open (Hourly)	FOH	(Ref. C5.5)	$1.27 \times 10\text{-}7$	$7.74 \times 10\text{-}7$	$4.70 \times 10\text{-}6$	6.1
Transfers Closed	SPO	(Ref. C5.23)	$8.00 \times 10\text{-}8$	$7.81 \times 10\text{-}7$	$3.27 \times 10\text{-}4$	5.0
Transfers Open	SPO	(Ref. C5.23)	$8.00 \times 10\text{-}8$	$7.81 \times 10\text{-}7$	$3.27 \times 10\text{-}4$	5.0

NOTE: EF = error factor; FOH = fails per hour; LEK = leaks; SPO = spurious operation..

Source: Original

At this stage of the analysis, it remains to decide which data is appropriate to keep and include in the data pool and which are discarded. The criteria for this process are discussed below.

The guidelines shown in Table C1.2-4 are based on observations of the analysts of their preferences and rationales during the data selection process among the data available at the time.

Table C1.2-4. Guidelines for Industry-wide Data Selection

	Data Selection Guidelines
1.	Preference for greater than zero failures (but not always able to exclude on this basis)
2.	Population of at least 5
3.	Denominator greater than 1,000 hours or 100 demands
4.	If mean or median values, some expression of uncertainty surrounding these values (either upper or lower bounds or lognormal error factor)
5.	Data analyst's confidence in the applicability of the data to the YMP based on: • Component design • Driver/operator • Size • Component application • Active versus passive service • Materials/fluids moved (e.g., water versus caustic versus viscous) • Component boundary • What's included and excluded in component definition (e.g., motor, electrical connections) • Failure modes • Operating environment • Physical (e.g., heat, humidity, corrosive) • Functional (e.g., operation, maintenance, and testing frequency)

NOTE: YMP = Yucca Mountain Project.

Source: Original

Given the fact that the YMP will be a relatively unique facility (although portions will be similar to the spent fuel handling and aging areas of commercial nuclear plants), the data development perspective was to collect as much relevant industry-wide failure estimate information as possible to cover the spectrum of equipment operational experience. It is assumed that the YMP equipment would fall within this spectrum (Assumption 3.2.1). The scope of the sources selected for this data set was deliberately broad to increase the probability that YMP operational experience would fall within the bounds. A combined estimate that reflected the uncertainty ranges defined by the data source values was developed. This process is addressed further in the Bayesian estimation Section C2.

Every attempt was made to find more than one data source for each TYP-FM, although the unique nature of many equipment types made this difficult. Data was extracted from several sources in many cases, then combined using Bayesian estimation (as described further below), and compared by plotting the individual and combined distributions. However, the comparison process often resulted in one source being selected as most representative of the TYP-FM. Ultimately, 53% of the TYP-FMs were quantified with one data source, 8% with two data sources, 8% with three data sources, and 31% with four or more data sources.

C1.3 CRANE AND SPENT FUEL TRANSFER MACHINE DROP ESTIMATES

Industry-wide data was used to quantify the likelihood of experiencing a drop from the 200-ton crane while handling waste forms and their associated containers and for estimating drop probability for jib cranes and cranes used to maneuver waste packages. In addition, drop likelihoods for the spent fuel transfer machine (SFTM) were estimated using industry-wide data.

The rationale for using industry-wide data for these estimates was that a significant amount of crane experience exists within the commercial nuclear power industry and other applications and that this experience could be used to bound the anticipated crane performance at YMP. Further, the repository is expected to have training for crane operators and maintenance programs similar to those of nuclear power plants.

Handling incidents that resulted in a drop were included in the drop probability regardless of cause; they may have been caused by equipment failures (including failures in the yokes and grapples), human error, or some combination of the two.

The industry-wide data for cranes was taken from NUREG-0612 (Ref. C5.35), *A Survey of Crane Operating Experience at U.S. Nuclear Power Plants from 1968 through 2002.* NUREG-1774 (Ref. C5.26), and the *Probabilistic Risk Assessment (PRA) of Bolted Storage Casks, Updated Quantification and Analysis Report* (Ref. C5.8). NUREG-0612 (Ref. C5.35) has several appendices that contain crane data from the Occupational Safety and Health Act Administration, the U.S. Navy, Waste Isolation Pilot Plant, Licensee Event Reports, and from the results of a fault tree analysis. The *Probabilistic Risk Assessment (PRA) of Bolted Storage Casks, Updated Quantification and Analysis Report* (Ref. C5.8) provides estimates from Savannah River Site crane experience in addition to fault tree analysis. Crane failure information was also obtained from quantitative risk study performed for the U.S. Army chemical weapons destruction program (Ref. C5.41).

The information from each of these sources was evaluated in terms of quality, applicability to YMP, and to ensure that the events cited included both equipment failures and human failures. For the industry-wide data provided in terms of the number of events, another major factor was the ability to reasonably and justifiably estimate a meaningful denominator of number of lifts (demands) conducted by the crane population considered in the data source. If this could not be done, the source information could not be used.

A key consideration in evaluating the industry-wide crane data for the 200-ton cranes was the NOG-1 (Ref. C5.3) design requirements that will be placed upon the YMP cranes versus the crane design features reflected in the input data sources. NUREG-1774 (Ref. C5.26, Table 12, pp. 61 – 63) provides a list of the nuclear power plants that had upgraded their cranes to single-failure-proof status consistent with licensee response to U.S. Nuclear Regulatory Commission (NRC) *NRC Bulletin 96-02* (Ref. C5.9) which requested specific information relating to their heavy loads programs and plans consistent with the recommendations of NUREG-0554 (Ref. C5.34). This information was used to constrain the denominator of the number of very heavy load lifts from NUREG-1774 (54,000) by using a percentage of percent of nuclear power plants reporting single failure proof cranes out of total plants (42/110).

Conversely, a separate category of non-single-failure-proof cranes for the waste package manipulating cranes was developed using the remaining percentage (68/110) to adjust the number of lifts. The jib crane lifts were estimated using the NUREG-1774 (Ref. C5.26, Appendix D) table of the types of cranes involved in accidents; mobile and tower cranes using jibs are cited as being involved in ~76% of accidents while bridge and gantry (used for very heavy loads) are ~19%. The percentage of accidents that did not involve jib cranes was therefore believed to reside somewhere between 19% and 24% (100% – 76%). So, the 20,620 lifts estimated for very heavy loads by single failure proof cranes was divided by 21.2% to yield a round number estimate of 97,250 jib crane lifts.

The number of crane drop incidents used as the numerator of the 200-ton crane drop estimate from NUREG-1774 (Ref. C5.26) was also restricted to those involving very heavy loads (defined in NUREG-1774 as >30 tons) of single-failure-proof cranes. Drops occurring during sling lifts were parsed into a separate category and used to estimate the sling lift-related drop likelihood.

Load drop likelihood due to two-blocking was also estimated using industry-wide data. NUREG-0612 (Ref. C5.35) describes a two-blocking event as: "The act of continued hoisting to the extent that the upper head block and the load block are brought into contact, and unless additional measures are taken to prevent further movement of the load block, excessive loads will be created in the rope reeving system, with the potential for rope failure and dropping of the load." Two-blocking events in the various data sources were evaluated based upon the type of crane involved, as was done for the drop likelihood estimates.

As a result, several categories of crane drop estimates were developed, were coded with TYP-FM designators, and were included in the template database for input to SAPHIRE:

CRN-DRP	200-ton Crane Load Drop	3.2E-05/demand
CRN-TBK	200-ton Crane Two Block Causing Load Drop	4.4E-07/demand
CRS-DRP	200-ton Crane using Slings Load Drop	1.2E-04/demand
CRJ-DRP	Jib Crane Load Drop	2.6E-05/demand
CRW-DRP	Waste Package Crane (Not Single Failure Proof) Load Drop	1.1E-04/demand
CRW-TBK	Waste Package Crane (Not Single Failure Proof) Two-Block Causing Load Drop	4.5E-05/demand

In each of these cases, as with the other active component reliability estimates, an effort was made to include a variety of operating experience and combine it together using a parametric empirical Bayes approach. However, for the CRS, CRJ and CRW estimates, since only NUREG-1774 (Ref. C5.26), data was considered to be applicable, a Jeffrey's non-informative prior approach for the Beta distribution was used, since the estimates were per lift (demand).

These crane incident estimates were combined in the SAPHIRE models with the number of estimated YMP crane lifts.

One potential issue regarding the applicability of the industry-wide crane data was the inclusion of hard-wired interlock features on the YMP cranes that might not exist at the nuclear power plants or naval installations from which the industry-wide experience resulted. In other instances, there was concern that interlocks included in the design for use in normal operations, on grapples to verify installation or engagement, could be defeated during maintenance actions where bypasses are permitted to move tools or pallets, since a particular grapple interlock is not standard in industry but is unique to YMP. Further, PCSA is not crediting the grapple interlock function and it was considered that having such interlocks in place would not make the estimated failure probability worse. Therefore the estimates from industry-wide data were considered to be reasonable in that they provided experience-based, and perhaps somewhat pessimistic measures of anticipated crane performance.

Estimates were also developed from industry-wide data source information for the likelihood of SFTM drop, collision, and raising the fuel too high but not dropped (for potential personnel exposure considerations). The primary source for this information was NUREG-1774 (Ref. C5.26, Table 4), which provides brief descriptions of SFTM incidents at U.S. nuclear power plants from 1968 through 2002. A separate study (McKenna/Framatome) (Ref. C5.20) was reviewed, which also included SFTM incidents at U.S. nuclear power plants categorized in terms of Human Error, Equipment Failure, or Misload. Some of these were the same incidents included in NUREG-1774 (Ref. C5.26) so care was taken not to double-count any events. Each of the incidents described was reviewed in detail to evaluate their relevance to the failure modes of interest to the study and their applicability to spent fuel transfers. Incidents related to all types of fuel transfers, such as refueling or new fuel receipt, were used to estimate upper bounds (95th percentiles of a lognormal distribution) and to develop the error factor uncertainty information input to SAPHIRE along with the mean value.

It should be noted that events prior to 1985 were removed from consideration since the number of plants in operation (and therefore the number of lifts per year) would significantly differ from that cited in McKenna/Framatome (Ref. C5.20). Also, McKenna/Framatome stated that reporting practices were inconsistent prior to 1985.

The number of fuel movements used as the denominator of the SFTM estimates was based upon information from McKenna/Framatome (Ref. C5.20), which gave 1,198,723 fuel movements for the 15 year study data window, from 1985 through 1999, or a rough estimate of 79,914.87 per year. Since the numerator information from NUREG-1774 (Ref. C5.26) was based upon 17 years of data, from 1985 through 2002, the estimated denominator was calculated for consistency as 79,914.87 × 17 or 1,358,553 SFTM lifts.

As a result, several categories of SFTM event estimates were developed, were coded with TYP-FM designators, and were included in the template database for input to SAPHIRE:

SFT-COL	SFTM Collision/Impact	2.9E-06/demand
SFT-DRP	SFTM Load Drop	5.2E-06/demand
SFT-RTH	SFTM Fuel Raised Too High (but not dropped)	7.4E-07/demand

These SFTM incident estimates were combined in the SAPHIRE models with the number of estimated YMP fuel assembly transfers, specifically: 66,188 based on two transfers each of 33,094 assemblies (Ref. C5.7, Table 4, pg. 27).

The results of the industry-wide data search are documented, organized by component type and failure mode, and can be found in the Excel spreadsheet file “YMP Active Comp Database.xls”, located on the CD in Attachment H.

C2 BAYESIAN DATA COMBINATION

The application of industry-wide data sources or expert elicitation introduces uncertainty in the input parameters used in basic events and, ultimately, the quantification of probabilities of event sequences. Uncertainty is a probabilistic concept that is inversely proportional to the amount of knowledge, with less knowledge implying more uncertainty. Bayes’ theorem is a common method of mathematically expressing a decrease in uncertainty gained by an increase in knowledge (for example, knowledge about failure frequency gained by in-field experience).

A typical application of Bayes’ theorem is illustrated as follows: a failure rate for a given component is needed for fault tree (e.g., a fan motor in the heating, ventilation, and air conditioning (HVAC) system). There is no absolute value but there are several data sources for the same kind of fan and/or similar fans that may exhibit considerable variability for many reasons. Applying any or all of the available data introduces uncertainty in the analysis of the reliability of the HVAC system. Bayes’ theorem provides a mechanism for systematically treating the uncertainty and applying λj data sources using the following steps:

1. Initially, estimate the failure rate to be within some range with a probability distribution. This is termed the “prior” probability of having a certain value of the failure rate that expresses the state of knowledge before any new information is applied.

2. Characterize the test information, or evidence, in the form of a likelihood function that expresses the probability of observing the number of failures in the given number of trial if the failure rate is a certain value. The evidence comprises observations or test results on the number of failure events that occur in over a certain exposure, operational, or test duration.

3. Update the probability distribution for the failure rate based on the new body of evidence using the mathematical expression of Bayes’ theorem.

The mathematical expression for applying Bayes' theorem to data analysis is briefly described here. Let λ_j be one failure rate of a set of possible failure rates of the fan motor (component j). Initially, the state of knowledge of the "true value" of λ_j is expressed by the probability distribution $P(\lambda)$, the "prior." The choice of the analytic or discrete form of the prior distribution is made by the data analyst. Let E be a new body of evidence, e.g., a new set of test data or field observations. The new evidence improves the data analyst's state of knowledge. The revised, or "updated," probability distribution for the "true value" of λ_j is represented as $P(\lambda_j|E)$. Bayes' theorem gives:

$$P(\lambda_j \mid E) = \frac{P(\lambda_j)L(E \mid \lambda_j)}{\sum_j P(\lambda_j)P(E \mid \lambda_j)} \quad \text{(Eq. C-1)}$$

In summary, Equation C-1 states that the knowledge of the "updated" probability of λ_j, given the new information E, equals the "prior" probability of λ_j before any new information times the likelihood function, $L(E|\lambda_j)$. The likelihood function expresses the probability of observing the number of failures in the evidence if the failure rate λ_j has a certain value. The likelihood function is defined by the analyst in accordance with the kind of evidence. For time-based failure data, a Poisson model is used for the likelihood function. For demand-based failure data, a binomial model is used. The numerator in Equation C-1 is divided by a normalization factor, which must be such that the sum of the probabilities over the entire set of λ_j equals unity.

There are several approaches for applying Bayes' theorem to data management and combining data sources, as described in NUREG/CR-6823 (Ref. C5.4). For the YMP PCSA, the method known as "parametric empirical Bayes" was used. This permitted a variety of different sources to be statistically combined and compared, whether the inputs were expressed as the number of failures and exposure time or demands, or as a mean and error factor. Examples of the methods used for several combinatorial cases are provided below.

C2.1 PARAMETER ESTIMATION USING DATA FROM DIFFERENT SOURCES

Using multiple reliability databases will typically cause a given active component to have various reliability estimates, each one from a different source. These various estimates can be viewed as independent samples from the same distribution, *g*, representing the source-to-source variability, also called population variability, of the component reliability (Ref. C5.4, Section 8.1). The objective of this section is to outline the methodology for developing the population-variability distribution of active components in the preclosure safety analysis. In a Bayesian approach to reliability estimation, the population-variability distribution of a component constitutes an informative prior distribution for its reliability. This distribution is to be updated, as operating experience becomes available, to produce a reliability distribution specific to the component operated under geologic repository operations area (GROA) conditions. For the time being however, the components anticipated for use at the GROA are yet to be procured and operated. As a consequence, the population-variability distributions developed in this section both aim at and are limited to encompassing the actual component reliability distributions that will be observed at the GROA when operating experience becomes available.

A parametric empirical Bayes method is used to develop the population-variability distributions of active components considered in the preclosure safety analysis. As indicated in "Bayesian Parameter Estimation in Probabilistic Risk Assessment." (Ref. C5.44, Section 5.1.2), this method is a pragmatic approach that has been used in PRA-related applications; it involves specifying the functional form of the prior population-variability distribution, and fitting the prior to available data, using classical techniques, for example, the maximum likelihood method. A discussion of the adequacy of the parametric empirical Bayes method for determining the population-variability distribution is given at the end of this section.

Applying the parametric empirical Bayes method requires first to categorize the reliability data sources into two types: those that provide information on exposure data (i.e., the number of failures that were recorded over an exposure time (in case of a failure rate) or over a number of demands (in case of a failure probability), and those that do not provide such information). In the latter case, reliability estimates for a failure rate or failure probability are provided in the form of a mean or a median value, along with an uncertainty estimate, typically an error factor.

For each data source, the reliability information about a component's failure rate of failure probability is mathematically represented by its likelihood function. If exposure data are provided, the likelihood function takes the form of a Poisson distribution (for failure rates), or a binomial distribution (for failure probabilities) (Ref. C5.44, Section 4.2). When no exposure data are available, the reliability estimates for failure rates or failure probabilities are interpreted as expert opinion, for which an adequate representation of the likelihood function is a lognormal distribution ((Ref. C5.44, Section 4.4) and (Ref. C5.27, pp. 312, 314, and 315)).

The next step is to specify the form of the population-variability distribution. In its simplest form, the parametric empirical Bayes method only considers exposure data and employs distributions that are conjugate to the likelihood function (i.e., a gamma distribution if the likelihood is a Poisson distribution, and a beta distribution if the likelihood is binomial) (Ref. C5.4, Section 8.2.1), which have the advantage of resulting in relatively simpler calculations. This technique however is not applicable when both exposure data and expert opinion are to be taken into consideration, because no conjugate distribution exists in this situation. Following the approach of "The Combined Use of Data and Expert Estimates in Population Variability Analysis,"(Ref. C5.27, Section 3.1), the population-variability distribution in this case is chosen to be lognormal. More generally, for consistency, the parametric empirical Bayes method is applied using the lognormal functional form for the population-variability distributions regardless of the type of reliability data available for the component considered (exposure data, expert opinion, or a combination of the two). In the rest of this section, the population-variability distribution in its lognormal form is noted $g(x,\nu,\tau)$, where x is the reliability parameter for the component (failure rate or failure probability), and ν and τ, the two unknowns to be determined, are respectively the mean and standard deviation of the normal distribution associated with the lognormal. The use of a lognormal distribution is appropriate for modeling the population-variability of failure rates and failure probabilities, provided in the latter case that any tail truncation above $x = 1$ has a negligible effect (Ref. C5.44, p. 99). The validity of this can by confirmed by selecting the failure probability with the highest mean and the most skewed lognormal distribution and calculating what the probability is of exceeding 1. In Table C4-1, PRV-FOD fits this profile, with a mean failure probability of 6.54E-03 and an error factor of 27.2. The probability that the distribution exceeds 1 is 2E-04. Stated equivalently, 99.98 percent of the values taken by the distribution are less than 1. This confirms that the use of a truncated lognormal distribution to represent the probability distribution is appropriate.

To determine ν and τ, it is first necessary to express the likelihood for each data source as a function of ν and τ only (i.e., unconditionally on x). This is done by integrating, over all possible values of x, the likelihood function evaluated at x, weighted by the probability of observing x, given ν and τ. For example, if the data source i indicates that r failures of a component occurred out of n demands, the associated likelihood function $L_i(\nu,\tau)$, unconditional on the failure probability x, is as follows:

$$L_i(\nu,\tau) = \int_0^1 Binom(x,r,n) \times g(x,\nu,\tau)dx \qquad \text{(Eq. C-2)}$$

where $Binom(x,r,n)$ represents the binomial distribution evaluated for r failures out of n demands, given a failure probability equal to x, and $g(x,\nu,\tau)$ is defined as previously indicated. This equation is similar to that shown in "Bayesian Parameter Estimation in Probabilistic Risk Assessment." (Ref. C5.44, Equation 37). If the component reliability was expressed in terms of a failure rate and the data source provided exposure data, the binomial distribution in Equation C-2 would be replaced by a Poisson distribution. If the data source provided expert opinion only (no exposure data), the binomial distribution in Equation C-2 would be replaced by a lognormal distribution.

The maximum likelihood method is an acceptable method to determine ν and τ (Ref. C5.44, p. 101). The maximum likelihood estimators for ν and τ are obtained by maximizing the likelihood function for the entire set of data sources. Given the fact that the data sources are independent, the likelihood function is the product of the individual likelihood functions for each data source (Ref. C5.27, Equation 4). To find the maximum likelihood estimators for ν and τ, it is equivalent and computationally convenient to maximize the log-likelihood function, which is the sum of the logarithms of the likelihood function for each data source.

The calculation of ν and τ completely determines the population-variability distribution *g* for the reliability of a given active component. The associated parameters to be plugged into SAPHIRE are the mean and the error factor of the lognormal distribution *g*, which are calculated using the formulas given in NUREG/CR-6823 (Ref. C5.4, Section A.7.3). Specifically, the mean of the lognormal distribution is equal to $\exp(\nu + \tau^2/2)$ and the error factor is equal to $\exp(1.645 \times \tau)$.

The selection of the parametric empirical Bayes method to determine the population-variability distribution is now discussed. This method provides a single "best" solution, while other techniques, such as the hierarchical Bayes method (Ref. C5.4, Section 8.3) differ by using a weighted mix of distributions of the chosen model, which incorporate epistemic (state of knowledge) uncertainty about the model. The parametric empirical Bayes method does not embed epistemic uncertainty but was nevertheless employed because of its satisfactory results for the majority of active components modeled in the preclosure safety analysis. The general adequacy of the method was confirmed by comparing its results to those obtained based on an example using a state-of-knowledge-informed approach (Ref. C5.27). The example involves twelve hypothetical data sources, each documenting the failure rate of motor-driven pumps either in terms of expert judgment or exposure data (Ref. C5.27, Table 1). Table C2.1-1 compares the percentiles predicted by the parametric empirical Bayes method and those found in "The Combined Use of Data and Expert Estimates in Population Variability Analysis." (Ref. C5.27, Table 4). Overall, the percentiles appear to be similar, with a key metric of the distributions, their mean, being nearly identical, and the medians being comparable. Percentiles at the tails of the distributions show more differences, the parametric empirical Bayes method yielding a population-variability distribution more spread out overall than the state-of-knowledge-informed distribution (Ref. C5.27).

Table C2.1-1. Comparison of Results of Parametric Empirical Bayes and Results Reported by Lopez Droguett et al.

Population-Variability Value	Parametric Empirical Bayes Method[a]	Lopez Droguett Results[b]
Mean	6.00×10^{-5}	6.05×10^{-5}
1st percentile	1.32×10^{-7}	3.16×10^{-7}
5th percentile	4.75×10^{-7}	1.38×10^{-6}
10th percentile	9.38×10^{-7}	2.67×10^{-6}
50th percentile (median)	1.04×10^{-5}	1.61×10^{-5}
90th percentile	1.14×10^{-4}	7.79×10^{-5}
95th percentile	2.26×10^{-4}	1.36×10^{-4}
99th percentile	8.10×10^{-4}	4.85×10^{-4}

NOTE: [a] Derivation of the results is given in the following section, Example of Development of Population-Variability Distribution.

[b] ("The Combined Use of Data and Expert Estimates in Population Variability Analysis." *Reliability Engineering and System Safety, 83* (Ref. C5.27, Table 1)

Source: (Ref. C5.27, Table 1).

An adjustment to the parametric empirical Bayes method was done in a few instances where the error factor of the calculated lognormal distribution was found to be excessive. In a synthetic examination of the failure rates of various components, "External Maintenance Rate Prediction and Design Concepts for High Reliability and Availability on Space Station Freedom," *Reliability Engineering and System Safety, 47* (Ref. C5.19, Figure 3) finds that electromechanical and mechanical components have, overall, a range of variation approximately between 2×10^{-8}/hr (5th percentile) and 6×10^{-5}/hr (95th percentile). Using the definition of the error factor given in NUREG/CR-6823, (Ref. C5.4, Section A.7.3), this corresponds to an error factor of $\sqrt{6 \cdot 10^{-5} / 2 \cdot 10^{-8}} = 55$. Therefore, in the preclosure safety analysis, it is considered that lognormal distributions resulting from the empirical Bayes method that yield error factors with a value greater than 55 are too diffuse to adequately represent the population-variability distribution of a component. In such instances (two such cases in the entire PCSA database, when the error factors from the Bayesian estimation were greater than 200), the lognormal distribution used to represent the population-variability is modified as follows. It has the same median as that predicted by the parametric empirical Bayes method, and its error factor is assigned a value of 55. The median is selected as the unvarying parameter because, contrary to the mean, it is not sensitive to the behavior of the tails of the distribution and therefore is unaffected by the value taken by the error factor. Based on NUREG/CR-6823, (Ref. C5.4, Section A.7.3), the median is calculated as exp(ν), where ν is obtained by the maximum likelihood estimation.

A limitation of the parametric empirical Bayes method that prevented its use for all active components of the preclosure safety analysis is that the calculated lognormal distribution can sometimes have a very small error factor (with a value around 1), corresponding to a distribution overly narrow to represent a population-variability distribution. As indicated in NUREG/CR-6823, (Ref. C5.4, p. 8-4), this situation can arise when the reliability data sources provide similar estimates for a component reliability. The inadequacy of the parametric empirical Bayes method in such situations is made apparent by plotting the probability density function of the lognormal distribution and comparing it with the likelihood functions associated with the reliability estimates of each data source. In the cases where the lognormal distribution does not approximately encompass the likelihood functions yielded by the data sources, it is not used to model the population-variability distribution. Instead, this distribution is modeled using a data source that yields a more diffuse likelihood. In the other cases, the lognormal distribution approximately encompasses the likelihood functions yielded by the data sources, showing that the parametric empirical Bayes method is adequate. An illustration of a graph plotting the population-variability distribution along with the likelihood functions from data, based on the example of the Lopez Droguett et al. paper (Ref. C5.27) is provided below.

Example of Development of Population-Variability Distribution

Mathcad is used to calculate the population-variability distribution of active components. An illustration of such a calculation is given using the example in "The Combined Use of Data and Expert Estimates in Population Variability Analysis." (Ref. C5.27, Table 1). In this example, several data sources supply information about the reliability of motor-driven pumps, as follows:

Four data sources supply point estimates of the failure rates, along with a range (error) factor. This information is given in the following matrix, where the first column contains the estimated hourly failure rate (considered to be a median value) and the second column the associated error factor:

$$A := \begin{pmatrix} 3.0\cdot10^{-5} & 5 \\ 2.1\cdot10^{-5} & 3 \\ 2.0\cdot10^{-5} & 10 \\ 2.53\cdot10^{-5} & 10 \end{pmatrix}$$

In addition, eight data sources supply exposure data, which are given in the following matrix, where a recorded number of failures is shown in the first column, and the associated operating time (in hours) is shown in the second.

$$B := \begin{pmatrix} 0 & 76000 \\ 0 & 152000 \\ 0 & 74000 \\ 2 & 74000 \\ 0 & 48000 \\ 3 & 76000 \\ 9 & 10200 \\ 2 & 48000 \end{pmatrix}$$

The population-variability distribution g of the failure rate x is approximated by a lognormal distribution whose unknown parameters, ν and τ, respectively the mean and standard deviation of the associated normal distribution, are to be determined. Calculating ν and τ involves calculating the likelihood function associated with the reliability information in each data source. This is done as follows:

For a data source providing a failure rate point estimate, the likelihood function is a lognormal distribution, function of the failure rate x, and characterized by its median value and associated error factor shown in the matrix A. In Mathcad, the parameters required for defining a lognormal distribution are the mean and standard deviation of the associated normal distribution. Based on the formulas given in NUREG/CR-6823 (Ref. C5.4, Section A.7.3), the mean of the associated normal distribution is the natural logarithm of the median failure rate, and the standard deviation of the associated normal distribution is $\ln(EF)/1.645$, where EF is the error factor.

Because the unknowns to be determined are ν and τ, the likelihood function is expressed as a function unconditional on the value of x. This is done by integrating the likelihood function over all possible values of x (i.e., theoretically, from 0 to infinity) and weighting by the probability of having a value of x, conditional on observing ν and τ. In practice, to facilitate the numerical integration on Mathcad, the integration is performed on a range that encompasses credible values for x. In this example, the failure rate range considered varies from 10^{-8}/hr to 10^{-2}/hr. Thus, the likelihood functions, unconditional on x, for each of the data source in the matrix A, are calculated as follows:

$a := 1..4$

$$fe(a,x) := dlnorm\left(x, ln\left(A_{a,1}\right), \frac{ln\left(A_{a,2}\right)}{1.645}\right) \quad \text{(Eq. C-3)}$$

$$LA(a,\nu,\tau) := \int_{10^{-8}}^{10^{-2}} fe(a,x)\cdot dlnorm(x,\nu,\tau)\, dx \quad \text{(Eq. C-4)}$$

(In the above formulas, a is an index used to particularize a likelihood function to a data source in the matrix A.)

For a data source providing exposure data (given in the form of a number n of recorded failures over an exposure time t), the likelihood function is a Poisson distribution, expressing the probability that n failures are observed when the expected number of failures is x times t. Here also, the likelihood needs to be expressed as a function unconditional on the failure rate x, which is done by integrating x out, in a similar manner as above:

$b := 1..8$

$$fd(b,x) := dpois\left(B_{b,1}, B_{b,2}\cdot x\right) \quad \text{(Eq. C-5)}$$

$$LB(b,\nu,\tau) := \int_{10^{-8}}^{10^{-2}} fd(b,x)\cdot dlnorm(x,\nu,\tau)\, dx \quad \text{(Eq. C-6)}$$

(In the above formulas, b is an index used to particularize a likelihood function to a data source in the matrix B.)

The maximum likelihood method is used to calculate ν and τ. This involves maximizing the likelihood function for the entire set of data sources. This likelihood function is the product of the individual likelihood function for each data source (this is because the data sources are independent from each other). It is equivalent and computationally convenient to find the maximum likelihood estimators for ν and τ by using the sum of the log-likelihood (logarithm of the likelihood) of each data source.

Therefore, the log-likelihood function to be maximized is:

$$L(\nu,\tau) := \sum_{a=1}^{4} ln\left(LA(a,\nu,\tau)\right) + \sum_{b=1}^{8} ln\left(LB(b,\nu,\tau)\right) \quad \text{(Eq. C-7)}$$

To maximize a function, Mathcad requires guess values and a range over which to search for maxima. The quantity ν represents the logarithm of a failure rate, which is expected to be in the 10^{-6}/hr range. Therefore, a guess value for ν is:

$$\nu := ln(10^{-6}) \qquad \nu = -13.8$$

Based on a typical error factor value of 10, a guess value for τ is:

$$\tau := \frac{ln(10)}{1.645} \qquad \tau = 1.4$$

A reasonable range over which to perform the likelihood maximization is as follows:

$$Given \qquad \nu > -20 \qquad \nu < -1$$

$$\tau > 0.01 \qquad \tau < 5$$

The maximum likelihood estimators for ν and τ are:

$$L := Maximize(L, \nu, \tau) \qquad \nu := L_1 \qquad \nu = -11.478$$

$$\tau := L_2 \qquad \tau = 1.874$$

Therefore, the mean and error factors of the population-variability distribution for the failure rate are (based on the formula in NUREG/CR-6823 (Ref. C5.4, Section A.7.3)):

$$m := exp\left(\nu + \frac{\tau^2}{2}\right) \qquad m = 6.00 \times 10^{-5} \quad \text{per hour}$$

$$EF := exp(1.645 \cdot \tau) \qquad EF = 21.8$$

Notable percentiles of the population-variability distribution are as follows (expressed as hourly failure rates) and shown in Figure C2.1-1:

1st percentile: $qlnorm(0.01, \nu, \tau) = 1.32 \times 10^{-7}$

5th percentile: $qlnorm(0.05, \nu, \tau) = 4.75 \times 10^{-7}$

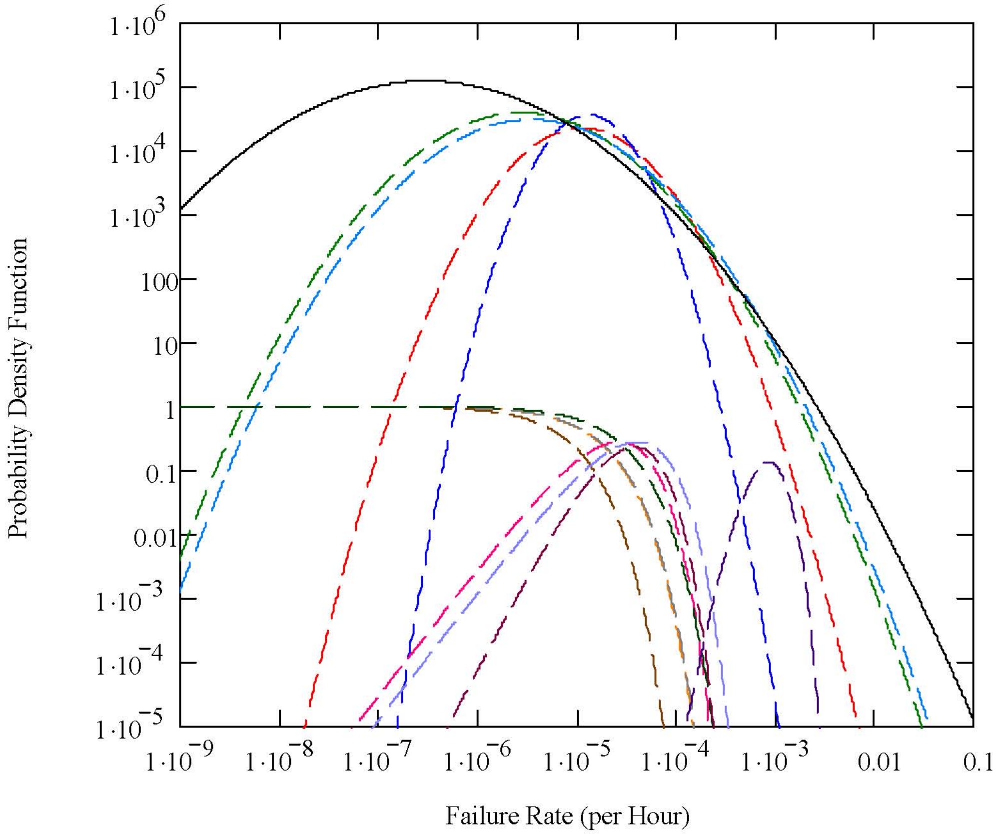
10th percentile: $qlnorm(0.10, \nu, \tau) = 9.38 \times 10^{-7}$

50th percentile: $qlnorm(0.50, \nu, \tau) = 1.04 \times 10^{-5}$

90th percentile: $qlnorm(0.90, \nu, \tau) = 1.14 \times 10^{-4}$

95th percentile: $qlnorm(0.95, \nu, \tau) = 2.26 \times 10^{-4}$

99th percentile: $qlnorm(0.99, \nu, \tau) = 8.10 \times 10^{-4}$



Source: Original

Figure C2.1-1. Likelihood Functions from Data Sources (Dashed Lines) and Population-Variability Probability Density Function (Solid Line)

C2.2 PARAMETER ESTIMATION IN CASE ONLY ONE DATA SOURCE IS AVAILABLE

To be developed, a population-variability distribution requires at least two data sources, and therefore the previous method is not applicable when only one data source is available. In this case, the probability distribution for the reliability parameter of an active component is that yielded by the data source. For example, if the data source provides a mean and an error factor for the component reliability parameter, the probability distribution is modeled in SAPHIRE as a lognormal distribution with that mean and that error factor. If the data source does not readily provide a probability distribution, but instead exposure data (i.e., a number of recorded failures over an exposure time for failure rates, or over a number of demands for failure probabilities) the probability distribution for the reliability parameter is developed through a Bayesian update using Jeffrey's noninformative prior distribution. As indicated in NUREG/CR-6823 (Ref. C5.4, Section 6.2.2.5.2), this noninformative prior conveys little prior belief or information, thus allowing the data to speak for themselves.

As mentioned in "Bayesian Parameter Estimation in Probabilistic Risk Assessment," (Ref. C5.44, Section 4.2), the likelihood function associated with exposure data is either a Poisson distribution (in the case of failure rates), or a binomial distribution (in the case of failure probabilities).

Applying Bayes' theorem with Jeffrey's noninformative prior in conjunction with a Poisson likelihood function characterized by r recorded failures over an exposure time t results in a closed-form posterior distribution, namely a gamma distribution, characterized by a shape parameter equal to $0.5 + r$, and a scale parameter equal to t; the mean of this distribution is $(0.5 + r)/t$ (Ref. C5.4, Sections 6.2.2.5.2 and A7.6). In SAPHIRE, this distribution is characterized by its mean and by its shape parameter (i.e., $0.5 + r$).

Applying Bayes' theorem with Jeffrey's noninformative prior in conjunction with a binomial likelihood function characterized by r recorded failures out of n demands results in a closed-form posterior distribution, namely a beta distribution, characterized by a parameter "a" equal to $0.5 + r$, and a parameter "b" equal to $n - r + 0.5$; the mean of this distribution is $(0.5 + r)/(n + 1)$ (Ref. C5.4, Sections 6.3.2.3.2 and A7.8). In SAPHIRE, this distribution is characterized by its mean and by the parameter "b" (i.e., $n - r + 0.5$).

C3 COMMON CAUSE FAILURE DATA

Dependent failures are modeled in event tree and fault tree logic models, with potential dependent failures modeled explicitly via the logic models, whenever possible. For example, failure of the HVAC system is explicitly dependent upon failures in the electrical supply systems that are modeled in the fault trees. Similarly, the effects of erroneous calibration or other human failure events can be explicitly included in the system fault tree models and the basic event probabilities considered during the human reliability analysis. Otherwise, potential dependencies known as common-cause failures are included in fault tree logic, but their probabilities are quantified by an implicit, parametric method. Therefore, another subtask of the active component reliability data analysis is to estimate common cause failure probabilities.

Surveys of failure events in the nuclear industry have led to several parameter models. Of these, three are most commonly used: the Beta Factor method (Ref. C5.18), the Multiple Greek Letter method (Ref. C5.29) and (Ref. C5.30), and the Alpha Factor method (Ref. C5.31). These methods do not require an explicit knowledge of the dependence failure mode. For the YMP PCSA, common-cause failure rates or probabilities were estimated using the alpha factor method described in NUREG/CR-5485 (Ref. C5.31).

The vast majority of the equipment types for which common cause failure basic events were modeled in the YMP PCSA are not covered by the detailed component-specific alpha factor sources based on commercial nuclear plant equipment. Therefore, it was necessary to use alpha factors to address the common cause failure estimates for crane hoist wire ropes, gear boxes, over-torque sensors and the like.

The alpha factor method provides a model to treat common cause failure (CCF) probabilities of k-of-m components. In addition, industry-wide alpha factors have been developed for the U.S. Nuclear Regulatory Commission from experience data collected at nuclear power plants. The data analysis reported in NUREG/CR-5485 (Ref. C5.31) consisted of:

1. Identifying the number of redundant components in each subsystem being reported (e.g., two, three, or four (this is termed the CCF group size, CCCG of size m)).

2. Partitioning the total number of reported failure events for a given component into the number of components that failed together, i.e., $k = 1$ for one component at a time, $k = 2$ for two components at a time, $k = 3$ for three components at a time, up to m for failure of all components in a given CCF group.

3. Estimating the alpha factor for a given component type based on its definition as the fraction of total failure events that involve k component failures due to common cause, for a system of m redundant components, using the alpha factor equation from NUREG/CR-5485 (Ref, C5.31, Table 5-10), as shown in Figure C3-1.

$$\alpha_k^m = \frac{n_k}{\sum_{j=1}^{m} n_j} \qquad k = 1, \ldots, m$$

Source: NUREG/CR-5485, p. 70 (Ref. C5.31)

Figure C3-1. Alpha Factor

4. Performing statistical analysis and curve fitting to define the mean and uncertainty range for alpha factors for various CCF group sizes up to eight.

The data analysis also produced industry-wide prior distributions for the alpha factors for each CCCG size, based on all CCF events in their database. Events were mapped to a given CCCG size, the maximum likelihood estimator obtained and fit to a constrained noninformative prior distribution. The parameter A_T of a Dirichlet distribution was then calculated for each alpha and the results combined using the geometric mean. The results are the industry-wide mean alpha factors and uncertainty bounds reported in of NUREG/CR-5485 (Ref. C5.31, Table 5-11) shown in Table C3-1:

Table C3-1. Alpha Factor Table

Table 5-11. Generic prior distributions for various system sizes.

CCCG Size m	α-Factor	Distributions Parameters		Percentiles			Mean
		a	b	P_{05}	P_{50}	P_{95}	
2	α_1	9.5300	0.470	8.20E-01	9.78E-01	1.00E-00	0.95300
	α_2	0.4700	9.530	1.42E-04	2.16E-02	1.81E-01	0.04700
3	α_1	15.2000	0.800	8.42E-01	9.67E-01	9.99E-01	0.95000
	α_2	0.3872	15.613	2.10E-05	8.79E-03	1.01E-01	0.02420
	α_3	0.4128	15.587	3.45E-05	1.01E-02	1.05E-01	0.02580
4	α_1	24.7000	1.300	8.67E-01	9.61E-01	9.95E-01	0.95000
	α_2	0.5538	25.446	1.44E-04	1.08E-02	7.81E-02	0.02130
	α_3	0.2626	25.737	2.98E-07	1.99E-03	4.82E-02	0.01010
	α_4	0.4836	25.516	6.29E-05	8.42E-03	7.17E-02	0.01860
5	α_1	38.042	1.958	8.86E-01	9.58E-01	9.91E-01	0.95106
	α_2	0.7280	39.272	3.72E-04	1.10E-02	6.05E-02	0.01820
	α_3	0.4120	39.588	1.32E-05	3.93E-03	4.22E-02	0.01030
	α_4	0.2336	39.766	4.57E-08	8.97E-04	2.89E-02	0.00584
	α_5	0.5840	39.416	1.24E-04	7.66E-03	5.27E-02	0.01460
6	α_1	50.4724	2.528	8.97E-01	9.58E-01	9.89E-01	0.95231
	α_2	0.7791	52.221	3.76E-04	9.20E-03	4.78E-02	0.01470
	α_3	0.5406	52.459	6.04E-05	5.02E-03	3.79E-02	0.01020
	α_4	0.3127	52.687	9.28E-07	1.56E-03	2.66E-02	0.00590
	α_5	0.2433	52.757	5.77E-08	7.67E-04	2.24E-02	0.00459
	α_6	0.6519	52.348	1.66E-04	6.93E-03	4.27E-02	0.01230
7	α_1	74.5360	3.464	9.12E-01	9.59E-01	9.86E-01	0.95559
	α_2	0.9906	77.009	6.44E-04	8.84E-03	3.79E-02	0.01270
	α_3	0.6817	77.318	1.39E-04	5.05E-03	2.99E-02	0.00874
	α_4	0.4891	77.511	2.21E-05	2.82E-03	2.42E-02	0.00627
	α_5	0.2941	77.706	3.39E-07	8.97E-04	1.74E-02	0.00377
	α_6	0.2051	77.795	3.84E-09	2.94E-04	1.35E-02	0.00263
	α_7	0.8034	77.197	2.89E-04	6.52E-03	3.32E-02	0.01030
8	α_1	97.6507	4.349	9.20E-01	9.60E-01	9.84E-01	0.95736
	α_2	1.1118	100.888	7.25E-04	7.91E-03	3.13E-02	0.01090
	α_3	0.7915	101.209	2.07E-04	4.87E-03	2.52E-02	0.00776
	α_4	0.6253	101.375	6.92E-05	3.34E-03	2.17E-02	0.00613
	α_5	0.4417	101.558	8.51E-06	1.76E-03	1.74E-02	0.00433
	α_6	0.2581	101.742	6.09E-08	4.74E-04	1.21E-02	0.00253
	α_7	0.1969	101.803	1.59E-09	1.93E-04	1.00E-02	0.00193
	α_8	0.9241	101.076	3.82E-04	6.12E-03	2.78E-02	0.00906

Source: NUREG/CR-5485 (Ref. C5.31)

These values were used in the YMP PCSA by multiplying the mean failure rate for the TYP-FM data by the appropriate alpha factor for k-of-n components for failure-on-demand events (e.g., pump failure to start) and by using the alpha factor divided by two for failure-to-operate events (e.g., pump fails to run) as per the guidance in NUREG/CR-5485 (Ref. C5.31). For example, for a 2-out-of-2 failure on demand event, the mean alpha factor of 0.047 shown in the far right column of Table C3-1 associated with α_2 was multiplied by the mean failure probability for the appropriate component type and failure mode (from Table C4-1) to yield the common cause failure probability.

This approach was considered to provide conservative CCF data for all the component types for which common causes were modeled. This was considered particularly important since the

YMP has never operated and therefore the applicability of conventional nuclear plant alpha factors could not be justified.

The conservatism of this approach can be demonstrated by comparing the alpha factors used for the PCSA diesel generator CCF events to those posted on the U.S. Nuclear Regulatory Commission website for use in Probabilistic Risk Assessment studies of commercial nuclear power plants in the U.S.

The alpha factor used for the PCSA for 2 of 2 diesel generators failing to start was the 0.047 value cited earlier, while the mean alpha factor for a CCCG=2 cited by the NRC (Ref. C5.36) is 0.0136.

Diesel generators are the only component types for which such a comparison can be made since the other YMP component types for which common cause failures were modeled were not covered by the NRC equipment-specific alpha factors.

C4 ACTIVE COMPONENT RELIABILITY ESTIMATES INPUT TO SAPHIRE

Since the primary active component reliability data task objective is to support the quantification of fault tree models developed in SAPHIRE by the system analysts, the output data had to conform to the format appropriate for input to the SAPHIRE code.

SAPHIRE provides template data to the fault tree models in the form of three input comma delimited files:

- BEA – attributes to assign information to the proper SAPHIRE fields
- BED – descriptions of the component type name and failure mode
- BEI – information on the failure rate or probability estimates and distributions used.

Demonstration files for the .BEA, .BED and .BEI template data files provided with SAPHIRE were originally used to construct the PCSA template data files to ensure the proper formatting of the data for use by the fault tree models. In general, the .BEA file provides attribute designators for the code to implement such that the template data is properly assigned to the appropriate fields in SAPHIRE. The .BED file allows description information to be entered and linked to the template data name or designator (which in the YMP PCSA case was the TYP-FM coding). Examples of descriptions used for the PCSA template data were Clutch Failed to Operate, Relay Spurious Operation, Position Sensor Fails on Demand, and Wire Rope Breaks. The .BEI file contains the actual active component reliability parameters, namely the mean value and uncertainty parameter, either the Lognormal Error Factor, or the shape parameter of the Beta or Gamma distributions.

Geometric means of the input parameters from the industry-wide data sources were initially used as screening values for each TYP-FM and were entered into the .BEI file, along with a default Error Factor of 10. Once the Bayesian combination process was completed for all 275 TYP-FM combinations, mean and uncertainty parameter information was entered into the BEA files, and tested in SAPHIRE before being distributed to the systems analysts.

Failure probability per demand information was entered as SAPHIRE Calculation Type 1 for a simple probability and failure rate per hour information was entered as SAPHIRE Calculation Type 3 as a mean failure rate in the lambda field. Calc Type 3 uses the formula $P = 1 - \exp(-\lambda T_m)$, where λ is the mean failure rate (or lambda) and T_m is the mission time. Mission time is defined in the SAPHIRE Basics manual as "…the period of time that a component is required to operate in order to characterize the component operation as successful." Since the template data was to be used for all YMP facilities while the mission times would be system-specific, the mission time field in the three template data files was left blank and these times were instead input individually by the systems analysts.

The correlation class field was also used for the YMP template data files "to account for data dependencies among like events in the database" during the uncertainty analysis, as stated in the SAPHIRE Basics manual. This meant that all components in the same correlation class would be treated the same during the uncertainty analysis. This feature of SAPHIRE is based upon the observations documented (Ref. C5.2) that in the risk models, all components of the same type are quantified with the same failure rate or probability, therefore it is appropriate to group together the experience of all the nominally identified components in the same facility. Therefore, all components of the same type and failure mode are aggregated into a single number, meaning that the dependency between components of the same class must somehow be addressed. For example, if multiple motor-operated valves needed to open for success and all are assigned the same failure probability, then these basic events needed to be correlated via being assigned the same correlation class in the .BEI file. However, if different probabilities were to be used for different motor-operated valves based on the data, then the basic events would not be correlated. In all cases, a correlation class identifier, using the TYP-FM acronyms, was input to the .BEI file to indicate that all equipment with in the same TYP-FM should be correlated by the SAPHIRE model. SAPHIRE then would sample from one distribution and then use this sampled probability for all other basic events with the same correlation class.

The template data was also identified by TYP-FM combination and was utilized by the fault tree models by being imported into SAPHIRE using the MAR-D portion of the code, then by using the Modify Event feature to link the template data to each basic event in the fault tree. This permitted each active component of the same type and failure mode to utilize the same failure estimate and uncertainty information, based on the results of the industry-wide data investigation and Bayesian combination process.

Table C4-1 shows the active component reliability estimates that were input to SAPHIRE as template data for fault tree model quantification.

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
AHU-FTR	Air Handling Unit Failure to Run	G	5.00E-01[b]		3.80E-06[b]	1 source; N/D	NUREG/CR-6928 (Ref. C5. 16)
ALM-SPO	Alarm/Annunciator Spurious Operation	L	1.30E+01		4.74E-07	5 sources N/D; 1 source mean	IEEE-500 (Ref. C5.23), NPRD-95 (Ref. C5.40)
AT-FOH	Actuator (Electrical) Failure	L	1.24E+01		7.54E-05	3 sources; N/D	NPRD-95 (Ref. C5.40)
ATH-FOH	Actuator (Hydraulic) Failure	L	3.81E+01		8.91E-04	4 sources; N/D	NPRD-95 (Ref. C5.40)
ATP-SPO	Actuator (Pneumatic Piston) Spurious Operation	L	5.00E+00		1.34E-06	1 source; mean + EF	NPRD-95 (Ref. C5.40)
AXL-FOH	Axle Failure	G	5.00E-01[b]		1.60E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
B38-FOH	Bearing Failure	L	1.13E+01		2.50E-06	8 sources; N/D	NPRD-95 (Ref. C5.40)
BEA-BRK	Lifting Beam/Boom Breaks	G	1.50E+00		2.40E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
BLD-RUP	Air Bag Ruptures	B	1.10E+04	1.36E-04		1 source; N/D	BSC 2007 (Ref. C5.7)
BLK-FOD	Block or Sheaves Failure on Demand	B	1.30E+06	1.15E-06		1 source; N/D	NPRD-95 (Ref. C5.40)
BRH-FOD	Brake (Hydraulic) Failure on Demand	L	5.50E+01	8.96E-06		3 sources N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40)
BRK-FOD	Brake Failure on Demand	L	6.30E+00	1.46E-06		3 sources; mean + EF	EPRI PRA (Ref. C5.8)
BRK-FOH	Brake (Electric) Failure	G	2.50E+00		4.40E-06	1 source; N/D	NPRD-95 (Ref. C5.40)
BRP-FOD	Brake (Pneumatic) Failure on Demand	L	2.55E+00	5.02E-05		4 sources; N/D	NPRD-95 (Ref. C5.40)
BRP-FOH	Brake (Pneumatic) Failure	L	2.55E+00		8.38E-06	4 sources; N/D	NPRD-95 (Ref. C5.40)
BTR-FOD	Battery No Output Given Challenge	B	6.05E+01	8.20E-03		1 source; N/D	NUREG/CR-4639 (Ref. C5.39)
BTR-FOH	Battery Failure	L	4.30E+00		4.29E-06	12 sources N/D; 8 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5. 16), SAIC Umatilla (Ref. C5.41)
BUA-FOH	AC Bus Failure	L	3.08E+00		6.10E-07	3 sources; N/D	IEEE 493 (Ref. C5. 22), NUREG/CR-6928 (Ref. C5. 16)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
BUD-FOH	DC Bus Failure	L	8.70E+01		2.40E-07	1 source mean + EF	IEEE-500 (Ref. C5.23)
BYC-FOH	Battery Charger Failure	L	1.00E+01		7.60E-06	1 source mean + EF	CCPS (Ref. C5.1)
C52-FOD	Circuit Breaker (AC) Fails on Demand	L	9.80E+00	2.24E-03		19 sources N/D; 1 source mean + EF	CCPS (Ref. C5.1), NUREG/CR-4639 (Ref. C5.39), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
C52-SPO	Circuit Breaker (AC) Spurious Operation	L	2.29E+01		5.31E-06	12 sources N/D; 1 source mean + EF	CCPS (Ref. C5.1), MIL-HDBK-217F (Ref. C5.12), NUREG/CR-6928 (Ref. C5.16), NUREG/CR-4639 (Ref. C5.39), SAIC Umatilla (Ref. C5.41)
C72-SPO	Circuit Breaker (DC) Spurious Operation	L	1.20E+00		1.07E-06	3 sources N/D; 1 source mean + EF	CCPS (Ref. C5.1), MIL-HDBK-217F (Ref. C5.12), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16)
CAM-FOH	Cam Lock Fails	L	8.30E+01		3.19E-06	4 sources N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40)
CBP-OPC	Cables (Electrical Power) Open Circuit	G	5.00E-01		9.13E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
CBP-SHC	Cables (Electrical Power) Short Circuit	G	5.00E-01		1.88E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
CKV-FOD	Check Valve Fails on Demand	L	1.36E+01	6.62E-04		4 sources N/D; 7 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5. 16), SRS Reactors (Ref. C5.5)
CKV-FTX	Check Valve Fails to Check	L	1.50E+01	2.20E-03		1 source; mean + EF	CCPS (Ref. C5.1)
CON-FOH	Electrical Connector (Site Transporter) Failure	G	5.00E-01		7.14E-05	1 source; N/D	NPRD-95 (Ref. C5.40)
CPL-FOH	Coupling (Automatic) Failure	L	5.00E+00		1.90E-06	1 source mean + EF	AIAA (Ref. C5.11)
CPO-FOH	Control System Onboard [TEV or Trolley] Failure	G	9.85E+01		2.10E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
CRD-FOH	Card Reader Failure	L	5.00E+00		4.55E-05	1 source mean + EF	HID (Ref. C5.21)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
CRJ-DRP	Jib Crane Drop	B	9.72E+04	2.60E-05		1 source; N/D	NUREG-1774 (Ref. C5.26)
CRN-DRP	200 Ton Crane Drop	L	4.35E+01	3.21E-05		2 sources N/D; 4 sources mean + EF	NUREG-0612 (Ref. C5.35), NUREG-1774 (Ref. C5.26), EPRI PRA (Ref. C5.8)
CRN-TBK	200 Ton Crane Two Block Drop	L	1.15E+01	4.41E-07		1 source N/D; 3 sources mean + EF	NUREG-0612 (Ref. C5.35), NUREG-1774 (Ref. C5.26)
CRS-DRP	200 Ton Crane Sling Drop	B	2.06E+04	1.21E-04		1 source; N/D	NUREG-1774 (Ref. C5.26)
CRW-DRP	WP (Non-Single Failure Proof) Crane Drop	B	3.34E+04	1.05E-04		1 source; N/D	NUREG-1774 (Ref. C5.26)
CRW-TBK	WP (Non-Single Failure Proof) Crane Two Block Drop	B	3.34E+04	4.49E-05		1 source; N/D	NUREG-1774 (Ref. C5.26)
CSC-FOH	Cask Cradle Failure	G	1.50E+00		4.81E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
CT-FOD	Controller Mechanical Jamming	L	5.00E+00[b]	4.00E-06		1 source; mean + EF	EPRI PRA (Ref. C5.8)
CT-FOH	Controller Failure	L	1.00E+01		6.88E-05	1 source mean + EF	CCPS (Ref. C5.1)
CT-SPO	Controller Spurious Operation	L	1.00E+01		2.27E-05	1 source mean + EF	CCPS (Ref. C5.1)
CTL-FOD	Logic Controller Fails on Demand	L	1.10E+01	2.03E-03		3 sources; N/D	NUREG/CR-6928 (Ref. C5.16)
DER-FOM	Derailment Failure per Mile	G	3.97E+03		1.18E-05	1 source; N/D	Federal Railroad Administration (Ref. C5.17)
DG-FTR	Diesel Generator Fails to Run	L	1.51E+01		4.08E-03	8 sources N/D; 1 source mean + EF	CCPS (Ref. C5.1), IEEE 493 (Ref. C5.22), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-3831 (Ref. C5.24), NUREG/CR-6890 (Ref. C5.15), NUREG/CR-6928 (Ref. C5.16), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
DG-FTS	Diesel Generator Fails to Start	L	3.50E+00	8.38E-03		9 sources N/D; 1 source mean + EF	CCPS (Ref. C5.1), IEEE 493 (Ref. C5.22), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-3831 (Ref. C5.24), NUREG/CR-6890 (Ref. C5.15), NUREG/CR-6928 (Ref. C5.16), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
DGS-FTR	Diesel Generator - Seismic - Fails to Run for 29 Days	G	5.05E+01		8.27E-04	1 source, N/D	NUREG/CR-6890 (Ref. C5.15)
DM-FOD	Drum Failure on Demand	L	1.00E+01	4.00E-08		2 sources mean + EF	EPRI PRA (Ref. C5.8)
DM-MSP	Drum Misspooling (Hourly)	G	5.00E-01		6.86E-07	1 source, N/D	NPRD-95 (Ref. C5.40)
DMP-FOH	Damper (Manual) Fails to Operate	L	4.30E+00		5.94E-06	3 sources mean + EF	IEEE-500 (Ref. C5.23), N-Reactor (Ref. C5.46), Moss (Ref. C5.32)
DMP-FRO	Damper (Manual) Fails to Remain Open (Transfers Closed)	L	3.20E+00		8.38E-08	2 sources N/D; 2 sources mean + EF	NUREG/CR-3154 (Ref. C5.6), NUREG/CR-1363 (Ref. C5.28), NUREG/CR-4639 (Ref. C5.39), SAIC Umatilla (Ref. C5.41)
DMS-FOH	Demister (Moisture Separator) Failure	L	5.00E+00		9.12E-06	1 source mean + EF	EPRI AP-2071 (Ref. C5.10)
DRV-FOH	Drive (Adjustable Speed) Failure	G	5.0E-01		2.5E-04	1 source; N/D	NPRD-95 (Ref. C5.40)
DRV-FSO	Drive (Adjustable Speed) Failure to Stop on Demand	B	2.5E+02		3.4E-05	1 source; N/D	NPRD-95 (Ref. C5.40)
DTC-RUP	Duct Ruptures	L	2.6E+01		3.7E-06	9 sources N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40), SRS Reactors (Ref. C5.5), SAIC Umatilla (Ref. C5.41)
DTM-FOD	Damper (Tornado) Failure on Demand	L	5.0E+00	8.7E-04		1 source; mean + EF	IEEE-500 (Ref. C5.23)
DTM-FOH	Damper (Tornado) Failure	L	7.9E+00		2.3E-05	2 sources N/D; 1 source mean + EF	IEEE-500 (Ref. C5.23), Moss (Ref. C5.32)
ECP-FOH	Position Encoder Failure	G	5.0E-01		1.8E-06	2 sources; N/D	NPRD-95 (Ref. C5.40)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
ESC-FOD	Emergency Stop Button Controller Failure to Stop (on Demand)	L	5.0E+00	2.5E-04		1 source; mean + EF	EPRI PRA (Ref. C5.8)
FAN-FTR	Fan (Motor-Driven) Fails to Run	L	4.6E+01		7.21E-05	11 sources N/D; 6 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
FAN-FTS	Fan (Motor-Driven) Fails to Start on Demand	L	1.0E+01	2.0E-03		7 sources N/D; 5 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
FRK-PUN	Forklift Puncture	L	1.06E+01		1.20E-05	1 source mean + EF	SAIC Umatilla (Ref. C5.41)
G65-FOH	Governor Failure	G	1.82E+02		1.16E-05	1 source; N/D	NPRD-95 (Ref. C5.40)
GPL-FOD	Grapple Failure on Demand	B	1.30E+06	1.15E-06		1 source; N/D	NPRD-95 (Ref. C5.40)
GRB-FOH	Gear Box Failure	L	1.40E+01		2.21E-04	1 source N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40)
GRB-SHH	Gear box Shaft/Coupling Shears	L	5.00E+00		2.40E-06	1 source; mean + EF	EPRI PRA (Ref. C5.8)
GRB-STH	Gear Box Stripped	L	5.00E+00		7.86E-08	1 source; mean + EF	NPRD-95 (Ref. C5.40)
HC-FOD	Hand Held Radio Remote Controller Failure to Stop (on Demand)	L	8.39E+01	1.74E-03		1 source N/D; 3 sources mean + EF	EPRI PRA (Ref. C5.8), NPRD-95 (Ref. C5.40)
HC-SPO	Hand Held Radio Remote Controller Spurious Operation	G	5.00E-01		5.23E-07	1 source N/D	NPRD-95 (Ref. C5.40)
HEP-LEK	Filter (HEPA) Leaks [Bypassed]	L	1.00E+01		3.00E-06	1 source; mean + EF	SRS Reactors (Ref. C5.5)
HEP-PLG	Filter (HEPA) Plugs	L	9.5E+00		4.3E-06	3 sources N/D; 2 sources mean + EF	IEEE-500 (Ref. C5.23), NUREG/CR-4639 (Ref. C5.39), SAIC Umatilla (Ref. C5.41)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
HOS-LEK	Hose Leaking	L	2.47E+01		1.48E-05	same as HOS-RUP with factor of 10	CCPS (Ref. C5.1), NPRD-95 (Ref. C5.40), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
HOS-RUP	Hose Ruptures	L	2.47E+01		1.48E-06	2 sources N/D; 3 sources mean + EF	CCPS (Ref. C5.1), NPRD-95 (Ref. C5.40), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
IEL-FOD	Interlock Failure on Demand	L	5.0E+00	2.8E-05		1 source; mean + EF	NPRD-95 (Ref. C5.40)
IEL-FOH	Interlock Failure	L	5.50E+01		3.43E-05	4 sources; N/D	NPRD-95 (Ref. C5.40)
LC-FOD	Level Controller Failure on Demand	B	6.07E+03	6.25E-04		1 source; N/D	NUREG/CR-6928 (Ref. C5.16)
LRG-FOH	Lifting Rig or Hook Failure	G	4.65E+01		7.45E-07	1 source; N/D	NPRD-95 (Ref. C5.40)
LVR-FOH	Lever (two position; up-down) Failure	G	9.85E+01		2.10E-06	1 source; N/D	NPRD-95 (Ref. C5.40)
MCC-FOH	Motor Control Centers (MCCs) Failure	L	1.00E+01		7.49E-06	composite of Relay (RLY-FTP) + Motor Starter (MST FOH) + Limit Switch (ZS-FOH)	
MOE-FOD	Motor (Electric) Fails on Demand	L	5.00E+00	6.00E-05		1 source; mean + EF	EPRI PRA (Ref. C5.8)
MOE-FSO	Motor (Electric) Fails to Shut Off	L	1.07E+01		1.35E-08	1 source N/D; 1 source mean + EF	CCPS (Ref. C5.1), MIL-HDBK-217F (Ref. C5.12)
MOE-FTR	Motor (Electric) Fails to Run	L	9.50E+00		6.50E-06	8 sources N/D; 2 sources mean + EF	NPRD-95 (Ref. C5.40), NSWC-98-LE1 (Ref. C5.37), NUREG/CR-4639 (Ref. C5.39), OREDA-2002 (Ref. C5.43)
MOE-FTS	Motor (Electric) Fails to Start (Hourly)	L	1.90E+01		7.14E-06	5 sources N/D; 2 sources mean + EF	NPRD-95 (Ref. C5.40)
MOE-SPO	Motor (Electric) Spurious Operation	L	1.07E+01		6.74E-07	1 source N/D; 1 source mean + EF	CCPS (Ref. C5.1), MIL-HDBK-217F (Ref. C5.12)
MSC-FOH	Motor Speed Control Module Failure	G	5.00E-01		1.28E-04	1 source; N/D	NPRD-95 (Ref. C5.40)
MST-FOH	Motor Starter Failure	L	1.33E+00		1.43E-07	2 sources; N/D	IEEE 493 (Ref. C5.22)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
NZL-FOH	Nozzle Failure	L	7.50E+00		2.85E-06	5 sources N/D; 1 source mean + EF	IEEE-500 (Ref. C5.23), NPRD-95 (Ref. C5.40), SAIC Umatilla (Ref. C5.41)
PIN-BRK	Pin (Locking or Stabilization) Breaks	L	1.46E+00		2.12E-09	4 sources; N/D	NPRD-95 (Ref. C5.40)
PLC-FOD	Programmable Logic Controller Fails on Demand	B	1.35E+03	3.69E-04		1 source; N/D	NPRD-95 (Ref. C5.40)
PLC-FOH	Programmable Logic Controller Fails to Operate	L	1.00E+01		3.26E-06	5 sources N/D; 1 source mean + EF	MIL-HDBK-217F (Ref. C5.12), NPRD-95 (Ref. C5.40), SAIC Umatilla (Ref. C5.41)
PLC-SPO	Programmable Logic Controller Spurious Operation	L	1.00E+01		3.65E-07	5 sources N/D; 1 source mean + EF	MIL-HDBK-217F (Ref. C5.12), NPRD-95 (Ref. C5.40), SAIC Umatilla (Ref. C5.41)
PMD-FTR	Pump (Motor Driven) Fails to Run	L	9.9E+00		3.5E-05	6 sources N/D; 87 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-1205 (Ref. C5.45), NUREG/CR-2886 (Ref. C5.13), NUREG/CR-6928 (Ref. C5.16), OREDA-2002 (Ref. C5.43), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
PMD-FTS	Pump (Motor Driven) Fails to Start on Demand	L	3.80E+00	2.50E-03		7 sources N/D; 80 sources mean + EF	N-Reactor (Ref. C5.46), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-1205 (Ref. C5.45), NUREG/CR-2886 (Ref. C5.13), NUREG/CR-6928 (Ref. C5.16), OREDA-2002 (Ref. C5.43), SAIC Umatilla (Ref. C5.41), SRS Reactors (Ref. C5.5)
PPL-RUP	Piping (Lined) Catastrophic	L	1.50E+01		4.42E-07	1 source; mean + EF	CCPS (Ref. C5.1)
PPM-PLG	Piping (Water) Plugs	L	1.35E+01		7.26E-07	1 source N/D; 2 sources mean + EF	DuPont (Ref. C5.14), EPRI Pipe Failure Study (Ref. C5.10), SAIC Umatilla (Ref. C5.41)
PPM-RUP	Piping (Water) Ruptures	L	2.00E+01		8.75E-10	1 source; mean + EF	NUREG/CR-6928 (Ref. C5.16)
PR-FOH	Passive restraint (bumper) Failure	G	2.09E+02		4.45E-10	1 source; N/D	NPRD-95 (Ref. C5.40)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
PRM-FOH	eProm (HVAC Speed Control) Failure	G	5.00E-01		5.38E-07	1 source; N/D	MIL-HDBK-217F (Ref. C5.12)
PRV-FOD	Pressure Relief Valve Fails on Demand	L	2.72E+01	6.54E-03		6 sources N/D; 2 sources mean + EF	CCPS (Ref. C5.1), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16)
PV-SPO	Pneumatic Valve Spurious Operation	G	5.00E-01		2.92E-05	1 source; N/D	NPRD-95 (Ref. C5.40)
QDV-FOH	Quick Disconnect Valve Failure	L	3.56E+00		4.26E-06	4 sources N/D	NPRD-95 (Ref. C5.40)
RCV-FOH	Air Receiver Fails to Supply Air	L	1.00E+01		6.00E-07	1 source; mean + EF	IEEE-500 (Ref. C5.23)
RLY-FTP	Relay (Power) Fails to Close/Open	G	5.00E-01		8.77E-06	1 source N/D	NPRD-95 (Ref. C5.40)
SC-FOH	Speed Control Failure	G	5.00E-01		1.28E-04	1 source N/D	NPRD-95 (Ref. C5.40)
SC-SPO	Speed Control Spurious Operation	G	5.00E-01		3.20E-05	1 source N/D	NPRD-95 (Ref. C5.40)
SEL-FOH	Speed Selector Fails	L	5.34E+00		4.16E-06	3 sources N/D	NPRD-95 (Ref. C5.40)
SEQ-FOD	Sequencer Fails on Demand	B	7.49E+02	3.33E-03		1 source N/D	NUREG/CR-6928 (Ref. C5.16)
SFT-COL	Spent Fuel Transfer Machine (SFTM) Collision or Impact	L	4.00E+00	2.94E-06		2 sources N/D	NUREG-1774 (Ref. C5.26), McKenna (Ref. C5.20)
SFT-DRP	Spent Fuel Transfer Machine (SFTM) Drop	L	3.00E+00	5.15E-06		2 sources N/D	NUREG-1774 (Ref. C5.26), McKenna (Ref. C5. 20)
SFT-RTH	Spent Fuel Transfer Machine (SFTM) Raised Fuel Too High	L	7.00E+00	7.36E-07		2 sources N/D	NUREG-1774 (Ref. C5.26), McKenna (Ref. C5.20)
SJK-FOH	Screw Jack [TEV] Failure	G	5.00E-01		8.14E-06	1 source; N/D	NPRD-95 (Ref. C5.40)
SRF-FOH	Flow Sensor Failure	G	5.00E-01		1.07E-06	1 source; N/D	NUREG/CR-4639 (Ref. C5.39)
SRP-FOD	Pressure Sensor Fails on Demand	B	1.25E+02	4.00E-03		1 source; N/D	NPRD-95 (Ref. C5.40)
SRP-FOH	Pressure Sensor Fails	L	1.21E+01		2.95E-06	8 sources N/D	NPRD-95 (Ref. C5.40), NUREG/CR-6928 (Ref. C5.16)
SRR-FOH	Radiation Sensor Fails	L	5.00E+00		2.00E-05	1 source; mean + EF	Laurus (Ref. C5.25)
SRS-FOH	OverSpeed Sensor Fails	G	1.28E+02		2.14E-05	1 source; N/D	NPRD-95 (Ref. C5.40)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
SRT-FOD	Temperature Sensor/Transmitter Fails on Demand	L	2.10E+00	7.33E-04		2 sources N/D	NUREG/CR-6928 (Ref. C5.16), OREDA-92 (Ref. C5.42)
SRT-FOH	Temperature Sensor/Transmitter Fails	L	1.41E+01		7.05E-07	4 sources N/D; 2 sources mean + EF	NPRD-95 (Ref. C5.40), NUREG/CR-6928 (Ref. C5.16), OREDA-2002 (Ref. C5.43)
SRT-SPO	Temperature Sensor Spurious Operation	L	2.80E+01		2.23E-06	1 source; mean + EF	OREDA-2002 (Ref. C5.43)
SRU-FOH	Ultrasonic Sensor Fails	G	5.00E-01		9.62E-05	1 source; N/D	NPRD-95 (Ref. C5.40)
SRV-FOH	Vibration Sensor (Accelerometer) Fails	L	1.07E+01		9.40E-05	4 sources N/D	NPRD-95 (Ref. C5.40)
SRX-FOD	Optical Position Sensor Fails on Demand	B	3.18E+03	1.10E-03		1 source; N/D	SAIC Umatilla (Ref. C5.41)
SRX-FOH	Optical Position Sensor Fails	L	5.00E+00		4.70E-06	1 source; mean + EF	NPRD-95 (Ref. C5.40)
STU-FOH	Structure (truck or railcar) Failure	G	1.50E+00		4.81E-08	1 source; N/D	NPRD-95 (Ref. C5.40)
SV-FOD	Solenoid Valve Fails on Demand	L	1.17E+01	6.28E-04		4 sources N/D; 5 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NSWC-98-LE1 (Ref. C5.37), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16), SRS Reactors (Ref. C5.5)
SV-FOH	Solenoid Valve Fails	L	1.70E+01		4.87E-05	1 source; mean + EF	CCPS (Ref. C5.1)
SV-SPO	Solenoid Valve Spurious Operation	L	3.00E+00		4.09E-07	1 source; mean + EF	CCPS (Ref. C5.1)
SWA-FOH	Auto-Stop Switch (CTT hose travel) Fails	G	6.50E+00		3.12E-06	1 source; N/D	NPRD-95 (Ref. C5.40)
SWG-FOH	13.8kV Switchgear Fails	G	2.85E+01		1.31E-07	1 source; N/D	IEEE 493 (Ref. C5.22)
SWP-FTX	Electric Power Switch Fails to Transfer	G	6.50E+00		3.59E-07	1 source; N/D	IEEE 493 (Ref. C5.22)
SWP-SPO	Electric Power Switch Spurious Transfer	G	6.50E+00		1.55E-07	1 source; N/D	IEEE 493 (Ref. C5.22)
TD-FOH	Transducer Failure	L	4.70E+00		9.84E-05	3 sources N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
TDA-FOH	Transducer (Air Flow) Failure	L	6.21E+00		1.65E-04	2 sources N/D	NPRD-95 (Ref. C5.40), NSWC-98-LE1 (Ref. C5.37)
TDP-FOH	Transducer (Pressure) Fails	L	5.35E+01		2.20E-04	23 sources N/D; 2 sources mean + EF	NPRD-95 (Ref. C5.40), NSWC-98-LE1 (Ref. C5.37)
TDT-FOH	Transducer (Temperature) Fails	L	2.95E+01		1.04E-04	12 sources N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40)
THR-BRK	Third Rail Breaks	L	1.00E+01		1.01E-08	1 source; mean + EF	NPRD-95 TRK-BRK adjusted with failure information from Federal Railroad Administration Safety Data website (Ref. C5.17)
TKF-FOH	Fuel Tank Fails	L	1.11E+01		4.40E-07	15 sources; N/D	NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16)
TL-FOH	Torque Limiter Failure	G	8.05E+01		8.05E-05	1 source N/D	NPRD-95 (Ref. C5.40)
TRD-FOH	Tread (Site Transporter)	L	3.40E+00		5.89E-07	1 source N/D; 1 source mean + EF	NPRD-95 (Ref. C5.40), Rand (Ref. C5.38)
UDM-FOH	Damper (Backdraft) Failure	L	7.90E+00		2.26E-05	2 sources N/D; 1 source mean + EF	IEEE-500 (Ref. C5.23), Moss (Ref. C5.32)
UPS-FOH	Uninterruptible Power Supply (UPS) Failure	L	5.08E+00		2.02E-06	10 sources; N/D	NPRD-95 (Ref. C5.40)
WNE-BRK	Wire Rope Breaks	L	5.00E+00	2.00E-06		1 source; mean + EF	EPRI PRA (Ref. C5.8)
XMR-FOH	Transformer Failure	L	1.53E+01		2.91E-07	13 sources N/D; 2 sources mean + EF	CCPS (Ref. C5.1), MIL-HDBK-217F (Ref. C5.12), NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16)
XV-FOD	Manual Valve Failure on Demand	L	1.00E+01	6.48E-04		3 sources N/D; 12 sources mean + EF	CCPS (Ref. C5.1), N-Reactor (Ref. C5.46), NUREG/CR-4639 (Ref. C5.39), NUREG/CR-6928 (Ref. C5.16), SRS Reactors (Ref. C5.5)
ZS-FOD	Limit Switch Failure on Demand	L	5.7E+00	2.9E-04		3 sources N/D	MIL-HDBK-217F (Ref. C5.12), NPRD-95 (Ref. C5.40), SRS Reactors (Ref. C5.5)

Table C4-1. Active Component Reliability Estimates Entered into SAPHIRE Models (Continued)

TYP-FM	Component Name & Failure Mode	Dist Type	Uncert Value	Demand Proba-bility	Hourly Failure Rate	Number of Inputs	Input Data Sources[a]
ZS-FOH	Limit Switch Fails	L	6.03E+00		7.23E-06	3 sources N/D	MIL-HDBK-217F (Ref. C5.12), NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39)
ZS-SPO	Limit Switch Spurious Operation	L	5.56E+00		1.28E-06	3 sources N/D	MIL-HDBK-217F (Ref. C5.12), NPRD-95 (Ref. C5.40), NUREG/CR-4639 (Ref. C5.39)

NOTE: [a] Refer to Section C1.2 for specific citation to data sources.
[b]There are minor differences between the specific values tagged by this footnote and those used to quantify the SAPHIRE model. Such differences are not meaningful in the context of this analysis because (a) the difference pertains only to the uncertainty of the component reliability or (b) the uncertainty in the reliability value is much greater than difference between the value given here and that used in the model.

B = Beta Distribution; EF = Lognormal Error Factor; G = Gamma Distribution; L = Lognormal Distribution; N/D = Numerator/Denominator

Source: Original

C5 REFERENCES; DESIGN INPUTS

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

The inputs in this Section noted with an (*) indicate that they fall into one of the designated categories described in Section 4.1, relative to suitability for intended use.

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C5.3 ASME NOG-1-2004. 2005. *Rules for Construction of Overhead and Gantry Cranes (Top Running Bridge, Multiple Girder).* New York, New York: American Society of Mechanical Engineers. TIC: 257672. ISBN: 0-7918-2939-1.

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C5.7 BSC 2007 (Bechtel SAIC Company). *Waste Form Throughputs for Preclosure Safety Analysis.* 000-PSA-MGR0-01800-000-00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071106.0001.

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C5.9 *Crutchfield, D.M. 1996. "Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment." NRC Bulletin 96-02. Washington, D.C.: U.S. Nuclear Regulatory Commission. Accessed February 12, 2008. ACC: MOL.20080213.0021. URL: http://www.nrc.gov/reading-rm/doc-collections/gen-comm/bulletins/1996/bl96002.html

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ATTACHMENT D
PASSIVE EQUIPMENT FAILURE ANALYSIS

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ACRONYMS AND ABBREVIATIONS

Acronyms

ASME	American Society of Mechanical Engineers
CDF	cumulative distribution function
COV	coefficient of variation
CTM	canister transfer machine
DOE	U.S. Department of Energy
DPC	dual-purpose canister
EPS	equivalent (or effective) plastic strain
ETF	expended toughness fraction
FEA	finite element analysis
HLW	high-level radioactive waste
INL	Idaho National Laboratory
LLNL	Lawrence Livermore National Laboratory
MCO	multicanister overpack
PCSA	preclosure safety analysis
PDF	probability density function
PWR	pressurized water reactor
SAR	Safety Analysis Report
SFC	spent fuel canister
SLS	steel-lead-steel
SNF	spent nuclear fuel
TAD	transportation, aging, and disposal
TEV	transport and emplacement vehicle
WPTT	waste package transfer trolley

ACRONYMS AND ABBREVIATIONS (Continued)

Abbreviations

C	Celsius
cm	centimeter
F	Fahrenheit
ft	foot, feet
hr, hrs	hour, hours
J	joule
K	Kelvin
kg	kilogram
kV	kilovolt
kW	kilowatt
LOS	loss of shielding
m	meter
min	minute, minutes
m/s	meters/second
mrem	millirem
MPa	megapascal
mph	miles per hour
psig	pounds per square inch gauge
rem	roentgen equivalent man
W/m K	watt per meter Kelvin
W/m^2K	watt per square meter Kelvin

ATTACHMENT D
PASSIVE EQUIPMENT FAILURE ANALYSIS

Many event sequences described in Section 6.1 include pivotal events that arise from loss of integrity of a passive component, namely one of the aging overpacks, casks, or canisters that contain a radioactive waste form. Such pivotal events involve (1) loss of containment of radioactive material that may result in airborne releases, or (2) loss of shielding effectiveness. Both types of pivotal events may be failure modes caused by either physical impact to the container or by thermal energy transferred to the container. This attachment presents the results of passive failure analyses that provide conditional probability of loss of containment or loss of shielding. Many scenarios were selected for analysis as representative or bounding for anticipated scenarios in the risk assessment. Results of some scenarios may not have been used in the final event sequence quantification.

D1 LOSS OF CONTAINMENT DUE TO DROPS AND IMPACTS

The category of passive equipment includes canisters and casks used during transport, aging, and disposal of spent nuclear fuel. The canisters and casks contain the spent fuel and provide containment of radioactive material. During transport and handling, the canisters and casks could be subjected to drops, impacts, or fires, which may result in loss of containment. The probabilities of loss of containment due to various physical or thermal challenges are evaluated primarily through structural and thermal analysis and drop test data.

Passive equipment (e.g., transportation casks, storage canisters, and waste packages) may fail from abnormal use such as defined by the event sequences. Studies were performed and passive equipment failure probabilities were determined using the methodologies summarized in Section 4.3.2.2. The probability of loss of containment (breach) was determined for several types of containers, including transportation casks (analyzed without impact limiters), shielded transfer casks, waste packages, TAD canisters, DPCs, DOE standardized canisters, MCOs, HLW canisters, and naval SNF canisters. The mechanical breach of TAD canisters, DPCs and naval SNF canisters were analyzed as representative canisters as described in Section D1.1. The structural analysis of DOE standardized canisters and MCOs for breaches is described in Section D1.2 and then the probabilistic methodology of Section D1.1 was applied. Transportation casks, site transfer casks (STCs) and horizontal STCs were analyzed as representative transportation casks as describe in Section D1.1. The probabilistic estimation of breach from mechanical loads of all other waste containers is described in Sections D1.3 through D1.6. The analysis of loss or degradation of shielding of casks and overpacks against mechanical loads is described in Section D3. The probabilistic analysis of fire severity and the associated effects on casks, canisters, and overpacks with respect to both containment breach and shielding degradation or loss is described in Section D2. The analysis of mechanical failures and thermal failures included the specific configuration defined by the event sequences. For example, if the event sequence occurred during a process in which the canister is within a transportation casks or aging overpack, the analysis is performed in that configuration.

D1.1 LAWRENCE LIVERMORE NATIONAL LABORATORY ANALYSIS OF CANISTERS AND CASKS

Lawrence Livermore National Laboratory (LLNL) performed the FEA using Livermore Software–Dynamic Finite Element Program (LS-DYNA) to model drops and impacts for casks and canisters with selected properties for use as representative containers expected to be delivered to Yucca Mountain (Ref. D4.1.27). LS-DYNA, which has been used in nuclear facility and non-nuclear industrial applications, is appropriate to model nonlinear, transient responses of a passive component to a structural challenge such as a drop or an impact. Existing commercial casks and canisters that would likely be used on the Yucca Mountain Project (YMP) were identified and characterized. The cases analyzed are listed in Table D1.2-1.

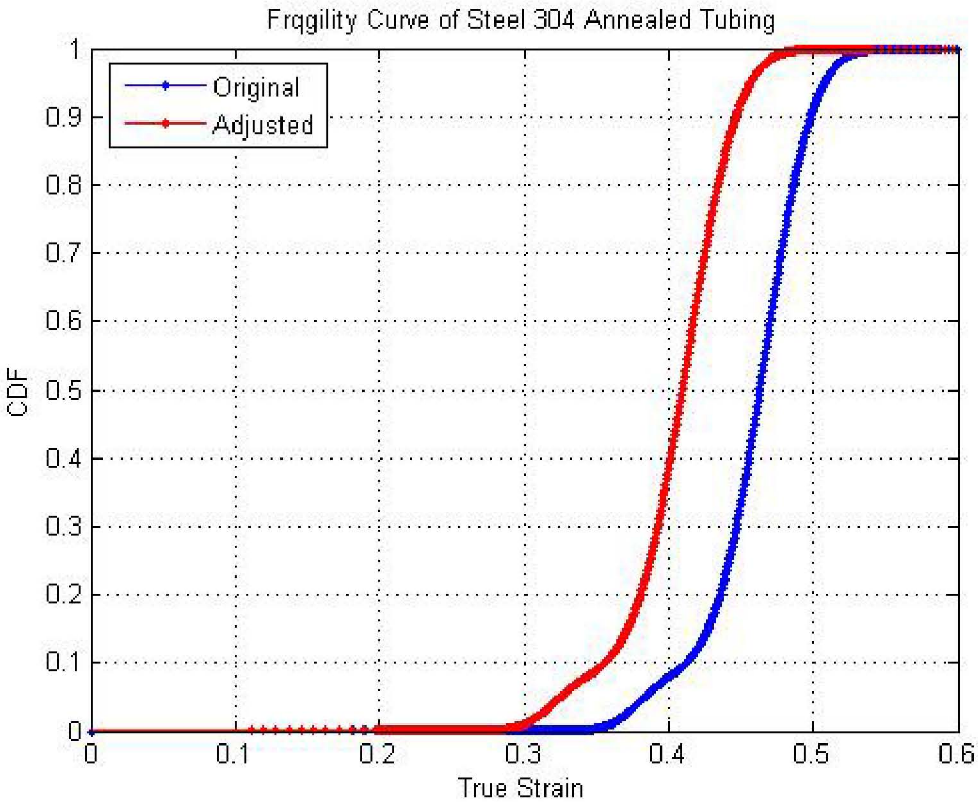
Appropriate finite element models were developed for the representative cask, selected container types, configurations, and drop types. The level of detail for each model was selected to understand deformation and damage patterns, possible failure mode(s) in each structural element, and failure-related response. Special attention was required to properly model the bottom-weld and closure regions to ensure that coarser mesh of the simplified model would capture failure-related response with acceptable accuracy. A consistent failure criterion for each case was identified as part of the detailed analyses. The effective plastic strain in each element, in combination with material ductility data, was used to predict failure measures.

The maximum strain for each scenario was compared with the capacity distribution based on material properties to obtain containment failure probabilities using the methodology described in Section 4.3.2.2. For simplicity and consistency in interpreting results, the impact-surface conditions, including both the ground and the falling 10-ton load for the analyses, were considered infinitely stiff and unyielding, which is conservative.

The results of these cases are summarized in Tables D1.2-2 through D1.2-4. The bases for these results are summarized in the following paragraphs. If a probability for the event sequence is less than 1.0×10^{-8}, additional conservatism is incorporated in the PCSA by using a failure probability of 1.0×10^{-5}, which are termed "LLNL, adjusted". This additional conservatism is added to account for a) future evolutions of cask and canister designs, and b) uncertainties, such as undetected material defects, undetected manufacturing deviations, and undetected damage associated with handling before the container reaches the repository, which are not included in the tensile elongation data.

LLNL developed a fragility curve for the base metal by fitting a mixture of two normal probability density functions (PDFs) to the engineering (tensile) strain data (Ref. D4.1.4). Both the data and their corresponding log-transforms were found to be non-normally distributed ($p < 10^{-4}$) by the Shapiro-Wilk test (Ref. D4.1.62). These data collected at 100°F were determined to be reasonably well modeled as a sample from a weighted mixture of two normal distributions, one with a mean of 46% and a standard deviation of 2.24% (weight = 7.84%), and the other with a mean of 59.3% and a standard deviation of 4.22% (weight = 92.16%), with the goodness of fit (p = 0.939) assessed by the Kolmogorov-Smirnov 1 sample test (Ref. D4.1.33).

The stainless steel used in the LLNL (Ref. D4.1.27) analysis is alloy 304L. The un-annealed alloys have relatively shorter elongations at failure than annealed 304L. Therefore, the base fragility cumulative distribution function (CDF) model was adjusted to different steels used in a typical design and to meet the code specification of the material model used in LS-DYNA. The adjustment consisted of shifting the distribution by -8.3% (Ref. D4.1.27, p. 93). Thus the initial fragility curve was shifted by 8.3% to a lower value of minimum elongation. The fragility curves before and after the shift are shown in Figure D1.1-1 and tabulated in Table D1.1-1. 316L stainless steel might be used for construction of some canisters and casks, but the stress-strain curves would be similar.



Source: Ref. D4.1.27, Figure 6.3.7-3

Figure D1.1-1. Original and Shifted Cumulative Distribution Functions (CDF) for Capacity (or Fragility) Plotted as a Function of True Strain

Table D1.1-1. Probability of Failure versus True Strain Tabulated for Figure D1.1-1

True Strain (*TS*)	$\frac{TS - TS_{mean}}{TS_{std}}$	Probability of Failure Original	Probability of Failure Adjusted (-8.3% shift)	True Strain (*TS*)	$\frac{TS - TS_{mean}}{TS_{std}}$	Probability of Failure Original	Probability of Failure Adjusted (-8.3% shift)
0.00	-1.70	0.0000E+00	1.6754E-15	0.36	0.05	1.0506E-02	1.0973E-01
0.01	-1.65	2.0924E-16	1.8688E-15	0.37	0.10	2.3978E-02	1.4282E-01
0.02	-1.60	4.1848E-16	2.0622E-15	0.38	0.15	4.3259E-02	1.9679E-01
0.03	-1.55	6.2772E-16	2.2555E-15	0.39	0.19	6.2863E-02	2.7687E-01

Table D1.1-1. Probability of Failure versus True Strain Tabulated for Figure D1.1-1 (Continued)

True Strain (*TS*)	$\frac{TS - TS_{mean}}{TS_{std}}$	**Probability of Failure Original**	**Probability of Failure Adjusted (-8.3% shift)**	**True Strain (*TS*)**	$\frac{TS - TS_{mean}}{TS_{std}}$	**Probability of Failure Original**	**Probability of Failure Adjusted (-8.3% shift)**
0.04	-1.50	8.3696E-16	2.4489E-15	0.40	0.24	7.9100E-02	3.8310E-01
0.05	-1.45	1.0462E-15	2.6422E-15	0.41	0.29	9.5539E-02	5.0814E-01
0.06	-1.41	1.2554E-15	2.8356E-15	0.42	0.34	1.2068E-01	6.3823E-01
0.07	-1.36	1.4647E-15	3.0290E-15	0.43	0.39	1.6410E-01	7.5736E-01
0.08	-1.31	1.6739E-15	3.2223E-15	0.44	0.44	2.3393E-01	8.5309E-01
0.09	-1.26	1.8832E-15	3.4157E-15	0.45	0.48	3.3371E-01	9.2036E-01
0.10	-1.21	2.0924E-15	3.6090E-15	0.46	0.53	4.5893E-01	9.6161E-01
0.11	-1.16	2.3016E-15	3.8024E-15	0.47	0.58	5.9615E-01	9.8363E-01
0.12	-1.11	2.5109E-15	2.8601E-14	0.48	0.63	7.2682E-01	9.9385E-01
0.13	-1.07	2.7201E-15	2.3645E-13	0.49	0.68	8.3454E-01	9.9797E-01
0.14	-1.02	2.9294E-15	1.6225E-12	0.50	0.73	9.1117E-01	9.9941E-01
0.15	-0.97	3.1386E-15	9.7686E-12	0.51	0.78	9.5806E-01	9.9985E-01
0.16	-0.92	3.3478E-15	5.2952E-11	0.52	0.82	9.8270E-01	9.9997E-01
0.17	-0.87	3.5571E-15	2.6233E-10	0.53	0.87	9.9379E-01	9.9999E-01
0.18	-0.82	3.7663E-15	1.2513E-09	0.54	0.92	9.9807E-01	1.0000E+00
0.19	-0.78	2.1733E-14	6.9107E-09	0.55	0.97	9.9948E-01	1.0000E+00
0.20	-0.73	2.1209E-13	2.6769E-08	0.56	1.02	9.9988E-01	1.0000E+00
0.21	-0.68	1.7358E-12	1.1600E-07	0.57	1.07	9.9998E-01	1.0000E+00
0.22	-0.63	1.1373E-11	4.8126E-07	0.58	1.11	1.0000E+00	1.0000E+00
0.23	-0.58	6.4625E-11	1.9316E-06	0.59	1.16	1.0000E+00	1.0000E+00
0.24	-0.53	4.1126E-10	7.5246E-06	0.60	1.21	1.0000E+00	1.0000E+00
0.25	-0.48	2.4773E-09	2.8566E-05	0.61	1.26	1.0000E+00	1.0000E+00
0.26	-0.44	1.2132E-08	1.0566E-04	0.62	1.31	1.0000E+00	1.0000E+00
0.27	-0.39	5.2343E-08	3.7635E-04	0.63	1.36	1.0000E+00	1.0000E+00
0.28	-0.34	2.4478E-07	1.2625E-03	0.64	1.41	1.0000E+00	1.0000E+00
0.29	-0.29	1.0945E-06	3.8474E-03	0.65	1.45	1.0000E+00	1.0000E+00
0.30	-0.24	4.7123E-06	1.0185E-02	0.66	1.50	1.0000E+00	1.0000E+00
0.31	-0.19	1.9709E-05	2.2466E-02	0.67	1.55	1.0000E+00	1.0000E+00
0.32	-0.15	7.9860E-05	4.0237E-02	0.68	1.60	1.0000E+00	1.0000E+00
0.33	-0.10	3.1104E-04	5.9110E-02	0.69	1.65	1.0000E+00	1.0000E+00
0.34	-0.05	1.1366E-03	7.5125E-02	0.70	1.70	1.0000E+00	1.0000E+00
0.35	**0.00**	**3.7379E-03**	**8.9858E-02**				

NOTE: The mean for true strain is 0.35, shown in bold. The standard deviation (std) of true strain is 0.21.

Source: Ref. D4.1.27, Table 6.3.7.3-1

The weldment at best can have the same mechanical properties as the hosting metal (native metal), but it is usually more brittle than the hosting metal. The failure likelihood of the weldment substructure was considered, reflecting weighting factors of both 1.0 and 0.75 applied to estimated true strain at failure.

The capacity function is based on coupon tensile strength tests in uniaxial tension. However, cracking of a stainless steel may not be determined simply by comparing the calculated plastic strain to the true strain of failure, because the equivalent (or effective) plastic strain (EPS) is calculated from a complex 3-D state of stress, while the true strain at failure was based on data from a 1-D state of stress. A 3-D state of stress may constrain plastic flow in the material and lower the EPS at which failure occurs. This loss of ductility is accounted for by the use of a triaxiality factor, which is the ratio of normal stress to shear stress on the octahedral plane, normalized to unity for simple tension. For the purpose of determining the probability of structural failure, LLNL (Ref. D4.1.27) set the ductility ratio to 0.5. This is equivalent to a triaxiality factor of 2, which corresponds to a state of biaxial tension.

Failure of containment can occur when strain in a component is of sufficient magnitude that it results in breakage or puncture of the container. The probability of failure is calculated based on the maximum strain for a single finite element brick obtained from LS-DYNA simulations. Fracture propagation takes place on the milliseconds time-scale and thus propagates across the canister wall thickness very quickly, compared to the time-frame of the LS-DYNA simulations. Furthermore, the fragility curve is obtained on the basis of a maximum average strain over the thickness of the respective specimens, which are 2 in. long stainless steel 304L specimens. Although LS-DYNA results provide multiple values of the strain through the thickness of the canister wall (the wall thickness being represented by multiple finite element layers), it is more conservative to use the maximum strain value at a single finite element brick than the average of the multiple values across the thickness of the wall.

The probability of failure for each impact scenario is evaluated by finding the maximum strain at a location in which a through-wall crack would constitute a radionuclide release. A probability of failure is determined from the CDF of capacity or fragility curve (as discussed below) from the global maximum strain.

A conservative approach and aid to computational efficiency is achieved by performing calculations focusing on the regions of the container having high strain (and deformation) after a drop (“hot zones”). An importance sampling strategy was used which places greater-than-random emphasis on ranges of input-variable values, and/or on combinations of such value ranges, that are more likely to affect output. This approach is an alternative to Monte Carlo methods with the important advantage that possible combinations of upper-bound variable values are in fact incorporated into each probabilistic estimate of expected model output (which is not always guaranteed by uniform sampling).

Using the general probabilistic approach summarized here, LLNL (Ref. D4.1.27) calculated failure probabilities for representative canisters in an aging overpack, and in a transportation cask, and for the representative canister itself, as presented in Tables D1.2-2 through D1.2-5. For the drop of a 10-metric-ton load onto a cask, the falling mass is modeled as a rigid (unyielding) wall, oriented normal to longitudinal axis of the cask.

D1.2 IDAHO NATIONAL LABORATORY ANALYSIS OF SPENT NUCLEAR FUEL CANISTERS AND MULTICANISTER OVERPACKS

Drop tests of prototype canisters conducted by the Idaho National Laboratory (INL) confirmed that the stainless steel shell material can undergo significant strains without material failure leading to loss of containment. These drop tests also validated analytical models used to predict strains under various drop scenarios. Table D1.2-6 shows scenarios selected to address potential drop scenarios at YMP facilities and the predicted strains.

INL performed FEA (using ABAQUS/Explicit, which, like LS-DYNA, has been used in nuclear facility and non-nuclear industrial applications, and is appropriate to model nonlinear, transient responses of a passive component to a structural challenge such as a drop or an impact) of 23-foot drops, three degrees off vertical, to determine the extent of strain at various positions in the bottom head, cylindrical shell, and joining weld. The strain was evaluated and reported for the inside, outside, and middle layers (Ref. D4.1.64). The U.S. Department of Energy (DOE) standardized spent nuclear fuel (SNF) canisters were modeled at 300°F, the maximum skin temperature expected due to the heat evolved by the fuel (based on review of thermal analyses performed by transportation casks vendors), resulting in diminished casing material strength. It was found that greater strains would be expected in the multicanister overpacks (MCOs) at ambient temperatures than at elevated temperatures.

During a canister drop event, the majority of the kinetic energy at impact performs work on the material, which causes the worst locations to exhibit plastic strain. A good measure of this work is equivalent plastic strain, which is a cumulative strain measure that takes into account the deformation history starting at impact. From the peak equivalent plastic strain, LLNL (Ref. D4.1.27) developed failure probabilities using the method described in Section D1.1 for an 18 in. and 24 in. DOE standard canister and an MCO. Results are summarized in Table D1.2-7.

Table D1.2-1. Container Configurations and Loading Conditions

Container	Configuration	Drop Type/Impact Condition[a]	Drop Height
AO (aging overpack) cell with canister inside	Representative canister inside AO	A IC 1: End with vertical orientation	3-ft vertical
		A IC 2: Slapdown from a vertical orientation and 2.5 mph horizontal velocity	0-ft vertical
Transportation cask with spent nuclear fuel (SNF) canister inside	Representative canister inside representative cask	T IC 1a: End, with 4 degree off-vertical orientation	12-ft vertical
		T.IC 1b: Same as T.IC 1a	13.1-ft vertical
		T.IC 1c: Same as T.IC 1a	30-ft vertical
		T IC 2a: End, with 4 degree off-vertical orientation, and approximated slapdown	13.1-ft vertical
		T.IC 2b: Same as T.IC 2a, with no free fall	0-ft vertical
		T IC 3: Side, with 3 degree off-horizontal orientation	6-ft vertical
		T IC 4: Drop of 10-metric-ton load onto top of cask	10-ft vertical
DPC (Dual purpose canister) TAD (Transportation, aging, and disposal) canister	Representative canister	D IC 1a: End, with vertical orientation	32.5-ft vertical
		D IC 1b: Same as D.IC 1a	40-ft vertical
		D IC 2a: End, with 4 degree off-vertical orientation	23-ft vertical
		D IC 2b: Same as D.IC 2a	10-ft vertical
		D IC 2c: Same as D.IC 2a	5-ft vertical
		D IC 3: 40 ft/min horizontal collision inside the CTM bell	No drop
		D IC 4: Drop of 10-metric-ton load onto top of canister	10-ft vertical
		D.IC 2a: Hourglass-control study for end drop, with 4 degree off-vertical orientation	23-ft vertical
		D.IC 2a: Friction coefficient sensitivity study for end drop, with 4 degree off-vertical orientation	23-ft vertical
		D.IC 2a: Mesh density study for end drop, with 4 degree off-vertical orientation	23-ft vertical
		D.IC 2a: Shell- and bottom-lid-thickness sensitivity study for end drop, with 4 degree off-vertical orientation	23-ft vertical
DSNF (DOE spent nuclear fuel) canister	INL-analyzed case	O.IC 1: End, with 3-degree-off vertical orientation	23-ft vertical

NOTE: A = aging overpack; (AO) CTM = canister transfer machine; ft = foot; D = dual purpose canister; IC = impact condition; min = minute; mph = miles per hour; O = DOE SNF canister; SNF = spent nuclear fuel; T = transportation cask.

Source: [a] Ref. D4.1.27, Table 4.3.3-1a.

Table D1.2-2. Failure Probabilities with and without Triaxiality Factor, with and without the Fragility Curve Adjustment, for Representative Canister within an Aging Overpack

Container Type/ Impact Condition[a]	Impact Condition Description	Max EPS[b]	Failure Probability[b]			
			Original CDF Fragility Curve w/o Adjustment		CDF Fragility Curve Adjusted for Minimum Elongation (-8.3% Shift)	
			w/o Triaxiality	with Triaxiality	w/o Triaxiality	with Triaxiality
A.IC 1	3-ft end drop, with vertical orientation	0.16%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
A.IC 2	Slapdown from a vertical orientation and 2.5-mph horizontal velocity	0.82%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$

NOTE: [a]"A" stands for aging overpack. "IC" stands for impact condition, which are defined in Table D1.2-1.
[b]Values of Max EPS and failure probability are applicable to the SNF canister.

Source: Ref. D4.1.27, Table 6.3.7.6-1.

Table D1.2-3. Failure Probabilities with and without Triaxiality Factor, with and without Fragility Curve Adjustment, for Representative Canister

Container Type/ Impact Condition[a]	Impact Condition Description	Max EPS[b]	Failure Probability[b]			
			Original CDF Fragility Curve w/o Adjustment		CDF Fragility Curve Adjusted for Minimum Elongation (-8.3% Shift)	
			w/o Triaxiality	with Triaxiality	w/o Triaxiality	with Triaxiality
D.IC 1a	32.5-ft end drop, with vertical orientation	2.13%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
D.IC 1b	40-ft end drop, with vertical orientation	2.65%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
D.IC 2a	23-ft end drop, with 4-degree off-vertical orientation	24.19%	$<1 \times 10^{-8}$	**7.71×10^{-1}**	**9.72 × 10–$^{-6}$**	**9.96×10^{-1}**
D.IC 2b	10-ft end drop, with 4-degree off-vertical orientation	19.71%	$<1 \times 10^{-8}$	**7.01×10^{-2}**	**1.73×10^{-8}**	**3.19×10^{-1}**
D.IC 2c	5-ft end drop, with 4-degree off-vertical orientation	15.76%	$<1 \times 10^{-8}$	**4.10×10^{-5}**	$<1 \times 10^{-8}$	**3.12×10^{-2}**
D.IC 3	40-ft/min horizontal side collision	0.16%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
D.IC 4	10-ft drop of 10-metric-ton load onto top of canister	0.75%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$

Table D1.2-3. Failure Probabilities with and without Triaxiality Factor, with and without Fragility Curve Adjustment, for Representative Canister (Continued)

Container Type/ Impact Condition[a]	Impact Condition Description	Max EPS[b]	Failure Probability[b]			
			Original CDF Fragility Curve w/o Adjustment		CDF Fragility Curve Adjusted for Minimum Elongation (-8.3% Shift)	
			w/o Triaxiality	with Triaxiality	w/o Triaxiality	with Triaxiality
D.IC 2a S1-L1	Same as D.IC 2a	24.19%	$<1 \times 10^{-8}$	**7.71×10^{-1}**	**9.72×10^{-6}**	**9.96×10^{-1}**
D.IC 2a S2-L1	Same as D.IC 2a	21.52%	$<1 \times 10^{-8}$	**1.66×10^{-1}**	**2.44×10^{-7}**	**7.62×10^{-1}**
D.IC 2a S3-L1	Same as D.IC 2a	16.53%	$<1 \times 10^{-8}$	**3.37×10^{-4}**	$<1 \times 10^{-8}$	**6.02×10^{-2}**
D.IC 2a S1-L2	Same as D.IC 2a	23.34%	$<1 \times 10^{-8}$	**5.52×10^{-1}**	**3.07×10^{-6}**	**9.78×10^{-1}**
D.IC 2a S1-L3	Same as D.IC 2a	25.15%	$<1 \times 10^{-8}$	**9.28×10^{-1}**	**3.48×10^{-5}**	**1.00**
D.IC 2a S2-L3	Same as D.IC 2a	22.57%	$<1 \times 10^{-8}$	**3.50×10^{-1}**	**1.07×10^{-6}**	**9.28×10^{-1}**
D.IC 2a S3-L3	Same as D.IC 2a	18.08%	$<1 \times 10^{-8}$	**1.22×10^{-2}**	$<1 \times 10^{-8}$	**1.14×10^{-1}**
D.IC 2a S2-L4	Same as D.IC 2a	24.07%	$<1 \times 10^{-8}$	**7.44×10^{-1}**	**8.27×10^{-6}**	**9.95×10^{-1}**
D.IC 2a S3-L4	Same as D.IC 2a	19.50%	$<1 \times 10^{-8}$	**6.29×10^{-2}**	**1.37×10^{-8}**	**2.77×10^{-1}**

NOTE: [a]"D" stands for dual purpose canister. "IC" stands for impact condition, which are defined in Table D1.2-1.

See Table 6.3.3.5-1 of Ref. D4.1.27 for definitions of H1, F1, M1, etc. See Table 6.3.3.6-1 of Ref. D4.1.27 for definitions of S1, L1, etc.

[b]Values of Max EPS and failure probability are applicable to the SNF canister. A range of canister shell and bottom plate thicknesses were evaluated. The values shown are for the configuration that yielded the highest strains (0.5-inch shell thickness and 2.313 inch bottom plate thickness)

Source: *Seismic and Structural Container Analyses for the PCSA* (Ref. D4.1.27, Table 6.3.7.6-3)

Table D1.2-4. Failure Probabilities with and without Triaxiality Factor, with and without the Fragility Curve Adjustment, for the Representative Canister inside the Transportation Cask

Container Type/ Impact Condition[a]	Impact Condition Description	Max EPS[b]	Failure Probability[b]			
			Original CDF Fragility Curve w/o Adjustment		CDF Fragility Curve Adjusted for Minimum Elongation (-8.3% Shift)	
			w/o Triaxiality	with Triaxiality	w/o Triaxiality	with Triaxiality
T.IC 1a	12-ft end drop, with 4-degree off-vertical orientation	3.53%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 1b	13.1-ft end drop, with 4-degree off-vertical orientation	4.06%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 1c	30-ft end drop, with 4-degree off-vertical orientation	5.77%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 2a	13.1-ft end drop, with 4-degree off-vertical orientation, and approximated slapdown	4.35%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 2b	Approximated slapdown from vertical orientation	1.25%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 3	6-ft side drop, with 3-degree off-horizontal orientation	2.07%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 4	10-ft drop of 10-metric-ton load onto top of cask	0.96%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 5a	30-ft end drop, with vertical orientation	3.55%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 5b	30-ft end drop, with 4-degree off-vertical orientation	5.77%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 5c	30-ft end drop, with 45-degree off-vertical orientation	6.41%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 5d	30-ft end drop, with center of gravity over corner (i.e., point of impact)	6.63%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$

NOTE: [a]"T" stands for transportation cask. "IC" stands for impact condition, which are defined in Table D1.2-1.
[b]Values of Max EPS and failure probability are applicable to the SNF canister.

Source: Ref. D4.1.27, Table 6.3.7.6-2

Table D1.2-5. Failure Probabilities with and without Triaxiality Factor, with and without the Fragility Curve Adjustment, for the Transportation Cask

Container Type/ Impact Condition[a]	Impact Condition Description	Max EPS[b]	Failure Probability	
			CDF Fragility Curve Adjusted for Minimum Elongation (-8.3% Shift)	
			w/o Triaxiality	with Triaxiality
T.IC 1a	12-ft end drop, with 4-degree off-vertical orientation	9.20%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 1b	13.1-ft end drop, with 4-degree off-vertical orientation	9.37%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 1c	30-ft end drop, with 4-degree off-vertical orientation	11.25%	$<1 \times 10^{-8}$	9×10^{-7}
T.IC 2a	13.1-ft end drop, with 4-degree off-vertical orientation, and approximated slapdown	9.94%	$<1 \times 10^{-8}$	3×10^{-8}
T.IC 2b	Approximated slapdown from vertical orientation	5.30%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 3	6-ft side drop, with 3-degree off-horizontal orientation	7.42%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 4	10-ft drop of 10-metric-ton load onto top of cask	1.76%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 5a	30-ft end drop, with vertical orientation	3.17%	$<1 \times 10^{-8}$	$<1 \times 10^{-8}$
T.IC 5b	30-ft end drop, with 4-degree off-vertical orientation	11.25%	$<1 \times 10^{-8}$	9×10^{-7}
T.IC 5c	30-ft end drop, with 45-degree off-vertical orientation	70.56%	1	1
T.IC 5d	30-ft end drop, with center of gravity over corner (i.e., point of impact)	44.88%	0.9	1

NOTE: [a]"T" stands for transportation cask. "IC" stands for impact condition, which are defined in Table D1.2-1.
[b]Values of Max EPS and failure probability are applicable to the structural body of the transportation cask, which excludes the shield and shield shell.

Source: Probabilities calculated using Table D1.1-1 based on strains reported in *Seismic and Structural Container Analyses for the PCSA* (Ref. D4.1.27, Table 6.3.7.6-2)

Table D1.2-6. Strains at Various Canister Locations Due to Drops

Canister	Component	Maximum PEEQ Strains (%) Outside Surface	Mid-Surface	Inside Surface	Load Case/ Conditions
18-inch DOE STD canister	Lower head	8	3	6	300°F, 23-foot drop, 3 degrees off-vertical Material: ASME Code minimum strengths
	Lower head-to-main shell weld	2	2	3	
	Main shell	2	2	3	
	Upper head-to-main shell weld	0	0	0	
	Upper head	1	0.2	2	
24-inch DOE STD canister	Lower head	2	0.7	1	300°F, 23-foot drop, 3 degrees off-vertical Material: ASME Code minimum strengths
	Lower head-to-main shell weld	0.2	0.3	0.5	
	Main shell	0.2	0.3	0.5	
	Upper head-to-main shell weld	0	0	0	
	Upper Head	0	0	0	
MCO	Lower head	35	16	14	70°F, 23-foot drop, 3 degrees off-vertical Material: Actual material properties (significantly higher than ASME Code minimums)
	Lower head-to-main shell weld	21	11	11	
	Main shell	13	15	29	
	Upper head-to-main shell weld	0	0	0	
	Upper head	0	0	0	

NOTE: ASME = The American Society of Mechanical Engineers; DOE STD = U.S. Department of Energy standard; MCO = multicanister overpack; PEEQ = peak equivalent.

Source: Ref. D4.1.64, Tables 13, 14, and 16

Table D1.2-7. Failure Probabilities for the DOE Spent Nuclear Fuel (DSNF) Canisters and Multicanister Overpack (MCO)

Component	Peak Equivalent Plastic Strain (%)			Probability of Failure					
				Original CDF			CDF adjusted to min elongation		
	Outside Surface	Middle	Inside Surface	Outside Surface	Middle	Inside Surface	Outside Surface	Middle	Inside Surface
18-inch standard canister containment PEEQ strains, 3 degrees off vertical drop, 300°F									
Lower Head	8	3	6	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Lower Head-to-Main Shell Weld	2	2	3	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Main Shell	2	2	3	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Upper Head-to-Main Shell Weld	0	0	0	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Upper Head	1	0.2	2	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
24-inch standard canister containment PEEQ strains, 3 degrees off vertical drop, 300°F									
Lower Head	2	0.7	1	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Lower Head-to-Main Shell Weld	0.2	0.3	0.5	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Main Shell	0.2	0.3	0.5	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Upper Head-to-Main Shell Weld	0	0	0	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Upper Head	0	0	0	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
4 MCO containment PEEQ strains, 3 degrees off vertical drop, 70°F									
Bottom	35	16	14	3.74E-03	<1E-08	<1E-08	8.99E-02	<1E-08	<1E-08
Bottom-to-Main Shell	21	11	11	<1E-08	<1E-08	<1E-08	1.16E-07	<1E-08	<1E-08
Main Shell	13	15	29	<1E-08	<1E-08	1.09E-06	<1E-08	<1E-08	3.85E-03
Collar	0	0	0	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08
Cover	0	0	0	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08	<1E-08

NOTE: ASME = The American Society of Mechanical Engineers; CDF = cumulative distribution function; DOE STD = U.S. Department of Energy standard; MCO = multicanister overpack; PEEQ = peak equivalent.

Source: Ref. D4.1.27, Tables 6.3.7.6-4 and 6.3.7.6-5

D1.3 PROBABILITIES OF FAILURE OF HIGH LEVEL WASTE CANISTERS DUE TO DROPS

The probability of failure for drops of high-level radioactive waste (HLW) canisters was assessed by evaluating actual drop test data. Several series of tests were conducted including vertical, top, and corner drops of steel containers. The reports on these tests are summarized in *Leak Path Factors for Radionuclide Releases from Breached Confinement Barriers and*

Confinement Areas (Ref. D4.1.17). No leaks were found after 27 tests, 14 of which were from 23 feet and 13 of which were from 30 feet. These tests can be interpreted as a series of Bernouilli trials, for which the outcome is the breach, or not, of the tested canister. The observation of zero failures in 13 tests was interpreted using a beta-binomial conjugate distribution Bayes analysis.

A uniform prior distribution, which indicates prior knowledge that the probability of failure is between 0 and 1, may be represented as a Beta(r,s) distribution in which both r and s equals 1. The conjugate pair likelihood function for a Beta(r,s) distribution is a Binomial(n, N) where n represents the number of failures within the tests and N represents the number of tests. The posterior distribution resulting from the conjugate pairing is also a Beta distribution with parameters r' and s', which are defined as follows:

$$r' = r + n \quad \text{and} \quad s' = s + N - n \qquad \text{(Eq. D-1)}$$

The mean, μ, and standard deviation, σ, of the posterior distribution are determined using the following equations:

$$\mu = r' / (r' + s') \quad \text{and} \quad \sigma = \{r's' / [(r' + s' + 1)(r' + s')^2]\}^{1/2} \qquad \text{(Eq. D-2)}$$

For n = 0 and N = 13, Equation D-2 results in $\mu = 0.067$ and $\sigma = 0.062$. For n = 0 and N = 27, $\mu = 0.034$ and $\sigma = 0.033$. These values are used for the failure probability of a dropped HLW canister, for example during its transfer by a canister transfer machine.

One element of the Nuclear Safety Design Basis (Section 6.9) requires that the transportation cask, which will deliver HLW and DOE standardized canisters, be designed to preclude contact between the canister and a transportation cask lid or other heavy object that might fall. Similarly, other large heavy objects are precluded from damaging these canisters, when residing within a co-disposal waste package by the design of the waste package, which includes separator plates that extend well above the canisters. These scenarios are not quantitatively analyzed herein.

The combined INL and LLNL analyses discussed previously conclude that a DOE SNF canister has a probability of breach less than 1E-08 for a 23 foot drop, 4 degrees off-normal (i.e., 4 degrees from vertical) onto an unyielding rigid surface. The LLNL results demonstrate that generally strains from impact and probability of failure is higher for off-normal drops than normal (i.e., vertical) drops for the same height. The LLNL results further show that a 10 ton load dropped from 10 feet onto a representative canister also results in a probability of breach of less than 1E-08. INL analysis EDR-NSNF-087 entitled Qualitative Analysis of the Standardized DOE SNF Canister for Specific Canister-on-Canister Drop Events at the Repository states that canister integrity was maintained for a 30 foot drop test onto a rigid, unyielding surface. The report discusses drop of a HLW canister on a DOE SNF canister and drop of a DOE SNF canister onto another one. Drops of these canisters onto canisters in the IHF or CRCF would occur with drop heights of less than 10 feet. Two main differences are noted between a drop of a DOE SNF and a drop of a HLW canister onto a DOE SNF. The first is that substantially lower kinetic energy of impact of the latter drop would result in significantly less skirt deformation. The non-flat bottom nature of the HLW/DOE SNF interaction would have a different skirt deformation pattern that the flat bottomed drop. INL concludes that the skirt would be expected to absorb the bulk of the heaviest HLW canister (4.6 tons) drop energy and DOE SNF canister integrity would be maintained. A difference between a 10 ton drop of a load onto a representative canister and a drop onto a DOE SNF canister results from the difference diameters of the target as well as different materials and lid thicknesses. Nevertheless, INL concludes that the impact from 10 feet of a HLW canister onto a DOE SNF canister is less challenging than impact from a 30 foot drop. Since the probability from a 23 foot drop was calculated to be less than 1E-08, it is conservative to use a value of 1E-05 for the probability of failure of an HLW on DOE SNF impact. The increased value is assigned to account for uncertainties owing to the differences noted above.

D1.4 PROBABILITIES OF FAILURE OF WASTE PACKAGES DUE TO DROPS AND IMPACTS

The probabilities of containment failure are evaluated by comparing the challenge load with the capacity of the waste package to withstand that challenge in a manner similar to that described in *Interim Staff Guidance HLWRS-ISG-02, Preclosure Safety Analysis - Level of Information and Reliability Estimation.* HLWRS-ISG-02 (Ref. D4.1.56), and summarized in Section 4.3.2.2. Three scenarios are evaluated for the potential loss of containment by waste packages due to drops and impacts:

- Two-foot horizontal drop
- 3.4-mph end-to-end impact
- Rockfall on waste package in subsurface tunnels.

An additional scenario, drop of a waste package shield ring onto a waste package, is considered in Section D1.4.4.

For this assessment, the potential load has been determined by FEA in the calculations cited below as the sources of inputs. The load is expressed in terms of stress intensities and as expended toughness fraction (ETF), which is the ratio of the stress intensity to the true tensile strength. The ETF is used to obtain the failure probability by the following:

$$P = \int_{-\infty}^{x} N(t)\,dt \quad and \quad x = \frac{ETF - 1}{COV} \qquad \text{(Eq. D-3)}$$

where

P = probability of failure

$N(t)$ = standard normal distribution with mean of zero and standard deviation of one

t = variable of integration

ETF = expended toughness fraction

COV = coefficient of variation = ratio of standard deviation to mean for strain capacity distribution, applied here to stress capacity or true tensile strength

The capacity is the true tensile strength of the material, the stress the material can withstand before it separates. The minimum true tensile strength, σ_u, for the Alloy 22 typically used for the outer corrosion barrier (OCB) of the waste package is 971 MPa (Ref. D4.1.20, Section 7.7, p. 162). The variability in the capacity is expressed as the standard deviation of a normal distribution that includes strength variation data and variability of the toughness index, I_T, computed without triaxialty adjustments (uniaxial test data). The standard deviation as percent of the mean of σ_u is 25% (Ref. D4.1.20, Section 7.6, p. 162). The distribution of elongations used for defining the fragility curve in the LLNL analysis was expressed as two normal distributions, the larger of which was with a mean of 59.3% elongation and a standard distribution of 4.22% elongation, or a COV of 0.0712 (Ref. D4.1.27, Section 6.3.7.3). Thus the 0.073 reported for the OCB material is conservative compared with the LLNL data and is used for the COV in the expression above. The possibility of waste package weld defects is not explicitly considered in the analysis. However, as noted in Section D.1.4.5, weld defects are not expected to contribute significantly to the probability of waste package failure due to drops or other impacts.

D1.4.1 Waste Package Drop

A study investigating the structural response of the naval long waste package to a drop while it is being carried on the emplacement pallet, found the ETF for the outer corrosion barrier (OCB) to be 0.29 for a 10 m/s flat impact (Ref. D4.1.20, Table 7-15, pg. 117), equivalent to a 16.7-foot drop. This corresponds to a failure probability of less than 1×10^{-8}. The failure of the OCB is used to define the loss of containment, taking no credit for the inner vessel and the canister within. The description of the transport and emplacement vehicle (TEV) provided in *Mechanical Handling Design Report: Waste Package Transport and Emplacement Vehicle* (Ref. D4.1.12) mentions that the floor plate is lifted by four jacks and guided by a roller. The guide roller precludes tilted drops of the flat bed of the TEV. As was done for the results from LLNL, to introduce an additional measure of conservatism, a failure probability of 1×10^{-5} is used for the probability that the waste package containment would fail due to a two-foot horizontal drop, which is much less severe than the modeled 16.7-foot drop.

D1.4.2 Rockfall onto a Waste Package

A seismic event during the preclosure period could cause rocks to fall from the ceiling of a drift onto the waste packages stored there prior to deployment of the drip shields. The extent of damage has been predicted for several levels of impact energy of falling rocks (Ref. D4.1.26). The maximum credible impact energy from a falling rock is about 1×10^6 joules (J) (Ref. D4.1.21, p. 57). The maximum ETF resulting from rockfall impacting with approximately 1×10^6 J is about 0.11 (Ref. D4.1.26, p. 54, Table 5), corresponding to a failure probability less than 1×10^{-8}. As was done for the results from LLNL, to introduce an additional measure of conservatism, a failure probability of 1×10^{-5} should be used for the probability that the waste package containment would fail due to rockfall on the waste package.

D1.4.3 Results for the Three Assessed Scenarios

The failure probabilities for the three scenarios, derived from the results in the cited reports, are summarized in Table D1.4-1.

Table D1.4-1. Waste Package Probabilities of Failure for Various Drop and Impact Events

Event	Probability of Failure
2-Foot Horizontal Drop	$< 1 \times 10^{-5}$
3.4-mph end-to-end impact	$< 1 \times 10^{-5}$
20 metric ton Rockfall on Waste Package with and without Rock Bolt[a] Impacting the Waste Package	$< 1 \times 10^{-5}$

NOTE: [a]A rock bolt is a long anchor bolt, for stabilizing rock excavations, which may be tunnels or rock cuts.

Source: Original.

D1.4.4 Drop of a Waste Package Shield Ring onto a Waste Package

After the co-disposal waste package has been welded closed in the Waste Package Positioning Room, the shield ring is lifted from it before the waste package transfer trolley is moved into the load out area. Grapple failures might cause the drop to occur at a variety of orientations relative to the top of the waste package. A frequency of canister breach from a potential drop as high as 10 feet is considered here. For a canister breach to occur, the shield ring must penetrate the 1-inch thick outer lid made of SB 575 (Alloy 22) and the 9 inch thick stainless steel inner lid (SA 240) before having an opportunity to impact the canister (Ref. D4.1.13). There are six inches separating the inner and outer lids. In the radial center area of that space, which would be directly above the DOE SNF canister, is a stainless steel lifting device attached to the inner lid. This adds another layer of energy absorption.

The shield ring weighs approximately 15 tons and is made of stainless steel with a lighter weight neutron absorber material. The impact energy of a 15-ton shield ring dropping 10 feet would be 0.4 MJ. The frequency of penetration of the sides of a waste package from a 20 metric ton rock impacting the side of the waste package with impact energy of 1 MJ is less than 1×10^{-8} (Table D1.4-1). The sides of a waste package are approximately three inches thick compared to a cumulative thickness (excluding lifting fixture) of 10 inches at the top. Although the impact energy could be more focused, the impact energy for the shield ring against the top of the waste package is less than the impact energy of the rockfall against the side and the top is much thicker than the side. The probability of failure due to shield ring impact against the top of the waste package is expected to be no worse than for the impact of a rock against the side. A conservative value of 1×10^{-5} is used in the analysis for this probability.

D1.4.5 Waste Package Weld Defects

Waste package closure involves engaging and welding the inner lid spread ring, inerting the waste package with helium, setting and welding the outer lid to the outer corrosion barrier, performing leak testing on the inner vessel closure, performing nondestructive examination of welds, and conducting postweld stress mitigation on the outer lid closure weld.

The weld process of the waste package closure subsystem is controlled as a special process by the Quality Assurance Program (Ref. D4.1.29, Section 9.0). The activities performed by the system are controlled by approved procedures.

The principal components of the system include welding equipment; nondestructive examination equipment for visual, eddy current, and ultrasonic inspections of the welds and leak detection; stress mitigation equipment for treatment of the outer lid weld; inerting equipment; and associated robotic arms. Other equipment includes the spread ring expander tool, leak detection tools, cameras, and the remote handling system. The system performs its functions through remote operation of the system components.

The capability of the waste package closure subsystem will be confirmed by demonstration testing of a full-scale prototype system. The prototype includes welding, nondestructive examinations, inerting, stress mitigation, material handling, and process controls subsystems. The objective of the waste package closure subsystem prototype program is to design, develop, and construct the complete system required to successfully close the waste package. An iterative process of revising and modifying the waste package closure subsystem prototype will be part of the design process. When prototype construction is finalized, a demonstration test of the closure operations will be performed on only the closure end of the waste package; thus, the mock-up will be full diameter but not full height as compared to the waste package. The purpose of the demonstration test is to verify that the individual subsystems and integrated system function in accordance with the design requirements and to establish closure operations procedures. This program is coordinated with the waste package prototype fabrication program.

The principal functions of the waste package closure subsystem are to:

- Perform a seal weld between the spread ring and the inner lid, the spread ring and the inner vessel, and the spread ring ends; perform a seal weld between the purge port cap and the inner lid; and perform a narrow groove weld between the outer lid and the outer corrosion barrier.

- Perform nondestructive examination of the welds to verify the integrity of the welds and repair any minor weld defects found.

- Purge and fill the waste package inner vessel with helium gas to inert the environment.

- Perform a leak detection test of the inner lid seals to ensure the integrity of the helium environment in the inner vessel.

- Perform stress mitigation of the outer lid groove closure weld to induce compressive residual stresses.

The gas tungsten arc welding process is used for waste package closure welds and weld repairs. Welding is performed in accordance with procedures qualified to the *2001 ASME Boiler and Pressure Vessel Code* (Ref. D4.1.5, Section IX), as noted below:

- The spread ring and purge port cap welds are two-pass seal welds.
- The outer lid weld is a multipass full-thickness groove weld.

Welding process procedures will be developed that identify the required welding parameters. The process procedures will:

- Identify the parameters necessary to consistently achieve acceptable welds.
- State the control method for each weld parameter and the acceptable range of values.

The welds are inspected in accordance with examination procedures developed using *2001 ASME Boiler and Pressure Vessel Code* (Ref. D4.1.5, Section V and Section III, Division 1, Subsection NC) as a guide, with modification as appropriate:

- Seal welds—visual inspection
- Groove welds—visual, eddy current, and ultrasonic inspection.

A weld dressing end effector is used for weld repairs. The defect is removed, resulting in an excavated cavity of a predetermined contour. The excavated cavity surface is inspected using the eddy current inspection end effectors. Then the cavity is welded and inspected in accordance with the welding and inspection procedures.

The stress mitigation process for the outer lid closure weld is controlled plasticity burnishing. Controlled plasticity burnishing is a patented method of controlled burnishing to develop specifically tailored compressive residual stress with associated controlled amounts of cold work at the outer surface of the waste package outer lid closure weld.

The inner vessel of the waste package is evacuated and backfilled with helium through a purge port on the inner lid. The inerting process is in accordance with the inerting process described in NUREG-1536 (Ref. D4.1.54, Sections 8.0 and V.1). After the waste package inner vessel is backfilled by helium, both the spread ring welds and the purge port plug are leak tested in accordance with *2001 ASME Boiler and Pressure Vessel Code* (Ref. D4.1.5, Section V, Article 10, Appendix IX) to verify that no leakage can be detected that exceeds the rate of 10^{-6} std cm^3/s.

Waste package closure welding, nondestructive examination, stress mitigation, and inerting are conducted in accordance with approved administrative controls. The processes for waste package closure welding, nondestructive examination, stress mitigation, and inerting will be developed in accordance with the codes and standards identified below. The processes are monitored by qualified operators, and resulting process data are checked and verified as acceptable by qualified individuals.

Waste package closure welding, nondestructive examination, stress mitigation, and inerting normal operating procedures will specify, for example, the welding procedure specification, nondestructive examination procedure, qualification and proficiency requirements for operators and inspectors, and acceptance and independent verification records for critical process steps.

The waste package closure subsystem–related welds, weld repairs, and inspections are performed in accordance with *2001 ASME Boiler and Pressure Vessel Code* (Ref. D4.1.5, Section II, Part C; Section III, Division I, Subsection NC; Section IX; Section V).

The inerting of the waste package is performed in accordance with the applicable sections of NUREG-1536 (Ref. D4.1.54).

PCSA event sequences involving waste packages include challenges ranging from low velocity collisions to a 20 metric ton rockfall to a spectrum of fires. Waste package failure probabilities are calculated to be very low. Furthermore, a significant conservatism in the analysis is that the containment associated with the canister is not included in the probability of containment breach. In other words, if the waste package breaches, radionuclide release is analyzed as if the canister has breached (if the event sequence is in Category 1 or 2). Analytically, the canister is not relied upon for event sequences involving waste packages. The analytical results from the LLNL analysis show a significant reduction in canister strains is achieved by transportation cask and aging overpack protection. Although not analyzed, a similar ameliorating effect on the canister would be expected to be provided by the waste package.

The weld, inspection and repair process ensures no significant defects to a high reliability. The event sequence analysis shows that all event sequences associated with waste package breach are Beyond Category 2. In the context of the event sequence analysis, a significant defect is one that would have increased the probability of breach of the canister within the waste package by orders of magnitude. Even for significant weld defects, the protection offered by the waste package to the canister containment function would remain. Therefore, the effect of waste package weld failure on loss of canister containment during event sequences is not further considered.

D1.4.6 Waste Package End-to-End Impact

An oblique impact of a long naval SNF waste package inside TEV) was modeled to assess the structural response (Ref. D4.1.19). Most of the runs were with initial impact velocity of 3.859 m/s corresponding to a drop height of 0.759 m (2.49 ft). The maximum ETF for the 3.859 m/s (12.66 ft/sec) oblique impact in the OCB is about 0.7 (Ref. D4.1.19, page 37, Table 7-3, runs 1, 2, and 3), corresponding to a failure probability of about 2×10^{-5}. The oblique impact should be bounding for a direct end impact Using equation D-4, an ETF of 0.11 is estimated for the hypothesized 3.4 mph end-to-end collision (two TEVs each traveling 1.7 mph), corresponding to a failure probability of less than 1×10^{-8}. The failure of the OCB is used to define the loss of containment, taking no credit for the inner vessel and the canister within. As was done for the results from LLNL, to introduce an additional measure of conservatism, a failure probability of 1×10^{-5} is used for the probability that the waste package containment would fail due to a 3.4-mph end-to-end impact.

D1.5 PREDICTING OUTCOMES OF OTHER SITUATIONS BY EXTRAPOLATING STRAINS FOR MODELED SCENARIOS

Equation 17 in Section 6.3.2.2 demonstrates use of the probability of failure at a given drop height together with the COV to predict probabilities at other drop heights. A similar approach can be used to extrapolate from one strain to another to find the corresponding failure probability. The work done on damaging the container expressed in the form of strain should be roughly proportional to the energy input to the material due to the impact. The impact energy is proportional to the drop height or to the square of the impact velocity. Finite element modeling demonstrated that the increase in strain is actually less than proportional to increase in drop height (Tables D1.2-3 and D1.2-4), so increasing the strain proportionally with drop height or the square of impact velocity is conservative. The strain is extrapolated by multiplying it by the square of the ratio of the velocity of interest to the reference velocity.

$$\tau_i = \tau_{ref}\left(\frac{v_i}{v_{ref}}\right)^2 \qquad \text{(Eq. D-4)}$$

where

τ_i = strain at velocity of interest (dimensionless)

τ_{ref} = strain at reference velocity (dimensionless)

v_i = velocity of interest (same units as v_{ref})

v_{ref} = reference velocity (same units as v_i)

In case D.IC.3, a 0.16% strain (τ_{ref}) was predicted for a side impact of 40 ft/min (v_{ref}). Using Equation D-4 to extrapolate for an impact velocity of 2.5 miles/hr gives an estimated strain of 4.84%.

The estimated strain is then compared with the fragility curve tabulated in D1.1-1. A failure rate of less than 1×10^{-8} is predicted for a strain of 4.84%. Probabilities of failure for a range of impact velocities are listed in Table D1.5-1.

Table D1.5-1. Calculated Strains and Failure Probabilities for Given Side Impact Velocities

Impact Velocity			
(ft/sec)	(ft/min)	% strain	Probability of failure
0.67	40	0.16	$< 1\times 10^{-8}$
1	60	0.36	$< 1\times 10^{-8}$
2	120	1.44	$< 1\times 10^{-8}$
4	240	5.76	$< 1\times 10^{-8}$
6	360	13	$< 1\times 10^{-8}$
8	480	23	$< 1\times 10^{-5}$

Source: Original

A similar approach is applied to estimate failure probabilities for vertical drops greater than 40 feet. The strains are extrapolated using the ratio of drop heights rather than the squared ratio of impact velocities in Equation D-4.

For the DPC, the maximum EPS is 2.65% for a 40-foot end drop (case D.IC.1b in Table D1.2-3). Strains of 2.98% and 3.31% are estimated for 45- and 50-foot drops, respectively. Doubling the strains to account for triaxiality and comparing these strains with Table D1.1-1 shows the probabilities of failure are both < 1×10^{-8}. As before, conservative probabilities of 1×10^{-5} are used in the event sequence quantification.

For the DOE standard canister the maximum strain is 8% in the lower head of the 18-inch canister resulting from a 23-foot drop 3 degrees off vertical (Table D1.2-6). By the same approach as above, 10.4%, 15.7%, and 17.4% strains are estimated for 30-foot, 45-foot, and 50-foot drops. Doubling these strains and comparing with Table D1.1-1 yields the failure probabilities of 1×10^{-7}, 3×10^{-2}, and 9×10^{-2} for the 30-foot, 45-foot, and 50-foot drops, respectively. A conservative probability of 1×10^{-5} is used for the 30-foot drop of the DOE standardized canister.

D1.6 MISCELLANEOUS SCENARIOS

D1.6.1 Localized Side Impact on a Transportation Cask

One of the requirements specified for transportation casks is they be robust enough to survive a 40-inch horizontal drop onto an unyielding 6-inch diameter upright cylinder (Ref. D4.2.2, Paragraph 71.73). The impact energy for such a scenario involving a 250,000 pound cask (a typical weight for a loaded cask) – the NAC STC has a loaded weight of 260,000 pounds (Ref. D4.1.50, p. 1.1-1) is about 1.1 MJ. The maximum weight of a forklift is considerably less than 20,000 kg. At a maximum speed of 2.5 mph (1.12 m/s), the maximum impact energy would be 12.5 kJ, a factor of 90 less than the impact energy for the 40-inch drop of the cask. If the resultant strain is proportional to the impact energy and the drop event in the Safety Analysis Report (SAR) is just below the failure threshold (i.e. the median impact energy for failure), the impact energy due to the 2.5-mph impact would be a maximum of 1/90th of the median failure impact energy, or 1 – 1/90 COVs less than a normalized median of 1. Equation D-3 is applicable substituting the ratio of impact energy to median failure impact energy for the factor ETF. Using 1/90 (=0.011) in place of the ETF in Equation D-3 gives a probability of failure of much less than 1×10^{-8} due to impact of a forklift against a transportation cask. If the impact speed were 9 mph instead of 2.5 mph, the impact energy would be about 1/7th of the energy in the SAR drop event, 0.14 would be used in place of the ETF in Equation D-3, and the probability of failure would still be less than 1×10^{-8}.

D1.6.2 Screening Argument for TAD Weld Defects

TAD canister closure is the process that closes the loaded TAD canister by welding the shield plug and fully draining and drying the TAD canister interior, followed by backfilling the TAD canister with helium and fully welding the TAD canister lid around its circumference onto the body of the TAD canister.

The process control program for the closure welds produced by the TAD canister closure system is controlled as a special process by the Quality Assurance Program (Ref. D4.1.29, Section 9.0).

TAD canister closure is done at the TAD canister closure station in the cask preparation area. The shielded transfer cask containing a loaded TAD canister is transferred from the pool to the TAD canister closure station using the cask handling crane. The shielded transfer cask lid is unbolted and then removed using the TAD canister closure jib crane. The TAD canister is then partially drained via the siphon port in order to lower the water level below the shield plug in preparation for welding. The TAD canister welding machine is positioned onto the TAD canister shield plug using the TAD canister closure jib crane, and the shield plug is welded in place. After a weld is completed, visual examination of the weld is performed in addition to the eddy current testing and ultrasonic testing that are performed by the TAD canister welding machine.

A draining, drying, and inerting system is connected to the siphon and vent ports in the shield plug and used to dry the interior of the TAD canister, followed by backfilling it with helium gas. Port covers are then placed over the siphon and vent ports and welded in place using the TAD canister welding machine. The TAD canister welding machine is removed, and the outer lid is placed onto the TAD canister using the TAD canister closure jib crane. The TAD canister welding machine is positioned onto the TAD canister outer lid, and the lid is welded in place. The TAD canister welding machine is removed, and the shielded transfer cask lid is placed onto the shielded transfer cask using the TAD canister closure jib crane and installed. Hoses are connected to the fill and drain ports on the shielded transfer cask, and the water is sampled for contamination. If the water is clean, the ports are opened to drain the annulus between the TAD canister and the shielded transfer cask. If the water is contaminated, then the annulus is flushed with treated borated water as needed. A drying system is then used to dry the annulus. The potential for contamination is kept to a minimum by the use of the inflatable seal.

The qualification of the TAD canister final closure welds is in accordance with ISG-18 (Ref. D4.1.55) as specified in *Basis of Design for the TAD Canister-Based Repository Design Concept* (Ref. D4.1.15, Section 33.2.2.36). Adherence to this guidance is deemed to provide reasonable assurance that weld defects occur at a low rate. However, TAD canister weld cracks are considered an initiating event after the TAD canister welding process in the Wet Handling Facility (WHF). If this occurs, the radionuclide release would be minimal because the incoming casks and canisters have already been opened. After TAD canisters are welded, they are placed in aging overpacks and moved by the site transporter to the Canister Receipt and Closure Facility (CRCF). The probability of TAD canister failure during removal from the aging overpack handling in the CRCF and placement into a waste package is considered in the CRCF event sequence analysis. The conditional probability of TAD canister failures during handling in the CRCF has been shown to be small. The low probability of weld defects and their size would not alter this result. After the TAD canister is placed in the waste package, the containment is considered to be the waste package and the TAD canister is no longer relied upon in event sequences involving mechanical impacts.

D2 PASSIVE FAILURE DUE TO FIRE

A risk assessment must consider a range of fires that can occur, as well as variations in the dynamics of the heat transfer and uncertainties in the failure temperature of the target. This section presents an analysis to determine the probability that a waste container will lose containment integrity or lose shielding in a fire. Section D2.1 addresses loss of containment and Section D2.2 addresses loss of shielding.

APPENDIX E.IV
SELECTION OF METHODS FOR DETAILED QUANTIFICATION

There are a number of methods available for the detailed quantification of HFEs (preliminary quantification is discussed in Appendix E.III of this analysis). Some are more suited for use for the YMP PCSA than others. A number of methods were considered, but many were rejected as inapplicable or insufficient for use in quantification. Several sources were examined as part of the background analysis for selecting a method for detailed quantification (i.e., Ref. E8.1.17; Ref. E8.1.13; Ref. E8.1.24; Ref. E8.1.21). As discussed in Section E3.2 the following four were chosen:

- ATHEANA expert judgment (Ref. E8.1.22).
- CREAM (Ref. E8.1.18)
- HEART (Ref. E8.1.28)/NARA (Ref. E8.1.11)
- THERP (Ref. E8.1.26)

This appendix discusses the selection process.

Basis for Selection—The selection process was conducted with due consideration of the HRA quantification requirements set forth in the ASME Level 1 PRA standard (Ref. E8.1.4) to the extent that those requirements, which were written for application to NPP PRA, apply to the types of operations conducted at the YMP. Certainly, all of the high level HRA quantification requirements were considered to be applicable. Further, all of the supporting requirements to these high level requirements were considered applicable, at least in regards to their intent. In some cases, the specifics of the supporting requirements are only applicable to NPP HRA and some judgment is needed on how to apply them. This was particularly true of those supporting requirements that judged certain specific quantification methods acceptable. This appendix lays out the specific case for the methods selected for use at the YMP (or, more to the point, the exclusion of certain methods that would normally be considered acceptable under the standard, but are deemed inappropriate for use for the YMP PCSA).

Differences between NPP and the YMP Relevant to HRA Quantification—There are a number of contrasts between the operations at the YMP and the operations at a NPP that affect the selection of approaches to performing detailed HRA quantification (Table E.IV-1).

Table E.IV-1. Comparison between NPP and YMP Operations

NPP	YMP
Central control of operations maintained in control room.	Decentralized (local), hands on control for most operations.
Most important human actions are in response to accidents.	Most important human actions are initiating events.
Post-accident response is important and occurs in minutes to hours. Short time response important to model in HRA.	Post-accident response evolves more slowly (hours to days). Short time response not important to model.

Table E.IV-1. Comparison between NPP and YMP Operations (Continued)

NPP	YMP
Multiple standby systems are susceptible to pre-initiator failures.	Standby systems do not play major role in the YMP safeguards, therefore few opportunities for pre-initiator failures.
Auxiliary operators sent by central control room operators to where needed in the plant.	Local control reduces time to respond.
Most actions are controlled by automatic systems.	Most actions are controlled by operators.
Reliance on instrumentation /gauges as operators' "eyes".	Most actions are local, either hands on or televised. Less reliance on man–machine interface.
High complexity of systems, interactions, and phenomena. Actions may be skill, rule, or knowledge based.	Relatively simple process with simple actions. Actions are largely skill based.
Many in operation for decades; HRA may include walk-downs and consultation with operators.	First of a kind; HRA performed for construction application, therefore walk-downs and consultation with operators not feasible.

NOTE: HRA = human reliability analysis; NPP = nuclear power plant; YMP = Yucca Mountain Project.

Source: Original

Assessment of Available Methods—There are essentially four general types of quantification approaches available:

1. Procedure focused methods:

   A. Basis: These methods concentrate on failures that occur during step-by-step tasks (i.e., during the use of written procedures). They are generally based on observations of human performance in the completion of manipulations without much consideration of the root causes or motivations for the performance (e.g., how often does an operator turn a switch to the left instead of to the right).

   B. Methods considered: THERP (Ref. E8.1.26).

   C. Applicability: This method is of limited use for the YMP because important actions are not procedure driven. Many operations are skill-based and/or semi-automated (e.g., crane operation, trolley operation, CTM operation, TEV operation). However, there are some instances where such an approach would be applicable to certain unsafe actions within an HFE. In addition, the THERP dependency model is adopted by NARA as being appropriate to use within a context-based quantification approach.

   D. Assessment: THERP is retained as an option in the detailed quantification for its dependency model and for limited use when simple, procedure-driven unsafe actions are present within an HFE.

2. Time-response focused methods:

   A. Basis: These methods focus on the time available to perform a task, versus the time required, as the most dominant factor in the probability of failure. They are, for the most part, based on NPP control room observations, studies, and simulator exercises. They also tend to be correlated with short duration simulator exercises (i.e., where there is a clear time pressure in the range of a few minutes to an hour to complete a task in response to a given situation).

B. As discussed in *Human Reliability Analysis: A Systems Engineering Approach with Nuclear Power Plant Applications* (Ref. E8.1.13), examples of time-response methods include: HCR (Ref. E8.1.13) and TRCs (Ref. E8.1.15).

C. Applicability: These methods are not applicable to the YMP because most actions do not occur in a control room and, in addition, are generally not subject to time pressure. This is particularly true of the most important HFEs, those that are human-induced initiators. Other than a desire to complete an action in a timely fashion to maintain production schedules, time is irrelevant to these actions, especially in the context of the type of time pressure considered by these methods. Even those actions at the YMP that may take place in a control room in response to an event sequence and have time as a factor would only require response in the range of hours or days, which is outside the credible range for these methods.

D. Assessment: No use can be identified for these methods within the YMP PCSA. None of them are retained.

3. Context and/or cognition driven methods:

A. Basis: These methods focus on the context and motivations behind human performance rather than the specifics of the actions, and as such are independent of the specific facility and process. To the extent that some of the methods are data-driven (i.e., they collect and use observations of human performance) the data utilized is categorized by GTT rather than by the type of facility or equipment where the human failure occurred. This makes them more broadly applicable to various industries, tasks, and situations, in large part because they allow context-specific PSFs to be considered. This allows for them to support a variety of contexts, individual performance factors (e.g., via PSFs) and human factor approaches.

B. Methods considered: HEART (Ref. E8.1.28; Ref. E8.1.29)/NARA (Ref. E8.1.11), CREAM (Ref. E8.1.18), and ATHEANA expert judgment (Ref. E8.1.22).

C. Applicability: The broad applicability of these methods and their flexibility of application make them most suited for application at the YMP. The use of information from a broad range of facilities and other performance regimes (e.g., driving, flying) support their use as facility-independent methods. The generic tasks considered can be applied to the types of actions of most concern to the YMP (i.e., human-induced initiators) as opposed to the more narrow definitions used in other approaches that make it difficult to use them for other than post-initiator or pre-initiator actions.

D. Assessment: Optimally it would be convenient to use only one of the three methods of this type for all the detailed quantification. However, HEART (Ref. E8.1.28)/NARA (Ref. E8.1.11) and CREAM (Ref. E8.1.18) approach their GTTs slightly differently and also use different PSFs and adjustment factors. There are unsafe actions within the YMP HFEs that would best fit the HEART (Ref. E8.1.28)/NARA (Ref. E8.1.11) approach and others that would best fit the CREAM (Ref. E8.1.18) approach. In addition, the union of the two approaches still has some gaps that would not cover a small subset of unsafe actions for the YMP (primarily in the area of unusual acts of commission). One gap relates to dependencies between actions, but in this case NARA (Ref. E8.1.11) specifically endorses the THERP (Ref. E8.1.26) approach and so this is used. However, other gaps exist. For these cases, the ATHEANA (Ref. E8.1.22) expert judgment approach provides a viable and structured framework for the use of judgment to establish the appropriate HEP values in a manner that would meet the requirements of the ASME RA-S-2002 (Ref. E8.1.4) standard. Therefore, all three of these methods are retained for use and the selection of one versus the other is made based on the specific unsafe action being quantified. This is documented as appropriate in the actual detailed quantification of each HFE.

4. Simplified methods:

A. Basis: These methods use the results of past PRAs to focus attention on those HFEs that have dominated risk. These are essentially PRA results from NPPs. As such, they presuppose NPP situations and actions, and define important PSFs based on these past NPP PRAs. They have very limited (if any) ability to investigate context, individual and human factors that are beyond NPP experience. The HEPs that result from applying these methods are calibrated to other NPP methods.

B. Methods considered: ASEP (Ref. E8.1.25), SPAR-H (Ref. E8.1.14).

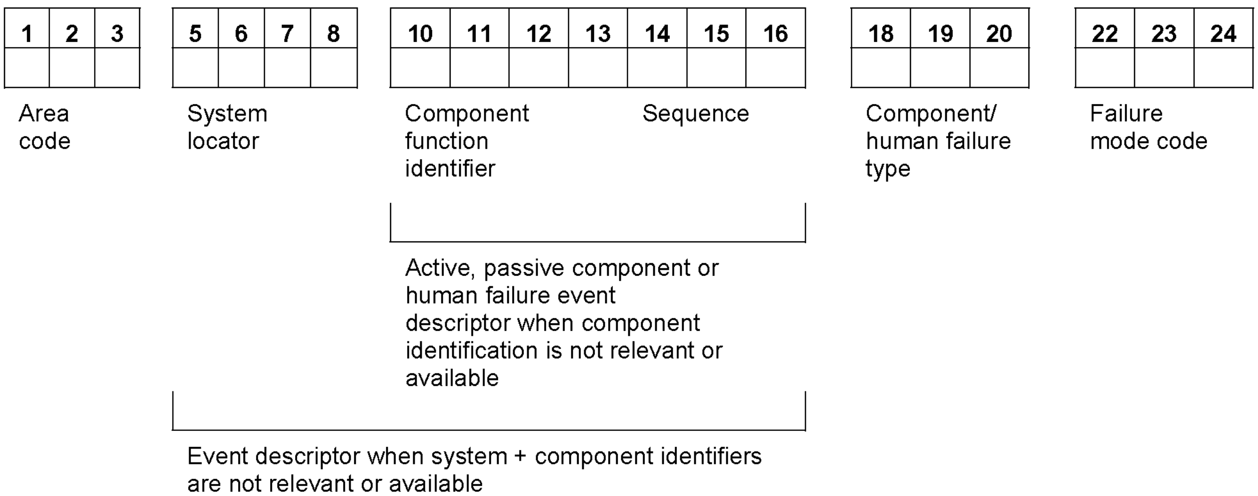
C. Applicability: These methods are clearly biased by their very close dependence on the results of past NPP PRAs. They are too limited for application beyond the NPP environment. They are not simply inappropriate for this application, but it would be extremely difficult to make a sound technical case regarding technical validity.

D. Assessment: No use can be identified for these methods within the YMP PCSA or any technical case made supporting them for a non-NPP application. None of them are retained.

APPENDIX E.V
HUMAN FAILURE EVENTS NAMING CONVENTION

Event names for HFEs in the YMP PCSA model follow the general structure of the naming convention for fault tree basic events. This is true whether the HFE is modeled in a fault tree, directly on an event tree, or as an initiating event. The convention, as adapted for HFEs, is as follows:

This basic event naming convention in Figure E.V-1 below is provided to ensure consistency with project standards and to permit this information to fit into a 24-character SAPHIRE field such that each basic event can be correlated to a unique component or human failure.



Source: Original

Figure E.V-1. Basic Event Naming Convention

The area code, taken from *Engineering Standard for Repository Area Codes Identifiers* (Ref. E8.1.8), defines the physical design or construction areas where a component would be installed. These codes are used rather than the facility acronyms to maintain consistency with Engineering. In this system, the Canister Receipt and Closure Facility is designated by area code 060, the Wet Handling Facility is 050, the RF is 200, the Initial Handling Facility is 51A, and Subsurface is 800. Intra-Site Operations could fall under one of several repository area codes and therefore the most appropriate code to use was the repository general area code. However, this code was insufficient for the purposes of this analysis, and a designator of ISO was substituted instead. For the majority of cases, the area coding of HFEs in Attachment E reflects the location of the operations being evaluated, such as ISO for Intra-Site Operations. However, for certain HFEs, the coding corresponds to the location of the systems impacted by the human failure, such as HVAC, which is specific to the CRCF and therefore retains the 060 coding, and AC power, which retains the 26x and 27x coding. For these specific instances, such coding provides better traceability of the HFE back to the affected equipment.

The system locator code identifies operational systems and processes. System locator codes (four characters) are listed in Table 1 of *Repository System Codes* (Ref. E8.1.9). These are generally three or four characters long, such as VCT for tertiary confinement HVAC.

The component function identifiers identify the component function and are listed in the *Engineering Standard for Repository Component Function Identifiers* (Ref. E8.1.7). These are generally three or four characters long. Some Bechtel SAIC Company, LLC component function identifiers for typical components are shown in Table E.V-1, but in cases where there is not an equivalent match, the most appropriate PCSA type code should be used (also given in Table E.V-1).

The sequence code is a numeric sequence and train assignment (suffix), if appropriate, that uniquely identifies components within the same area, system, and component function.

If an HFE is related to the failure of an individual component with an existing component function identifier and sequence code, the naming scheme should utilize these codes in the event name. If an HFE is such that these codes do not apply, the basic event name can be a free form field for describing the nature of the event, such as HCSKSCF for operator topples cask during scaffold movement or HFCANLIDAJAR for operator leaves canister lid ajar, utilizing either seven characters when there is a relevant system locator code, or 12 characters when no system codes are applicable.

The human failure type and failure mode codes are three characters each, consistent with the coding provided in Table E.V-1 below.

For HFEs, the type code always begins with HF and continues with a one letter designator for the HFE temporal phase: P for pre-initiator, I for human-induced initiator, N for non-recovery post-initiator, R for recovery post-initiator (this latter code is not used during preliminary analysis).

Table E.V-1. Human Failure Event Type Codes and Failure Mode Codes

PRE-INITIATOR HFEs; TYP=HFP		**FMC=**
Fail to properly restore a standby system to service		RSS
Failure to properly restore an operating system to service when the degraded state is not easily detectable		ROH
Failure to properly restore an operating system to service when the degraded state is easily detectable		ROE
Calibration error		CAL
HUMAN-INDUCED INITIATOR HFEs; TYP=HFI		
Failure to properly conduct an operation	Operation is performed on a daily basis.	NOD
	Operation is performed on a very regular basis (on the order of once per week)	NOW
	Operation is performed only very infrequently (once per month or less)	NOM

Table E.V-1. Human Failure Event Type Codes and Failure Mode Codes (Continued)

PRE-INITIATOR HFEs; TYP=HFP		FMC=
Operation is extremely complex OR conducted under environmental or ergonomic stress	Operation is performed on a daily basis.	COD
	Operation is performed on a very regular basis (on the order of once per week)	COW
	Operation is performed only very infrequently (once per month or less)	COM
Operation is extremely complex AND conducted under environmental or ergonomic stress	Operation is performed on a daily basis.	CSD
	Operation is performed on a very regular basis (on the order of once per week)	CSW
	Operation is performed only very infrequently (once per month or less)	CSM
NON-RECOVERY POST-INITIATOR HFEs; TYP=HFN		
Not trained or proceduralized, time pressure		NPT
Not trained or proceduralized, no time pressure		NPN
Trained and/or proceduralized, time pressure		TPT
Trained and/or proceduralized, no time pressure		TPN
RECOVERY POST-INITIATOR HFEs; TYP=HFR		
Not trained or proceduralized, time pressure		NPT
Not trained or proceduralized, no time pressure		NPN
Trained and/or proceduralized, time pressure		TPT
Trained and/or proceduralized, no time pressure		TPN

NOTE: FMC = failure mode code; HFE = human failure event; HFI = human-induced initiator HFE; HFN = human failure non-recovery post-initiator HFE; HFP = pre-initiator HFE; HFR = human failure recovery post-initiator HFE; TYP = type.

Source: Original

ATTACHMENT F
FIRE ANALYSIS

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ACRONYMS

CTM	canister transfer machine
CTT	cask transport trolley
DPC	dual-purpose canister
EPRI	Electrical Power Research Institute
GROA	geologic repository operations area
HEPA	high-efficiency particulate air
HVAC	heating, ventilation, and air-conditioning
MCC	motor control center
NFPA	National Fire Protection Association
NRC	U.S. Nuclear Regulatory Commission
P&ID	piping & instrument diagram
PCSA	Preclosure Safety Analysis
RF	Receipt Facility
RWF	residence weighting factor
TAD	transportation, aging, and disposal
TTC	transportation cask in the tilted position
VTC	a transportation cask that is upended on a railcar
YMP	Yucca Mountain Project

F1 INTRODUCTION

This document describes the work scope, definitions, and terms, method, and results for the fire analysis performed as a part of the Yucca Mountain Project (YMP) preclosure safety analysis (PCSA). Fire PCSA is divided into four major areas:

- Initiating event identification
- Initiating event quantification (including both ignition frequency and propagation probability)
- Fragility analysis (including convolution of fragility and hazard curves)
- Fire analysis model development and quantification.

Within the task, the internal events PCSA model is evaluated with respect to fire initiating events and modified as necessary to address fire-induced failures that lead to exposures. The lists of fire-induced failures that are included in the model are evaluated as to fire vulnerability, and fragility analyses are conducted as needed. All calculations are performed in Excel and included in Attachment H in *RF Fire Frequency_ no suppression.xls* and *RF CB Report.xls*.

F2 REFERENCES

Design Inputs

The PCSA is based on a snapshot of the design. The reference design documents are appropriately documented as design inputs in this section. Since the safety analysis is based on a snapshot of the design, referencing subsequent revisions to the design documents (as described in EG-PRO-3DP-G04B-00037, *Calculations and Analyses* (Ref. 2.1.1, Section 3.2.2.F)) that implement PCSA requirements flowing from the safety analysis would not be appropriate for the purpose of the PCSA.

The inputs in this Section noted with an asterisk (*) indicate that they fall into one of the designated categories described in Section 4.1, relative to suitability for intended use.

F2.1 ANSI/ANS 58.23-2007. *Fire PRA Methodology*. La Grange Park, Illinois: American Nuclear Society. TIC: 259894.

F2.2 ASME RA-S-2002. *Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications*. New York, New York: American Society of Mechanical Engineers. TIC: 255508. ISBN: 0-7918-2745-3.

F2.3 BSC 2007. *CRCF, RF, WHF, and IHF Cask Transfer Trolley Process and Instrumentation Diagram*. 000-M60-HM00-00301-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071119.0013.

F2.4 BSC 2007. *Equipment Motor Horsepower and Electrical Requirements Analysis*. 000-M0A-H000-00100-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070816.0001.

F2.5 *BSC 2007. *Preliminary Throughput Study for the Receipt Facility*. 200-30R-RF00-00300-000-000. REV 002. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071227.0021.

F2.6 *BSC 2007. *Receipt Facility Cask Cavity Gas Sampling System Piping & Instrument. Diagram*. 200-M60-MRE0-00101-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070328.0009.

F2.7 *BSC 2007. *Receipt Facility Chilled Water System Piping & Instrument. Diagram*. 200-M60-PSC0-00101-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070910.0017.

F2.8 *BSC 2007. *Receipt Facility Chilled Water System Piping & Instrument. Diagram*. 200-M60-PSC0-00102-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070910.0018.

F2.9 *BSC 2007. *Receipt Facility Chilled Water System Piping & Instrument. Diagram*. 200-M60-PSC0-00103-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070910.0019.

F2.10 *BSC 2007. *Receipt Facility Composite Vent Flow Diagram Non-Confinement Non-ITS HVAC Sys Support & Operations*. 200-M50-VNI0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071011.0016.

F2.11 BSC 2007. *Receipt Facility Composite Vent Flow Diagram Tertiary Conf ITS HVAC Systems, Elect & Battery RMS*. 200-M50-VCT0-00301-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071002.0022.

F2.12 BSC 2007. *Receipt Facility Composite Vent Flow Diagram Tertiary Confinement Non-ITS HVAC Supply & Exhaust System*. 200-M50-VCT0-00201-000 REV 00C. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071221.0003.

F2.13 BSC 2007. *Receipt Facility Composite Vent Flow Diagram Tertiary Confinement Non-ITS HVAC Supply Sys & ITS Exhaust*. 200-M50-VCT0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071002.0021.

F2.14 BSC 2007. *Receipt Facility Confinement ITS Battery Room Exhaust System - Train A Ventilation & Instrumentation Diagram*. 200-M80-VCT0-00302-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071201.0004.

F2.15 BSC 2007. *Receipt Facility Confinement ITS Battery Room Exhaust System - Train B Ventilation & Instrumentation Diagram*. 200-M80-VCT0-00304-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071201.0005.

F2.16 BSC 2007. *Receipt Facility Confinement ITS Electrical Room HVAC System - Train A Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00301-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071002.0027.

F2.17 BSC 2007. *Receipt Facility Confinement ITS Electrical Room HVAC System - Train B Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00303-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071002.0029.

F2.18 *BSC 2007. *Receipt Facility Confinement Non-ITS HEPA Exhaust System Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00205-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071221.0004.

F2.19 *BSC 2007. *Receipt Facility Confinement South Areas HVAC Supply System Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00201-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0005.

F2.20 *BSC 2007. *Receipt Facility Confinement 2nd Floor North Areas HVAC Supply System Ventilation & Instrumentation Diagram*. 200-M80-VCT0-00206-000 REV 00A. Las Vegas, NV: Bechtel SAIC Company. ACC: ENG.20071010.0010.

F2.21 *BSC 2007. *Receipt Facility General Arrangement Ground Floor Plan.* 200-P10-RF00-00102-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071212.0011.

F2.22 BSC 2007. *Receipt Facility General Arrangement Second Floor Plan.* 200-P10-RF00-00103-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071212.0012.

F2.23 BSC 2007. *Receipt Facility General Arrangement Third Floor Plan.* 200-P10-RF00-00104-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071212.0013.

F2.24 *BSC 2007. *Receipt Facility Hot Water System Piping & Instrument. Diagram*. 200-M60-PSH0-00101-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070918.0010.

F2.25 *BSC 2007. *Receipt Facility Hot Water System Piping & Instrument. Diagram*. 200-M60-PSH0-00102-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20070918.0011.

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F2.27 BSC 2007. *Receipt Facility ITS Confinement Areas HEPA Exhaust System - Train A Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071204.0017.

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F2.29 *BSC 2007. *Receipt Facility ITS Confinement Areas HVAC Supply System Ventilation & Instrumentation Diagram.* 200-M80-VCT0-00103-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071002.0025.

F2.30 BSC 2007. *Receipt Facility ITS UPS Train A 200-EEU0-UJX-00001 Single Line Diagram.* 200-E10-EEU0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0020.

F2.31 BSC 2007. *Receipt Facility ITS UPS Train B 200-EEU0-UJX-00002 Single Line Diagram.* 200-E10-EEU0-00201-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0021.

F2.32 *BSC 2007. *Receipt Facility UPS 200-EEP0-UJX-00001 Single Line Diagram*. 200-E10-EEP0-00101-000 REV 00B. Las Vegas, NV: Bechtel SAIC Company. ACC: ENG.20071217.0013.

F2.33 *BSC 2007. *Receipt Facility LLW Vestibule Non-Confinement HVAC System Ventilation & Instrumentation Diagram*. 200-M80-VNI0-00106-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0018.

F2.34 *BSC 2007. *Receipt Facility Non-Confinement Areas HVAC Supply System Ventilation & Instrumentation Diagram*. 200-M80-VNI0-00101-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0013.

F2.35 Not used.

F2.36 *BSC 2007. *Receipt Facility Site Transp Cask Vestibule Annex Non-Confinement HVAC System Ventilation & Instrumentation Diagram*. 200-M80-VNI0-00105-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0017.

F2.37 *BSC 2007. *Receipt Facility Site Transporter Vestibule Non-Confinement HVAC System Ventilation & Instrumentation Diagram*. 200-M80-VNI0-00104-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0016.

F2.38 *BSC 2007. *Receipt Facility Transportation Cask Vestibule Non-Confinement HVAC System Ventilation & Instrumentation Diagram*. 200-M80-VNI0-00103-000 REV 00A. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071010.0015.

F2.39 BSC 2007. *Receipt Facility 480V ITS MCC Train A 200-EEE0-MCC-00001 Single Line Diagram.* 200-E10-EEE0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0016.

F2.40 BSC 2007. *Receipt Facility 480V ITS MCC Train B 200-EEE0-MCC-00002 Single Line Diagram.* 200-E10-EEE0-00201-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0017.

F2.41 Not used.

F2.42 Not used.

F2.43 *BSC 2007. *Receipt Facility 480V Load Center 200-EEN0-LC-00001 Single Line Diagram.* 200-E10-EEN0-00101-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0003.

F2.44 Not used.

F2.45 *BSC 2007. *Receipt Facility 480V MCC 200-EEN0-MCC-00001 Single Line Diagram.* 200-E10-EEN0-00201-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0004.

F2.46 *BSC 2007. *Receipt Facility 480V MCC 200-EEN0-MCC-00002 Single Line Diagram.* 200-E10-EEN0-00301-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0005.

F2.47 *BSC 2007. *Receipt Facility 480V MCC 200-EEN0-MCC-00003 Single Line Diagram.* 200-E10-EEN0-00401-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0006.

F2.48 *BSC 2007. *Receipt Facility 480V MCC 200-EEN0-MCC-00004 Single Line Diagram.* 200-E10-EEN0-00501-000 REV 00B. Las Vegas, Nevada: Bechtel SAIC Company. ACC: ENG.20071217.0007.

F2.49 Not used.

F2.50 Not used.

F2.51 Not used.

F2.52 Not used.

F2.53 Not used.

F2.54 *EPRI (Electric Power Research Institute) and NRC (Nuclear Regulatory Commission) 2005. *Detailed Methodology*. Volume 2 of *EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities*. EPRI TR-1011989 and NUREG/CR-6850. Palo Alto, California: Electric Power Research Institute. ACC: MOL.20070323.0062.

F2.55 *EPRI and NRC 2005. *Summary & Overview*. Volume 1 of *EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities*. EPRI-1011989 and NUREG/CR-6850. Palo Alto, California: Electric Power Research Institute. ACC: MOL.20070323.0061.

F2.56 *NFPA (National Fire Protection Association) 2000. *Fires in or at Industrial Chemical, Hazardous Chemical, and Plastic Manufacturing Facilities: 1988 - 1997 Unallocated Annual Averages and Narratives*. Quincy, Massachusetts: National Fire Protection Association. TIC: 259997.

F2.57 *NFPA 2007. *Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction, 1980-1998*. Quincy, Massachusetts: National Fire Protection Association. TIC: 259983.

F2.58 *SAIC (Science Applications International Corporation) 2002. *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology*. SAIC-01/2650. Abingdon, Maryland: Science Applications International Corporation. ACC: MOL.20080115.0138.

F2.59 *Tillander, K. 2004. *Utilisation of Statistics to Assess Fire Risks in Buildings*. Ph.D. dissertation. Espoo, Finland: VTT Technical Research Centre of Finland. TIC: 259928.

F2.60 *Winkler, R. L., and Hays, W. L. 1975. *Statistics: Probability, Inference, and Decision*., Series in Quantitative Methods for Decision Making. 2nd Edition. Winkler, R.L., ed., New York, New York: Holt, Rinehart, and Winston. TIC: 259976. ISBN-10: 0030140110.

F3 BOUNDARY CONDITIONS

The general boundary conditions used during the analysis of fire vulnerabilities and fire model development are clearly stated and documented. In general, the boundary conditions are compatible with those ones usually applied to internal events due to fire events. The principal boundary conditions for the fire analysis are listed below:

F3.1 Plant Operational State

Initial state of the facility is normal with each system operating within its limiting condition of operation limits.

F3.2 Credit for Automatic Fire Suppression Systems

The automatic fire suppression systems, although designed to meet all requirements and standards for fire suppression systems in nuclear facilities, are considered non-important to safety and thus no credit is taken for their operation.

F3.3 Number of Fire Event to Occur

The facility is analyzed to respond to one fire event at a given time. Additional fire events as a result of independent causes or of re-ignition once a fire is extinguished are not considered.

F3.4 Ignition Source Counting

Ignition sources are counted in accordance with applicable counting guidance contained in NUREG/CR-6850 (Ref. F2.54) and (Ref. F2.55).

F3.5 Fire Cable and Circuit Failure Analysis

Unlike nuclear power plants, which depend on the continued operation of equipment to prevent fuel damage, the YMP facilities cease operating on loss of power or control. Therefore, fire damage in rooms that do not contain waste cannot result in an increased level of radiological exposure. Cable and circuit analysis in these rooms is not required.

F3.6 Heating, Ventilation, and Air Conditioning (HVAC) Fire Analysis

HVAC is not relied upon to mitigate potential releases associated with large fire event sequences. In recognition of a large amount of fire generated, non-radiological particulates could render the HVAC filters ineffective. HVAC can be credited for localized fires unless HVAC control or power circuits are present in the area of the fire.

F3.7 No Other Simultaneous Initiating Events

It is standard practice to not consider the occurrence of other initiating events (human-induced and naturally occurring) during the time span of an event sequence because (a) the probability of two simultaneous initiating events within the time span is small and, (b) each initiating event will cease operations of the waste handling facility, which further reduces the conditional probability of the occurrence of a second initiating event, given the first has occurred.

F3.8 Data Collection Scope

The fire ignition data collection and analysis are performed for locations relevant to waste handling in the facilities.

F3.9 Component Failure Modes

The failure mode of a structure, system, or component affected by a fire is the most severe with respect to consequences. For example, the failure mode for a canister could be the overpressurization of a reduced strength canister.

F3.10 Component Failure Probability

Fires large enough to fail waste containment components will be large enough to fail all active components in the same room. Active components fail in a de-energized state for such fires.

F3.11 Internal Events PCSA Model

To implement the systems analysis guidance contained herein, the fire preclosure safety analysis (PCSA) team uses the internal events PCSA model, which is developed concurrently with the fire PCSA. This internal events PCSA is used as the basis for the fire PCSA. The internal events PCSA is in general conformance with the ASME PRA *Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications* (Ref. F2.2).

F4 ANALYSIS METHOD

F4.1 Introduction

Nuclear power plant fire risk assessment techniques, as discussed in the following sections, have limited applicability to facilities such as the Receipt Facility (RF) or other facilities in the geologic repository operations area (GROA). The general methodological basis of this analysis is the *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology* (Ref. F2.58), which are similar to those in the GROA in that these facilities are handling and disposal facilities for highly hazardous materials. This is a "data based" approach in that it utilizes actual historical experience on fire ignition and fire propagation to determine fire initiating event frequencies. That approach has been adapted to utilize data applicable to the YMP waste handling facilities. To the extent applicable to a non-reactor facility, NUREG/CR-6850 (Ref. F2.54) and *Summary & Overview*. Volume 1 of *EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities*. EPRI-1011989 and NUREG/CR-6850 (Ref. F2.55) are also considered in the development of this analysis method. The method complies with the applicable requirements of the ANS fire PRA standard (Ref. F2.1) that is relevant to a non-reactor facility. Many of the definitions, modeling approximations, and requirements of these documents were used to develop this document.

F4.2 Identification of Initiating Events

Current techniques in fire risk assessment for nuclear power plants focus on fire that can damage electrical and control circuits or impact other equipment that can compromise process and safety systems. This type of approach is not generally applicable to YMP because loss of electric power is a safe state except for the need for HVAC after a release of radionuclides. In general, when systems are affected by fire, they cease to function. While at a nuclear power plant this is of concern, at YMP this means that fuel handling stops and initiating events capable of producing elevated levels of radioactivity are essentially unrealizable. While it is theoretically possible that a fire could inadvertently result in a drop of a cask or canister, it is difficult (if not impossible) to identify any mechanisms by which this would occur due to fire that would not be much more likely to occur by other means. Of much greater concern at YMP is the potential for a fire to directly affect the waste containers and cause a breach that would result in a release. The fire analysis, therefore, focused on potential for a fire to directly affect the waste containers and cause a breach that would result in a release, rather than analyzing fires that would remove power from fuel handling systems. After a release of radionuclides, the HVAC system, with its high-efficiency particulate air filter (HEPA) filtration, aids in the abatement of radioactivity that is released from buildings. However, the occurrence of fires tends to significantly reduce the effectiveness of HEPA filtration and the fire event sequence analysis, therefore, does not rely on this system. Consideration is given both to fires that start in rooms containing waste and fires that start in other rooms and propagate to where the waste is located. The steps of this process are outlined in Section F4.2.1 thru F4.2.4.

F4.2.1 Identify Fire-Rated Barriers and Designate Fire Zones

The facility is broken into fire zones based on the location of fire-rated barriers. The rating of the barriers is not significant to the methodology, so all rated barriers are considered. In order for a fire zone to exist, the penetrations, doorways, and ducts must also be limited to the perimeter of the zone. Note that a floor is always considered to be a fire barrier as long as it is solid. Zones are identified by a number determined by the analyst, and will consist of one or more rooms.

F4.2.2 Identify the Rooms Where Waste can be Present

Each room where waste can be present, even if only for a brief time, is listed. The first set of fire initiating events to be considered in the PCSA is fires that affect each of these rooms, but do not affect other rooms that could contain waste.

F4.2.3 Define Local Initiating Events

Fire ignition occurrences are identified for each room within a fire zone. The total occurrences of a fire within a room containing a waste form is composed of the occurrences of ignitions in that room plus the occurrences of ignitions in surrounding rooms, within the fire zone, which propagate across room boundaries to the room containing the waste form. The locations of fire initiating events were identified in the master logistic diagram.

F4.2.4 Define Large Fire Initiating Events

Traditional fire risk studies for nuclear power plants have tended to ignore large fires, arguing that the fire barriers in place will prevent such occurrences. However, actual observed historical data shows that large fires in buildings occur. Large fires are defined for this study as those that spread to encompass the entire building. This is recognized in the latest fire risk guidance from Nuclear Regulatory Commission (NRC) and Electrical Power Research Institute (EPRI) (Ref. F2.54, Section 11.5.4 and Ref. F2.55). There, potential large fire initiating events are identified. The general approach is as follows:

In the YMP facilities waste forms, except during the short time being lifted by a canister transfer machine (CTM), are on the ground floor. Continuing with the focus on rooms that contain waste forms, large fires may be divided two ways. One is associated with fires that start on the ground floor and spread to the entire building. The other is a fire that starts anywhere else in the building and spreads to the entire building.

As a practical analysis technique, any fire that spreads out of a fire area is considered a large fire.

F4.3 Quantification of Fire Ignition Frequency

The quantification of initiating event frequency involves three steps. First, the overall frequency of fire ignition for the facility is determined, then that frequency is allocated to the individual room in the facility based on the number and types of ignition sources in the rooms. Types of ignition sources are characterized in general terms such as mechanical, electrical, combustible liquid. Finally, propagation probabilities are applied to determine the overall frequency that a fire reaches the area of the waste. Quantification uses data from the following sources for equipment ignition frequencies and conditional probabilities of propagation:

Detailed Methodology. Volume 2 of EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities. EPRI TR-1011989 and NUREG/CR-6850 (Ref. F2.54).

Summary & Overview. Volume 1 of EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities. EPRI-1011989 and NUREG/CR-6850 (Ref. F2.55).

Fires in or at Industrial Chemical, Hazardous Chemical, and Plastic Manufacturing Facilities: 1988 - 1997 Unallocated Annual Averages and Narratives (Ref. F2.56).

Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction (Ref. F2.57).

Chemical Agent Disposal Facility Fire Hazard Assessment Methodology (Ref. F2.58).

Utilisation of Statistics to Assess Fire Risks in Buildings (Ref. F2.59).

F4.3.1 Determine the Overall Facility Fire Frequency

There is insufficient data available regarding the total frequency of fires in facilities comparable to YMP. NUREG/CR-6850 (Ref. F2.54) and (Ref. F2.55) provides an overall frequency for a typical nuclear power plant, but these are much larger and complex than the YMP facilities. Therefore, it has been decided to use a more generic fire ignition frequency approach that relates building size to total fire frequency for various broad categories of facilities (Ref. F2.59). This approach applies the following equation to overall fire ignition frequency.

Determine the Fire Frequency per Unit Area – The frequency per unit area is expressed by the following equation:

$$f_m(A) = c_1 A^r + c_2 A^s \quad \text{(Eq. F-1)}$$

where f_m is the fire ignition frequency per m^2-yr, A is the floor area (in m^2) and c_1, c_2, r, and s are coefficients that were determined from historical data observations for different types of facilities.

For industrial buildings, the parameter values are as follows:

c1 = 3×10-4; c2 = 5×10-6; r = -0.61; and s = -0.05

This first equation relates the frequency per unit area to the total area of the facility. This correlation was determined from the historical data, which showed that total fire frequency was not linearly related to the size of the facility. Rather, the frequency per unit area was affected by the size of the facility, and the larger the facility the lower the frequency per unit area was.

Determine the Total Fire Frequency for the Facility – The total frequency of fire ignition for the building is thus represented by the following equation:

$$f_{fire} = f_m(A) \times A \quad \text{(Eq. F-2)}$$

F4.3.2 Determine the Fire Ignition Frequency in Each Room

The approach to allocating the fire ignition frequency is based on the approach used in NUREG/CR-6850 (Ref. F2.54), (Ref. F2.55), and *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology* (Ref. F2.58). Both of these approaches determine the fraction of the total facility ignition frequency associated with various categories of equipment (i.e., ignition source category), then determine a facility-specific ignition frequency for each piece of equipment in each category, and then determine the total ignition frequency in the room based on the ignition source population in the room.

F4.3.2.1 Fraction of Fire Ignition Frequency Associated with Each Ignition Source Category

NUREG/CR-6850 (Ref. F2.54) and (Ref. F2.55) have data for these fractions for nuclear power plants, and *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology* (Ref. F2.58) has data for these frequencies for chemical process plants. Neither of these data sets is the best for the facilities at YMP. Therefore, the NFPA was requested to provide an analysis (Ref. F2.57) of the data in their proprietary database on the distribution of fires by equipment type in all nuclear facilities of non-combustible construction. NFPA distinguishes between a large number of equipment types that can cause ignition of a fire. There is an insufficient amount of data to justify retaining this number of equipment types, so the equipment types were consolidated into a set of ignition source categories. These categories are defined in Appendix F.I.

Using the data by category, an analysis is performed to determine the fraction of fires that are caused by each category. That analysis is documented in Appendix F.II.

The total fire ignition frequency from Section F4.3.1 is multiplied by each of these factors to determine the total fire ignition frequency due to each equipment type. For example, the total ignition frequency due to electrical equipment for a given facility is:

$$f_{elec\text{-}all} = f_{fire} \times 0.086 \quad \text{(Eq. F-3)}$$

F4.3.2.2 Individual Ignition Source Fire Ignition Frequency

The next step is to determine the fire ignition frequency from each piece of equipment in each category. As is done in NUREG/CR-6850 (Ref. F2.54), (Ref. F2.55), and *Chemical Agent Disposal Facility Fire Hazard Assessment Methodology* (Ref. F2.58), divide the frequency contribution for each equipment type by the total number of pieces of equipment in the facility. For example, take the case following from the above example for the frequency of fire ignition from electrical equipment. If there are 50 pieces of electrical equipment in the facility, the ignition frequency for each piece of equipment is:

$$f_{elec\text{-}each} = f_{elec\text{-}all} / 50 \quad \text{(Eq. F-4)}$$

For the case of the category "no equipment involved" the ignition frequency is per unit area, so the total for this category is divided by the total floor area of the facility (which was already determined in Section F4.3.1).

F4.3.2.3 Allocation of Fire Ignition Frequency to Each Room

The final step is to use the per equipment values to allocate fire frequency to each room. This is done by counting the number of ignition sources of each type contained in each room, multiplying by the ignition frequency for each ignition source type, and summing across all types. For example, if Room 1 has six pieces of electrical equipment, then the ignition frequency in that room due to electrical equipment is:

$$f_{elec\text{-}1} = f_{elec\text{-}each} \times 6 \qquad \text{(Eq. F-5)}$$

Doing this for each ignition source type (including multiplying the "no equipment involved" per unit area by the floor area of the room) and summing them together yields the total fire ignition frequency for the room:

$$f_1 = f_{elec\text{-}1} + f_{hvac\text{-}1} + f_{\ldots\text{-}1} \qquad \text{(Eq. F-6)}$$

F4.4 Determine Initiating Event Frequency

The definition of each initiating event includes the implicit condition that the fire actually threatens a target that contains radioactive material. Therefore, for each initiating event, the initiating event frequency considers two aspects; the fraction of time there is a waste container in the room, and the probability a fire propagates to that waste container.

F4.4.1 Probability of Presence of a Target

The probability of the presence of a target waste form is the fraction of time that the waste form(s) is in the area affected by the fire (e.g., for a room fire it is the fraction of time a waste form is in the room). For use in initiating event frequency equations, the probability is represented as follows:

P_{wr} = probability that a particular waste form is in room i during the preclosure period

P_{wz} = probability that a particular waste form is in zone i during the preclosure period

P_{wfi} = probability that a particular waste form is on floor i during the preclosure period

P_{wb} = probability that a particular waste form is in the building during the preclosure period.

Note the specific phrasing. This probability pertains to each individual waste form (i.e., one of the approximately 11,000 waste forms that will be handled at YMP). For example, if each waste form that passes through the RF spends 60 minutes in the Cask Preparation Room, the probability that it is present when a fire occurs is 60 min/(50 yrs × 8,760 hrs/yr × 60 min/hr). This is used to correct the final initiating event frequency for fires (normally expressed as per year) to be per operation over the preclosure period so that it is equivalent to the other internal initiating events (e.g., drops) and can be multiplied by the number of operations in same manner.

F4.4.2 Probability of Propagation to a Target

Of key interest for assessing the fire risk, is the extent to which fires that start in a "benign" area can spread to sensitive areas (i.e., areas where nuclear waste is present). The likelihood of fire propagation within the building is strongly dependent on the building construction and the presence of automatic fire suppression systems.

Both probabilities of exceedance and conditional probabilities were determined. The probabilities of exceedance are the probabilities that a fire propagates up to a specified limit or beyond. The conditional probabilities are probabilities that a fire spreads to a specified limit.

Probabilities of exceedance are not independent, but rather represent the total probability that a fire spreads up to the specified limit or beyond. These values are provided because, for many fire sequences there will only be one case of interest, (i.e., there will be only one target of concern, and once the fire reaches that target the fact that the fire may propagate even further does not change the outcome of the sequence in terms of release). For example, this value could be applied to a case where a fire that spreads throughout a room affects the waste form in that room, and there are no additional waste forms in adjacent rooms or fire zones.

Conditional probabilities are independent, as they represent the probability that a fire spreads to precisely the specified limit. These values are provided to address those cases where the extent of propagation will define the number of targets involved in the fire. For example, these values would be applied when a fire that spreads throughout a room affects a waste form in that room; but if it spreads to adjacent rooms, additional forms would be involved.

There are two types of propagation that are considered: propagation within a room and propagation between rooms.

F4.4.2.1 Fire Propagation Within Rooms

An important consideration in the fire risk assessment is propagation within a given room. This will be referred to as "in-room propagation." Propagation within the room is important for fires initiated in a room where waste is present. In this case, the question is whether the fire, which can ignite wherever there is an ignition source in the room, reaches the area within the room in which the waste is located.

This section provides a table with the in-room propagation values for the cases with and without automatic fire suppression systems functioning. To use this table to determine whether the fire spreads sufficiently to threaten waste forms, it is necessary to consider where the fire occurs in the room of interest. The steps in this process are as follows:

- Determine the distribution of the ignition sources (identified under Section F4.3.2.3) within the room by counting the total number of potential ignition sources that are "at," "near," or "far from" the target waste form.[1]
- Calculate the fraction of ignition sources "at," "near," and "far from" the target waste form by dividing the number at each location by the total in the room.
- Calculate the frequency of the fire reaching the waste form using the following equation:

$$fier\text{-}i = Pwri\ [fi\ (FRa + (FRn \times (Ppc + Prc)) + (FRf \times Prc))] \quad \text{(Eq. F-7)}$$

where

$f_{ier\text{-}I}$ = frequency of fire affecting waste form, i-th room

P_{wri} = probability that a waste form is in the i-th room

f_i = frequency of ignition, i-th room

FR_a = fraction of ignition sources at the waste form

FR_n = fraction of ignition sources near the waste form

P_{pc} = conditional probability for fire confined to part of room of origin

FR_f = fraction of ignition sources far from the waste form

P_{rc} = conditional probability for confined to room of origin.

The values for P in the previous equation were developed from the analysis performed by NFPA (Ref. F2.57). The derivation of the values is provided in Appendix F.II for two cases (automatic fire suppression available and automatic fire suppression unavailable). The frequency f_i is the sum of frequencies of ignition of all ignition sources in the room. The fraction of ignition sources at, near, and far from the waste form was developed from equipment layout drawings such as:

Receipt Facility General Arrangement Ground Floor Plan. (Ref. F2.21).

F4.4.2.2 Fire Propagation Beyond Rooms

This section provides propagation probabilities for fires spreading beyond the room in which they start. This type of propagation will be referred to as "ex-room propagation."

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[1] In the context of this method, an ignition source within a few feet of the waste source would be "at" the source, whereas an ignition source beyond this distance, but within a few yards of the waste source would be "near" the source. Ignition sources more that a few yards distant would be "far from" the waste source. This definition coordinates with the fire response model given in Attachment D.

This section provides a table with the ex-room propagation values for the cases with and without automatic fire suppression systems functioning. To use this table to determine whether the fire spreads sufficiently to threaten waste forms, it is necessary to consider the various rooms where the fire could start and spread to the extent defined by the initiating event. The steps in this process are as follows:

- For each initiating event, identify all of the rooms within the area defined by the initiating event. For example, for a fire involving a specific fire zone, list all the rooms in that zone. For a fire involving an entire floor, list all the rooms on the floor. For a fire involving the entire building, list all rooms in the building.

- For each room, calculate the probability that a fire that starts within the room is not confined to the next smaller fire initiating event but is confined to less than the definition of the next largest initiating event by multiplying the ignition frequency for the room by the conditional probability (or sum of conditional probabilities) that the fire spreads at least as far as defined, but no further. For example, for a fire involving a floor where there is also an initiating event for a fire involving a zone on the floor and an initiating event involving the entire building (multiple floors or beyond), the equation is:

$$f_{ief\text{-}fj\text{-}ri} = f_i \times P_{fc} \qquad \text{(Eq. F-8)}$$

where

$f_{ief\text{-}fj\text{-}ri}$ = frequency of fire in zone j starting in room i

f_i = frequency of ignition, i-th room

P_{fc} = conditional probability for fire confined to floor of origin.

Similarly, for a fire involving a floor where there is an initiating event for a fire in a zone on the floor and no specific initiating event for a fire involving the entire building the equation is:

$$f_{ief+\text{-}ri} = f_i \times (P_{fc} + P_{bc} + P_{b+c}) \qquad \text{(Eq. F-9)}$$

where

$f_{ief+\text{-}ri}$ = frequency of fire involving an entire floor or greater starting in room i

f_i = frequency of ignition, i-th room

P_{fc} = conditional probability for fire confined to floor of origin

P_{bc} = conditional probability for fire confined to building of origin

P_{b+c} = conditional probability for fire extending beyond building of origin.

The total fire frequency of the defined severity is the sum across all rooms relevant to the initiating event, as discussed above.

F4.4.3 Initiating Event Frequency

The final initiating event frequency is determined by multiplying the frequency of the fire reaching the waste form (in occurrences over the 50-year preclosure period) times the probability that a waste form is present (fraction of time over the 50-year preclosure period per waste form). This yields the initiating event frequency for a fire of a specific severity affecting a waste form, per waste form processed, over the preclosure period.

F5 ANALYSIS

F5.1 Introduction

Fire initiating event frequencies have been calculated using Excel spreadsheets (RF Fire Frequency_NoSuppression.xls and RF CB Report.xls in Attachment H) for each fire initiating event identified for the RF. This section details the analysis performed to determine these frequencies, using the methodology documented in Section F4. The discussion of the analysis below presupposes that the reader has developed a thorough understanding of the details of that methodology, as those details are not repeated in this section. Note that the tables presented in this section, unless otherwise noted, are images of the actual spreadsheets used to perform the calculations. Therefore, there are no typographical errors in the translation of the results of the calculations into this report. The spreadsheet cells are color-coded to aid the analyst. Green numbers indicate values that are input by the analyst specific to the facility. Black numbers result from “off-line” calculations performed for this study. That is, they are facility-specific parameters whose values were determined as part of this analysis, but are not directly linked to the cell (i.e., they needed to be entered by the analyst). The source for these values is indicated in the text description of the spreadsheet. Orange numbers are values based on the analysis of operational experience (e.g., NFPA data), and should generally not be changed unless the analysis of operational experience changes or is updated. Red numbers are calculated values and should never be changed by the analyst. Green shaded cells are parameters that are assigned distributions that are used for the Crystal Ball Monte Carlo simulation runs discussed in section F5.8. The aqua shaded cells are the final initiating event frequencies. The values shown in the cells are the baseline, point estimate values. The Monte Carlo simulation runs convert these values into distributions for use in the event sequence quantification.

F5.2 Initiating Event Frequencies

Fire ignition frequencies are based upon the total floor area of the building. Thus, the assessment of the area of each room of the RF is the first step in obtaining initiating event frequencies. Table F5.2-1 shows the calculations that were performed to identify individual room areas, total ignition frequency, and uncertainty distributions.

F5.2.1 Room Area

Dimensions for room area calculations were obtained from the following RF general layout drawings:

Receipt Facility General Arrangement Ground Floor Plan (Ref. F2.21)

Receipt Facility General Arrangement Second Floor Plan (Ref. F2.22)

Receipt Facility General Arrangement Third Floor Plan (Ref. F2.23).

In some cases, the dimension intervals shown on the general arrangement drawings matched the boundaries of the rooms. Where this was the case these values were used to define the dimensions of the rooms. In cases where these the dimension intervals did not accurately represent a room, the drawing scale and a straightedge was utilized to determine the dimensions. The length and width figures obtained were entered into the L1(ft) and L2(ft) columns of Table F5.2-1 and multiplied to produce the area in square feet. Rooms 1002 and 2007 occupy two floors of building space. The area obtained for these rooms was doubled to account for this. Similarly, rooms 1017/1017A and 1028 occupy three floors of building space, and the area for these rooms was tripled. Rooms 1003E, 1017/1017A, 1028A, 1029, 1201A, 2029, and 3029 are not of a standard rectangular shape whose area can be calculated by a single length and width. Thus, these rooms were divided into two to three rectangles, each with a determined length and width. Addition of the area of these rectangles provides the total room area. Rooms 1005, 1018, 1019, 1020, 1221, 1223, 2005, and 2012 contain smaller room(s) within themselves. To account for this, the red text indicates a reference to the cells that contain the dimensions of the smaller room(s), the area of which is subtracted from the area of the room containing it. All areas calculated in square feet were multiplied by 0.09290304 to obtain the area in square meters, since Equation F-1 is based in square meters.

Table F5.2-1. Room Areas and Total Ignition Frequency

Room	L1(ft)	L2(ft)	A(sq-ft)	A(sq-m)	L3 (ft)	L4(ft)			
1001	39	46	1794	167					
1002	43	46	3956	368	*Area multiplied by two - Room extends two floors				
1003A	6	71	426	40					
1003B	82	10	820	76					
1003C	82	7	574	53					
1003D	84	18	1512	140					
1003E	155.667	8	1429	133	8	23			
1003F	10	72	720	67					
1003G	6	80	480	45					
1004	51	55	2805	261					
1004A	51	21	1071	99					
1005	38.66667	71	2529	235	24	9			
1005A	24	9	216	20					
1011	35	30	1050	98					
1012	74	43	3182	296					
1013	41	46	1886	175					
1014	33	46	1518	141					
1015	41	41	1681	156					
1016	33	41	1353	126					
1017/1017A	74	91	21452.34	1993	40.3334	31	*Area multiplied by three (3 floors)		
1018	46	72	2760	256	46	12			
1018A	46	12	552	51					
1019	50	72	2850	265	50	15			
1019A	50	15	750	70					
1020	38.667	72	2550	237	26	9			
1020A	26	9	234	22					
1021	40.3334	51	2057	191					
1021A	47	80	3760	349					
1021B	11	12	132	12					
1022	32	17	544	51					
1023	34	17	578	54					
1025	32	19	608	56					

Table F5.2-1. Room Areas and Total Ignition Frequency (Continued)

Room	L1(ft)	L2(ft)	A(sq-ft)	A(sq-m)	L3 (ft)	L4(ft)			
1026	33	13	429	40					
1027	13	25	325	30					
1028	18	15	810	75	*Area multiplied by three - Room extends three floors				
1028A	30	24	552	51	12	14			
1029	23	24	454	42	7	14			
1030	18	18	324	30					
1031	18	19	342	32					
1200	9	9	81	8					
1201A	7	64	508	47	10	6			
1201B	108	10	1080	100					
1202	15	15	225	21					
1203	20	25	500	46					
1204	15	25	375	35					
1205	10	9	90	8					
1206	15	26	390	36					
1207	23	32	736	68					
1208	16	34	544	51					
1209	17	34	578	54					
1210	18	34	612	57					
1211	11	34	374	35					
1212	35	12	420	39					
1212A	10	8	80	7					
1213	8	17	136	13					
1214	8	17	136	13					
1215	19	17	323	30					
1216	10	17	170	16					
1217	45	9	405	38					
1218	13	17	221	21					
1219	13	17	221	21					
1220	20	17	340	32					
1221	18	34	522	48	10	9			
1222	9	5	45	4					
1223	16	26	371	34	9	5			

Table F5.2-1. Room Areas and Total Ignition Frequency (Continued)

Room	L1(ft)	L2(ft)	A(sq-ft)	A(sq-m)	L3 (ft)	L4(ft)			
1224	28	28	784	73					
2001	39	46	1794	167					
2002A	9	82	738	69					
2002B	142	10	1420	132					
2002C	9	20	180	17					
2002D	10	94	940	87					
2002E	196	10	1960	182					
2002F	9	72	648	60					
2002G	9	20	180	17					
2003	50	72	3600	334					
2004	38.667	72	2784	259					
2005	52.333	72	3588	333	9	20			
2006	74	43	3182	296					
2007	74	105	15540	1444	*Area multiplied by two - Room extends two floors				
2008	33	87	2871	267					
2009	46	72	3312	308					
2010	50	72	3600	334					
2011	38.667	72	2784	259					
2012	52.333	72	3588	333	9	20			
2022	32	18	576	54					
2023	34	17	578	54					
2025	31	19	589	55					
2026	31	14	434	40					
2027	15	27	405	38					
2029	23	24	454	42	7	14			
3001	20	13	260	24					
3026	31	14	434	40					
3029	23	24	454	42	7	14			
Total Area (sq-m)				12842		50% Value		97.5% Value	
Ignition Frequency (per sq-m/yr)				4.05E-06	4.05E-06	4.05E-06		9.64E-06	
Ignition Frequency (per yr)				5.20E-02					
Ignition Frequency (50 years - preclosure period)				2.60E+00					

NOTE: A = area; ft = foot; m = meter; sq = square.

Source: Original

F5.2.2 Building Ignition Frequency

Ignition frequency calculations are presented at the bottom of Table F5.2-1, and begin with the total area calculation. This is obtained by summing the areas (in square meters) of all rooms in the building. The ignition frequency per square meter per year line implements Equation F-1. The ignition frequency per year line implements Equation F-2. The ignition frequency over the 50 year period is obtained by multiplying the latter value by 50. As can be seen from the table, the expected number of ignition events over the preclosure period is approximately four.

The values shown are the baseline mean values for ignition frequency. An uncertainty analysis was performed on the results of Equation F-1 for the use of Crystal Ball software to run Monte Carlo simulations to obtain fire initiating event frequency distributions. The geometric mean and 97.5 percent values of the resulting distribution for Equation F-1 are shown on the table. Refer to Appendix F.II for the calculations performed to develop the uncertainty distribution.

F5.3 Ignition Source Frequency

As discussed in Section F4.3.2.1, an industrial building fire can begin as the result of numerous types of ignition sources, which have been grouped into nine categories:

- Electrical
- HVAC
- Mechanical equipment
- Heat generating equipment
- Torches, welders, and burners
- Internal combustion engines
- Office/kitchen equipment
- Portable equipment
- No equipment involved.

Each category has a fraction representing the probability that, given an ignition, that category is the source of the ignition. The mean values of these fractions are shown in the column labeled Category Fraction in Table F5.3-1. The derivation of these values is discussed in Appendix F.II. The column labeled Category Frequency (50 years) implements the generic form of Equation F-3 to determine the mean ignition frequency associated with each ignition source. The next column, Category Population, contains the total number of ignition sources in each category in the facility. This is either the actual count of sources, a weighted point score of sources, or (for the case of no equipment involved) the total floor area of the facility. The source of the count or score is presented in the next section. The floor area is taken from Table F5.2-1, fourth row from the bottom. The fifth column uses the previous two columns to implement Equation F-4 to determine the frequency per ignition source unit (i.e., per ignition source, per ignition source weighted point, or per square meter of floor area). These values are used in the next section to allocate fire ignition frequency to each room in the facility.

As stated previously, these are mean values. The right hand group of columns is used by Crystal Ball to apply an uncertainty distribution to each of the category fraction values for the purpose of developing uncertainty distributions on initiating event frequency. The Mean Fraction, 97.5%

Value, and 97.5th percentile add columns show the parameters of these distributions. The development of all of the values is detailed in Appendix F.II. When Crystal Ball is run, it creates a sampled value for each fraction in the sampled value column. The spreadsheet then determines a normalized value by first assuring that each sampled value is not negative (minimum value of zero) and then normalizing the values so that the sum is always equal to one. The normalized value for each trial then replaces the category fraction value in the calculation. These probabilities must always add to one, as the groupings include all possible sources of ignition.

Table F5.3-1. Ignition Frequency by Ignition Source

Category	Category Fraction	Category Frequency (50 years)	Category Population	Frequency per Unit (50 years)			Sampled Value	Mean Fraction	97.5% Value	97.5th percentile add
Electrical	0.086	2.22E-01	157	1.42E-03		0.086	0.086	0.086	1.26E-01	4.05E-02
HVAC	0.080	2.09E-01	36	5.79E-03		0.080	0.080	0.080	1.20E-01	3.93E-02
Mechanical Equipment	0.139	3.62E-01	32	1.13E-02		0.139	0.139	0.139	1.89E-01	5.01E-02
Heat Generating Equipment	0.155	4.03E-01	0	0.00E+00		0.155	0.155	0.155	2.07E-01	5.24E-02
Torches, welders, burners	0.219	5.69E-01	440	1.29E-03		0.219	0.219	0.219	2.79E-01	5.99E-02
Internal combustion engines	0.021	5.46E-02	200	2.73E-04		0.021	0.021	0.021	4.23E-02	2.09E-02
Office/kitchen equipment	0.064	1.66E-01	10	1.66E-02		0.064	0.064	0.064	9.97E-02	3.55E-02
Portable Equipment	0.102	2.65E-01	36	7.37E-03		0.102	0.102	0.102	1.45E-01	4.37E-02
No equipment involved	0.134	3.48E-01	12842	2.71E-05		0.134	0.134	0.134	1.83E-01	4.93E-02
	1.000	2.6E+00				1.000				

NOTE: HVAC = heating, ventilation, and air conditioning.

Source: Original

F5.4 Ignition Source Distribution (Equipment List)

Compiling an initiating event frequency for the RF is dependant on identifying many characteristics of the building, to include ignition sources. Ignition sources are defined as items which exist in the rooms of the building that have the potential to contribute to the initiation and/or propagation of a fire. These sources are grouped into eight categories: electrical equipment; mechanical/electrical HVAC equipment; mechanical process equipment; heat generating process equipment; torches, welders and burners; internal combustion engines; office/kitchen equipment; and portable and special equipment. Once the grouping for a source is determined, it is assigned a count (points), a number which specifies the significance of the source by its contribution to fire ignition. Counts are integral to the calculations, as the total count for each category and room are multiplied by the ignition source frequency and summed to obtain the room ignition frequency. Table F5.4-1 shows the results of the ignition source distribution assessment for the RF. The red numbers on this table highlight the actual count used, so as to make identification of the equipment count values easy to pick out from the other equipment identification information provided. The x-out information shows pieces of equipment that are in the room in question, but they do not count as ignition sources per the counting rules. The following sections describe how the equipment was identified, categorized, and counted for the building.

Table F5.4-1. Ignition Source Population by Room

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1001 (Site Transporter Vestibule)		~~2 Site Transporter Vestibule Fan coil units~~ ~~200-VNI0-FCU-00003~~ ~~200-VNI0-FCU-00004~~ ~~3 HP (ea.)~~	~~Overhead Door~~ ~~2 motors @ 3hp ea.~~			**7% Site Transporter** 1002 & 1013 7 **points** **200 hp diesel/elec.**		
1002 (Lid Bolting Room)			**Lid Bolting Rm. 10 ton Crane** 200-HMC0-CRN-00001 • **1** + 2 **motors @ 25,** 1.5, & 3 **hp** • **29.5 hp** **Lid Bolting Platform** 200-HMC0-PLAT-00003 • **10 hp** • **2 motors @ 5, & 5 hp** ~~Overhead Door~~ ~~2 motors @ 3hp ea.~~			**59% Site Transporter** 1001 & 1013 • **59 points** • **200 hp diesel/elec.**		
1003A (Corridor)								
1003B (Corridor)								
1003C (Corridor)								
1003D (Corridor)								
1003E (Corridor)								
1003F (Corridor)								
1003G (Corridor)								
1003H (Utility Chase)								
1004 (HVAC Room)		**Exhaust Fan** 200-VCT0-EXH-00005 • **1 Motor** • **200 hp** **3 HEPA Filter Units** (hp n/a) 200-VCT0-FLT-00005 200-VCT0-FLT-00006 200-VCT0-FLT-00007			**Portable Welding Receptacle – WWF = 5 points**			**11.1% of all such equipment** • **4 points**
1004A (HVAC Room)		**2 Exhaust Fans** 200-VCT0-EXH-00009 200-VCT0-EXH-00010 • **7.5 hp (ea.)** **2 HEPA Filter Units** (hp n/a) 200-VCT0-FLT-00003 200-VCT0-FLT-00004						**5.6% of all such equipment** • **2 points**

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1005 (Electrical Room)	**480V Load Center** 200-EEE0-LC-00001 • **2 cabs** **480V MCC ITS** 200-EEE0-MCC-00001 • **10 cabs** **1 480V UPS ITS** 200-EEU0-UJX-00001 **1 45kVA ITS Dist. Xfmr** 200-EEE0-XFMR-00003 **1 480kVA ITS UPS** 200-EEE0-XFMR-00004 **1 40kVA ITS Bypass Xfmr** 200-EEU0-XFMR-00001 **1 208/120V Distribution Panel** 200-EEE0-PL-00003 **1 480/277V ITS Lighting Panel** 200-EUL0-PL-00002 **1 208/120V UPS Dist. Panel** 200-EEU0-PL-00001 **2 PLC Panels** **2 DCMIS**	**2 Fan Coil Units** 200-VCT0-FCU-00001 200-VCT0-FCU-00002 • **20 hp (ea.)**						
1005A (Battery Room)	**1 125V Battery** **200-EEU0-BTRY-00001**							
1011 (LLW Vestibule)		~~2 LLLW Entrance Vestibule Fan coil units~~ ~~200-VNI0-FCU-00007~~ ~~200-VNI0-FCU-00008~~ ~~3 HP (ea.)~~	~~Overhead Door~~ ~~1 motor @ 2hp~~					
1012 (LLW Staging Room)		~~MP LLW Liquid Samp. Pump~~ ~~200-MWL0-P-00001~~ ~~0.5 hp~~ ~~MP LLW Liquid Sump Pump~~ ~~200-MWL0-P-00002~~ ~~2 hp~~	~~Overhead Door~~ ~~1 motor @ 2hp~~		**Portable Welding Receptacle – WWF = 5 points**			
1013 (Loading Room)			**Shield Door** 200-RF00-DR-00002 **2 motors @ 7.5 & 7.5 hp**			**34% Site Transporter** 1001 & 1002 • **34 points** **200 hp diesel/elec.**		

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1014 (Maint. Room)			**2 Chilled Water Pumps** 200-PSC0-P-00001A 200-PSC0-P-00001B • **1 motor (ea.)** • **50 hp (ea.)** **2 Hot Water Pumps** 200-PSH0-P-00001A 200-PSH0-P-00001B • **1 motor (ea.)** **15 hp**		**Portable Welding Receptacle – WWF = 5 point**			
1015 (Cask Unloading Room)			**Shield Door** 200-RF00-DR-00001 • **2 motors @ 7.5 hp** • **15 hp** **3% in 1015** **Cask Transfer Trolley** 200-HM00-TRLY-00001 • **1 power drive x RWF 0.03** • **5 hp** Shared w/ room 1017					**2.8% of all such equipment** **1 point**
1016 (CTM Maint. Room)			~~Overhead Door~~ ~~1 motor @ 2hp~~					
1017/1017A (Cask Preparation Room and Annex)			**Cask Handling Crane** 200-HM00-CRN-00001 • **4 motors @ 90, 45, 7.5, & 30 hp** • **120 hp** **Cask Preparation Platform** 200-HMH0-PLAT-00001 • **10 hp** • **2 motors @ 5 hp ea** **Mobile Access Platform** 200-HMC0-PLAT-00001 • **40 hp** • **4 motors @ 1 hp** • **4 motors @ 4 hp** • **2 motors @ 10 hp** **97% in 1017** **Cask Transfer Trolley** 200-HM00-TRLY-00001 • **1 power drive x RWF 0.97** • **5 hp** Shared w/ room 1015 ~~Cask Handling Yoke~~ ~~200-HM00-BEAM-00001~~ ~~2 hp~~		**Primary Welding Station** **400 points**	**35% Site Prime Mover** • **35 points** **Split w/ rooms 1021 & 1021A**		**11.1% of all such equipment** **4 points**

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1018 (Electrical Room)	**480V Load Center** 200-EEN0-LC-00001 • **6 cabs** **480V MCCs** 200-EEN0-MCC-00001 • **11 cabs** 200-EEN0-MCC-00002 • **14 cabs** 200-EEN0-MCC-00003 • **14 cabs** 200-EEN0-MCC-00004 • **8 cabs** 200-EEN0-MCC-00005 • **6 cabs** 200-EEN0-MCC-00006 • **7 cabs** ~~2 Xfmrs~~ ~~200-EEN0-XFMR-00001~~ ~~200-EEN0-XFMR-00002~~ • ~~13.8 kVA~~ • ~~located outside~~ **1 480V UPS** 200-EEP0-UJX-00001 **1 208/120V UPS Panel** 200-EEP0-PL-00001 **2 75kVA Distribution Xfmrs** 200-EEN0-XFMR-00003 200-EEN0-XFMR-00004 **1 480-208/120V Bypass Xfmr** 200-EEP0-XFMR-00001 **2 208/120V Distribution Panels** 200-EEN0-PL-00003 200-EEN0-PL-00004 **3 480/277V Lighting Panels** 200-EUL0-PL-00001 200-EUL0-PL-00002 200-EUL0-PL-00006 **2 PLC Panels** **2 DCMIS**				**Portable Welding Receptacle – WWF = 5 points**			**5.6% of all such equipment** **2 points**
1018A (Battery Room)	**2 125V Batteries** 200-EEP0-BTRY-00001 200-EEP0-BTRY-00002							

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1019 (HVAC Room)		**Exhaust Fan** 200-VCT0-EXH-00006 • **1 motor** • **200 hp** **3 HEPA Filter Units** (hp n/a) 200-VCT0-FLT-00008 200-VCT0-FLT-00009 200-VCT0-FLT-00010						**11.1% of all such equipment** **4 points**
1019A (HVAC Room)		**2 Exhaust Fans** 200-VCT0-EXH-00011 200-VCT0-EXH-00012 • **15 hp (ea.)** **2 HEPA Filter Units** (HP n/a) 200-VCT0-FLT-00011 200-VCT0-FLT-00012						**5.6% of all such equipment** **2 points**
1020 (Electrical Room)	**1 ITS Xfmr** 200-EEE0-XFMR-00002 • **13.8kVA** **480V Load Center** 200-EEE0-LC-00002 • **2 cabs** **480V ITS MCC** 200-EEE0-MCC-00002 • **10 cabs** **1 480V ITS UPS** 200-EEU0-UJX-00002 **1 480kVA ITS Dist. Xfmr** 200-EEE0-XFMR-00004 **1 40kVA ITS Bypass Xfmr** 200-EEU0-XFMR-00002 **1 208/120V Distribution Panel** 200-EEE0-PL-00004 **1 480/277V ITS Lighting Panel** 200-EUL0-PL-00001-B **1 208/120V UPS Dist. Panel** 200-EEU0-PL-00002 **2 PLC Panels** **2 DCMIS**	**2 Fan coil units** 200-VCT0-FCU-00003 200-VCT0-FCU-00004 • **20 hp (ea.)**						**5.6% of all such equipment** **2 points**
1020A (Battery Room)	**1 125V Battery** 200-EEU0-BTRY-00002							
1021 (Transport Cask Vestibule Annex)		~~2 Transportation Cask Vestibule Fan coil units~~ ~~200-VNI0-FCU-00005~~ ~~200-VNI0-FCU-00006~~ ~~1.5 HP (ea.)~~	**Overhead Door** • **1 motor @ 5 hp**			**33% Site Prime Mover** • **33 points** **Split w/ rooms 1021A & 1017/1017A**		

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1021A (Transport Cask Vestibule)		**2 Transportation Cask Vest Fan Coil Units** 200-VNI0-FCU-00001 200-VNI0-FCU-00002 **7.5 hp (ea.)**	**2 Overhead Doors** • **1 motor ea. @ 5 hp**			**32% Site Prime Mover** • **32 points** **Split w/ rooms 1021 & 1017/1017A**		
1021B (Personnel Vestibule)								
1022 (Stair #1)								
1023 (Stair #2)								
1025 (Stair #3)								
1026 (Stair #4)								
1027(Stair #5)								
1028 (Freight Elevator)			**7000 lb Freight Elevator** • **50kVA** **1 motor**					
1028A (Vestibule)			~~Overhead Door~~ • ~~1 motor @ 2hp~~ ~~Elevator Door~~ • ~~1 motor @ 2hp~~					
1029 (Elevator Lobby)			~~Elevator Door~~ • ~~1 motor @ 2hp~~					
1030 (Fire Water Rinser Valve #1)								
1031 (Fire Water Rinser Valve #2)								
1200 (Entry/Exit Vestibule)								
1201A (Entry Lobby)								
1201B (Corridor)								
1202 (Security Post)								
1203 (RA Control Post)								
1204 (Mens Locker)		~~1 Exhaust Fan~~ ~~200-VNI0-EXH-00002~~ • ~~0.5 HP (ea.)~~						
1205 (RA Exit Vestibule)								
1206 (Women's Locker)		~~1 Exhaust Fan~~ ~~200-VNI0-EXH-00003~~ ~~0.5 HP (ea.)~~						
1207 (Operations Room)							**10% of all such equipment** **1 point**	

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
1208 (Communications Rm.)	**6 Equipment Racks**						**10% of all such equipment 1 point**	
1209 (RP Staff Work Room)							**20% of all such equipment 2 points**	
1210 (Briefing/ Break Rm.)							**20% of all such equipment 2 points**	
1211 (Janitor Closet)		~~1 Exhaust Fan 200-VNI0-EXH-00001 0.5 HP (ea.)~~						
1212 (RP Gear Supply Room)							**10% of all such equipment 1 point**	
1212A (RA Entrance Vestibule)								
1213 (Change Room 1)								
1214 (Change Room 2)								
1215 (RP Equipment Room)								
1216 (Respirator Room)								
1217 (Corridor)								
1218 (RP Lab / Count Room)							**10% of all such equipment 1 point**	
1219 (RP Lab/SamplePrep Rm.)							**10% of all such equipment 1 point**	
1220 (Decon Room)							**10% of all such equipment 1 point**	
1221 (RA Exit/PCM Room)								
1222 (Janitor Closet)								
1223 (Gas Sampling Room)			**Cask Cavity Gas Sample System** 200-MRE0-DET-00001 • **1 motor**					
1224 (RP Instrument Room)								
2001 (Ops/Maint.Storage Room)								
2002A (Corridor)								
2002B (Corridor)								

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
2002C (Corridor)								
2002D (Corridor)								
2002E (Corridor)								
2002F (Corridor)								
2002G (Corridor)								
2003 (HVAC Room)		**2 Air Handling Units** 200-VCT0-AHU-00001 200-VCT0-AHU-00002 • **125 hp (ea.)**			**Portable Welding Receptacle – WWF = 5 point**			**5.6% of all such eq.** **2 points**
2004 (HVAC Room)		**1 Air Handling Unit** 200-VCT0-AHU-00003 • **125 hp**						**5.6% of all such eq.** **2 points**
2005 (Instrument and Elec. Shop)					**Portable Welding Receptacle – WWF = 5 point**			
2006 (HVAC Room)		**3 Exhaust Fans** 200-VCT0-EXH-00001 200-VCT0-EXH-00002 200-VCT0-EXH-00013 • **75 hp (ea.)** **3 HEPA Filter Units** (hp n/a) 200-VCT0-FLT-00001 200-VCT0-FLT-00002 200-VCT0-FLT-00013						**5.6% of all such eq.** **2 points**
2007 (Canister Transfer Room)			**CTM Maintenance Crane** 200-HTC0-CRN-00001 • **44.5 hp** • **2 + 1 motors @ 35, 2, & 7.5 hp** • **62. kVA** **Canister Trans. Machine** 200-HTC0-FHM-00001 • **120.5 hp** • **5 + 1 motors @ 45, 3, 7.5, 7.5, 60, & 5 hp** • **133kVA (ea.)** ~~AO/STC Port Slide Gate~~ ~~200-HTC0-HTCH-00002~~ ~~2 motors @ 0.5 hp~~ ~~Cask Port Slide Gate~~ ~~200-HTC0-HTCH-00001~~ ~~2 motors @ 0.5 hp~~					**2.8% of all such eq.** **1 point**
2008 (HVAC Room)		**2 Air Handling Units** 200-VNI0-AHU-00001 200-VNI0-AHU-00002 • **40 hp (supply)** • **20 hp (return)**						**5.6% of all such eq.** **2 points**
2009 (HVAC Room)		**1 Air Handling Unit** 200-VCT0-AHU-00004 • **100 hp**						**5.6% of all such eq.** **2 points**

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
2010 (HVAC Room)		**2 Air Handling Units** 200-VCT0-AHU-00005 200-VCT0-AHU-00006 • **100 hp (ea.)**			**Portable Welding Receptacle – WWF = 5 points**			**5.6% of all such eq. 2 points**
2011 (HVAC Room)								**5.6% of all such eq. 2 points**
2012 (Receiver / Dryer Equipment Room)	**480V Load Center** 200-EEN0-LC-00002 • **5 cabs** **480V MCCs** 200-EEN0-MCC-00007 • **6 cabs** 200-EEN0-MCC-00008 • **7 cabs** **1 480kVA Distribution Xfmr** 200-EEN0-XFMR-00005 **1 208/120V Distribution Panel** 200-EEN0-PL-00005 **1 480/277V Lighting Panels** 200-EUL0-PL-00007				**Portable Welding Receptacle – WWF = 5 points**			
2022 (Stair #1)								
2023 (Stair #2)								
2025 (Stair #3)								
2026 (Stair #4)								
2027 (Stair #5)								
2029 (Elevator Lobby)			~~Elevator Door~~ ~~1 motor @ 2hp~~					

Table F5.4-1. Ignition Source Population by Room (Continued)

Ignition Source / Room Number	Electrical Equipment	Mechanical/Electrical HVAC Equipment	Mechanical Process Equipment	Heat Generating Process Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/Kitchen Equipment	Portable and Special Equipment
3001 (Corridor)								
3026 (Stair #4)								
3029 (Elevator Lobby)			~~Elevator Door 1 motor @ 2hp~~					

NOTE: 1.The equipment shown shaded in grey is included on the table to show completeness in the process of identifying equipment and locations. However, in accordance with the counting guidance cited in the methodology section these pieces of equipment are not considered as ignition sources because they are motors of less than 5 hp.
2.In accordance with the counting guidance, the cabinet count for each MCC is for energized cabinets only (i.e., cabinets that have a load assigned). De-energized (i.e., spare) cabinets are not counted.
3.RWF is room weighting factor for equipment that can be in multiple rooms. Factor represents the percentage of exposure (i.e., waste residence) time that the piece of equipment spends in the particular room.
4.WWF is the welding weighting factor, which represents the relative number of total welding activity (hours/year) that occurs in each location where welding is performed. The number of hours for maintenance-related welding is based on about 8 hours/week in the primary maintenance welding location and 5 hours per year in each satellite welding location (for repairs that must be performed locally). Waste package closure room welding is estimated based in the IHF throughput Gantt chart and the total number of waste packages expected to be handled, as follows: (1) the preclosure period is 50 years; (2) the welding machine actually operates for 13 hours per waste package; (3) here are three CRCFs, each with two closure welding machines, both of which are in the same room. Since they are both in the same room, the welding score for the room is 1/3 of the CRCF total; (4) the three CRCFs combined will process 10,911 waste packages. (10,911x13/50)/3 = 946 hours per year (both machines combined, 472 hrs/machine). Note that for any given waste package being processed, the total welding score is "at" the WP.
5.Power ratings are for each motor unless otherwise noted.

cabs = cabinet; DCIMS = digital control and management information system; Dist. = distribution; HEPA = high-efficiency particulate air (filter); hp = horsepower; HVAC = heating, ventilation, and air conditioning; ITS = important to safety; kVA = kilo-volt amperes; LLW = low-level radioactive waste; MCC = motor control center; PLC = programmable logic controller; RA = radiological access; RP = radiological protection; RWF = residence weighting factor; UPS = uninterruptable power supply; V = volt; WWF = welding weighting factor; Xfmr = transformer

Source: Original

F5.4.1 Electrical Equipment

Information regarding electrical equipment was gathered solely from the following single line diagrams and layout drawings:

Receipt Facility General Arrangement Ground Floor Plan (Ref. F2.21)

Receipt Facility General Arrangement Second Floor Plan (Ref. F2.22)

Receipt Facility 480V Load Center 200-EEN0-LC-00001 Single Line Diagram (Ref. F2.43)

*Receipt Facility 480V MCC 200-EEN0-MCC-00001 Single Line Diagra*m (Ref. F2. 45)

Receipt Facility 480V MCC 200-EEN0-MCC-00002 Single Line Diagram (Ref. F2.46)

Receipt Facility 480V MCC 200-EEN0-MCC-00003 Single Line Diagram (Ref. F2.47)

Receipt Facility 480V MCC 200-EEN0-MCC-00004 Single Line Diagram (Ref. F2.48)

Receipt Facility 480V ITS MCC Train A 200-EEE0-MCC-00001 Single Line Diagram (Ref. F2.39)

Receipt Facility 480V ITS MCC Train B MCC 200-EEE0-MCC-00002 Single Line Diagram (Ref. F2.40)

Receipt Facility ITS UPS Train A 200-EEU0-UJX-00001 Single Line Diagram (Ref. F2.30)

Receipt Facility ITS UPS Train B 200-EEU0-UJX-00002 Single Line Diagram (Ref. F2.31)

Receipt Facility UPS 200-EEP0-UJX-00001 Single Line Diagram (Ref. F2.32).

The electrical equipment category consists of computers, equipment racks, load centers, motor control centers (MCCs), uninterruptable power supply, transformers, lighting panels, digital control and management information system, programmable logic controller panels, batteries, and electrical panels. In general, each piece of electrical equipment constitutes a single ignition source and therefore has a count of one. However, MCCs, load centers, and equipment racks are assigned a count based on the total number of active vertical cabinets making up the overall unit. Every vertical cabinet in an equipment rack is active. In the case of MCCs and load centers, a cabinet is considered active if the single line diagram shows that a load is attached (i.e., unused breakers are not counted).

F5.4.2 HVAC Equipment

HVAC equipment locations and horsepower were obtained from the following facility general layout drawings and HVAC equipment lists:

Receipt Facility Composite Vent Flow Diagram Tertiary Confinement Non-ITS HVAC Supply Sys & ITS Exhaust (Ref. F2.13)

Receipt Facility Composite Vent Flow Diagram Tertiary Confinement Non-ITS HVAC Supply & Exhaust System (Ref. F2.12)

Receipt Facility Composite Vent Flow Diagram Tertiary Conf ITS HVAC Systems, Elect & Battery RMS (Ref. F2.11)

Receipt Facility Composite Vent Flow Diagram Non-Confinement Non-ITS HVAC Sys Support & Operations (Ref. F2.10)

Receipt Facility ITS Confinement Areas HEPA Exhaust System – Train A Ventilation & Instrumentation Diagram (Ref. F2.27)

Receipt Facility ITS Confinement Areas HEPA Exhaust System – Train B Ventilation & Instrumentation Diagram (Ref. F2.28)

Receipt Facility ITS Confinement Areas HVAC Supply System Ventilation & Instrumentation Diagram (Ref. F2.29)

Receipt Facility Confinement South Areas HVAC Supply System Ventilation & Instrumentation Diagram (Ref. F2.19)

Receipt Facility Confinement Non-ITS HEPA Exhaust System Ventilation & Instrumentation Diagram (Ref. F2.18)

Receipt Facility Confinement 2nd Floor North Areas HVAC Supply System Ventilation & Instrumentation Diagram (Ref. F2.20)

Receipt Facility Confinement ITS Electrical Room HVAC System – Train A Ventilation & Instrumentation Diagram (Ref. F2.16)

Receipt Facility Confinement ITS Battery Room Exhaust System – Train A Ventilation & Instrumentation Diagram (Ref. F2.14)

Receipt Facility Confinement ITS Electrical Room HVAC System – Train B Ventilation & Instrumentation Diagram (Ref. F2.17)

Receipt Facility Confinement ITS Battery Room Exhaust System – Train B Ventilation & Instrumentation Diagram (Ref. F2.15)

Receipt Facility Non-Confinement Areas HVAC Supply System Ventilation & Instrumentation Diagram (Ref. F2.34)

Receipt Facility Transportation Cask Vestibule Non-Confinement HVAC System Ventilation & Instrumentation Diagram (Ref. F2.38)

Receipt Facility Site Transporter Vestibule Non-Confinement HVAC System Ventilation & Instrumentation Diagram (Ref. F2.37)

Receipt Facility Site Transp Cask Vestibule Annex Non-Confinement HVAC System Ventilation & Instrumentation Diagram (Ref. F2.36)

Receipt Facility LLW Vestibule Non-Confinement HVAC System Ventilation & Instrumentation Diagram (Ref. F2.33).

HVAC equipment consists of HEPA filters, exhaust fans, air handling units, fan coil units, and sump pumps. Because any motor with a horsepower rating of 5 or more is considered to be an initiator, the number of motors and the horsepower of each motor is determined for all applicable HVAC equipment identified. A piece of equipment containing motors is assigned a count based on the number of motors with a horsepower of 5 or more. Because HEPA filter units are not applicable to this process, a count of one is assigned for each.

F5.4.3 Mechanical Process Equipment

Information regarding mechanical process equipment locations and horsepower were obtained from the following facility general layout drawings, mechanical equipment lists, and equipment piping & instrument diagram (P&ID) drawings.

Receipt Facility General Arrangement Ground Floor Plan (Ref. F2.21)

Receipt Facility General Arrangement Second Floor Plan (Ref. F2.22)

Receipt Facility General Arrangement Third Floor Plan (Ref. F2.23)

Equipment Motor Horsepower and Electrical Requirements Analysis (Ref. F2.4)

CRCF, RF, WHF, and IHF Cask Transfer Trolley Process and Instrumentation Diagram (Ref. F2.3)

Receipt Facility Chilled Water System Piping & Instrument. Diagram (Ref. F2.7)

Receipt Facility Chilled Water System Piping & Instrument. Diagram (Ref. F2.8)

Receipt Facility Chilled Water System Piping & Instrument. Diagram (Ref. F2.9)

Receipt Facility Hot Water System Piping & Instrument. Diagram (Ref. F2.24)

Receipt Facility Hot Water System Piping & Instrument. Diagram (Ref. F2.25)

Receipt Facility Hot Water System Piping & Instrument. Diagram (Ref. F2.26)

Receipt Facility Cask Cavity Gas Sampling System Piping & Instrument. Diagram (Ref. F2.6).

Mechanical process equipment includes most of the motorized equipment to include cranes, trolleys, doors, and platforms. These are counted in the method described in section F5.4.2 (each motor of 5 horsepower or more contributes a count of one). Because some of the equipment in this category is mobile, and counts are done for each room individually, it was necessary to consider the counts for equipment which can occupy more than one room. To accomplish this, the amount of time a piece of equipment spends in each room was identified using the process throughput Gantt charts (Ref. F2.5). The cask transfer trolley (CTT) was identified as the only piece of mobile equipment that occupies more than one room.

The total time the CTT spends in the Cask Unloading Room (1015) is calculated from the following procedures identified in the process throughput:

- 1.3.13 Move Transportation Cask into Cask Unloading Room – 20 minutes
- 2.1 Move TAD To Aging Overpack – 243 minutes
- 1.6.1 Move Transportation Cask into Cask Preparation Room – 20 minutes

The total time the CTT spends in the Cask Preparation Room (1017) is calculated by subtracting the total amount of time the CTT will be in room 1015 from the total time of the procedure (8,345 minutes).

The times a mobile equipment item spends in each room is utilized to determine the percentage of time the equipment occupies a room, which directly corresponds to the percentage of the total count assigned to that room. This is represented on the equipment list as the residence weighting factor (RWF).

F5.4.4 Heat Generating Process Equipment

This equipment refers to such things as furnaces, dryers, and other such equipment except for those associated with the HVAC, which are counted separately as discussed above. There is no equipment for any of the facilities that falls under this category.

F5.4.5 Torches, Welders, and Burners

Welding operations are the only contributors to this category. The assignment of residency in this case is based on the estimated number of hours per year that welding operations are expected to occur in the area. This provides a suitable relative weight for apportioning fire ignition caused by welding operations. Portable welding receptacles are provided in various areas of the facility for the purpose of occasional welding of stationary equipment that may require repair. These are provided for convenience, and are not expected to see significant use. Each station is estimated to see on the order of five hours of use per year, and so is assigned a score of five points each. The primary maintenance area also contains a welding receptacle (the "primary welding station"), intended to perform all of the maintenance related welding for repair and fabrication that does not require direct work on a stationary piece of equipment (including on components of stationary pieces of equipment that are easily removed). The primary welding station is estimated to be utilized about eight hours per week, and so is assigned a score of 400 points.

The locations of portable welding receptacles were determined as an engineering judgment on the part of the design team based on preliminary electrical and general layout drawings. The resultant fire initiating event frequencies are insensitive to the precise distribution of the portable welding receptacles, so a more rigorous analysis of the distribution is not required.

F5.4.6 Internal Combustion Engines

There are two transporters that utilize internal combustion engines in the RF, which provide the entire contribution of fire ignition to the internal combustion engines category. The site transporter and site prime mover are assigned a total of 100 points each. The points are allocated to the rooms where these vehicles could be located by use of a RWF, as discussed in section F5.4.3.

The site transporter occupies rooms 1001 (Site Transporter Vestibule), 1002 (Lid Bolting Room), and 1013 (Loading Room). The times necessary to determine the percentage of time the site transporter spends in each room are given in sections 1.4, 2.1, and 1.5 of the RF process throughput diagram. There are a total of 68 minutes that are assigned to two rooms because the doors between them are open. Resultant times are 56 minutes in the Site Transporter Vestibule (1001), 486 minutes in the Lid Bolting Room (1002), and 283 minutes in the Loading Room (1013).

The site prime mover/tractor occupies rooms 1017/1017A (Cask Preparation Room), 1021 (Transportation Cask Vestibule Annex), and 1021A (Transportation Cask Vestibule). The times necessary to determine the percentage of time the prime mover/tractor spends in each room are given in Section 1.1.1 of the RF process throughput diagram. There are 36 total minutes that are assigned to two or more rooms because the doors between them are open. Resultant times are 38 minutes in the Transportation Cask Vestibule (1021A), 36 minutes in the Transportation Cask Vestibule Annex (1021), and 40 minutes in the Cask Preparation Room (1017/1017A).

The times internal combustion engines spend in each room is utilized to determine the percentage of time the engine occupies a room, which directly corresponds to the percentage of the total count assigned to that room. This is represented on the equipment list as the RWF.

Locations of the internal combustion engines were determined solely from the general layout drawings.

F5.4.7 Office/Kitchen Equipment

This category consists of miscellaneous office and kitchen equipment such as: shredders, vending machines, microwaves, computers, radios, and printers. The location and quantity of such equipment was inferred by the description and layout of the rooms to come up with a reasonable distribution of such equipment in the facility. Work rooms, break rooms, briefing rooms, and offices were considered to possess such equipment. A judgment was made by the analysis team based on the function and size of the room as to how much of such equipment might reside in these rooms. Points were assigned to each room expected to contain office or kitchen equipment based on this judgment (one point per room). The resultant fire initiating event frequencies are quite insensitive to the precise distribution of this equipment, so a more rigorous analysis of the distribution is not required.

Locations of the office and kitchen equipment were determined solely from the general layout drawings.

F5.4.8 Portable and Special Equipment

This category consists of portable hand tools, monitoring devices, portable heaters, diagnostic equipment, and the like. Rooms where there were significant amounts of equipment that would expect to be maintained on a regular basis or where monitoring would take place were considered to possess such equipment. Determinations for the portable and special equipment category were inferred from the description and layout of the rooms, as described in Section F5.4.7. Each room containing such equipment was assigned one to two points, depending on the quantity expected in that room. The resultant fire initiating event frequencies are quite insensitive to the precise distribution of this equipment, so a more rigorous analysis of the distribution is not required.

F5.5 Room Ignition Frequency

Ignition Frequencies for each room are determined as a function of the number of units of ignition sources in the room, and the area of the room. The spreadsheet used to determine these frequencies is displayed as Table F5.5-1.

The major input to the spreadsheet is the number of units per category for each room (green text). These values are taken from the equipment list Table (F5.4-1), which is formulated from equipment and general layout drawings, and equipment lists (Section F5.4). The total number of units in each category is the result of a sum across all rooms, and can be found in the bottom total row. It is this value that is used in Table F5.3-1 in the column entitled "Category Population" for all categories except no equipment involved, as explained in Section F5.3.

The "No Equipment Involved" column of Table F5.5-1 is the area of the rooms, as a unit in this category is represented by a single square meter. These values are taken from Table F5.2-1, in the column entitled A (sq-m).

The final column on Table F5.5-1, entitled "Room Ignition Frequency," implements the generic forms of equations F-5 and F-6. It calculates the room ignition frequency, which utilizes the frequency per unit from section F5.3. It takes the required per unit ignition frequencies directly from the spreadsheet represented by Table F5.3-1, the column entitled "Frequency per Unit". Per Equation F-5, the number of units in each category (green text) is multiplied by the corresponding frequency per unit for that category. Per Equation F-6, summing these multiplications across a row provides the room ignition frequency for that room. The sum of all rooms is the building ignition frequency. This value is shown in the lower right hand column of the Table. Note that this value does not match the value shown at the bottom of Table F5.2-1. That value, which is based only on building area, pre-supposes that the ignition sources in the building cover each of entire ignition source categories used in the analysis. However, the RF does not have any equipment that fits the definition of heat generating equipment (welders have their own category), so this contribution does not apply to RF.

Table F5.5-1. Fire Ignition Frequencies by Room

	Ignition Source Category and Room-by-Room Population									
Room	Electrical	HVAC	Mechanical Equipment	Heat Generating Equipment	Torches, welders, burners	Internal combustion engines	Office/ kitchen equipment	Portable Equipment	No equipment involved	Room Ignition Frequency
1001						7			167	6.4E-03
1002			3			59			368	6.0E-02
1003A									40	1.1E-03
1003B									76	2.1E-03
1003C									53	1.4E-03
1003D									140	3.8E-03
1003E									133	3.6E-03
1003F									67	1.8E-03
1003G									45	1.2E-03
1004		4			5			4	261	6.6E-02
1004A		4						2	99	4.1E-02
1005	23	2							235	5.1E-02
1005A	1								20	2.0E-03
1011									98	2.6E-03
1012					5				296	1.4E-02
1013			2			34			175	3.7E-02
1014			4		5				141	5.5E-02
1015			2.03					1	156	3.5E-02
1016									126	3.4E-03
1017/1017A			8.97		400	35		4	1993	7.1E-01
1018	80				5			2	256	1.4E-01
1018A	2								51	4.2E-03
1019		4						4	265	6.0E-02
1019A		4						2	70	4.0E-02
1020	23	2						2	237	6.5E-02
1020A	1								22	2.0E-03
1021			1			33			191	2.5E-02
1021A		2	2			32			349	5.2E-02
1021B									12	3.3E-04
1022									51	1.4E-03

Table F5.5-1. Fire Ignition Frequencies by Room (Continued)

Room	Electrical	HVAC	Mechanical Equipment	Heat Generating Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/ Kitchen Equipment	Portable Equipment	No Equipment Involved	Room Ignition Frequency
1023									54	1.5E-03
1025									56	1.5E-03
1026									40	1.1E-03
1027									30	8.2E-04
1028			1						75	1.3E-02
1028A									51	1.4E-03
1029									42	1.1E-03
1030									30	8.2E-04
1031									32	8.6E-04
1200									8	2.0E-04
1201A									47	1.3E-03
1201B									100	2.7E-03
1202									21	5.7E-04
1203									46	1.3E-03
1204									35	9.5E-04
1205									8	2.3E-04
1206									36	9.8E-04
1207							1		68	1.8E-02
1208	6						1		51	2.7E-02
1209							2		54	3.5E-02
1210							2		57	3.5E-02
1211									35	9.4E-04
1212							1		39	1.8E-02
1212A									7	2.0E-04
1213									13	3.4E-04
1214									13	3.4E-04
1215									30	8.1E-04
1216									16	4.3E-04
1217									38	1.0E-03
1218							1		21	1.7E-02
1219							1		21	1.7E-02
1220							1		32	1.7E-02
1221									48	1.3E-03
1222									4	1.1E-04

Table F5.5-1. Fire Ignition Frequencies by Room (Continued)

Room	Electrical	HVAC	Mechanical Equipment	Heat Generating Equipment	Torches, Welders, Burners	Internal Combustion Engines	Office/ Kitchen Equipment	Portable Equipment	No Equipment Involved	Room Ignition Frequency
1223			1						34	1.2E-02
1224									73	2.0E-03
2001									167	4.5E-03
2002A									69	1.9E-03
2002B									132	3.6E-03
2002C									17	4.5E-04
2002D									87	2.4E-03
2002E									182	4.9E-03
2002F									60	1.6E-03
2002G									17	4.5E-04
2003		2			5			2	334	4.2E-02
2004		1						2	259	2.8E-02
2005					5				333	1.6E-02
2006		6						2	296	5.8E-02
2007			7					1	1444	1.3E-01
2008		2						2	267	3.4E-02
2009		1						2	308	2.9E-02
2010		2			5			2	334	4.2E-02
2011								2	259	2.2E-02
2012	21				5				333	4.5E-02
2022									54	1.5E-03
2023									54	1.5E-03
2025									55	1.5E-03
2026									40	1.1E-03
2027									38	1.0E-03
2029									42	1.1E-03
3001									24	6.6E-04
3026									40	1.1E-03
3029									42	1.1E-03
TOTAL	157	36	32	0	440	200	10	36		2.2E+00

NOTE: HVAC = heating, ventilation, and air conditioning.

Source: Original

F5.6 Propagation Probabilities

Propagation probabilities are utilized in this analysis to define the probability of a fire spreading to various defined points. The first two columns of Table F5.6-1 define the maximum extent of propagation, and the conditional probability column is the probability associated with that extent of propagation. The remaining columns in Table F5.6-1 are utilized in the uncertainty distribution for the conditional probability. The structure of this spreadsheet is analogous to Table F5.3-1. The right hand group of columns is used by Crystal Ball to apply an uncertainty distribution to each of the propagation probability values for the purpose of developing uncertainty distributions on initiating event frequency. The mean fraction, 97.5%, and 97.5th percentile add columns show the parameters of these distributions. The development of all of the values is detailed in Appendix F.II. When Crystal Ball is run, it creates a sampled value for each fraction in the sampled value column. The spreadsheet then determines a normalized value by first assuring that each sampled value is not negative (minimum value of zero) and then normalizing the values so that the sum is always equal to one. The normalized value for each trial then replaces the category fraction value in the calculation. These probabilities must always add to one, as the groupings include all possible propagation outcomes.

Table F5.6-1. Fire Propagation Probabilities

Automatic Suppression Functional		Conditional Probability			Sampled Value	Mean Fraction	97.5% Value	97.5th percentile add
Extent of Propagation	Alternative Definition							
Confined to Object of Origin	No Propagation	0.551		0.551	0.551	0.551	0.667	0.117
Confined to Part of Room of Origin	Spreads Through Part of Room of Origin	0.317		0.317	0.317	0.317	0.426	0.109
Confined to Room of Origin	Spreads Throughout Room of Origin	0.028		0.028	0.028	0.028	0.066	0.038
Confined to Fire-Rated Area of Origin	Spreads Throughout Fire-Rated Area of Origin	0.005		0.005	0.005	0.005	0.020	0.016
Confined to Floor of Origin	Spreads Throughout Floor of Origin	0.069		0.069	0.069	0.069	0.128	0.059
Confined to Structure of Origin	Spreads Throughout Building	0.028		0.028	0.028	0.028	0.055	0.028
Extended Beyond Structure of Origin	Breaches Building Boundary	0.005		0.005	0.005	0.005	0.020	0.016
		1.000		1.000				
Automatic Suppression Fails								
Extent of Propagation	Alternative Definition							
Confined to Object of Origin	No Propagation	0.621		0.621	0.621	0.621	0.725	0.104
Confined to Part of Room of Origin	Spreads Through Part of Room of Origin	0.149		0.149	0.149	0.149	0.226	0.076
Confined to Room of Origin	Spreads Throughout Room of Origin	0.004		0.004	0.004	0.004	0.017	0.013
Confined to Fire-Rated Area of Origin	Spreads Throughout Fire-Rated Area of Origin	0.057		0.057	0.057	0.057	0.107	0.050
Confined to Floor of Origin	Spreads Throughout Floor of Origin	0.004		0.004	0.004	0.004	0.017	0.013
Confined to Structure of Origin	Spreads Throughout Building	0.161		0.161	0.161	0.161	0.240	0.079
Extended Beyond Structure of Origin	Breaches Building Boundary	0.004		0.004	0.004	0.004	0.017	0.013
		1.000		1.000				

Source: Original

F5.7 Initiating Event Frequencies

Initiating event frequencies are the final results of the fire hazard analysis, and are a factor of all of the previously discussed data and residence fractions. The following sections shall describe the culmination of this data to conclude with initiating event frequencies.

F5.7.1 Residence Fractions

Residence fractions have been developed from process throughputs to determine the length of time a waste form will be vulnerable in a particular area of the building and in a particular configuration. The source for all of the times related to transportation, aging, and disposal (TAD) canisters and dual-purpose canisters (DPCs) is the RF throughput study (Ref. F2.5). Table F5.7-1 shows the vulnerabilities for the TAD canister, and the times that contribute to the overall time of vulnerability. The column labeled BFD Task refers to the task number from the process block flow diagram that was used in the throughput study. These numbers appear directly on the Gantt charts and provide a reference for the task that was considered. The total shows the total number of minutes that the waste form was in the specified configuration in the specified location. The fraction column implements the approach discussed in Section F4.4.1 to calculate the fraction of time that a specific waste form spends in the particular configuration and location over the 50-year pre-closure period. Similarly to the TAD canister residence fractions, the process throughputs have been utilized to determine residence fractions for DPC (TTC and VTC; Table F5.7-2), and DPC (HTC; Table F5.7-3).

Table F5.7-1. TAD Residence Fractions

RF Residence Times and Fractions					
Section I - Localized Fires					
BFD Task	Steps (if needed)	Time (m)	Fraction		
TC/TAD on Railcar/Trailer in Vestibule/Prep Area w/SPM/Truck (Diesel Present)					
1.1.1		56			
Total		**56**	**2.1E-06**		
TC/TAD on Railcar/Trailer in Prep Area w/o SPM/Truck (No Diesel Present)					
1.1.4		134			
Not in BFD	Visual inspection	55			
1.1.5		83			
1.1.6		90			
1.1.7		55			
1.3.1	Steps 1-2	15			
Total		**432**	**1.6E-05**		
TC/TAD on CTT in Prep Area					
1.3.1	Steps 3-6	35			
1.3.2		5			
1.3.3		108			
1.3.13		20			
Total		**168**	**6.4E-06**		
TC/TAD on CTT in Unloading Room					
1.3.13	again	20			
2.1.5		6			
2.1.6		1			
2.1.7		17			
2.1.8		22			
2.1.9		6			
2.1.10		1			
2.1.11	Steps 1-3	20			
Total		**93**	**3.5E-06**		
TAD in CTM in Transfer Room					
2.1.11	Step 3-4 (again)	15			
2.1.12		1			
2.1.13		5			
2.1.14		1			
2.1.15	Step 1	10			
Total		**32**	**1.2E-06**		

Table F5.7-1. TAD Residence Fractions (Continued)

BFD Task	Steps (if needed)	Time (min)	Fraction
TAD in AO in Loading Room (Diesel)			
2.1.15	again	21	
2.1.16		1	
2.1.17		22	
2.1.18		5	
2.1.19		17	
2.1.20		1	
1.5.1		20	
Total		**87**	**3.3E-06**
TAD in AO in Lid Bolting Room (Diesel)			
1.5.1	Again	20	
1.5.2		242	
Not in BFD	Rad Inspection	30	
1.5.3	Steps 1-5	26	
Total		**318**	**1.2E-05**
Section II - Large Fire			
TC/TAD w/SPM/Truck Present (Diesel)			
1.1.1		56	
Total		**56**	**2.1E-06**
TC/TAD w/o SPM/Truck Present (No Diesel)			
1.1.4		134	
Not in BFD	Visual inspection	55	
1.1.5		83	
1.1.6		90	
1.1.7		55	
1.3.1	Steps 3-6	50	
1.3.2		5	
1.3.3		108	
1.3.13		20	
2.1.5		6	
2.1.6		1	
2.1.7		17	
2.1.8		22	
2.1.9		6	
2.1.10		1	
2.1.11	Steps 1-3	20	
Total		**673**	**2.6E-05**

Table F5.7-1. TAD Residence Fractions (Continued)

BFD Task	Steps (if needed)	Time (min)	Fraction
TAD in CTM			
2.1.11	Step 3-4 (again)	15	
2.1.12		1	
2.1.13		5	
2.1.14		1	
2.1.15	Step 1	10	
Total		**32**	**1.2E-06**
TAD in AO (Diesel Present)			
2.1.15	again	21	
2.1.16		1	
2.1.17		22	
2.1.18		5	
2.1.19		17	
2.1.20		1	
1.5.1		20	
1.5.2		242	
Not in BFD	Rad Inspection	30	
1.5.3	Steps 1-5	26	
Total		**385**	**1.5E-05**

NOTE: AO aging overpack; BFD = block flow diagram; CTT = cask transfer trolley; RF = Receipt Facility; SPM = site prime mover; TAD = transportation, aging, and disposal canister; TC = transportation cask.

Source: Original

Table F5.7-2. DPC (TTC & VTC) Residence Fractions

BFD Task	Steps (if needed)	Time (m)	Fraction		BFD Task	Steps (if needed)	Time (m)	Fraction	
TC/DPC (TTC) on Railcar/Trailer in Vestibule/Prep Area w/SPM/Truck (Diesel Present)					**TC/DPC (VTC) in Vestibule/Prep Area w/SPM/Truck (Diesel Present)**				
1.1.1		56			1.1.1		56		
Total		**56**	**2.1E-06**		**Total**		**56**	**2.1E-06**	
TC/DPC (TTC) on Railcar/Trailer in Prep Area w/o SPM/Truck (No Diesel Present)					**TC/DPC (VTC) on Railcar/Trailer in Prep Area w/o SPM/Truck (No Diesel Present)**				
1.1.8		83			1.1.4		138		
1.1.9		100			Not in BFD	Inspections and Surveys	55		
1.1.10		135			1.1.5		83		
Not in BFD	Inspections and Surveys	55			1.1.7		55		
1.1.11		105			1.3.1	Steps 1-2	15		
1.1.12		36			**Total**		**346**	**1.3E-05**	
1.1.13		70							
1.1.14		30							
1.3.1	Steps 1-2	15							
Total		**629**	**2.4E-05**						
TC/DPC (TTC) on CTT in Prep Area					**DPC (VTC) Same as TTC**				
1.3.1	Steps 3-8	50							
1.3.2		5							
1.3.3		108							
1.3.4		5							
1.3.5		35							
1.3.6		5							
1.3.7		40							
1.3.8		5							
1.3.9		50							
1.3.10		40							
1.3.11		5							
1.3.12		20							
1.3.13		20							
Total		**388**	**1.5E-05**						
TC/DPC (TTC) in CTT in Unloading Room					**DPC (VTC) Same as TTC**				
1.1.13	Again	20							
2.1.5		6							
2.1.6		1							
2.1.11	Steps 1-3	20							
Total		**47**	**1.8E-06**						

Table F5.7-2. DPC (TTC & VTC) Residence Fractions (Continued)

BFD Task	Steps (if needed)	Time (m)	Fraction		BFD Task	Steps (if needed)	Time (m)	Fraction
TC/DPC (TTC) in CTM in Transfer Room					**DPC (VTC) Same as TTC**			
2.1.11	Step 3-4 (again)	15						
2.1.12		1						
2.1.13		5						
2.1.14		1						
2.1.15	Step 1	10						
Total		**32**	**1.2E-06**					
TC/DPC (TTC) in AO in Loading Room (Diesel)					**DPC (VTC) Same as TTC**			
2.1.15	again	21						
2.1.16		1						
2.1.17		22						
2.1.18		5						
2.1.19		17						
2.1.20		1						
1.5.1		20						
Total		**87**	**3.3E-06**					
TC/DPC (TTC) in AO in Lid Bolting Room (Diesel)					**DPC (VTC) Same as TTC**			
1.5.1	Again	20						
1.5.2		242						
Not in BFD	Rad Inspection	30						
1.5.3	Steps 1-5	26						
Total		**318**	**1.2E-05**					
Section II - Large Fire								
TC/DPC (TTC) w/SPM/Truck (Diesel Present)					**TC/DPC (VTC) in Vestibule/Prep Area w/SPM/Truck (Diesel Present)**			
1.1.1		56			1.1.1		56	
Total		**56**	**2.1E-06**		**Total**		**56**	**2.1E-06**

Table F5.7-2. DPC (TTC & VTC) Residence Fractions (Continued)

BFD Task	Steps (if needed)	Time (m)	Fraction		BFD Task	Steps (if needed)	Time (m)	Fraction
TC/DPC (TTC) w/o SPM/Truck (No Diesel)					**TC/DPC (VTC) w/o SPM/Truck (No Diesel Present)**			
1.1.8		83			1.1.4		138	
1.1.9		100			Not in BFD	Inspections and Surveys	55	
1.1.10		135			1.1.5		83	
Not in BFD	Inspections and Surveys	55			1.1.7		55	
1.1.11		105			1.3.1	Steps 1-2	15	
1.1.12		36			1.3.1		65	
1.1.13		70			1.3.2		5	
1.1.14		30			1.3.3		108	
1.3.1		65			1.3.4		5	
1.3.2		5			1.3.5		35	
1.3.3		108			1.3.6		5	
1.3.4		5			1.3.7		40	
1.3.5		35			1.3.8		5	
1.3.6		5			1.3.9		50	
1.3.7		40			1.3.10		40	
1.3.8		5			1.3.11		5	
1.3.9		50			1.3.12		20	
1.3.10		40			1.3.13		20	
1.3.11		5			2.1.5		6	
1.3.12		20			2.1.6		1	
1.3.13		20			2.1.11	Steps 1-3	20	
2.1.5		6			**Total**		**776**	**3.0E-05**
2.1.6		1						
2.1.11	Steps 1-3	20						
Total		**1044**	**4.0E-05**					
TC/DPC (TTC) in CTM					**DPC (VTC) Same as TTC**			
2.1.11	Step 3-4 (again)	15						
2.1.12		1						
2.1.13		5						
2.1.14		1						
2.1.15	Step 1	10						
Total		**32**	**1.2E-06**					
TC/DPC (TTC) in AO (Diesel Present)					**DPC (VTC) Same as TTC**			
2.1.15	again	21						
2.1.16		1						
2.1.17		22						
2.1.18		5						
2.1.19		17						
2.1.20		1						
1.5.1		20						
1.5.2		242						
Not in BFD	Rad Inspection	30						
1.5.3	Steps 1-5	26						
Total		**385**	**1.5E-05**					

NOTE: AO = aging overpack; BFD = block flow diagram; CTT = cask transfer trolley; DPC = dual-purpose canister; m = minutes; SPM = site prime mover; TC = transportation cask; TTC = transportation cask in the tilted position; VTC = transportation cask in the vertical position.

Source: Original

Table F5.7-3. DPC (HTC) Residence Fractions

BFD Task	Steps (if needed)	Time (m)	Fraction		
TC/DPC (HTC) in Vestibule/Prep Area w/SPM/Truck (Diesel Present)					
1.1.1		56			
Not on Gantt	Move Outside Facilty	56			
Total		**112**	**4.3E-06**		
TC/DPC (HTC) on Railcar/Trailer in Prep Area w/o SPM/Truck (No Diesel Present)					
1.2.1		181			
1.2.2		50			
1.2.3		100			
1.2.4		265			
1.2.5		108			
Total		**704**	**2.7E-05**		
No other steps in processing of HTC DPCs					

BFD Task	Steps (if needed)	Time (m)	Fraction		
Section II - Large Fire					
TC/DPC (HTC) w/SPM/Truck (Diesel Present)					
1.1.1		56			
Not on Gantt	Move Outside Facilty	56			
Total		**112**	**4.3E-06**		
TC/DPC (HTC) w/o SPM/Truck (No Diesel Present)					
1.2.1		181			
1.2.2		50			
1.2.3		100			
1.2.4		265			
1.2.5		108			
Total		**704**	**2.7E-05**		
No other steps in processing of HTC DPCs					

NOTE: BFD = block flow diagram; DPC = dual-purpose canister; HTC=transportation cask in the horizontal position; m = minutes; SPM = site prime mover; TC = transportation cask.

Source: Original

F5.7.2 Localized Fires

Initiating event frequencies have been divided into two types of calculations; localized and large fires. Table F5.7-4 contains all of the calculations contributing to the localized fire initiating event frequencies.

Table F5.7-4. Localized Fire Initiating Event Frequencies

Localized Fires That Threatens Waste Form			0													
Contributions from Rooms Containing Waste Form																
Room of Origin (includes comments field as needed)	Ignition Source (If Applicable)	Number in Room	Frequency per Unit (50 years)	Number at Target	Number Near Target	Propagation Probability to Target	Number Away from Target	Propagation Probability to Target	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)
Entry represents a vulnerability due to the Site Transporter									TAD or DPC (TTC & VTC)							
1001 & 1002	Electrical	0	1.42E-03			0.211		0.061	1.2E-05	0.0E+00						
	HVAC	0	5.79E-03			0.211		0.061	1.2E-05	0.0E+00						
	Mechanical Equipment	3	1.13E-02	3		0.211		0.061	1.2E-05	4.1E-07						
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	1.2E-05	0.0E+00						
	Torches, welders, burners	0	1.29E-03			0.211		0.061	1.2E-05	0.0E+00						
	Internal combustion engines	66	2.73E-04	66		0.211		0.061	1.2E-05	2.2E-07						
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	1.2E-05	0.0E+00						
	Portable Equipment	0	7.37E-03			0.211		0.061	1.2E-05	0.0E+00						
	No equipment involved	534	2.71E-05	267	267	0.211		0.061	1.2E-05	1.1E-07						
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Vestibule/Lid Bolting Room (Diesel Present)										**7.3E-07**						
Entry represents a vulnerability due to the Site Transporter									TAD or DPC (TTC & VTC)							
1013 & 2007	Electrical	0	1.42E-03			0.211		0.061	3.3E-06	0.0E+00						
	HVAC	0	5.79E-03			0.211		0.061	3.3E-06	0.0E+00						
	Mechanical Equipment	9	1.13E-02	7		0.211	2	0.061	3.3E-06	2.7E-07						
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	3.3E-06	0.0E+00						
	Torches, welders, burners	0	1.29E-03			0.211		0.061	3.3E-06	0.0E+00						
	Internal combustion engines	34	2.73E-04	34		0.211		0.061	3.3E-06	3.1E-08						
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	3.3E-06	0.0E+00						
	Portable Equipment	0	7.37E-03			0.211		0.061	3.3E-06	0.0E+00						
	No equipment involved	1619	2.71E-05	175	120	0.211	1324	0.061	3.3E-06	2.5E-08						
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Loading Room (Diesel Present)										**3.2E-07**						

Table F5.7-4. Localized Fire Initiating Event Frequencies (Continued)

Room of Origin (includes comments field as needed)	Ignition Source (If Applicable)	Number in Room	Frequency per Unit (50 years)	Number at Target	Number Near Target	Propagation Probability to Target	Number Away from Target	Propagation Probability to Target	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)
Entry represents a vulnerability due to the Site Prime Mover (Diesel Present)									TC/TAD		TC/DPC (TTC)		TC/DPC (VTC)		TC/DPC (HTC)	
1017/1017A	Electrical	0	1.42E-03			0.211		0.061	2.1E-06	0.0E+00	2.1E-06	0.0E+00	2.1E-06	0.0E+00	4.3E-06	0.0E+00
1021	HVAC	2	5.79E-03		2	0.211		0.061	2.1E-06	5.2E-09	2.1E-06	5.2E-09	2.1E-06	5.2E-09	4.3E-06	1.0E-08
1021A	Mechanical Equipment	11.97	1.13E-02	9.97	2	0.211		0.061	2.1E-06	2.5E-07	2.1E-06	2.5E-07	2.1E-06	2.5E-07	4.3E-06	5.0E-07
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	2.1E-06	0.0E+00	2.1E-06	0.0E+00	2.1E-06	0.0E+00	4.3E-06	0.0E+00
	Torches, welders, burners	400	1.29E-03			0.211	400	0.061	2.1E-06	6.8E-08	2.1E-06	6.8E-08	2.1E-06	6.8E-08	4.3E-06	1.4E-07
	Internal combustion engines	100	2.73E-04	100		0.211		0.061	2.1E-06	5.8E-08	2.1E-06	5.8E-08	2.1E-06	5.8E-08	4.3E-06	1.2E-07
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	2.1E-06	0.0E+00	2.1E-06	0.0E+00	2.1E-06	0.0E+00	4.3E-06	0.0E+00
	Portable Equipment	4	7.37E-03		2	0.211	2	0.061	2.1E-06	8.5E-09	2.1E-06	8.5E-09	2.1E-06	8.5E-09	4.3E-06	1.7E-08
	No equipment involved	2533	2.71E-05	349	120	0.211	2064	0.061	2.1E-06	2.9E-08	2.1E-06	2.9E-08	2.1E-06	2.9E-08	4.3E-06	5.8E-08
Propagation from rooms in Fire Zone																
1016			3.41E-03			0.057			2.1E-06	4.2E-10	2.1E-06	4.2E-10	2.1E-06	4.2E-10	4.3E-06	8.4E-10
Localized Fire Threatens TC/TAD or TC/DPC in Vestibule/Preparation Area (Diesel Present)																
	Localized Fire Threatens TC/TAD in Vestibule/Preparation Area (Diesel Present)									**4.2E-07**						
	Localized Fire Threatens TC/DPC (TTC) in Vestibule/Preparation Area (Diesel Present)											**4.2E-07**				
	Localized Fire Threatens TC/DPC (VTC) in Vestibule/Preparation Area (Diesel Present)													**4.2E-07**		
	Localized Fire Threatens TC/DPC (HTC) in Vestibule/Preparation Area (Diesel Present)															**8.4E-07**
Entry represents a vulnerability due to the Railcar (No Diesel Present)									TC/TAD		TC/DPC (TTC)		TC/DPC (VTC)		TC/DPC (HTC)	
1017/1017A	Electrical	0	1.42E-03			0.211		0.061	1.6E-05	0.0E+00	2.4E-05	0.0E+00	1.3E-05	0.0E+00	2.7E-05	0.0E+00
1021	HVAC	2	5.79E-03		2	0.211		0.061	1.6E-05	4.0E-08	2.4E-05	5.8E-08	1.3E-05	3.2E-08	2.7E-05	6.5E-08
1021A	Mechanical Equipment	11.97	1.13E-02	9.97	2	0.211		0.061	1.6E-05	1.9E-06	2.4E-05	2.8E-06	1.3E-05	1.5E-06	2.7E-05	3.1E-06
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	1.6E-05	0.0E+00	2.4E-05	0.0E+00	1.3E-05	0.0E+00	2.7E-05	0.0E+00
	Torches, welders, burners	400	1.29E-03			0.211	400	0.061	1.6E-05	5.2E-07	2.4E-05	7.6E-07	1.3E-05	4.2E-07	2.7E-05	8.5E-07
	Internal combustion engines	0	2.73E-04			0.211		0.061	1.6E-05	0.0E+00	2.4E-05	0.0E+00	1.3E-05	0.0E+00	2.7E-05	0.0E+00
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	1.6E-05	0.0E+00	2.4E-05	0.0E+00	1.3E-05	0.0E+00	2.7E-05	0.0E+00
	Portable Equipment	4	7.37E-03		2	0.211	2	0.061	1.6E-05	6.6E-08	2.4E-05	9.6E-08	1.3E-05	5.3E-08	2.7E-05	1.1E-07
	No equipment involved	2533	2.71E-05	349	120	0.211	2064	0.061	1.6E-05	2.2E-07	2.4E-05	3.3E-07	1.3E-05	1.8E-07	2.7E-05	3.6E-07
Propagation from rooms in Fire Zone																
1016			3.41E-03			0.057			1.6E-05	3.2E-09	2.4E-05	4.7E-09	1.3E-05	2.6E-09	2.7E-05	5.3E-09
Localized Fire Threatens TC/TAD or TC/DPC in Preparation Area																
	Localized Fire Threatens TC/TAD in Preparation Area (No Diesel Present)									**2.8E-06**						
	Localized Fire Threatens TC/DPC (TTC) in Preparation Area (No Diesel Present)											**4.1E-06**				
	Localized Fire Threatens TC/DPC (VTC) in Preparation Area (No Diesel Present)													**2.2E-06**		
	Localized Fire Threatens TC/DPC (HTC) in Preparation Area (No Diesel Present)															**4.5E-06**

Table F5.7-4. Localized Fire Initiating Event Frequencies (Continued)

Room of Origin (includes comments field as needed)	Ignition Source (If Applicable)	Number in Room	Frequency per Unit (50 years)	Number at Target	Number Near Target	Propagation Probability to Target	Number Away from Target	Propagation Probability to Target	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)
Entry represents a vulnerability due to the Cask Transfer Trolley									TC/TAD		TC/DPC (incl. TTC & VTC)					
1017/1017A	Electrical	0	1.42E-03			0.211		0.061	6.4E-06	0.0E+00	1.5E-05	0.0E+00				
	HVAC	0	5.79E-03			0.211		0.061	6.4E-06	0.0E+00	1.5E-05	0.0E+00				
	Mechanical Equipment	8.97	1.13E-02	6.97		0.211	2	0.061	6.4E-06	5.1E-07	1.5E-05	1.2E-06				
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	6.4E-06	0.0E+00	1.5E-05	0.0E+00				
	Torches, welders, burners	400	1.29E-03			0.211	400	0.061	6.4E-06	2.0E-07	1.5E-05	4.7E-07				
	Internal combustion engines	35	2.73E-04			0.211	35	0.061	6.4E-06	3.7E-09	1.5E-05	8.6E-09				
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	6.4E-06	0.0E+00	1.5E-05	0.0E+00				
	Portable Equipment	4	7.37E-03		2	0.211	2	0.061	6.4E-06	2.6E-08	1.5E-05	5.9E-08				
	No equipment involved	1993	2.71E-05	175	120	0.211	1698	0.061	6.4E-06	5.3E-08	1.5E-05	1.2E-07				
Propagation from rooms in Fire Zone																
1016			3.41E-03			0.057			6.4E-06	1.3E-09	1.5E-05	2.9E-09				
1021			2.55E-02			0.057			6.4E-06	9.4E-09	1.5E-05	2.2E-08				
1021A			5.24E-02			0.057			6.4E-06	1.9E-08	1.5E-05	4.4E-08				
Localized Fire Threatens Waste Form in Preparation Area																
	Localized Fire Threatens TC/TAD in Preparation Area									**8.3E-07**						
	Localized Fire Threatens TC/DPC (VTC, incl TTC) in Preparation Area											**1.9E-06**				
Entry represents a vulnerability due to the Cask Transfer Trolley									TC/TAD		TC/DPC (TTC)		TC/DPC (VTC)			
1015 & 2007	Electrical	0	1.42E-03			0.211		0.061	3.5E-06	0.0E+00	1.8E-06	0.0E+00	1.8E-06	0.0E+00		
Cask Unloading Rr	HVAC	0	5.79E-03			0.211		0.061	3.5E-06	0.0E+00	1.8E-06	0.0E+00	1.8E-06	0.0E+00		
	Mechanical Equipment	9.03	1.13E-02	7.03	2	0.211		0.061	3.5E-06	3.0E-07	1.8E-06	1.5E-07	1.8E-06	1.5E-07		
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	3.5E-06	0.0E+00	1.8E-06	0.0E+00	1.8E-06	0.0E+00		
	Torches, welders, burners	0	1.29E-03			0.211		0.061	3.5E-06	0.0E+00	1.8E-06	0.0E+00	1.8E-06	0.0E+00		
	Internal combustion engines	0	2.73E-04			0.211		0.061	3.5E-06	0.0E+00	1.8E-06	0.0E+00	1.8E-06	0.0E+00		
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	3.5E-06	0.0E+00	1.8E-06	0.0E+00	1.8E-06	0.0E+00		
	Portable Equipment	1	7.37E-03	1		0.211		0.061	3.5E-06	2.6E-08	1.8E-06	1.3E-08	1.8E-06	1.3E-08		
	No equipment involved	1600	2.71E-05	30	120	0.211	1450	0.061	3.5E-06	1.4E-08	1.8E-06	7.0E-09	1.8E-06	7.0E-09		
Propagation from rooms in Fire Zone																
1016			3.41E-03			0.057			3.5E-06	6.9E-10	1.8E-06	3.5E-10	1.8E-06	3.5E-10		
1021			2.55E-02			0.057			3.5E-06	5.2E-09	1.8E-06	2.6E-09	1.8E-06	2.6E-09		
1021A			5.24E-02			0.057			3.5E-06	1.1E-08	1.8E-06	5.4E-09	1.8E-06	5.4E-09		
Localized Fire Threatens Waste Form in Cask Unloading Room																
	Localized Fire Threatens TC/TAD in Cask Unloading Room									**3.5E-07**						
	Localized Fire Threatens TC/DPC (TTC) in Cask Unloading Room											**1.8E-07**				
	Localized Fire Threatens TC/DPC (VTC) in Cask Unloading Room													**1.8E-07**		

Table F5.7-4. Localized Fire Initiating Event Frequencies (Continued)

Room of Origin (includes comments field as needed)	Ignition Source (If Applicable)	Number in Room	Frequency per Unit (50 years)	Number at Target	Number Near Target	Propagation Probability to Target	Number Away from Target	Propagation Probability to Target	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)	Target Exposure Time (Fraction)	Contribution to IE Frequency (per waste form over 50 years)
Entry represents a vulnerability due to the Canister Transfer Machine									TAD or DPC (TTC & VTC)							
2007	Electrical	0	1.42E-03			0.211		0.061	1.2E-06	0.0E+00						
	HVAC	0	5.79E-03			0.211		0.061	1.2E-06	0.0E+00						
	Mechanical Equipment	7	1.13E-02	7		0.211		0.061	1.2E-06	9.6E-08						
	Heat Generating Equipment	0	0.00E+00			0.211		0.061	1.2E-06	0.0E+00						
	Torches, welders, burners	0	1.29E-03			0.211		0.061	1.2E-06	0.0E+00						
	Internal combustion engines	0	2.73E-04			0.211		0.061	1.2E-06	0.0E+00						
	Office/kitchen equipment	0	1.66E-02			0.211		0.061	1.2E-06	0.0E+00						
	Portable Equipment	0	7.37E-03			0.211		0.061	1.2E-06	0.0E+00						
	No equipment involved	1444	2.71E-05	30	120	0.211	1294	0.061	1.2E-06	4.4E-09						
Localized Fire Threatens TAD or DPC (incl TTC & VTC) in Transfer Room										**1.0E-07**						

NOTE: AO = aging overpack; DPC = dual-purpose canister; HTC = transportation cask in the horizontal position; HVAC = heating, ventilation, and air conditioning; IE = initiating event; TAD = transportation, aging, and disposal canister; TC = transportation cask; TTC = transportation cask in the tilted position; VTC = transportation cask in the vertical position.

Source: Original

F5.7.2.1 Room Groupings

The first column of Table F5.7-4 identifies the room(s) of origin. If the vulnerability is expected to occur in a single room with no gates or doors open and that is surrounded by qualified fire barriers (i.e., it is a single room fire area), this room is listed as the only room of origin. However, there are several cases in which the vulnerability takes place as the waste form moves between multiple rooms, or the room where the vulnerability occurs has open doors or gates with other rooms, or it shares a qualified fire area with other rooms. Table F5.7-5 lists all of the vulnerabilities that have more than one room of origin, and the justification for the multiple room listing. Whenever such a condition exists, the quantification of the localized fire considers not only fires that start in the room where the waste form resides, but also the contribution of other rooms that could directly communicate with that room through non-qualified or open fire barriers. Rooms within the same fire area of a room of origin are listed under each vulnerability in the column labeled "Propagation From Rooms in Fire Zone" heading.

For rooms of origin, the Frequency per Unit column is populated by the results in Section F5.3. This is discussed further in Section F5.7.2.2. Propagation rooms populate the Frequency per Unit column with the total ignition frequency for that room, as calculated and reviewed in Section F4.4 (Room Ignition Frequency).

Table F5.7-5. Localized Fire Initiating Events with Multiple Rooms of Origin

Rooms	Vulnerability	Justification
1001 1002	Site Transporter	Rooms open to each other due to open doors as the Site Transporter moves from 1001 into 1002
1013 2007	Site Transporter	Rooms open to each other due to the open port slide gate for the Canister Transfer Machine
1017/1017A 1021 1021A	Site Prime Mover (Diesel Present) / Railcar (No Diesel Present)	Rooms open to each other due to open doors as the Site Prime Mover/Railcar moves from 1021A to 1021 to 1017/1017A
1015 2007	Cask Transfer Trolley	Rooms open to each other due to the open cask port slide gate for the Canister Transfer Machine
1026 1027	Site Transporter	Rooms open to each other due to open doors as the Site Transporter moves from 1027 to 1026

Source: Original

F5.7.2.2 Ignition Source Distribution Within a Room

Per the methodology discussion in Section F4.4.2.1, the location of the ignition sources within the room are identified relative to the target and assigned a location at the target, near the target, and away from the target. This is shown in the so-named columns of Table F5.7-4, and must sum to the 'Number in Room' column entry. These columns are designators of where the ignition sources are in relation to the vulnerable waste form.

For all categories except no equipment involved, the distribution is determined by analysis of the room layout to determine whether the ignition source unit is at a distance within about three meters (at target), between about 3 and 7 meters (near target), or further (away from target) of the vulnerable waste form. For vulnerable waste forms in motion (e.g., in the railcar), ignition sources within the aforementioned distances of any portion of the path of motion are counted in the class representing its closest point to the waste form.

The ignition source units for the no equipment involved category are the area of the room (square meters). For vulnerabilities that are not waste forms in motion, the numbers for at target and near target are 30 and 120, respectively (i.e., a floor area of approximately 30 square meters is considered at the target and the next 120 square meters near the target). The remaining square meters are entered as away from target. For vulnerable waste forms in motion, the "at target" value is the total square meters covered by the full range of motion plus a three meter ring. Similarly, the number near target is figured to be a seven meter ring around the at target area. Remaining square meters are entered as away from target.

The distribution of ignition sources are used to determine how far a fire must spread before it reaches the vulnerable waste form. The propagation values are taken from Table F5.6-1 for the no suppression case, per the boundary conditions, in accordance with the guidance discussed in Section F4.4.2 (in particular, F4.4.2.1). The Frequency per [ignition source] Unit is taken from Table F5.3-1, the column labeled Frequency per Unit. The Target Exposure Time (Fraction), which is the probability that there is a waste form in the room, is taken from Tables F5.7-1, F5.7-2, and F5.7-3 as appropriate. The column labeled "Contribution to IE Frequency" implements Equation F-7 to provide the total initiating event frequency contribution from fire that start in the room where the waste form resides.

There is also a section of Table F5.7-4 that addresses the contribution from nearby rooms in the same fire area (i.e., that are separated from the room by walls or doors, but those barriers are not qualified fire barriers). In this case, the location of the ignition sources within these rooms is not important, only the probability that the fire spreads beyond the room within the same fire area matters as to whether the fire reaches the target. In this case, the Frequency per Unit column refers to the overall frequency of ignition in the room, which comes from the last column in Table F5.5-1. In this case, the appropriate propagation value for spread of a fire beyond the room is taken from Table F5.6-1, again for the no suppression case, as discussed in Section F4.4.2 (in particular, F4.4.2.2). For these rooms, the Contribution to IE Frequency column implements the generic form of Equation F-8, as applied to a fire throughout a fire area (zone) where the next largest fire is a floor fire.

The overall fire initiating event frequency, provided in a shaded cell for each defined initiating event shown in bold, is the sum of all the individual contributors.

F5.7.3 Large Fires

Calculation of the Initiating Event Frequencies is completed similarly to the localized fire contributions from other rooms. Table F5.7-6 provides the analysis. In this case, the fire can start in any room in the facility and become a large fire. Since the fire can start in any room, and the methodology applies the same probability of fire propagation to each room, the starting point is the total ignition frequency from all rooms, from Table F5.6-1. The propagation probability is applied as discussed in Section F4.4.2 (in particular, F4.4.2.2) to implement Equation F-9. The target exposure time (fraction) is once again taken from Tables F5.7-1, F5.7-2, and F5.7-3. Large fires always propagate beyond the fire area of the room of origin.

Table F5.7-6. Large Fire Initiating Event Frequencies

	Total Ignition Frequency	Propagation Probability Beyond Fire -rated Area	Target Exposure Time (Fraction)	**IE Frequency, per waste form, over 50 years**
Large Fire Threatens TC/TAD or TC/DPC (TTC & VTC) (Diesel Present)	2.20E+00	0.169	2.1E-06	**7.8E-07**
Large Fire Threatens TC/TAD (No Diesel)	2.20E+00	0.169	2.6E-05	**9.6E-06**
Large Fire Threatens TAD or DPC (TTC & VTC) in CTM	2.20E+00	0.169	1.2E-06	**4.4E-07**
Large Fire Threatens TAD or DPC (TTC & VTC) in AO (Diesel Present)	2.20E+00	0.169	1.5E-05	**5.6E-06**
Large Fire Threatens TC/DPC (TTC) (No Diesel)	2.20E+00	0.169	4.0E-05	**1.5E-05**
Large Fire Threatens TC/DPC (VTC) (No Diesel)	2.20E+00	0.169	3.0E-05	**1.1E-05**
Large Fire Threatens TC/DPC (HTC) (Diesel Present)	2.20E+00	0.169	4.3E-06	**1.6E-06**
Large Fire Threatens TC/DPC (HTC) (No Diesel)	2.20E+00	0.169	2.7E-05	**1.0E-05**

NOTE: AO = aging overpack; CTM = canister transfer machine; DPC = dual-purpose canister; HTC = transportation cask in the horizontal position; IE = initiating event; TAD = transportation, aging, and disposal canister; TC = transportation cask; TTC = transportation cask in the tilted position; VTC = transportation cask in the vertical position.

Source: Original

F5.7.4 Contribution to Initiating Event Frequency

The probability of a fire reaching the vulnerable waste form and the target exposure time (residence fractions; refer to section F5.7.1) contribute to the final calculation of the contribution to initiating event frequency (cells highlighted in blue on Tables F5.7-4 and F5.7-6). Section F4.4 details the calculations performed to arrive at the initiating event frequency.

F5.8 Monte Carlo Simulation/Uncertainty Distributions

F5.8.1 Uncertainty Distributions

Uncertainty distributions are utilized in the contribution to initiating event frequency calculations to account for the potential of variance in the data. For example, the ignition frequency presented in Table F5.2-1, Section F5.1 is the result of a calculation based on room area. The equation utilized to perform this calculation was derived from data collected on building fires. While the data collected and the equation developed to fit the data have a good R-squared (percentage of variability accounted for in the equation) value (90), an uncertainty distribution is necessary to account for the natural variability of the frequency of ignition.

The uncertainty distributions utilized for this analysis are normally distributed, with the exception of one lognormal distribution (skewed bell curve shape, with the median value at the top of the curve). Both distributions can be accurately represented by a median (50 percent value; equal to the mean for normal distributions) and a 97.5 percent value. The 97.5 percent value is a figure that represents a point at which only 2.5 percent of all possible outcomes will vary from the mean more significantly.

Three uncertainty distributions were developed for this analysis: ignition frequency, category fraction, and conditional probability. The distribution for ignition frequency is discussed in detail in Appendix F.III. The distributions for category fraction and conditional probability are discussed in Appendix F.II.

F5.8.2 Monte Carlo Simulation

Monte Carlo simulations are performed to determine the mean, standard deviation, variance, minimum, and maximum values of each of the initiating event frequencies based on the variance of the contributing data. To accomplish this, the Microsoft Excel add-on package Crystal Ball was used. This software requires input of the necessary uncertainty distribution figures and the figures that the simulation will produce results for (initiating event frequencies). Crystal Ball software uses the mean or median and 97.5% value to calculate the equation which represents the distribution. The software then randomly selects a value from the possibilities defined by the distribution. This is set within the software to be done 10,000 times to ensure accurate results.

F5.9 Results

The results of the analysis are the fire initiating event frequencies and their associated distributions. The initiating event frequencies represent the probability, over the length of the pre-closure period, that a fire will threaten the stated waste form during the stated vulnerability. Because data used to obtain these results are based on existing fire data, it was necessary to determine the uncertainty distribution for each initiating event. Figure F5.7-1 displays the Crystal Ball results for a localized fire threatening a transportation cask/TAD canister in the CTT in the Cask Unloading Room.

These results provide a statistical reference for the variance of each initiating event frequency. As seen in Section F5.7.2, Table F5.7-4, the baseline initiating event frequency for this case is 3.5. The Crystal Ball results give insight into this, showing that given the variability of the inputs, the true result could lie anywhere between 5.5 and 1.9 , with a mean of 3.9 , a standard deviation of 1.9 , and a lognormal shape. Crystal Ball was run for all of the initiating events, and a summary of the results, giving the distribution parameters of each distribution, is shown in Table F5.7-7. The 97.5 percentile values in Table F5.7-7 are not provided in the Crystal Ball full report. Instead, these values were obtained by utilizing the Extract Data option, which allows the analyst to specify the percentile values necessary. Also not included in the Crystal Ball report is the Error Factors (EF), these figures were calculated from the mean and median as discussed in Appendix F.V. It was determined via methods described in Appendix F.IV that all of the resultant distributions are lognormal. The complete output from Crystal Ball and the 97.5 percentile values are provided in Appendix F.VI. In addition to showing the initiating event frequency distribution, it also shows the input distribution for the parameters that were varied, which match the distributions developed and documented in Appendices F.II and F.III.

Table F5.7-7. Fire Initiating Events Results Summary

Initiating Event	Equipment	Mean	Median	97.5% Value	Error Factor	Type
Localized Fire Threatens Waste Form in AO in Vestibule/Lid Bolting Room (Diesel Present)	**Site Transporter**					
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Vestibule/Lid Bolting Room (Diesel Present)		8.1E-07	7.3E-07	1.80E-6	2.1	Lognormal
Localized Fire Threatens Waste Form in AO in Loading Room (Diesel Present)	**Site Transporter**					
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Loading Room (Diesel Present)		3.5E-07	3.2E-07	7.9E-07	2.0	Lognormal
Localized Fire Threatens Waste Form in Vestibule/Preparation Area (Diesel Present)	**Site Prime Mover**					
Localized Fire Threatens TC/TAD in Vestibule/Preparation Area (Diesel Present)		4.6E-07	4.2E-07	1.0E-06	2.0	Lognormal
Localized Fire Threatens TC/DPC (TTC) in Vestibule/Preparation Area (Diesel Present)		4.6E-07	4.2E-07	1.0E-06	2.0	Lognormal
Localized Fire Threatens TC/DPC (VTC) in Vestibule/Preparation Area (Diesel Present)		4.6E-07	4.2E-07	1.0E-06	2.0	Lognormal
Localized Fire Threatens TC/DPC (HTC) in Vestibule/Preparation Area (Diesel Present)		9.3E-07	8.3E-07	2.1E-06	2.2	Lognormal
Localized Fire Threatens Waste Form in Preparation Area	**Railcar**					
Localized Fire Threatens TC/TAD in Preparation Area (No Diesel Present)		3.1E-06	2.8E-06	6.9E-06	2.1	Lognormal
Localized Fire Threatens TC/DPC (TTC) in Preparation Area (No Diesel Present)		4.5E-06	4.0E-06	1.0E-05	2.2	Lognormal
Localized Fire Threatens TC/DPC (VTC) in Preparation Area (No Diesel Present)		2.5E-06	2.2E-06	5.5E-06	2.3	Lognormal
Localized Fire Threatens TC/DPC (HTC) in Preparation Area (No Diesel Present)		5.0E-06	4.5E-06	1.1E-05	2.1	Lognormal

Table F5.7-7. Fire Initiating Events Results Summary (Continued)

Initiating Event	Equipment	Mean	Median	97.5% Value	Error Factor	Type
Localized Fire Threatens Waste Form in Preparation Area	**Cask Transfer Trolley**					
Localized Fire Threatens TC/TAD in Preparation Area		9.1E-07	8.1E-07	2.1E-06	2.2	Lognormal
Localized Fire Threatens TC/DPC (VTC, incl TTC) in Preparation Area		2.1E-06	1.9E-06	4.8E-06	2.1	Lognormal
Localized Fire Threatens Waste Form in Cask Unloading Room	**Cask Transfer Trolley**					
Localized Fire Threatens TC/TAD in Cask Unloading Room		3.9E-07	3.5E-07	8.7E-07	2.1	Lognormal
Localized Fire Threatens TC/DPC (TTC) in Cask Unloading Room		2.0E-07	1.8E-07	4.4E-07	2.1	Lognormal
Localized Fire Threatens TC/DPC (VTC) in Cask Unloading Room		2.0E-07	1.8E-07	4.4E-07	2.1	Lognormal
Localized Fire Threatens Waste Form in Transfer Room	**Canister Transfer Machine**					
Localized Fire Threatens TAD or DPC (incl TTC & VTC) in Transfer Room		1.1E-07	9.9E-08	2.5E-07	2.1	Lognormal
Initiating Event		**Mean**	**Median**	**97.5% Value**	**Error Factor**	**Type**
Large Fire Threatens TC/TAD or TC/DPC (TTC & VTC) (Diesel Present)		8.6E-07	7.6E-07	2.0E-06	2.3	Lognormal
Large Fire Threatens TC/TAD (No Diesel)		1.1E-05	9.5E-06	2.5E-05	2.4	Lognormal
Large Fire Threatens TAD or DPC (TTC & VTC) in CTM		4.9E-07	4.4E-07	1.1E-06	2.1	Lognormal
Large Fire Threatens TAD or DPC (TTC & VTC) in AO (Diesel Present)		6.1E-06	5.5E-06	1.4E-05	2.1	Lognormal
Large Fire Threatens TC/DPC (TTC) (No Diesel)		1.6E-05	1.5E-05	3.8E-05	1.8	Lognormal
Large Fire Threatens TC/DPC (VTC) (No Diesel)		1.2E-05	1.1E-05	2.9E-05	2.0	Lognormal
Large Fire Threatens TC/DPC (HTC) (Diesel Present)		1.8E-06	1.6E-06	4.1E-06	2.2	Lognormal
Large Fire Threatens TC/DPC (HTC) (No Diesel)		1.1E-05	9.8E-06	2.6E-05	2.2	Lognormal

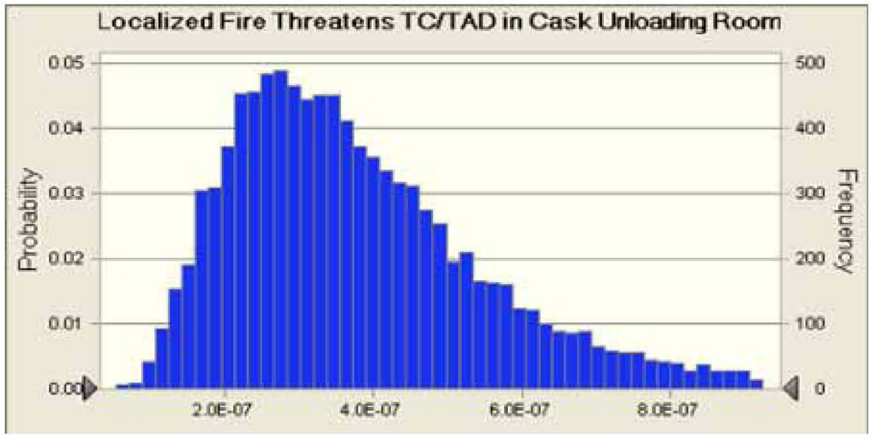
NOTE: AO = aging overpack; DPC = dual-purpose canister; HTC = transportation cask in the horizontal position; IE = initiating event; TAD = transportation, aging, and disposal canister; TC = transportation cask; TTC = transportation cask in the tilted position; VTC = transportation cask in the vertical position.

Source: Original

Forecast: Localized Fire Threatens TC/TAD in Cask Unloading Room **Cell: K99**

Summary:

Entire range is from 5.5E-08 to 1.9E-06
Base case is 3.5E-07
After 10,000 trials, the std. error of the mean is 1.9E-09



Statistics:	Forecast values
Trials	10,000
Mean	3.9E-07
Median	3.5E-07
Mode	1.8E-07
Standard Deviation	1.9E-07
Variance	3.6E-14
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.4911
Minimum	5.5E-08
Maximum	1.9E-06
Range Width	1.8E-06
Mean Std. Error	1.9E-09

Forecast: Localized Fire Threatens TC/TAD in Cask Unloading Room (cont'd) **Cell: K99**

Percentiles:	Forecast values
0%	5.5E-08
10%	1.9E-07
20%	2.3E-07
30%	2.7E-07
40%	3.1E-07
50%	3.5E-07
60%	3.9E-07
70%	4.5E-07
80%	5.2E-07
90%	6.3E-07
100%	1.9E-06

NOTE:

Source: Crystal Ball Software output.

Figure F5.7-1. Example of Crystal Ball Output for a Fire Initiating Event

APPENDIX F.I
DEFINITION OF IGNITION SOURCE CATEGORY

Table F.I-1. Definition of Ignition Source Category

Ignition Source Category	NFPA Equipment Categories Included
Electrical Equipment	**Fixed wiring**; **transformer, associated over current or disconnect equipment**; **meter, meter box**; power switchgear, over current protection devices; switch, receptacle, outlet; lighting fixture, lamp holder, ballast, sign; cord, plug; lamp, light bulb; unclassified or **unknown-type electrical distribution equipment**; electronic equipment; rectifier, charger
Mechanical and Electrical HVAC Equipment	**Central heating unit**; water heater; **fixed, stationary local heating unit**; central air conditioning, refrigeration equipment; water cooling device, tower; **fixed, stationary local refrigeration unit**; fixed, stationary local air conditioning unit; chimney, gas vent flue; chimney connector, vent connector; **heat transfer system**; unclassified heating systems; **other HVAC equipment**; unclassified air conditioning, refrigeration systems
Mechanical Equipment	Chemical process equipment; **waste recovery equipment**; **working, shaping machine**; coating machine; painting machine; unclassified process equipment; **separate motor or generator**; separate pump or compressor; conveyor, **unknown mechanical equipment**
Fixed Heat-Generating Process Equipment	**Casting, molding, or forging equipment**; heat-treating equipment; **dryers; furnaces; incinerators**
Torches/Welders	**Torches, welders, burners**
Internal Combustion Engines	**Internal combustion engines**
Office and Kitchen Equipment	**Television, radio, stereo; fixed food-warming appliance; fixed or stationary oven;** all other categories
Portable and Special Equipment	**Portable local heating unit**; **hand tools**; portable appliance designed to produce controlled heat; **portable appliance designed not to produce heat; unclassified special equipment; unclassified service or maintenance equipment; biomedical equipment or device**
No Equipment Involved	**No equipment**

NOTE: The entries shown in bold in the table were those that had caused fires in the data set. The other entries were included in the data set retrieval, but no fires were attributed to them. Given that there were only a total of 188 fires in the entire data set, the fact that certain items had not been associated with an observed fire cannot be taken to mean that they can be eliminated as potential ignition sources.
HVAC = heating, ventilation, and air conditioning; NFPA = National Fire Protection Association;

Source: Ref. F2.57

APPENDIX F.II
DERIVATION OF IGNITION SOURCE DISTRIBUTION AND FIRE PROPAGATION PROBABILITIES

Three independent data sets concerning fires in radioactive material working facilities (Tables F.II-1 through F.II-3) have been analyzed for statistical confidence. The data sets are in the format of a tally; each sample (fire) is placed in the appropriate category (equipment type, extent of flame damage, etc.), and the reported figure for each category is the number of fires that pertained to the category. All of these data sets reflect the operating history of nuclear facilities of non-combustible construction as defined by the NFPA. (Ref. F2.57).

The first data set provides a distribution of fire ignition as a function of the ignition source category, as defined in Appendix I. Table F.II-1 provides a summary of that data.

Table F.II-1. Fires in Radioactive Material Working Facilities by Originating Equipment

Ignition Source Category	Fires	
Electrical	16	9%
Mechanical/Electrical HVAC	15	8%
Mechanical	26	14%
Heat Generating	29	16%
Torches/Welders	41	22%
Internal Combustion	4	2%
Offices/Kitchen Equipment	12	6%
Portable Equipment	19	10%
No Equipment	25	13%
Total	187	100%

NOTE: HVAC = heating, ventilation, and air conditioning.

Source: Ref. F2.57

Table F.II-2. Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction and in which No Automatic Suppression System Was Present or the Automatic Suppression System Failed to Operate

Extent of Flame Damage	Fires	
Confined to object of origin	54	63%
Confined to part of room/area of origin	13	15%
Confined to room of origin	0	0
Confined to fire-rated compartment of origin	5	6%
Confined to floor of origin	0	0
Confined to structure of origin	14	16%
Extended beyond structure of origin	0	0
Total	86	100%

Source: Ref. F2.57

Table F.II-3. Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction and in which the Fire Was Too Small to Activate the Automatic Suppression System or the Automatic System Operated Properly

Extent of Flame Damage	Fires	
Confined to object of origin	40	56%
Confined to part of room/area of origin	23	32%
Confined to room of origin	2	3%
Confined to fire-rated compartment of origin	0	0%
Confined to floor of origin	5	7%
Confined to structure of origin	2	3%
Extended beyond structure of origin	0	0
Total	72	100%

Source: Ref. F2.57

The method chosen for calculating the confidence interval of the data is the margin of error calculation:

$$ME = \sqrt{\frac{p(1-p)}{n}} \times t$$

(Eq. FII-1)

where

ME = Margin of error

p = Event probability

n = Number of samples

t = t-distribution value (see Table F.II-4)

The Event Probabilities are in the second "Fire" column of Tables F.II-1 through F.II-3, and are converted to decimal format (divided by 100) for the calculations. Values for t are obtained from a standard t-distribution table, the necessary excerpt from which is provided in Table F.II-4.

Table F.II-4. t-Distribution

t-distribution			
		α	
		0.025	0.005
v	60	2.000	2.660
	120	1.980	2.617

Source: Ref. F2.60.

where

α = One minus the confidence interval (CI) divided by two (ex. A 95% CI corresponds to an α of 0.025)

v = Degrees of freedom (number of samples minus one)

For the data sets analyzed, Confidence Intervals (CI) of 95% and 99% were analyzed. This is done because while 95% is an accepted and commonly used CI, 99% is an extremely conservative CI.

Completed calculations and the ranges based on the margins of error are provided in Tables F.II-5 through F.II-10 below. To demonstrate the calculations performed in F.II-5 – F.II-10, an example will be completed from Table F.II-5, row 1. The Event Probability (p) is determined by dividing the number of occurrences (16) for that event by the total number of fires (187). Thus, 0.0856 is the event probability for an electrically originated fire. The margin of error (ME) is then calculated utilizing Equation 1 above, obtaining t from Table F.II-4. For this example, t is 1.98 because the degrees of freedom (v = n-1 = 186) is greater than 120, and the CI is 95%, making α=0.025. The ME obtained, ±0.0405, when subtracted from and added to the event probability provides a percentile range (Probability range column). It can be said with 95% confidence that the true event probability lies within this range. The final column is an occurrences range, which is calculated by converting the percentages of the preceding row to decimal format (dividing by 100), and multiplying them by the total number of fires (187). It can be said with 95% confidence that the true number of occurrences for any set of 187 fires is within this range. The calculations throughout Tables F.II-5 through F.II-10 are performed in the same manner, with the value of t depending on the number of samples (fires) and the CI.

Table F.II-5. Margin of Error Results at 95% CI for Fires in Radioactive Material Working Facilities by Originating Equipment

Equipment Type	Occurances	Probability		Margin of Error (95% confidence)	Probability range based on Margin of Error (%)			Occurances range based on Margin of Error		
Electrical	16	8.56E-02	±	4.05E-02	4.51	≤ p ≤	12.61	8.43	≤ O ≤	23.58
Mechanical/Electrical HVAC	15	8.02E-02	±	3.93E-02	4.09	≤ p ≤	11.95	7.65	≤ O ≤	22.35
Mechanical	26	1.39E-01	±	5.01E-02	8.89	≤ p ≤	18.91	16.62	≤ O ≤	35.36
Heat Generating	29	1.55E-01	±	5.24E-02	10.27	≤ p ≤	20.75	19.20	≤ O ≤	38.80
Torches/Welders	41	2.19E-01	±	5.99E-02	15.93	≤ p ≤	27.92	29.79	≤ O ≤	52.21
Internal Combustion	4	2.14E-02	±	2.09E-02	0.04	≤ p ≤	4.23	0.07	≤ O ≤	7.91
Offices/Kitchen Equipment	12	6.42E-02	±	3.55E-02	2.87	≤ p ≤	9.97	5.37	≤ O ≤	18.64
Portable Equipment	19	1.02E-01	±	4.37E-02	5.79	≤ p ≤	14.53	10.83	≤ O ≤	27.17
No Equipment	25	1.34E-01	±	4.93E-02	8.44	≤ p ≤	18.3	15.78	≤ O ≤	34.22
Total	187	1								

NOTE: HVAC = heating, ventilation, and air conditioning.

Source: Original

Table F.II-6. Margin of Error Results at 99% CI for Fires in Radioactive Material Working Facilities by Originating Equipment

Equipment Type	Occurances	Probability		Margin of Error (99% confidence)	Probability range based on Margin of Error (%)			Occurances range based on Margin of Error		
Electrical	16	8.56E-02	±	5.35E-02	3.2	≤ p ≤	13.91	5.98	≤ O ≤	26.01
Mechanical/Electrical HVAC	15	8.02E-02	±	5.20E-02	2.82	≤ p ≤	13.22	5.27	≤ O ≤	24.72
Mechanical	26	1.39E-01	±	6.62E-02	7.28	≤ p ≤	20.53	13.61	≤ O ≤	38.39
Heat Generating	29	1.55E-01	±	6.93E-02	8.58	≤ p ≤	22.44	16.04	≤ O ≤	41.96
Torches/Welders	41	2.19E-01	±	7.92E-02	14.01	≤ p ≤	29.84	26.20	≤ O ≤	55.80
Internal Combustion	4	2.14E-02	±	2.77E-02	-0.63	≤ p ≤	4.91	0.00	≤ O ≤	9.18
Offices/Kitchen Equipment	12	6.42E-02	±	4.69E-02	1.73	≤ p ≤	11.11	3.24	≤ O ≤	20.78
Portable Equipment	19	1.02E-01	±	5.78E-02	4.38	≤ p ≤	15.94	8.19	≤ O ≤	29.81
No Equipment	25	1.34E-01	±	6.51E-02	6.86	≤ p ≤	19.88	12.83	≤ O ≤	37.18
Total	187	1								

NOTE: HVAC = heating, ventilation, and air conditioning.

Source: Original

Table F.II-7. Margin of Error Results at 95% CI for Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction and in which No Automatic Suppression System Was Present or the Automatic Suppression System Failed to Operate

Extent of Flame Damage	Occurrences	**Probability**		**Margin of Error (95% confidence)**			**Probability range based on Margin of Error (%)**				**Occurrences range based on Margin of Error**		
Confined to object of origin	54	6.21E-01		±	1.04E-01		51.67	? p ?	72.48		44.78	? O ?	62.81
Confined to part of room/area of origin	13	1.49E-01		±	7.65E-02		7.3	? p ?	22.59		6.33	? O ?	19.58
Confined to room of origin	0.33	3.79E-03		±	1.32E-02		0	? p ?	1.7		0	? O ?	1.47
Confined to fire-rated compartment of origin	5	5.75E-02		±	4.99E-02		0.76	? p ?	10.74		0.66	? O ?	9.31
Confined to floor of origin	0.33	3.79E-03		±	1.32E-02		0	? p ?	1.7		0	? O ?	1.47
Confined to structure of origin	14	1.61E-01		±	7.88E-02		8.21	? p ?	23.97		7.11	? O ?	20.77
Extended beyond structure of origin	0.33	3.79E-03		±	1.32E-02		0	? p ?	1.7		0	? O ?	1.47
Total	86.99	1											

Source: Original

Table F.II-8. Margin of Error Results at 99% CI for Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction and in which No Automatic Suppression System Was Present or the Automatic Suppression System Failed to Operate

Extent of Flame Damage	Occurrences	Probability	Margin of Error (99% confidence)		Probability range based on Margin of Error (%)			Occurances range based on Margin of Error		
Confined to object of origin	54	6.21E-01	±	1.38E-01	48.24	≤ p ≤	75.91	41.8	≤ O ≤	65.78
Confined to part of room/area of origin	13	1.49E-01	±	1.02E-01	4.78	≤ p ≤	25.11	4.14	≤ O ≤	21.76
Confined to room of origin	0.33	3.79E-03	±	1.75E-02	0	≤ p ≤	2.13	0	≤ O ≤	1.85
Confined to fire-rated compartment of origin	5	5.75E-02	±	6.64E-02	0	≤ p ≤	12.39	0	≤ O ≤	10.74
Confined to floor of origin	0.33	3.79E-03	±	1.75E-02	0	≤ p ≤	2.13	0	≤ O ≤	1.85
Confined to structure of origin	14	1.61E-01	±	1.05E-01	5.61	≤ p ≤	26.57	4.86	≤ O ≤	23.03
Extended beyond structure of origin	0.33	3.79E-03	±	1.75E-02	0	≤ p ≤	2.13	0	≤ O ≤	1.85
Total	86.99	1								

Source: Original

Table F.II-9. Margin of Error Results at 95% CI for Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction and in which the Fire Was Too Small to Activate the Automatic Suppression System or the Automatic System Operated Properly

Extent of Flame Damage	Occurances	Probability		Margin of Error (95% confidence)			Probability range based on Margin of Error (%)				Occurances range based on Margin of Error		
Confined to object of origin	40	5.51E-01		±	1.17E-01		43.38	≤ p ≤	66.72		31.52	≤ O ≤	48.48
Confined to part of room/area of origin	23	3.17E-01		±	1.09E-01		20.74	≤ p ≤	42.57		15.07	≤ O ≤	30.93
Confined to room of origin	2	2.75E-02		±	3.84E-02		0	≤ p ≤	6.59		0	≤ O ≤	4.79
Confined to fire-rated compartment of origin	0.33	4.54E-03		±	1.58E-02		0	≤ p ≤	2.03		0	≤ O ≤	1.47
Confined to floor of origin	5	6.88E-02		±	5.94E-02		0.94	≤ p ≤	12.82		0.68	≤ O ≤	9.32
Confined to structure of origin	2	2.75E-02		±	3.84E-02		0	≤ p ≤	6.59		0	≤ O ≤	4.79
Extended beyond structure of origin	0.33	4.54E-03		±	1.58E-02		0	≤ p ≤	2.03		0	≤ O ≤	1.47
Total	72.66	1											

Source: Original

Table F.II-10. Margin of Error Results at 99% CI for Structure Fires in Radioactive Material Working Facilities and Nuclear Energy Plants of Non-Combustible Construction and in which the Fire Was Too Small to Activate the Automatic Suppression System or the Automatic System Operated Properly

Extent of Flame Damage	Occurances	Probability		Margin of Error (99% confidence)			Probability range based on Margin of Error (%)				Occurances range based on Margin of Error		
Confined to object of origin	40	5.51E-01		±	1.55E-01		39.53	≤ p ≤	70.57		28.72	≤ O ≤	51.28
Confined to part of room/area of origin	23	3.17E-01		±	1.45E-01		17.14	≤ p ≤	46.17		12.45	≤ O ≤	33.55
Confined to room of origin	2	2.75E-02		±	5.11E-02		0	≤ p ≤	7.86		0	≤ O ≤	5.71
Confined to fire-rated compartment of origin	0.33	4.54E-03		±	2.10E-02		0	≤ p ≤	2.55		0	≤ O ≤	1.85
Confined to floor of origin	5	6.88E-02		±	7.90E-02		0	≤ p ≤	14.78		0	≤ O ≤	10.74
Confined to structure of origin	2	2.75E-02		±	5.11E-02		0	≤ p ≤	7.86		0	≤ O ≤	5.71
Extended beyond structure of origin	0.33	4.54E-03		±	2.10E-02		0	≤ p ≤	2.55		0	≤ O ≤	1.85
Total	72.66	1											

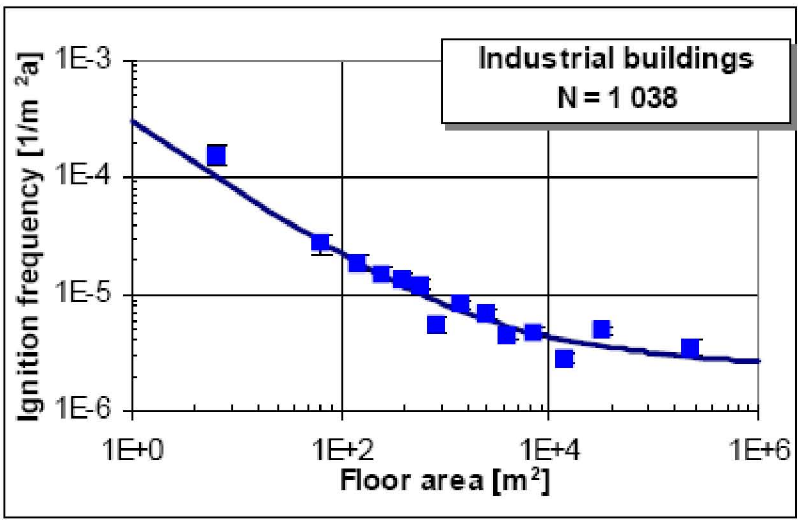
Source: Original

APPENDIX F.III
DERIVATION OF IGNITION FREQUENCY DISTRIBUTION

For proper consideration of the fire frequency analysis of the RF, it was necessary to develop an uncertainty distribution for the industrial building fire frequency. The *Utilisation of Statistics to Assess Fire Risks in Buildings* (Ref. F2.59) used to develop these frequencies presents an equation with floor area as an input to determine frequency. The following equation is developed based on sample data collected:

$$f''_m(A) = c_1 A^r + c_2 A^s \qquad \text{(Eq. F.III-1)}$$

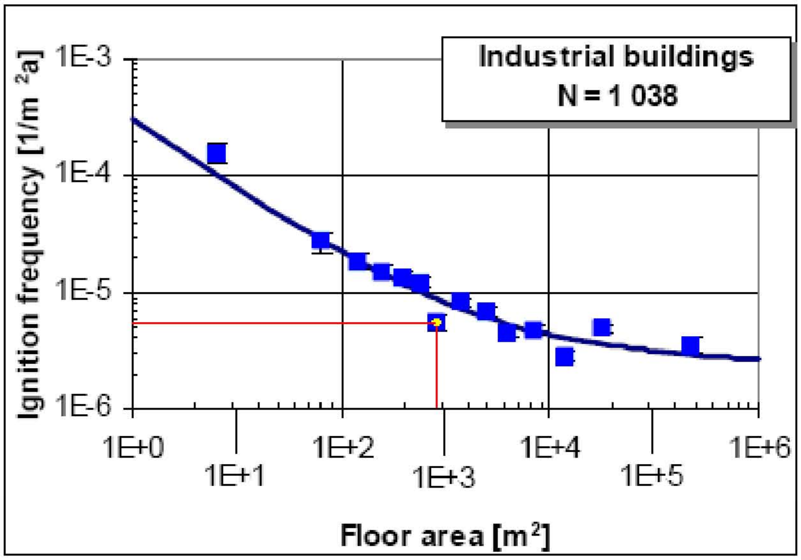
where f''_m is the annual fire frequency per square meter of floor area, A is the floor area, and the values c_1, c_2, r, and s are constants determined by the line of best fit derived from the data. For industrial buildings, the values for the constants are as follows: $c_1 = 3 \times 10^{-4}$, $c_2 = 5 \times 10^{-6}$, r = -0.61. and s = -0.05. The data for industrial buildings and the resulting line of best fit are presented in Figure F.III-1.



Source: Ref. F2.59

Figure F-III-1. Ignition Frequency Observations

Each data point in the graph represents the average of many data points. The individual data points and the average values were not provided in the reference. Because the data were only presented graphically, it was necessary to estimate the data for the purposes of this analysis. To accomplish this, the center of each data point was found, and x axis values were added such that the powers increase by a unit of one. Horizontal and vertical lines were drawn from each data point to the x and y axes. The ignition frequency and floor area were then estimated based on the relative distances between these lines and the major axis values. For the example shown in Figure F.III-2 below, the distance from the 1E+2 label to the red vertical line is divided by the distance from the 1E+2 to 1E+3 labels. In this case, the result is 0.925. Thus, the floor area for the data point is $10^{2.925}$. The ignition frequency is determined in an identical manner. The ignition frequency and floor area obtained in this manner are displayed in Table F.III-1. The ignition frequency predicted based on Equation F.III-1 is also provided in the table.



NOTE: m = meter.

Source: Original

Figure F.III-2. Data Point Determination

Table F.III.1. Ignition Frequency Data from Figure F.III-1 and Equation F.III-1

Graphically Determined Data Points		**From Equation F.III-1**
Floor Area (m^2)	**Ignition Frequency (1/yr m^2)**	**Predicted Frequency (1/yr m^2)**
7	1.6×10^{-4}	9.6×10^{-5}
65	2.8×10^{-5}	2.8×10^{-5}
150	1.9×10^{-5}	1.8×10^{-5}
240	1.5×10^{-5}	1.4×10^{-5}
380	1.4×10^{-5}	1.2×10^{-5}
570	1.2×10^{-5}	9.9×10^{-6}
840	5.6×10^{-6}	8.5×10^{-6}
1,400	8.9×10^{-6}	7.1×10^{-6}
2,500	7.0×10^{-6}	5.9×10^{-6}
4,100	4.6×10^{-6}	5.2×10^{-6}
7,100	4.8×10^{-6}	4.5×10^{-6}
14,000	2.9×10^{-6}	4.0×10^{-6}
33,000	5.1×10^{-6}	3.5×10^{-6}
230,000	3.6×10^{-6}	2.9×10^{-6}

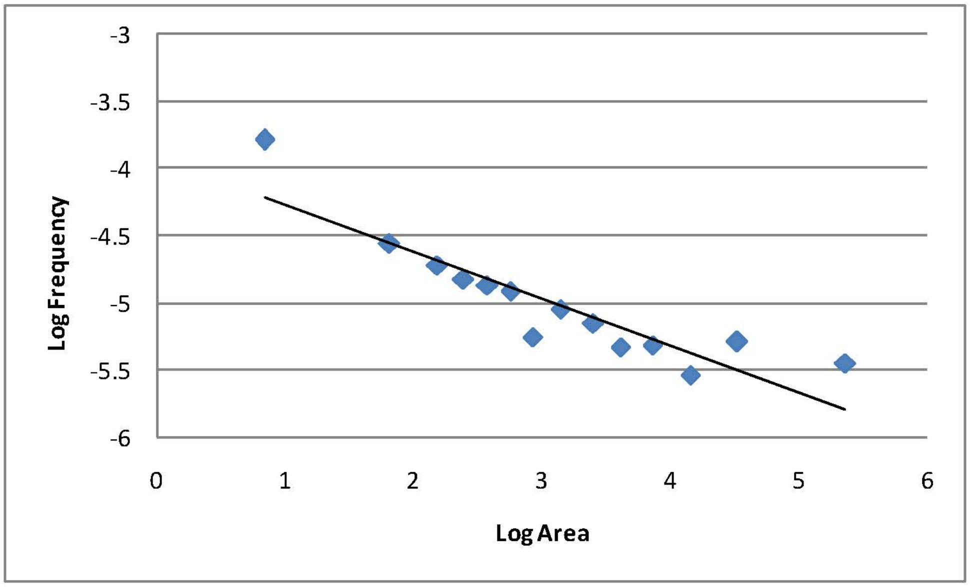
NOTE: m = meter; yr = year.

Source: Original

Because the ignition frequency is determined based on the line of best fit, the uncertainty distribution for the calculated ignition frequency can be determined by estimating the uncertainty in the ability of the best fit equation to predict the ignition frequency of any industrial building not included in the database. This is accomplished using the methodology presented below.

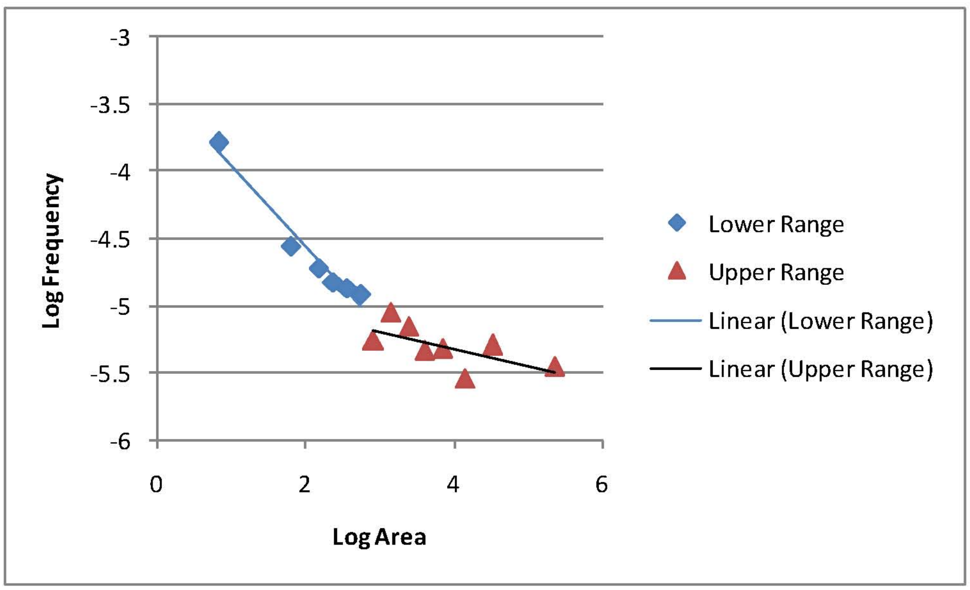
Statistics: Probability, Inference, and Decision (Ref. F2.60) outline a procedure to determine the confidence limits for a value predicted based on a linear regression equation. Though the ignition frequency and floor area are not linearly related, as illustrated by the figure and by equation F.III-1, the relationship between the log of the ignition frequency and the log of the floor area is approximately linear. This is illustrated in Figure F.III-3.

As shown in Figures F.III-1 and F.III-3, the portion of the curve for buildings less than 1,000m^2 has a steeper slope than the portion of the curve for buildings larger than 1,000m^2. For that reason, the data were divided into two ranges as shown in Figure F.III-4. Because all of the YMP facilities have floor areas larger than 1,000m^2, the remaining analysis focused on the upper end of the floor area range.



Source: Original

Figure F.III-3. Plot of Log(Ignition Frequency) as a Function of Log(Floor Area)



Source: Original

Figure F.III-4. Plot of Log(Ignition Frequency) as a Function of Log(Floor Area) Divided into Two Floor Area Ranges

To arrive at the confidence interval for the log of the ignition frequency, the follow equations are used:

$$\hat{y} \pm a \frac{s_{xy}}{\sqrt{n-2}} \sqrt{n+1+\frac{(x-m_x)^2}{s_x^2}} \qquad \text{(Eq. F.III-2)}$$

$$s_{xy} = \sqrt{s_y^2 (1 - r_{xy}^2)} \qquad \text{(Eq. F.III-3)}$$

$$r_{xy} = \frac{\sum_{i=0}^{i=n} (x_i - m_x)(y - m_y)}{n\, s_x\, s_y} \qquad \text{(Eq. F.III-4)}$$

where

$\hat{y}$ = the predicted value for the log of the ignition frequency using Equation C-1

x = the log of the corresponding floor area value

n = number of data points used in the linear regression analysis (8 for the upper floor area range)

a = the 1-(α/2) fractile of the t-distribution with n-2 degrees of freedom (for a 95% confidence interval, α is 5% and the value for a is 2.447)

x_i = the x data values (log of floor area)

y_i = the y data values (log of ignition frequency)

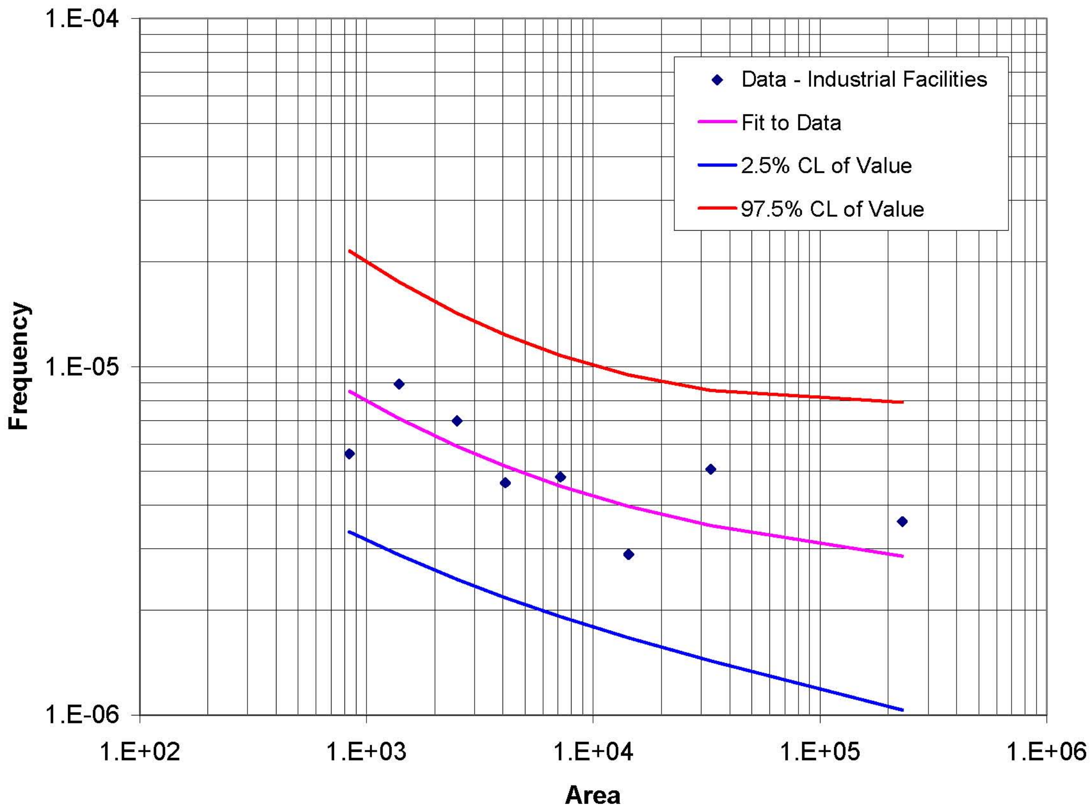
m_x = the mean of the x data values

m_y = the mean of the y data values

s_x = the standard deviation of the x data values

s_y = the standard deviation of the y data values

The upper and lower confidence limits (i.e., the 97.5% and 2.5% values) for any predicted value of the ignition frequency can be determined from Equations F.III-2 through F.III-4 using the x-y data for the upper end of the floor area range. The upper and lower confidence limits for the ignition frequency were then determined by taking the anti-log of the predicted y values. Figure F.III-5 is a plot showing the original data, the predicted values using Equation F.III-1, and the upper and lower confidence limits for the predicted values. The same approach can be used to determine the upper and lower confidence limits for the ignition frequency calculated for each of the YMP facilities. Those results are provided in Table F.III-2.



NOTE: CL = confidence limit.

Source: Original

Figure F.III-5. Plot of the Ignition Frequency Data, the Predicted Ignition Frequency, and Confidence Limits for the Predicted Value

Table F.III-2. Calculated Median and Confidence Limits for the YMP Facility Ignition Frequency

Facility	Ignition Frequency (Ignitions per sq-m per year)		
	Median	**2.5% LCL**	**97.5% UCL**
CRCF	3.78×10^{-6}	1.58×10^{-6}	9.08×10^{-6}
IHF	4.79×10^{-6}	2.02×10^{-6}	1.14×10^{-5}
RF	4.05×10^{-6}	1.70×10^{-6}	9.64×10^{-6}
WHF	3.93×10^{-6}	1.65×10^{-6}	9.39×10^{-6}

NOTE: CRCF = Canister Receipt and Closure Facility; IHF = Initial Handling Facility LCL = lower confidence limit; RF = Receipt Facility; UCL = upper confidence limit; WHF = Wet Handling Facility.

Source: Original

APPENDIX F.IV
PROOF OF LOGNORMAL DISTRIBUTION

The fire initiating event frequencies presented throughout this document are the result of a series of calculations performed using inputs in the form of three different probability distributions. Two of the input distributions (see Appendix II) are normally distributed, and the third (see Appendix III) is lognormally distributed. After the calculations were performed, it was necessary to determine what type of distribution best represented the results. The Crystal Ball output (see Appendix VI) shows the calculated distributions at ten percentile intervals. Crystal Ball also provides the mean and the median of the distributions.

Microsoft Excel has a function, LOGNORMDIST, which can be utilized to calculate the corresponding intervals for a lognormal distribution. The Excel function requires that the log mean (μ) and log standard deviation (σ) be provided. To perform this analysis, it was necessary to calculate μ and σ using Equations F.IV-1 and F.IV-2, where the mean and median in these equations are provided in the Crystal Ball results.

$$\mu = \ln(median) \qquad \text{(Eq. F.IV-1)}$$

$$\sigma = \sqrt{2\ln\left(\frac{mean}{median}\right)} \qquad \text{(Eq. F.IV-2)}$$

A comparison between the Crystal Ball and Excel percentile intervals reveals whether the data is a satisfactory fit to a lognormal distribution. Table F.IV-1 shows the result of this analysis. The table shows that the difference between the Excel calculated values and the Crystal Ball percentile values never exceeds 1 percent. Thus, it is concluded that the fire initiating events are lognormally distributed.

Table F.IV-1. Crystal Ball and Excel Percentile Interval Analysis of Longnormal Distributions

Forecast Values	Excel Calculated Percentiles	Crystal Ball Percentiles	Difference
1.60E-08	0.004	0	0.00
5.37E-08	9.220	10	0.78
6.64E-08	19.19	20	0.81
7.71E-08	29.19	30	0.81
8.77E-08	39.40	40	0.60
9.93E-08	50.00	50	0.00
1.12E-07	60.33	60	0.33
1.27E-07	70.25	70	0.25
1.47E-07	80.18	80	0.18
1.81E-07	90.31	90	0.31
5.64E-07	99.99	100	0.01
Mu	-16.1253	Mean	1.10E-07
Sigma	0.4626	Median	9.93E-08

Source: Original

APPENDIX F.V
DERIVATION OF ERROR FACTORS

It was necessary to provide an error factor (EF) for each initiating event frequency, which was calculated using data provided by Crystal Ball. The software output in Appendix F.VI provides the mean and median necessary to determine the EF. Equation F.V-1 is utilized to calculate the log standard deviation.

$$\sigma = \sqrt{2\ln\left(\frac{mean}{median}\right)} \qquad \text{(Eq. F.V-1)}$$

$$\text{EF} = e^{\sigma \times 1.645} \qquad \text{(Eq. F.V-2)}$$

The resultant EFs for each initiating event frequency are displayed in Table F5.7-7, as well as the mean and median utilized to calculate the EF.

Several of the initiating event frequencies were not utilized as originally anticipated, many were summed for the purpose of developing split fractions. It was necessary to develop EFs for these summed figures as well. This was accomplished by directly summing the figures, then defining the summation as a Crystal Ball forecast value. The Crystal Ball results (Table F5.7-7) provided a mean and median by which the EF can be calculated using Equations F.V-1 and F.V-2.

APPENDIX F.VI
CRYSTAL BALL FULL RESULTS

Initiating Event	97.5% Percentile
Large Fire Threatens TAD or DPC (TTC & VTC) in AO (Diesel Present)	1.4E-05
Large Fire Threatens TAD or DPC (TTC & VTC) in CTM	1.1E-06
Large Fire Threatens TC/DPC (HTC) (Diesel Present)	4.1E-06
Large Fire Threatens TC/DPC (HTC) (No Diesel)	2.6E-05
Large Fire Threatens TC/DPC (TTC) (No Diesel)	3.8E-05
Large Fire Threatens TC/DPC (VTC) (No Diesel)	2.9E-05
Large Fire Threatens TC/TAD (No Diesel)	2.5E-05
Large Fire Threatens TC/TAD or TC/DPC (TTC & VTC) (Diesel Present)	2.0E-06
Localized Fire Threatens TAD or DPC (incl TTC & VTC) in Transfer Room	2.5E-07
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Loading Room (Diesel Present)	7.9E-07
Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Vestibule/Lid Bolting Room (Diesel Present)	1.8E-06
Localized Fire Threatens TC/DPC (HTC) in Preparation Area (No Diesel Present)	1.1E-05
Localized Fire Threatens TC/DPC (HTC) in Vestibule/Preparation Area (Diesel Present)	2.1E-06
Localized Fire Threatens TC/DPC (TTC) in Cask Unloading Room	4.4E-07
Localized Fire Threatens TC/DPC (TTC) in Preparation Area (No Diesel Present)	1.0E-05
Localized Fire Threatens TC/DPC (TTC) in Vestibule/Preparation Area (Diesel Present)	1.0E-06
Localized Fire Threatens TC/DPC (VTC) in Cask Unloading Room	4.4E-07
Localized Fire Threatens TC/DPC (VTC) in Preparation Area (No Diesel Present)	5.5E-06
Localized Fire Threatens TC/DPC (VTC) in Vestibule/Preparation Area (Diesel Present)	1.0E-06
Localized Fire Threatens TC/DPC (VTC, incl TTC) in Preparation Area	4.8E-06
Localized Fire Threatens TC/TAD in Cask Unloading Room	8.7E-07
Localized Fire Threatens TC/TAD in Preparation Area	2.1E-06
Localized Fire Threatens TC/TAD in Preparation Area (No Diesel Present	6.9E-06
Localized Fire Threatens TC/TAD in Vestibule/Preparation Area (Diesel Present)	1.0E-06

NOTE: AO = aging overpack; DPC = dual-purpose canister; HTC = transportation cask in the horizontal position; TAD = transportation, aging, and disposal canister; TTC = transportation cask in the tilted position; VTC = transportation cask in the vertical position.

Source: Crystal Ball ‘extract data’ output.

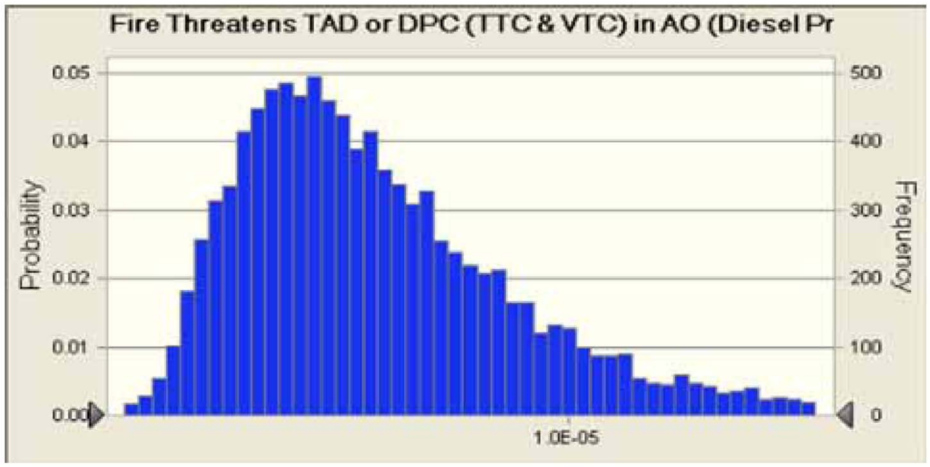
The Crystal Ball report, forecast worksheets, and “assumptions” follow. The term “assumptions” is used by Crystal Ball to denote the probability distributions of the inputs, and does not refer to assumptions as defined by the calculations and analysis procedure.

Worksheet: [RF Fire Frequency_NoSuppression.xls]Initiating Event Frequency

Forecast: Large Fire Threatens TAD or DPC (TTC & VTC) in AO (Diesel Present) Cell: K124

Summary:

Entire range is from 7.5E-07 to 3.1E-05
Base case is 5.6E-06
After 10,000 trials, the std. error of the mean is 3.2E-08



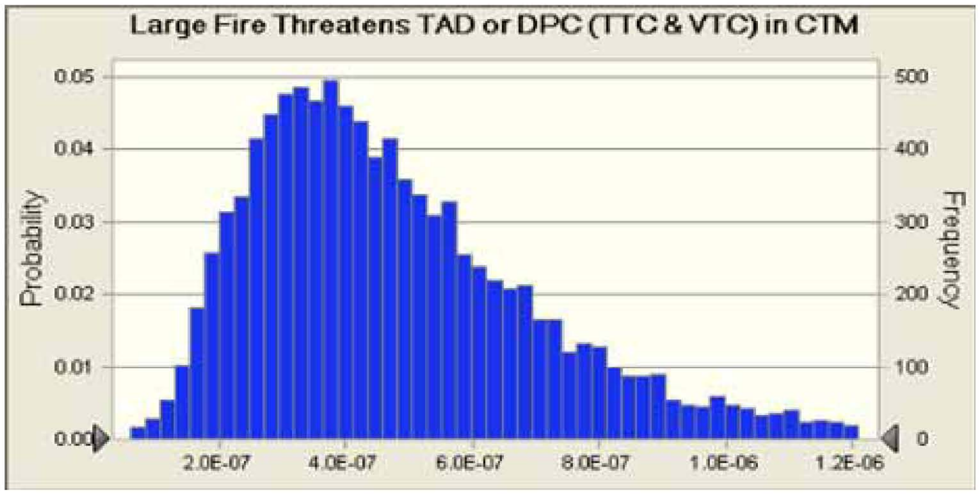
Statistics:	Forecast values
Trials	10,000
Mean	6.1E-06
Median	5.5E-06
Mode	3.0E-06
Standard Deviation	3.2E-06
Variance	1.0E-11
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	7.5E-07
Maximum	3.1E-05
Range Width	3.1E-05
Mean Std. Error	3.2E-08

Forecast: Large Fire Threatens TAD or DPC (TTC & VTC) in AO (Diesel Present) (cont'd) Cell: K124

Percentiles:	Forecast values
0%	7.5E-07
10%	2.8E-06
20%	3.6E-06
30%	4.2E-06
40%	4.8E-06
50%	5.5E-06
60%	6.2E-06
70%	7.1E-06
80%	8.3E-06
90%	1.0E-05
100%	3.1E-05

Forecast: Large Fire Threatens TAD or DPC (TTC & VTC) in CTM **Cell: K123**

Summary:
Entire range is from 6.0E-08 to 2.5E-06
Base case is 4.4E-07
After 10,000 trials, the std. error of the mean is 2.6E-09



Statistics:	Forecast values
Trials	10,000
Mean	4.9E-07
Median	4.4E-07
Mode	2.4E-07
Standard Deviation	2.6E-07
Variance	6.6E-14
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	6.0E-08
Maximum	2.5E-06
Range Width	2.5E-06
Mean Std. Error	2.6E-09

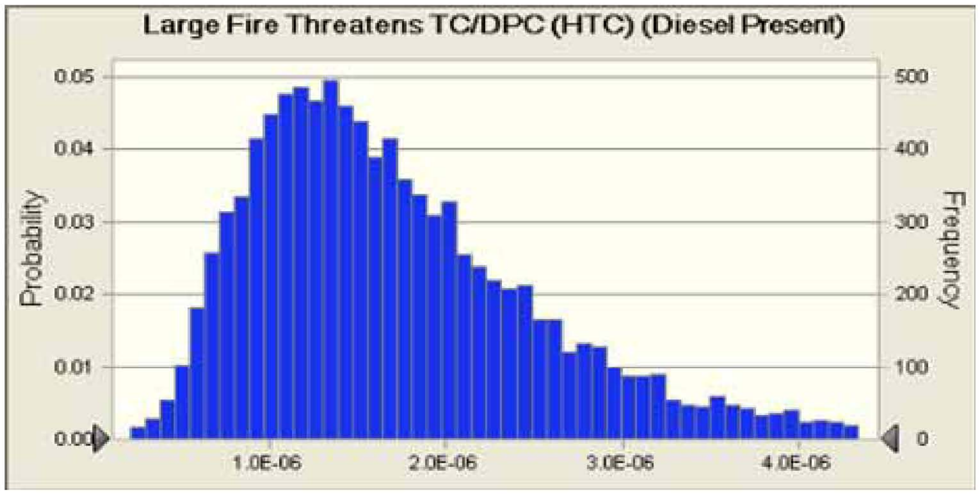
Forecast: Large Fire Threatens TAD or DPC (TTC & VTC) in CTM (cont'd) **Cell: K123**

Percentiles:	Forecast values
0%	6.0E-08
10%	2.3E-07
20%	2.9E-07
30%	3.4E-07
40%	3.8E-07
50%	4.4E-07
60%	5.0E-07
70%	5.7E-07
80%	6.6E-07
90%	8.2E-07
100%	2.5E-06

Forecast: Large Fire Threatens TC/DPC (HTC) (Diesel Present) **Cell: K127**

Summary:

Entire range is from 2.1E-07 to 9.0E-06
Base case is 1.6E-06
After 10,000 trials, the std. error of the mean is 9.2E-09



Statistics:	Forecast values
Trials	10,000
Mean	1.8E-06
Median	1.6E-06
Mode	8.7E-07
Standard Deviation	9.2E-07
Variance	8.4E-13
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	2.1E-07
Maximum	9.0E-06
Range Width	8.8E-06
Mean Std. Error	9.2E-09

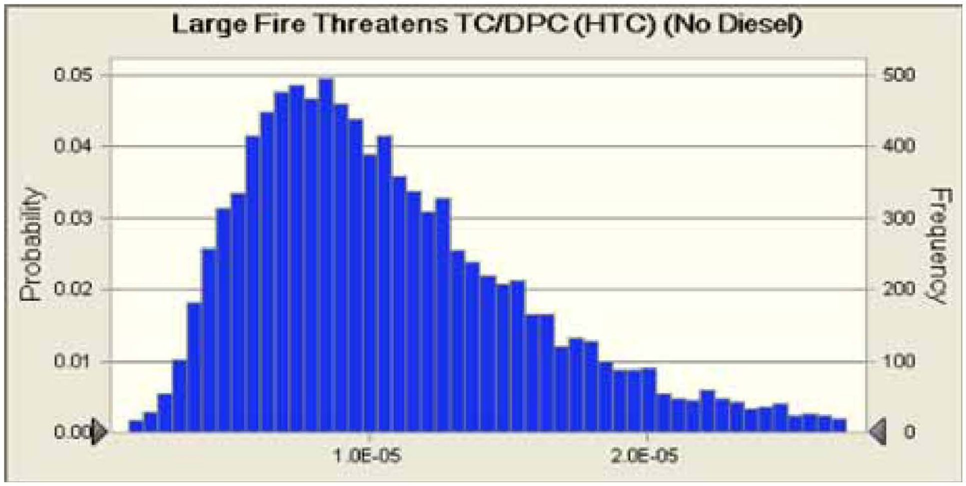
Forecast: Large Fire Threatens TC/DPC (HTC) (Diesel Present) (cont'd) **Cell: K127**

Percentiles:	Forecast values
0%	2.1E-07
10%	8.1E-07
20%	1.0E-06
30%	1.2E-06
40%	1.4E-06
50%	1.6E-06
60%	1.8E-06
70%	2.0E-06
80%	2.4E-06
90%	3.0E-06
100%	9.0E-06

Forecast: Large Fire Threatens TC/DPC (HTC) (No Diesel) **Cell: K128**

Summary:

Entire range is from 1.3E-06 to 5.7E-05
Base case is 1.0E-05
After 10,000 trials, the std. error of the mean is 5.8E-08



Statistics:	Forecast values
Trials	10,000
Mean	1.1E-05
Median	9.8E-06
Mode	5.5E-06
Standard Deviation	5.8E-06
Variance	3.3E-11
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	1.3E-06
Maximum	5.7E-05
Range Width	5.5E-05
Mean Std. Error	5.8E-08

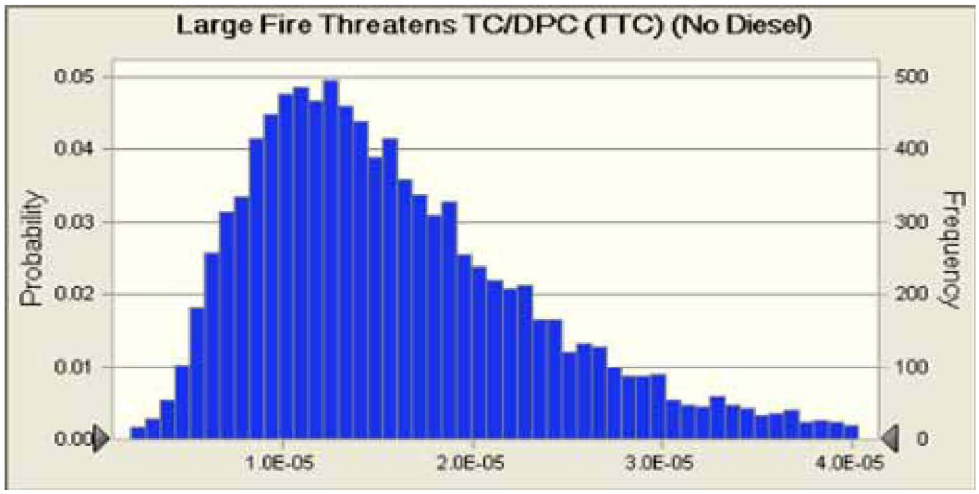
Forecast: Large Fire Threatens TC/DPC (HTC) (No Diesel) (cont'd) **Cell: K128**

Percentiles:	Forecast values
0%	1.3E-06
10%	5.1E-06
20%	6.4E-06
30%	7.5E-06
40%	8.6E-06
50%	9.8E-06
60%	1.1E-05
70%	1.3E-05
80%	1.5E-05
90%	1.9E-05
100%	5.7E-05

Forecast: Large Fire Threatens TC/DPC (TTC) (No Diesel) Cell: K125

Summary:

Entire range is from 2.0E-06 to 8.4E-05
Base case is 1.5E-05
After 10,000 trials, the std. error of the mean is 8.5E-08



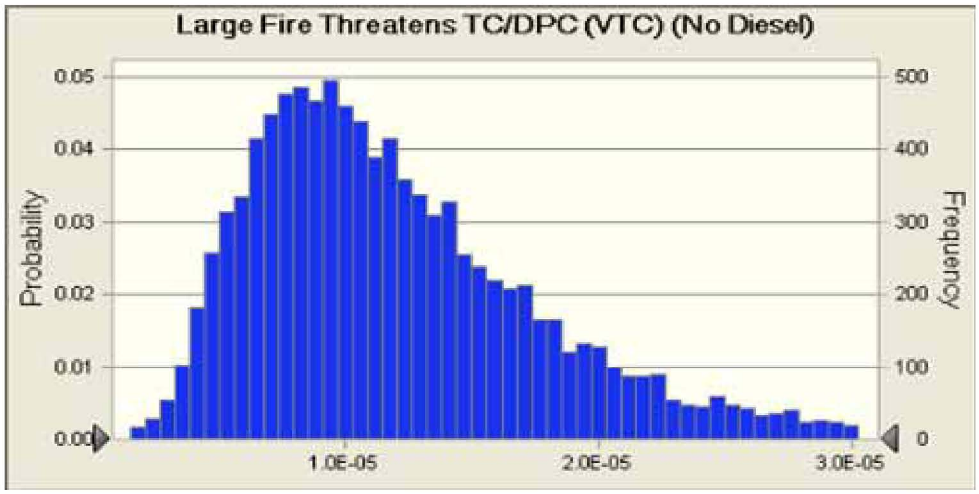
Statistics:	Forecast values
Trials	10,000
Mean	1.6E-05
Median	1.5E-05
Mode	8.1E-06
Standard Deviation	8.5E-06
Variance	7.3E-11
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	2.0E-06
Maximum	8.4E-05
Range Width	8.2E-05
Mean Std. Error	8.5E-08

Forecast: Large Fire Threatens TC/DPC (TTC) (No Diesel) (cont'd) Cell: K125

Percentiles:	Forecast values
0%	2.0E-06
10%	7.6E-06
20%	9.5E-06
30%	1.1E-05
40%	1.3E-05
50%	1.5E-05
60%	1.7E-05
70%	1.9E-05
80%	2.2E-05
90%	2.7E-05
100%	8.4E-05

Forecast: Large Fire Threatens TC/DPC (VTC) (No Diesel) **Cell: K126**

Summary:
Entire range is from 1.5E-06 to 6.3E-05
Base case is 1.1E-05
After 10,000 trials, the std. error of the mean is 6.4E-08



Statistics:	Forecast values
Trials	10,000
Mean	1.2E-05
Median	1.1E-05
Mode	6.1E-06
Standard Deviation	6.4E-06
Variance	4.1E-11
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	1.5E-06
Maximum	6.3E-05
Range Width	6.2E-05
Mean Std. Error	6.4E-08

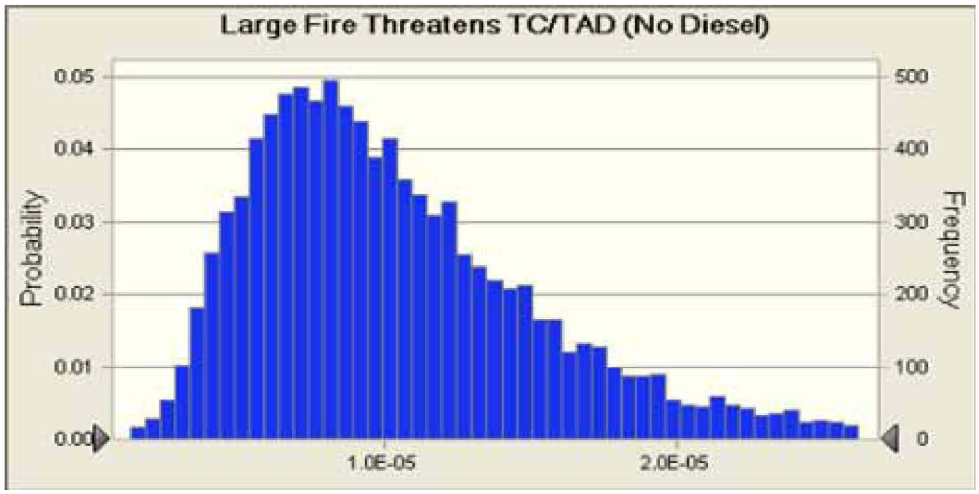
Forecast: Large Fire Threatens TC/DPC (VTC) (No Diesel) (cont'd) **Cell: K126**

Percentiles:	Forecast values
0%	1.5E-06
10%	5.7E-06
20%	7.2E-06
30%	8.4E-06
40%	9.6E-06
50%	1.1E-05
60%	1.2E-05
70%	1.4E-05
80%	1.7E-05
90%	2.1E-05
100%	6.3E-05

Forecast: Large Fire Threatens TC/TAD (No Diesel) **Cell: K122**

Summary:

Entire range is from 1.3E-06 to 5.5E-05
Base case is 9.6E-06
After 10,000 trials, the std. error of the mean is 5.5E-08



Statistics:	Forecast values
Trials	10,000
Mean	1.1E-05
Median	9.5E-06
Mode	5.3E-06
Standard Deviation	5.5E-06
Variance	3.1E-11
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	1.3E-06
Maximum	5.5E-05
Range Width	5.3E-05
Mean Std. Error	5.5E-08

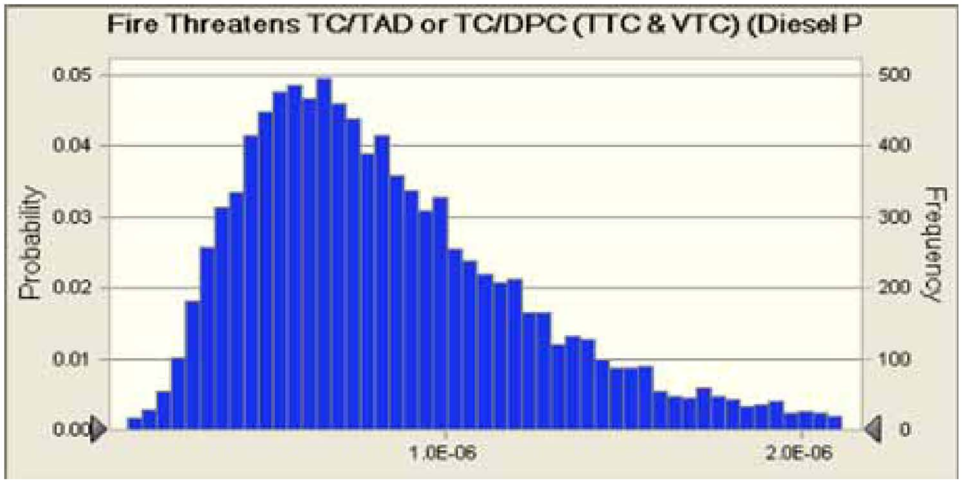
Forecast: Large Fire Threatens TC/TAD (No Diesel) (cont'd) **Cell: K122**

Percentiles:	Forecast values
0%	1.3E-06
10%	4.9E-06
20%	6.2E-06
30%	7.3E-06
40%	8.3E-06
50%	9.5E-06
60%	1.1E-05
70%	1.2E-05
80%	1.4E-05
90%	1.8E-05
100%	5.5E-05

Forecast: Large Fire Threatens TC/TAD or TC/DPC (TTC & VTC) (Diesel Present) Cell: K121

Summary:

Entire range is from 1.0E-07 to 4.4E-06
Base case is 7.8E-07
After 10,000 trials, the std. error of the mean is 4.5E-09



Statistics:	Forecast values
Trials	10,000
Mean	8.6E-07
Median	7.6E-07
Mode	4.3E-07
Standard Deviation	4.5E-07
Variance	2.0E-13
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.5204
Minimum	1.0E-07
Maximum	4.4E-06
Range Width	4.3E-06
Mean Std. Error	4.5E-09

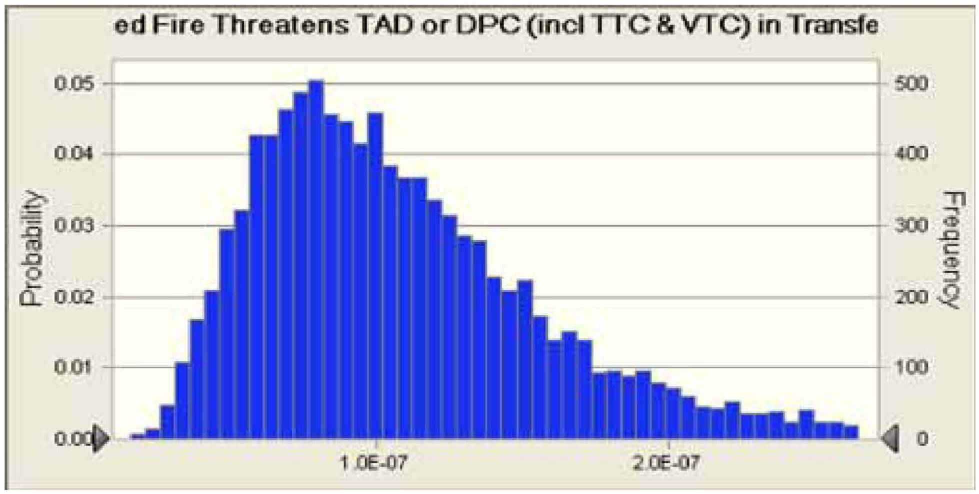
Forecast: Large Fire Threatens TC/TAD or TC/DPC (TTC & VTC) (Diesel Present) Cell: K121 (cont'd)

Percentiles:	Forecast values
0%	1.0E-07
10%	4.0E-07
20%	5.0E-07
30%	5.9E-07
40%	6.7E-07
50%	7.6E-07
60%	8.7E-07
70%	9.9E-07
80%	1.2E-06
90%	1.4E-06
100%	4.4E-06

Forecast: Localized Fire Threatens TAD or DPC (incl TTC & VTC) in Transfer RoomCell: K113

Summary:

Entire range is from 1.6E-08 to 5.6E-07
Base case is 1.0E-07
After 10,000 trials, the std. error of the mean is 5.5E-10



Statistics:	Forecast values
Trials	10,000
Mean	1.1E-07
Median	9.9E-08
Mode	5.1E-08
Standard Deviation	5.5E-08
Variance	3.0E-15
Skewness	1.51
Kurtosis	6.85
Coeff. of Variability	0.4968
Minimum	1.6E-08
Maximum	5.6E-07
Range Width	5.5E-07
Mean Std. Error	5.5E-10

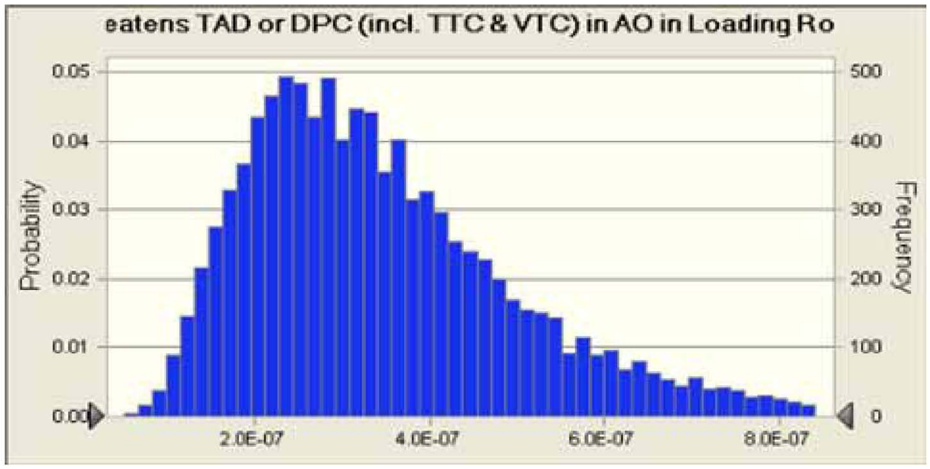
Forecast: Localized Fire Threatens TAD or DPC (incl TTC & VTC) in Transfer RoomCell: K113

Percentiles:	Forecast values
0%	1.6E-08
10%	5.4E-08
20%	6.6E-08
30%	7.7E-08
40%	8.8E-08
50%	9.9E-08
60%	1.1E-07
70%	1.3E-07
80%	1.5E-07
90%	1.8E-07
100%	5.6E-07

Forecast: Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Loading Room (Diesel Present) **Cell: K28**

Summary:

Entire range is from 5.1E-08 to 1.7E-06
Base case is 3.2E-07
After 10,000 trials, the std. error of the mean is 1.7E-09



Statistics:	Forecast values
Trials	10,000
Mean	3.5E-07
Median	3.2E-07
Mode	1.6E-07
Standard Deviation	1.7E-07
Variance	3.0E-14
Skewness	1.49
Kurtosis	6.73
Coeff. of Variability	0.4894
Minimum	5.1E-08
Maximum	1.7E-06
Range Width	1.7E-06
Mean Std. Error	1.7E-09

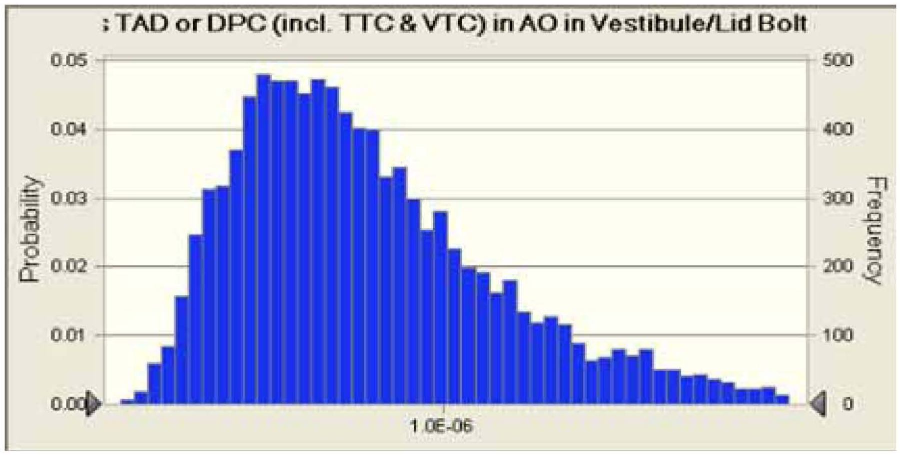
Forecast: Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Loading Room (Diesel Present) (cont'd) **Cell: K28**

Percentiles:	Forecast values
0%	5.1E-08
10%	1.7E-07
20%	2.1E-07
30%	2.5E-07
40%	2.8E-07
50%	3.2E-07
60%	3.6E-07
70%	4.1E-07
80%	4.7E-07
90%	5.8E-07
100%	1.7E-06

Forecast: Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Vestibule/Lid Bolting Room (Diesel Present) **Cell: K16**

Summary:

Entire range is from 1.2E-07 to 3.8E-06
Base case is 7.4E-07
After 10,000 trials, the std. error of the mean is 4.0E-09



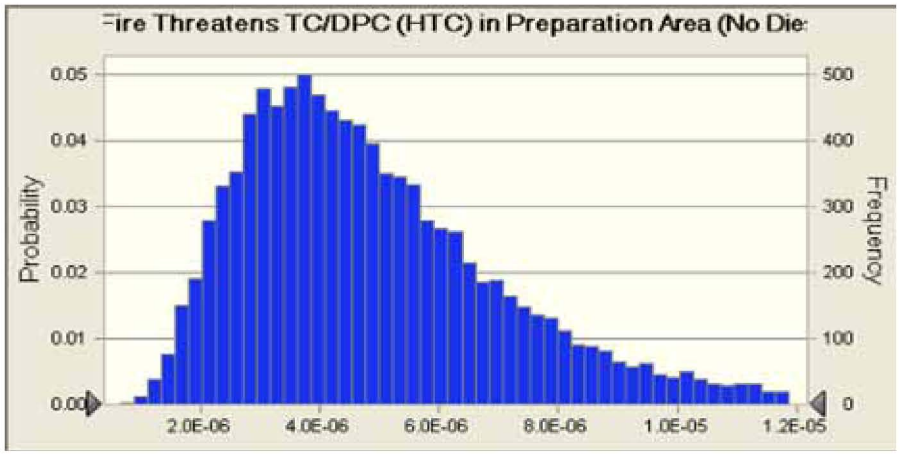
Statistics:	Forecast values
Trials	10,000
Mean	8.1E-07
Median	7.3E-07
Mode	3.8E-07
Standard Deviation	4.0E-07
Variance	1.6E-13
Skewness	1.49
Kurtosis	6.80
Coeff. of Variability	0.4965
Minimum	1.2E-07
Maximum	3.8E-06
Range Width	3.7E-06
Mean Std. Error	4.0E-09

Forecast: Localized Fire Threatens TAD or DPC (incl. TTC & VTC) in AO in Vestibule/Lid Bolting Room (Diesel Present) (cont'd) **Cell: K16**

Percentiles:	Forecast values
0%	1.2E-07
10%	3.9E-07
20%	4.9E-07
30%	5.7E-07
40%	6.5E-07
50%	7.3E-07
60%	8.2E-07
70%	9.3E-07
80%	1.1E-06
90%	1.3E-06
100%	3.8E-06

Forecast: Localized Fire Threatens TC/DPC (HTC) in Preparation Area (No Diesel Present) **Cell: Q64**

Summary:
Entire range is from 6.7E-07 to 2.1E-05
Base case is 4.5E-06
After 10,000 trials, the std. error of the mean is 2.5E-08



Statistics:	Forecast values
Trials	10,000
Mean	5.0E-06
Median	4.5E-06
Mode	2.3E-06
Standard Deviation	2.5E-06
Variance	6.0E-12
Skewness	1.50
Kurtosis	6.66
Coeff. of Variability	0.4912
Minimum	6.7E-07
Maximum	2.1E-05
Range Width	2.0E-05
Mean Std. Error	2.5E-08

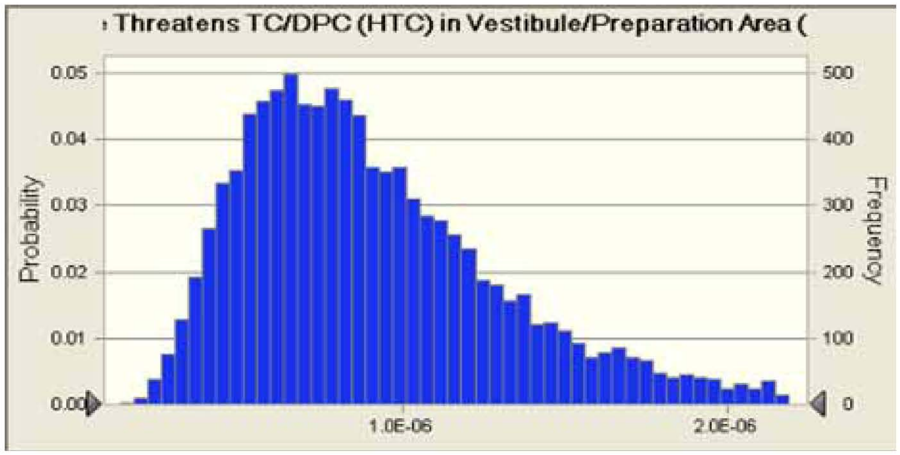
Forecast: Localized Fire Threatens TC/DPC (HTC) in Preparation Area (No Diesel Present) (cont'd) **Cell: Q64**

Percentiles:	Forecast values
0%	6.7E-07
10%	2.5E-06
20%	3.0E-06
30%	3.5E-06
40%	4.0E-06
50%	4.5E-06
60%	5.1E-06
70%	5.7E-06
80%	6.6E-06
90%	8.1E-06
100%	2.1E-05

Forecast: Localized Fire Threatens TC/DPC (HTC) in Vestibule/Preparation Area (Diesel Present) **Cell: Q46**

Summary:

Entire range is from 1.3E-07 to 4.2E-06
Base case is 8.4E-07
After 10,000 trials, the std. error of the mean is 4.5E-09



Statistics:	Forecast values
Trials	10,000
Mean	9.3E-07
Median	8.3E-07
Mode	4.3E-07
Standard Deviation	4.5E-07
Variance	2.0E-13
Skewness	1.48
Kurtosis	6.65
Coeff. of Variability	0.4877
Minimum	1.3E-07
Maximum	4.2E-06
Range Width	4.1E-06
Mean Std. Error	4.5E-09

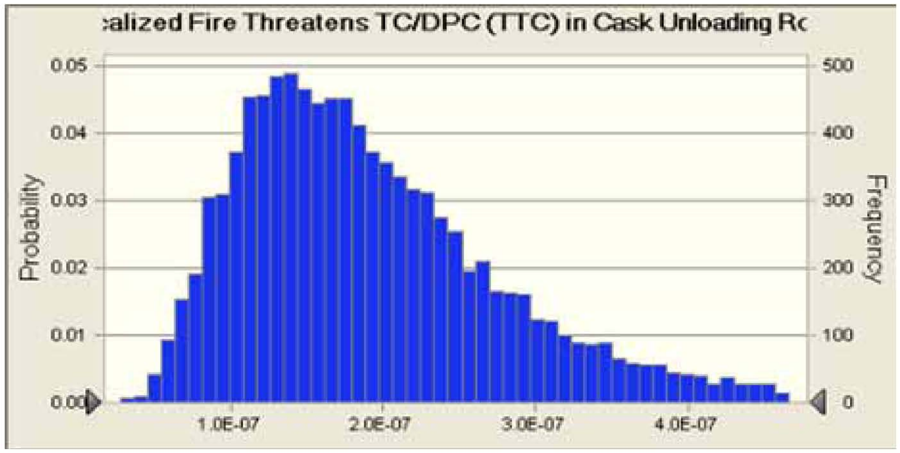
Forecast: Localized Fire Threatens TC/DPC (HTC) in Vestibule/Preparation Area (Diesel Present) (cont'd) **Cell: Q46**

Percentiles:	Forecast values
0%	1.3E-07
10%	4.6E-07
20%	5.6E-07
30%	6.5E-07
40%	7.4E-07
50%	8.3E-07
60%	9.4E-07
70%	1.1E-06
80%	1.2E-06
90%	1.5E-06
100%	4.2E-06

Forecast: Localized Fire Threatens TC/DPC (TTC) in Cask Unloading Room Cell: M100

Summary:

Entire range is from 2.8E-08 to 9.5E-07
Base case is 1.8E-07
After 10,000 trials, the std. error of the mean is 9.6E-10



Statistics:	Forecast values
Trials	10,000
Mean	2.0E-07
Median	1.8E-07
Mode	9.2E-08
Standard Deviation	9.6E-08
Variance	9.3E-15
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.4911
Minimum	2.8E-08
Maximum	9.5E-07
Range Width	9.2E-07
Mean Std. Error	9.6E-10

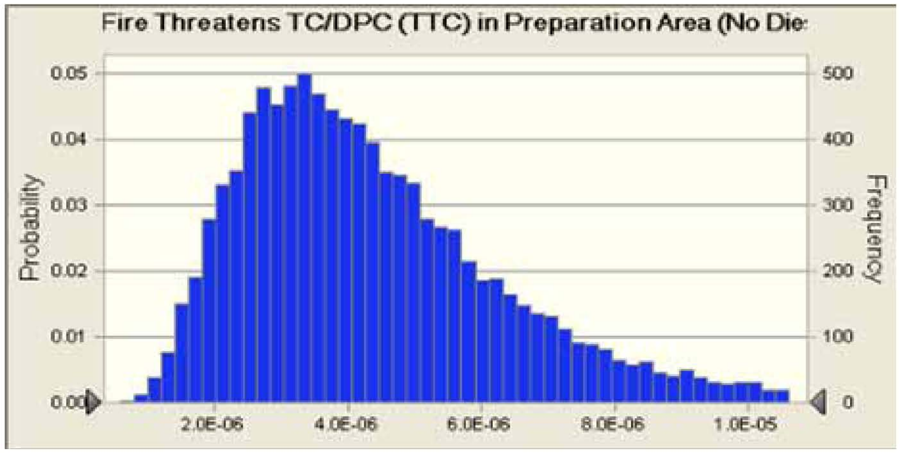
Forecast: Localized Fire Threatens TC/DPC (TTC) in Cask Unloading Room (cont'd) Cell: M100

Percentiles:	Forecast values
0%	2.8E-08
10%	9.6E-08
20%	1.2E-07
30%	1.4E-07
40%	1.6E-07
50%	1.8E-07
60%	2.0E-07
70%	2.3E-07
80%	2.6E-07
90%	3.2E-07
100%	9.5E-07

Forecast: Localized Fire Threatens TC/DPC (TTC) in Preparation Area (No Diesel Cell: M62

Summary:

Entire range is from 6.0E-07 to 1.9E-05
Base case is 4.1E-06
After 10,000 trials, the std. error of the mean is 2.2E-08



Statistics:	Forecast values
Trials	10,000
Mean	4.5E-06
Median	4.0E-06
Mode	2.1E-06
Standard Deviation	2.2E-06
Variance	4.8E-12
Skewness	1.50
Kurtosis	6.66
Coeff. of Variability	0.4912
Minimum	6.0E-07
Maximum	1.9E-05
Range Width	1.8E-05
Mean Std. Error	2.2E-08

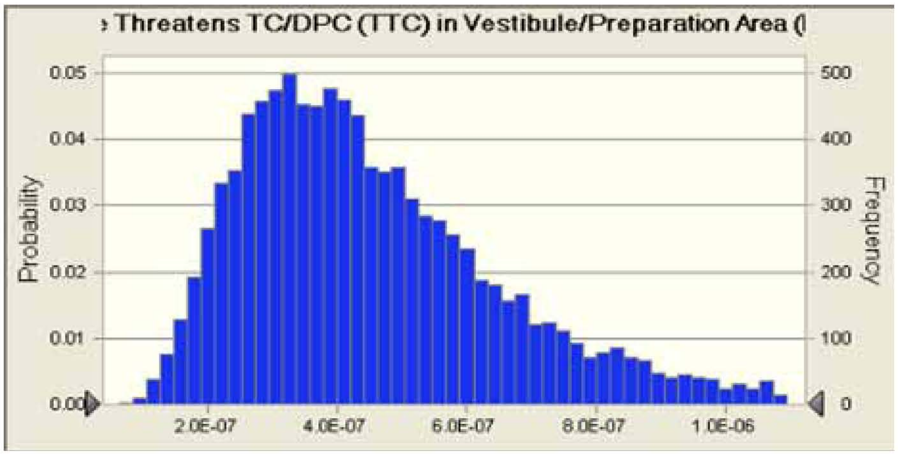
Forecast: Localized Fire Threatens TC/DPC (TTC) in Preparation Area (No Diesel Present) (cont'd) Cell: M62

Percentiles:	Forecast values
0%	6.0E-07
10%	2.2E-06
20%	2.7E-06
30%	3.1E-06
40%	3.6E-06
50%	4.0E-06
60%	4.5E-06
70%	5.1E-06
80%	5.9E-06
90%	7.3E-06
100%	1.9E-05

Forecast: Localized Fire Threatens TC/DPC (TTC) in Vestibule/Preparation Area (Diesel Present) **Cell: M44**

Summary:

Entire range is from 6.3E-08 to 2.1E-06
Base case is 4.2E-07
After 10,000 trials, the std. error of the mean is 2.3E-09



Statistics:	Forecast values
Trials	10,000
Mean	4.6E-07
Median	4.2E-07
Mode	2.1E-07
Standard Deviation	2.3E-07
Variance	5.1E-14
Skewness	1.48
Kurtosis	6.65
Coeff. of Variability	0.4877
Minimum	6.3E-08
Maximum	2.1E-06
Range Width	2.0E-06
Mean Std. Error	2.3E-09

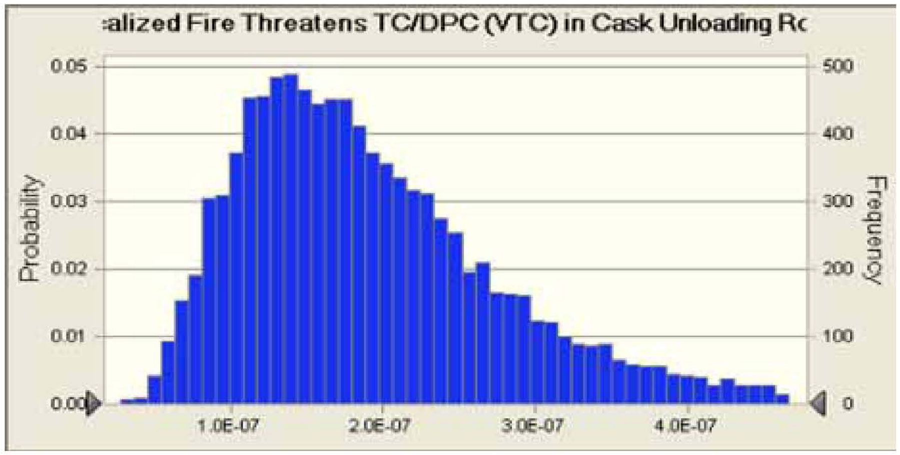
Forecast: Localized Fire Threatens TC/DPC (TTC) in Vestibule/Preparation Area (Diesel Present) (cont'd) **Cell: M44**

Percentiles:	Forecast values
0%	6.3E-08
10%	2.3E-07
20%	2.8E-07
30%	3.3E-07
40%	3.7E-07
50%	4.2E-07
60%	4.7E-07
70%	5.3E-07
80%	6.1E-07
90%	7.5E-07
100%	2.1E-06

Forecast: Localized Fire Threatens TC/DPC (VTC) in Cask Unloading Room **Cell: O101**

Summary:

Entire range is from 2.8E-08 to 9.5E-07
Base case is 1.8E-07
After 10,000 trials, the std. error of the mean is 9.6E-10



Statistics:	Forecast values
Trials	10,000
Mean	2.0E-07
Median	1.8E-07
Mode	9.2E-08
Standard Deviation	9.6E-08
Variance	9.3E-15
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.4911
Minimum	2.8E-08
Maximum	9.5E-07
Range Width	9.2E-07
Mean Std. Error	9.6E-10

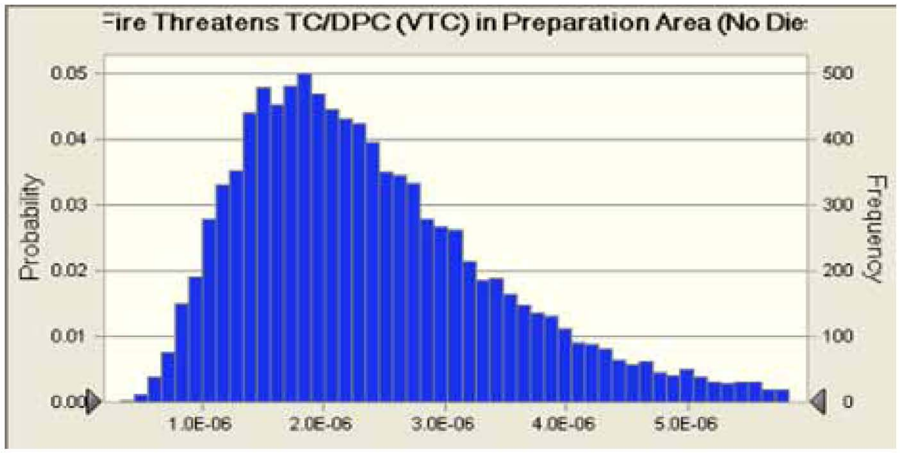
Forecast: Localized Fire Threatens TC/DPC (VTC) in Cask Unloading Room (cont'd) **Cell: O101**

Percentiles:	Forecast values
0%	2.8E-08
10%	9.6E-08
20%	1.2E-07
30%	1.4E-07
40%	1.6E-07
50%	1.8E-07
60%	2.0E-07
70%	2.3E-07
80%	2.6E-07
90%	3.2E-07
100%	9.5E-07

Forecast: Localized Fire Threatens TC/DPC (VTC) in Preparation Area (No Diesel Present) **Cell: O63**

Summary:

Entire range is from 3.3E-07 to 1.0E-05
Base case is 2.2E-06
After 10,000 trials, the std. error of the mean is 1.2E-08



Statistics:	Forecast values
Trials	10,000
Mean	2.5E-06
Median	2.2E-06
Mode	1.1E-06
Standard Deviation	1.2E-06
Variance	1.5E-12
Skewness	1.50
Kurtosis	6.66
Coeff. of Variability	0.4912
Minimum	3.3E-07
Maximum	1.0E-05
Range Width	1.0E-05
Mean Std. Error	1.2E-08

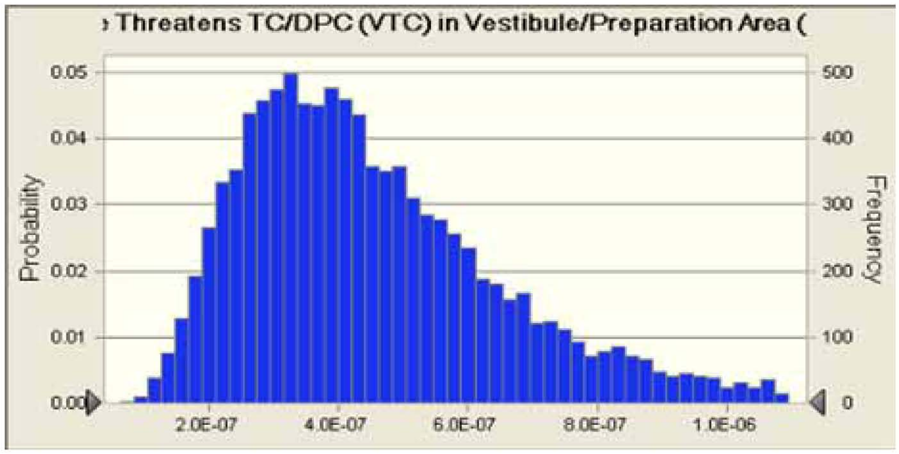
Forecast: Localized Fire Threatens TC/DPC (VTC) in Preparation Area (No Diesel Present) (cont'd) **Cell: O63**

Percentiles:	Forecast values
0%	3.3E-07
10%	1.2E-06
20%	1.5E-06
30%	1.7E-06
40%	2.0E-06
50%	2.2E-06
60%	2.5E-06
70%	2.8E-06
80%	3.3E-06
90%	4.0E-06
100%	1.0E-05

Forecast: Localized Fire Threatens TC/DPC (VTC) in Vestibule/Preparation Area (Diesel Present) **Cell: O45**

Summary:

Entire range is from 6.3E-08 to 2.1E-06
Base case is 4.2E-07
After 10,000 trials, the std. error of the mean is 2.3E-09



Statistics:	Forecast values
Trials	10,000
Mean	4.6E-07
Median	4.2E-07
Mode	2.1E-07
Standard Deviation	2.3E-07
Variance	5.1E-14
Skewness	1.48
Kurtosis	6.65
Coeff. of Variability	0.4877
Minimum	6.3E-08
Maximum	2.1E-06
Range Width	2.0E-06
Mean Std. Error	2.3E-09

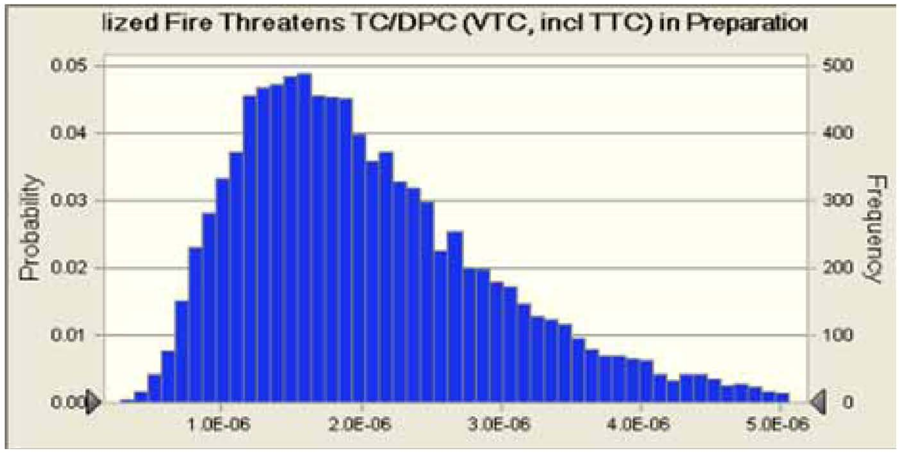
Forecast: Localized Fire Threatens TC/DPC (VTC) in Vestibule/Preparation Area (Diesel Present) (cont'd) **Cell: O45**

Percentiles:	Forecast values
0%	6.3E-08
10%	2.3E-07
20%	2.8E-07
30%	3.3E-07
40%	3.7E-07
50%	4.2E-07
60%	4.7E-07
70%	5.3E-07
80%	6.1E-07
90%	7.5E-07
100%	2.1E-06

Forecast: Localized Fire Threatens TC/DPC (VTC, incl TTC) in Preparation Area Cell: M82

Summary:

Entire range is from 2.8E-07 to 9.3E-06
Base case is 1.9E-06
After 10,000 trials, the std. error of the mean is 1.1E-08



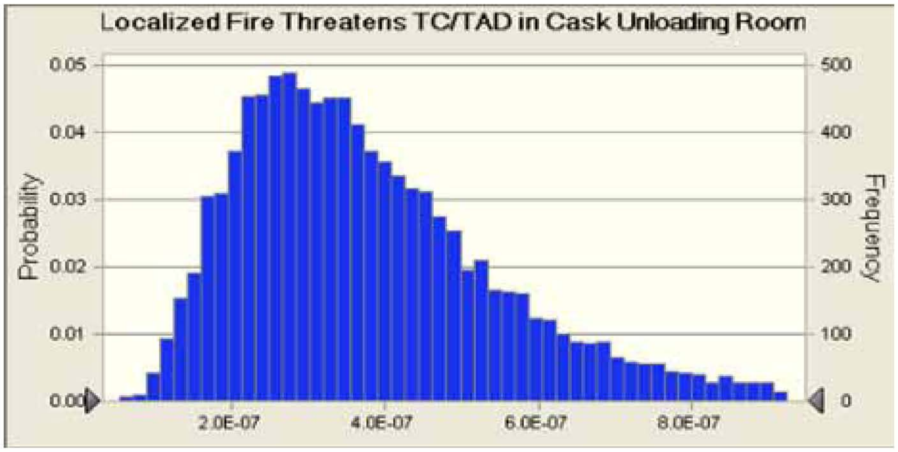
Statistics:	Forecast values
Trials	10,000
Mean	2.1E-06
Median	1.9E-06
Mode	9.6E-07
Standard Deviation	1.1E-06
Variance	1.1E-12
Skewness	1.52
Kurtosis	6.78
Coeff. of Variability	0.5005
Minimum	2.8E-07
Maximum	9.3E-06
Range Width	9.0E-06
Mean Std. Error	1.1E-08

Forecast: Localized Fire Threatens TC/DPC (VTC, incl TTC) in Preparation Area (cont'd) Cell: M82

Percentiles:	Forecast values
0%	2.8E-07
10%	1.0E-06
20%	1.3E-06
30%	1.5E-06
40%	1.7E-06
50%	1.9E-06
60%	2.1E-06
70%	2.4E-06
80%	2.8E-06
90%	3.5E-06
100%	9.3E-06

Forecast: Localized Fire Threatens TC/TAD in Cask Unloading Room **Cell: K99**

Summary:
Entire range is from 5.5E-08 to 1.9E-06
Base case is 3.5E-07
After 10,000 trials, the std. error of the mean is 1.9E-09



Statistics:	Forecast values
Trials	10,000
Mean	3.9E-07
Median	3.5E-07
Mode	1.8E-07
Standard Deviation	1.9E-07
Variance	3.6E-14
Skewness	1.49
Kurtosis	6.69
Coeff. of Variability	0.4911
Minimum	5.5E-08
Maximum	1.9E-06
Range Width	1.8E-06
Mean Std. Error	1.9E-09

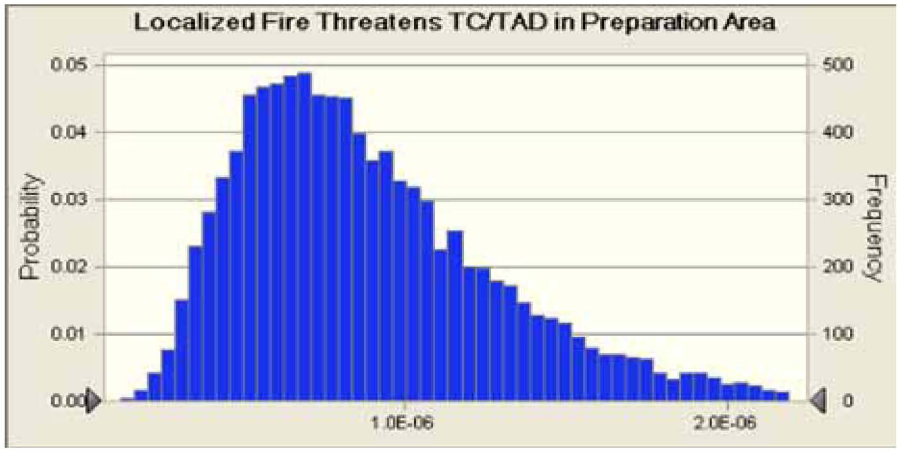
Forecast: Localized Fire Threatens TC/TAD in Cask Unloading Room (cont'd) **Cell: K99**

Percentiles:	Forecast values
0%	5.5E-08
10%	1.9E-07
20%	2.3E-07
30%	2.7E-07
40%	3.1E-07
50%	3.5E-07
60%	3.9E-07
70%	4.5E-07
80%	5.2E-07
90%	6.3E-07
100%	1.9E-06

Forecast: Localized Fire Threatens TC/TAD in Preparation Area **Cell: K81**

Summary:

Entire range is from 1.2E-07 to 4.0E-06
Base case is 8.3E-07
After 10,000 trials, the std. error of the mean is 4.6E-09



Statistics:	Forecast values
Trials	10,000
Mean	9.1E-07
Median	8.1E-07
Mode	4.2E-07
Standard Deviation	4.6E-07
Variance	2.1E-13
Skewness	1.52
Kurtosis	6.78
Coeff. of Variability	0.5005
Minimum	1.2E-07
Maximum	4.0E-06
Range Width	3.9E-06
Mean Std. Error	4.6E-09

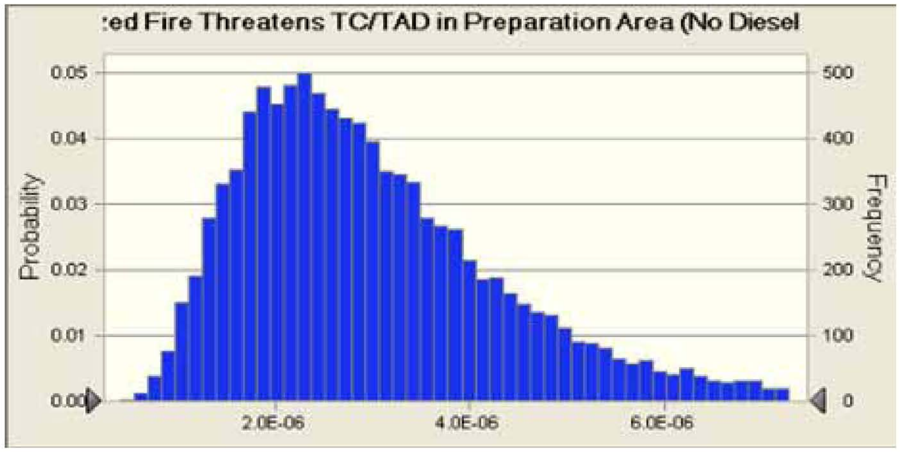
Forecast: Localized Fire Threatens TC/TAD in Preparation Area (cont'd) **Cell: K81**

Percentiles:	Forecast values
0%	1.2E-07
10%	4.4E-07
20%	5.5E-07
30%	6.4E-07
40%	7.2E-07
50%	8.1E-07
60%	9.2E-07
70%	1.0E-06
80%	1.2E-06
90%	1.5E-06
100%	4.0E-06

Forecast: Localized Fire Threatens TC/TAD in Preparation Area (No Diesel Present Cell: K61

Summary:

Entire range is from 4.1E-07 to 1.3E-05
Base case is 2.8E-06
After 10,000 trials, the std. error of the mean is 1.5E-08



Statistics:	Forecast values
Trials	10,000
Mean	3.1E-06
Median	2.8E-06
Mode	1.4E-06
Standard Deviation	1.5E-06
Variance	2.3E-12
Skewness	1.50
Kurtosis	6.66
Coeff. of Variability	0.4912
Minimum	4.1E-07
Maximum	1.3E-05
Range Width	1.3E-05
Mean Std. Error	1.5E-08

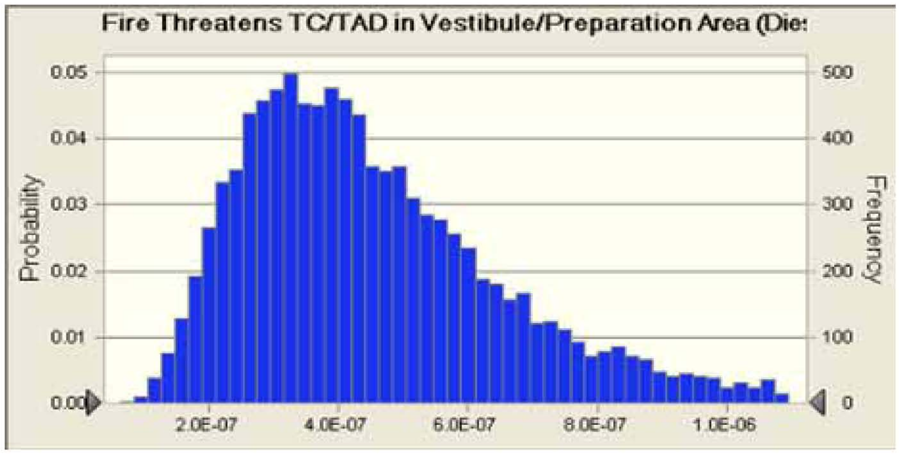
Forecast: Localized Fire Threatens TC/TAD in Preparation Area (No Diesel Present Cell: K61

Percentiles:	Forecast values
0%	4.1E-07
10%	1.5E-06
20%	1.9E-06
30%	2.2E-06
40%	2.4E-06
50%	2.8E-06
60%	3.1E-06
70%	3.5E-06
80%	4.1E-06
90%	5.0E-06
100%	1.3E-05

Forecast: Localized Fire Threatens TC/TAD in Vestibule/Preparation Area (Diesel Present) **Cell: K43**

Summary:

Entire range is from 6.3E-08 to 2.1E-06
Base case is 4.2E-07
After 10,000 trials, the std. error of the mean is 2.3E-09



Statistics:

	Forecast values
Trials	10,000
Mean	4.6E-07
Median	4.2E-07
Mode	2.1E-07
Standard Deviation	2.3E-07
Variance	5.1E-14
Skewness	1.48
Kurtosis	6.65
Coeff. of Variability	0.4877
Minimum	6.3E-08
Maximum	2.1E-06
Range Width	2.0E-06
Mean Std. Error	2.3E-09

Forecast: Localized Fire Threatens TC/TAD in Vestibule/Preparation Area (Diesel Present) (cont'd) **Cell: K43**

Percentiles:

	Forecast values
0%	6.3E-08
10%	2.3E-07
20%	2.8E-07
30%	3.3E-07
40%	3.7E-07
50%	4.2E-07
60%	4.7E-07
70%	5.3E-07
80%	6.1E-07
90%	7.5E-07
100%	2.1E-06

End of Forecasts

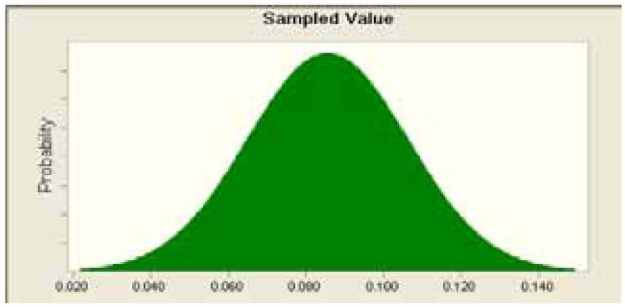
Assumptions

Worksheet: [RF Fire Frequency_NoSuppression.xls]Ignition Source Frequency

Assumption: Sampled Value **Cell: H2**

Normal distribution with parameters:

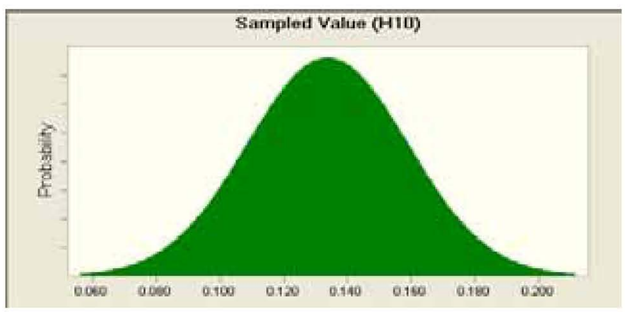
Mean	0.086	(=I2)
97.5%	0.126	(=J2)



Assumption: Sampled Value (H10) **Cell: H10**

Normal distribution with parameters:

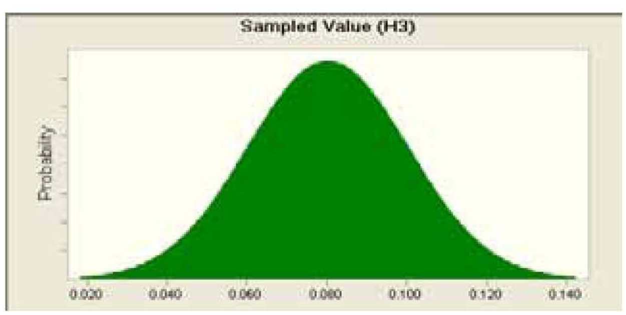
Mean	0.134	(=I10)
97.5%	0.183	(=J10)



Assumption: Sampled Value (H3) **Cell: H3**

Normal distribution with parameters:

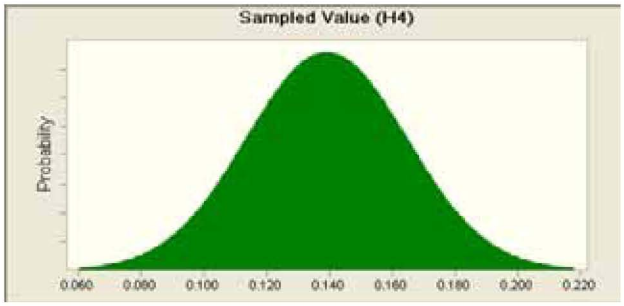
Mean	0.080	(=I3)
97.5%	0.120	(=J3)



Assumption: Sampled Value (H4) **Cell: H4**

Normal distribution with parameters:

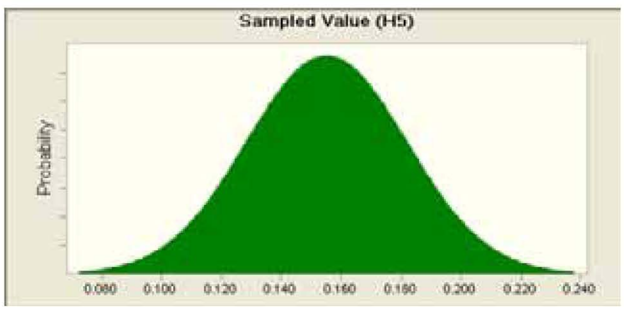
Mean	0.139	(=I4)
97.5%	0.189	(=J4)



Assumption: Sampled Value (H5) **Cell: H5**

Normal distribution with parameters:

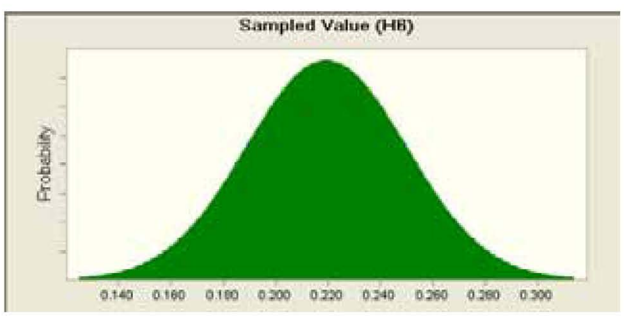
Mean	0.155	(=I5)
97.5%	0.207	(=J5)



Assumption: Sampled Value (H6) **Cell: H6**

Normal distribution with parameters:

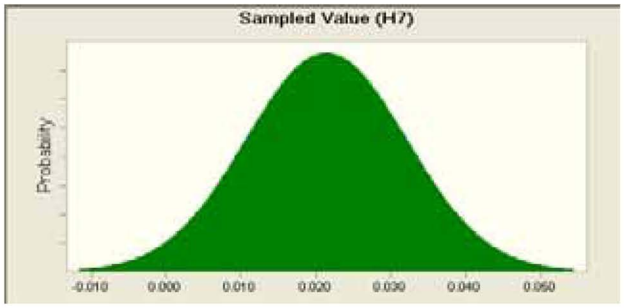
Mean	0.219	(=I6)
97.5%	0.279	(=J6)



Assumption: Sampled Value (H7) **Cell: H7**

Normal distribution with parameters:

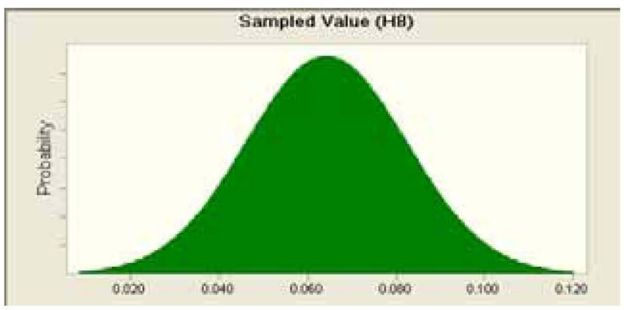
Mean	0.021	(=I7)
97.5%	0.042	(=J7)



Assumption: Sampled Value (H8) **Cell: H8**

Normal distribution with parameters:

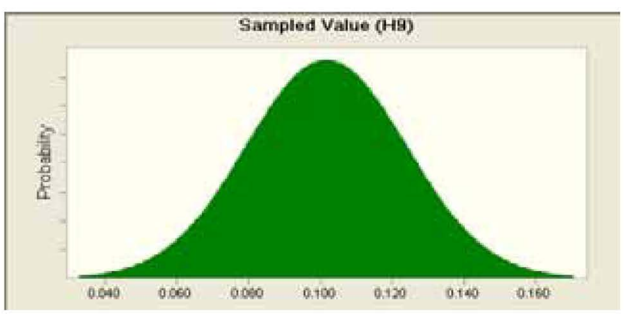
Mean	0.064	(=I8)
97.5%	0.100	(=J8)



Assumption: Sampled Value (H9) **Cell: H9**

Normal distribution with parameters:

Mean	0.102	(=I9)
97.5%	0.145	(=J9)

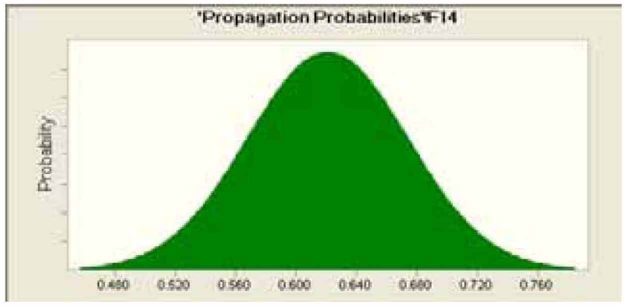


Worksheet: [RF Fire Frequency_NoSuppression.xls]Propagation Probabilities

Assumption: F14 **Cell: F14**

Normal distribution with parameters:

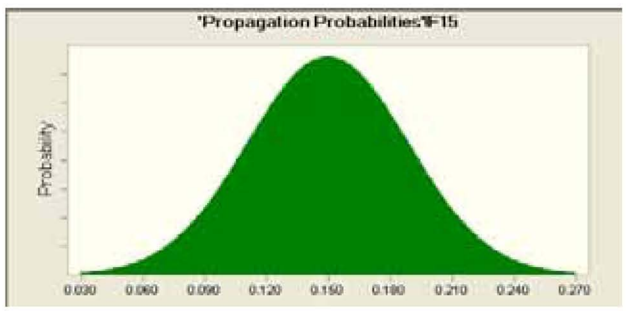
Mean	0.621	(=G14)
97.5%	0.725	(=H14)



Assumption: F15 **Cell: F15**

Normal distribution with parameters:

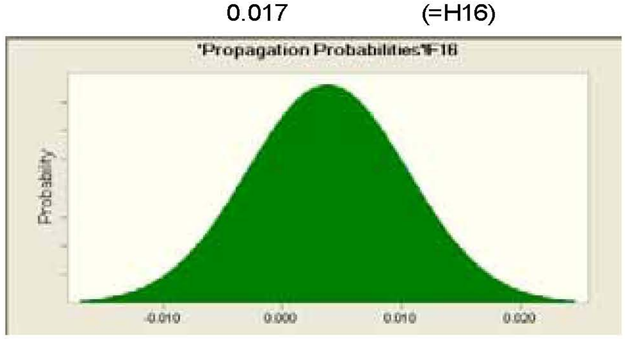
Mean	0.149	(=G15)
97.5%	0.226	(=H15)



Assumption: F16 **Cell: F16**

Normal distribution with parameters:

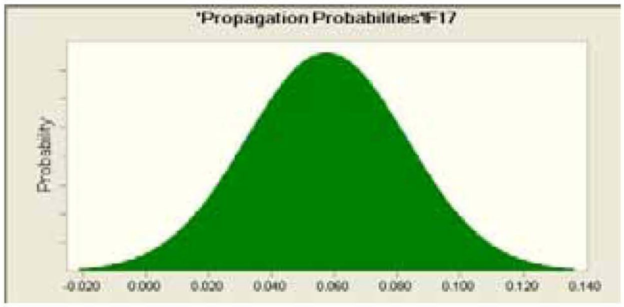
Mean	0.004	(=G16)
97.5%	0.017	(=H16)



Assumption: F17 **Cell: F17**

Normal distribution with parameters:

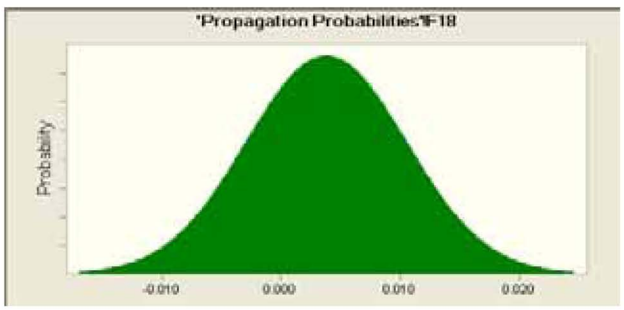
Mean	0.057	(=G17)
97.5%	0.107	(=H17)



Assumption: F18 **Cell: F18**

Normal distribution with parameters:

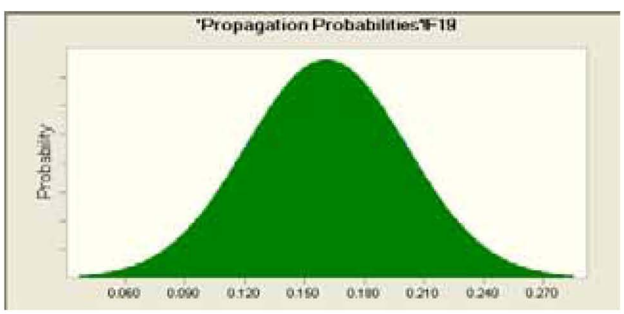
Mean	0.004	(=G18)
97.5%	0.017	(=H18)



Assumption: F19 **Cell: F19**

Normal distribution with parameters:

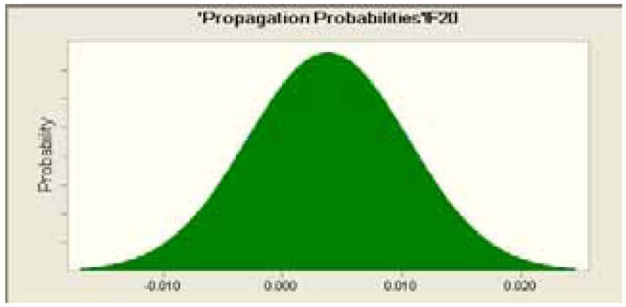
Mean	0.161	(=G19)
97.5%	0.240	(=H19)



Assumption: F20 **Cell: F20**

Normal distribution with parameters:

Mean	0.004	(=G20)
97.5%	0.017	(=H20)

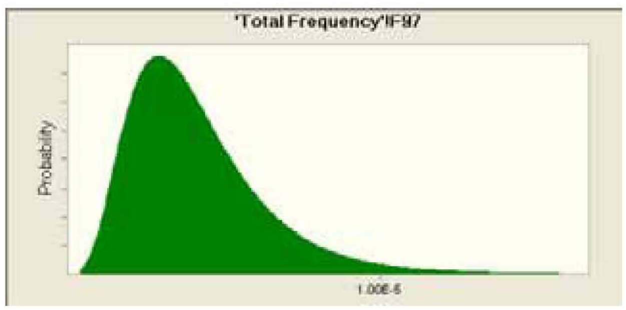


Worksheet: [RF Fire Frequency_NoSuppression.xls]Total Frequency

Assumption: F97 **Cell: F97**

Lognormal distribution with parameters:

50%	4.05E-6	(=G97)
97.5%	9.64E-6	(=I97)



End of Assumptions

NOTE:

Source: Crystal Ball software output.

ATTACHMENT G
EVENT SEQUENCE QUANTIFICATION SUMMARY TABLES

ATTACHMENT G
EVENT SEQUENCE QUANTIFICATION SUMMARY TABLES

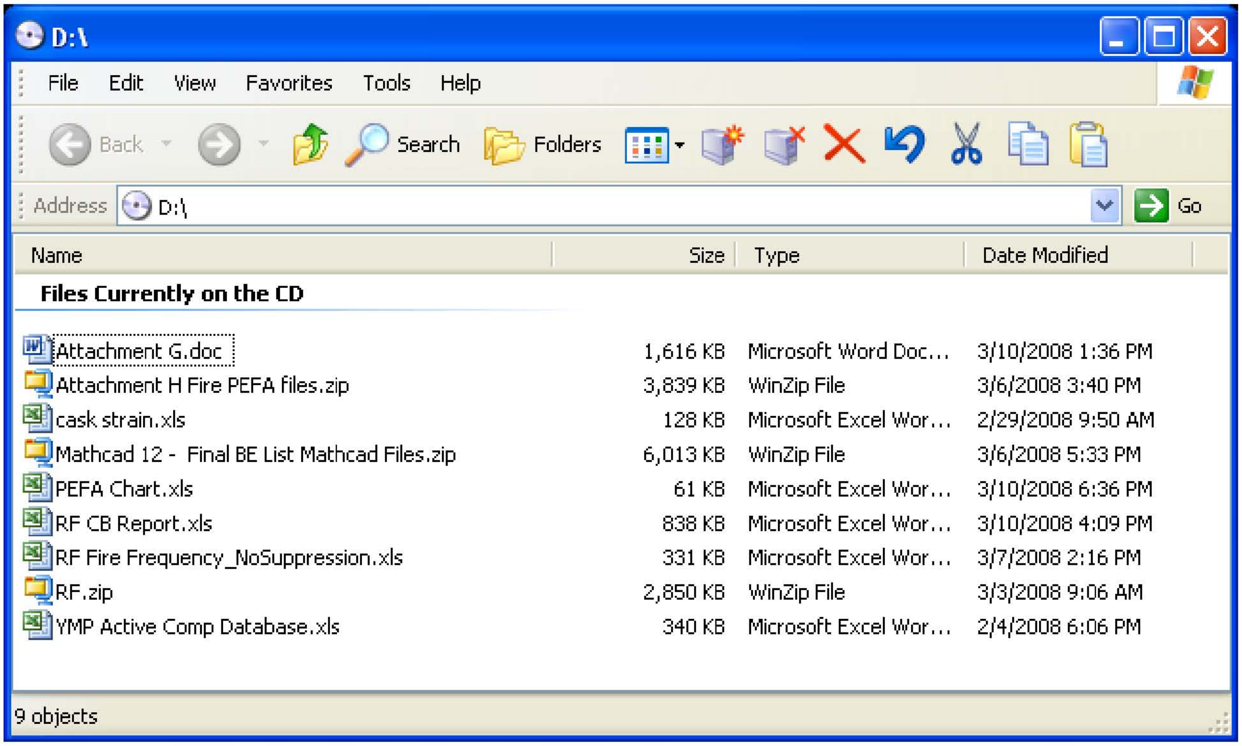
Attachment G contains the event sequence quantification summary table (Table G-1) referenced by Section 6.7. It also contains Table G-2, *Final Event Sequence Summary*; Table G-3, *Beyond Category 2 Final Event Sequences Summary*; and Table G-4, *Important to Criticality Final Event Sequences Summary* that are referenced in Section 6.8. Cells in these tables with 0.00E+00 indicate that the value is <E-12.

This attachment can be found on the CD in Attachment H, in a file named Attachment G.doc.

ATTACHMENT H
SAPHIRE MODEL AND SUPPORTING FILES

ATTACHMENT H
SAPHIRE MODEL AND SUPPORTING FILES

This attachment is the CD containing the SAPHIRE model and supporting files. The electronic files contained on the CD are identified below.



OFFICE OF CIVILIAN RADIOACTIVE WASTE MANAGEMENT
SPECIAL INSTRUCTION SHEET

1. QA: QA

This is a placeholder page for records that cannot be scanned.

2. Record Date	3. Accession Number
3/11/08	ATT. TO: ENG.20080312.0030
4. Author Name(s)	**5. Authorization Organization**
NORMAN GRAVES	BSC/PCSA

6. Title/Description

RECEIPT FACILITY RELIABILITY AND EVENT SEQUENCE CATEGORIZATION ANALYSIS (ATTACHMENT H)

7. Document Number(s)	8. Version Designator
200-PSA-RF00-00200-000	00A

9. Document Type	10. Medium
DATA	2 CD'S

11. Access Control Code

PUB

12. Traceability Designator

200-PSA-RF00-00200-000-00A

13. Comments

1 ORIGINAL
1 COPY

VALIDATION OF COMPLETE FILE TRANSFER. ALL FILES COPIED. SOFTWARE USED SAPHIRE, MS EXCEL, MSWORD, AND WINZIP.

14. RPC Electronic Media Verification

XREF MOL.20080312.0016

MAR 14 2008 TChurch /BSC

MD5 Validation

THIS IS AN ELECTRONIC ATTACHMENT

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 Volume in drive D is 080311_1048
 Volume Serial Number is 21AC-6079

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03/06/2008  03:40p           3,930,232 Attachment H Fire PEFA files.zip
02/29/2008  09:50a             130,560 cask strain.xls
03/06/2008  05:33p           6,157,289 Mathcad 12 -  Final BE List Mathcad Files.zip
03/10/2008  06:36p              62,464 PEFA Chart.xls
03/10/2008  04:08p             858,112 RF CB Report.xls
03/07/2008  02:16p             338,432 RF Fire Frequency_NoSuppression.xls
03/03/2008  09:06a           2,918,034 RF.zip
02/04/2008  06:06p             347,648 YMP Active Comp Database.xls

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03/06/2008  05:33p           6,157,289 Mathcad 12 -  Final BE List Mathcad Files.zip
03/10/2008  06:36p              62,464 PEFA Chart.xls
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02/04/2008  06:06p             347,648 YMP Active Comp Database.xls

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03/07/2008  02:16p             338,432 RF Fire Frequency_NoSuppression.xls
03/03/2008  09:06a           2,918,034 RF.zip
02/04/2008  06:06p             347,648 YMP Active Comp Database.xls
              27 File(s)     49,192,665 bytes

     Total Files Listed:
              27 File(s)     49,192,665 bytes
               0 Dir(s)               0 bytes free
```

BSC	**Calculation/Analysis Change Notice** *Complete only applicable items.*	1. QA: QA 2. Page 1 of 1

ENG.20080313.0001

3. Document Identifier:	4. Rev.:	5. CACN:
200-PSA-RF00-00200-000-00A	00A	001

6. Title:
Receipt Facility Reliability and Event Sequence Categorization Analysis

7. Reason for Change:
Provide additional clarification of the results and conclusions concerning the HVAC system,

8. Supersedes Change Notice:	☐ Yes If, Yes, CACN No.: ______	☒ No

9. Change Impact:

Inputs Changed:	☐ Yes	☒ No	Results Impacted:	☐ Yes	☒ No
Assumptions Changed:	☐ Yes	☒ No	Design Impacted:	☐ Yes	☒ No

10. Description of Change:
The following paragraph is added as the second paragraph in Section 7.

As stated in Section 2.2, the PCSA is based on a snapshot of the design. At the time the analysis was performed the design inputs referred to an ITS HVAC confinement system. The results of the analysis show that there are no Category 1 event sequences (Table 6.8-2), and there are no Category 2 event sequences that result in a radionuclide release (Table 6.8-3 and Table 7-2). Therefore, it is concluded that the confinement function of the HVAC system is not required to be ITS.

11. REVIEWS AND APPROVAL

Printed Name	Signature	Date
11a. Originator: Norman Graves	[signature]	3/13/08
11b. Checker: Kathy Ashley	[signature]	3/13/08
11c. EGS: Michael Frank	[signature]	3/13/08
11d. DEM: Mark Wisenburg	[signature]	3/13/2008
11e. Design Authority: Barbara Rusinko R.J. Tosetti	[signature]	3/13/08

Kla 3/13/08